April 26, 2000

Mr. Craig G. Anderson Vice President, Operations ANO Entergy Operations, Inc. 1448 S. R. 333 Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 2 - ISSUANCE OF AMENDMENT RE:

ELIMINATION OF UNNECESSARY MODE RESTRICTIONS ON SELECTED

TECHNICAL SPECIFICATIONS (TAC NO. MA8289)

Dear Mr. Anderson:

The Commission has issued the enclosed Amendment No. 214 to Facility Operating License No. NPF-6 for the Arkansas Nuclear One, Unit No. 2 (ANO-2). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 24, 2000.

The amendment revises TS 4.4.11 on reactor coolant system vent flow verification, TS 4.6.1.1.a on containment penetration closure verification (non-automatic), and TS 4.6.3.1.2 on containment isolation valve actuation verification. The changes eliminate unnecessary mode restrictions on these surveillance requirements.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Thomas W. Alexion, Project Manager, Section 1 Project Directorate IV & Decommissioning Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosures:

1. Amendment No. 214 to NPF-6

2. Safety Evaluation

cc w/encls: See next page

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DISTRIBUTION Docket No. 50-368

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W.Beckner Enclosures: PUBLIC 1. Amendment No. to NPF-6 PDIV-1 RF K.Brockman.RIV 2. Safety Evaluation G.Hill(2) L.Hurley,RIV

ACRS J.Kilcrease,RIV

cc w/encls: See next page OGC *SRXB request removal from

> concurrence, and work exclusively with RTSB, per F. Akstulewicz

03/14/00 e-mail

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Package No: ML003707604

Admendment No: ML003707505 OFFICIAL RECORD COPY TS Pages: ML003707505

Arkansas Nuclear One

CC:

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ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 214 License No. NPF-6

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated February 24, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations:
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-6 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No., 214 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Robert A. Gramm, Chief, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical

Specifications

Date of Issuance: April 26, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 214

FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	<u>Insert</u>
3/4 4-27 3/4 6-1	3/4 4-27 3/4 6-1
3/4 6-17	3/4 6-17

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 214 TO

FACILITY OPERATING LICENSE NO. NPF-6

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 2

DOCKET NO. 50-368

1.0 INTRODUCTION

By letter dated February 24, 2000, Entergy Operations, Inc. (the licensee), submitted a request for changes to the Arkansas Nuclear One, Unit No. 2, Technical Specifications (TSs). The requested changes would revise TS 4.4.11 on reactor coolant system vent flow verification, TS 4.6.1.1.a on containment penetration closure verification (non-automatic), and TS 4.6.3.1.2 on containment isolation valve actuation verification. These TS surveillances require testing to be performed during Modes 5 and/or 6. The proposed changes would eliminate unnecessary mode restrictions on these surveillance requirements.

2.0 DISCUSSION

TS 4.4.11 requires that each reactor coolant system vent path be demonstrated operable at least once per 18 months by verifying flow through the reactor coolant vent system vent paths during cold shutdown. The licensee proposes to remove the phrase "during cold shutdown" to obtain additional flexibility on when the surveillance is performed. The Nuclear Regulatory Commission (the Commission or NRC) staff notes that this change is not inconsistent with NUREG-1432, Revision 1 (Improved Standard Technical Specifications (ISTS), Combustion Engineering Plants, April 1995), since the ISTS do not have TSs for reactor coolant system vents.

TS 4.6.1.1.a requires that containment penetrations consisting of valves, blind flanges, and deactivated automatic valves which are located inside containment and are locked, sealed, or otherwise secured in the closed position, be verified closed during each cold shutdown, except that such verification need not be performed more often than once per 92 days. The licensee proposes to replace the phrase "during each cold shutdown" with "prior to entering Mode 4 from Mode 5," to obtain additional flexibility on when the surveillance is performed. The NRC staff notes that this change is consistent with the ISTS.

TS 4.6.3.1.2 requires that each containment isolation valve shall be demonstrated operable during the cold shutdown or refueling mode at least once per 18 months by verifying that on a

containment isolation test signal, each isolation valve actuates to its isolation position. The licensee proposes to remove the phrase "during the cold shutdown or refueling mode" to obtain additional flexibility on when the surveillance is performed. The NRC staff notes that this change is consistent with the ISTS.

3.0 EVALUATION

Deletion of the mode restrictions discussed above will not relieve the licensee of the responsibility for prudent planning, scheduling, and performance of testing activities, and may provide the licensee with lower-risk periods of opportunity for test performance. Also, the control of plant conditions appropriate to perform a surveillance test is typically an issue for procedures and scheduling. The inclusion of routine procedural guidance has previously been determined to be unnecessary as a TS restriction. As indicated in Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle," allowing this procedural control is consistent with the vast majority of other surveillance requirements that do not dictate specific plant conditions for surveillances.

Since the responsibility of the licensee in these matters remains unchanged by the proposed changes, the proposed changes should not significantly affect the plant or personnel safety. Based on the above, the proposed changes are found to be acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 15379, 03/22/00). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Alexion

Date: April 26, 2000