



**Duke Power Company**  
*A Duke Energy Company*

EC07H  
526 South Church Street  
P.O. Box 1006  
Charlotte, NC 28201-1006

**M. S. Tuckman**  
*Executive Vice President  
Nuclear Generation*

(704) 382-2200 OFFICE  
(704) 382-4360 FAX

April 17, 2000

U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555-001

ATTENTION: Document Control Desk

Subject: Duke Energy Corporation  
Catawba Nuclear Station, Units 1 and 2  
Docket Numbers 50-413 and 50-414  
McGuire Nuclear Station, Units 1 and 2  
Docket Number 50-369 and 50-370  
Implementation of Best-Estimate Large Break  
LOCA Methodology


Reference: 1) WCAP-12945-P-A, Volume 1 (Revision 2) and  
Volumes 2 through 5 (Revision 1), "Code  
Qualification Document for Best-Estimate  
Loss-of-Coolant Accident Analysis," March  
1998.

2) M. S. Tuckman (Duke) Letter to the  
Document Control Desk, April 10, 2000,  
Subject: Implementation of Best-Estimate  
Large Break LOCA Methodology

Reference 2 initially transmitted a Duke Energy Corporation  
submittal addressing the subject identified above.  
However, this April 10, 2000 letter did not include the  
necessary attachment. This letter includes the attachment.  
Duke Energy Corporation regrets this error.

Please address any comments or questions regarding this  
matter to J. S. Warren at (704) 382-4986.

Very truly yours,

  
M. S. Tuckman

ADD 1/1

U. S. Nuclear Regulatory Commission  
April 17, 2000  
Page 2

xc w/Attachment:

L. A. Reyes, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
Atlanta Federal Center  
61 Forsyth St., SW, Suite 23T85  
Atlanta, GA 30303

F. Rinaldi, Project Manager  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Mail Stop O-8 H12  
Washington, D. C. 20555-0001

C. P. Patel, Project Manager  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Mail Stop O-8 H12  
Washington, D. C. 20555-0001

S. M. Shaeffer  
NRC Senior Resident Inspector  
McGuire Nuclear Station

D. J. Roberts  
NRC Senior Resident Inspector  
Catawba Nuclear Station

U. S. Nuclear Regulatory Commission

April 17, 2000

Page 3

bxc w/Attachment:

G. B. Swindlehurst

R. C. Harvey

C. J. Thomas

M. T. Cash

K. L. Crane

G. D. Gilbert

K. E. Nicholson

T. K. Pasour (2)

J. S. Warren

ELL

## Attachment

### McGuire/Catawba Nuclear Stations Best-Estimate Large Break LOCA Model Development

The best estimate large break LOCA (BE LBLOCA) analysis for the McGuire and Catawba Units, incorporating a 1% higher power level, will be performed using a bounding unit approach, similar to what has been done for Farley and Diablo Canyon. In this approach, a WCOBRA/TRAC model is developed choosing bounding inputs for the plant configuration. Where the bounding direction is not known, sensitivity studies are performed to determine the limiting direction.

The differences between the four units will be divided into vessel and loop, as detailed below.

#### Vessel

Two vessel models will be built to capture the differences in the upper internals.

- McGuire Unit 1, with (14) 15x15 guide tubes
- Other three units, with (6) 15x15 guide tubes

Other minor differences will be bounded in the two vessel models as follows:

1. Barrel/baffle: All units are upflow, but the baffle plates and bypass flow fraction are different between them. A conservative composite approach will be used to model this area, including use of:
  - Thickest of the baffle plates (increases boiling rate, which decreases core flooding rate)
  - Maximum barrel/baffle volume (corresponding to thinnest plates, which decreases water available for core reflow)
  - Higher bypass flow (decreases water available for core reflow)

## Attachment

2. Cold leg nozzle loss coefficient (forward flow): Maximum value among the four units will be used which reduces the safety injection flow rate (higher injection pressure).
3. Balance of vessel: A bounding approach is used for other minor differences, similar to discussion of barrel/baffle region above. This approach will:
  - Maximize vessel volume where liquid is not available for core cooling, such as the lower plenum
  - Minimize vessel volume where liquid is available for core cooling, such as the upper head
  - Maximize metal mass

Based on these two vessel models, a limiting vessel will be chosen based on analysis results and determination of the phenomenological differences, which led to those results. Studies will then continue with the determination of limiting loop configuration.

### Loops

The major differences between the loops for the four units are the accumulators and steam generators. For the limiting vessel model determined above, the following studies will be performed to determine the limiting configuration.

1. Accumulator line friction (L/D): The highest and lowest values will be analyzed to determine the limiting direction.
2. Accumulator pressure: Base transient will use a nominal pressure. The range of pressures to encompass all units will be included in the initial condition uncertainty calculations, so it will not be considered here.
3. Accumulator water volume: Base transient will use the minimum nominal water volume. The high nominal value will be analyzed to determine the limiting direction.

## Attachment

4. Steam Generators: The two types of steam generators (W and BWC) will be analyzed with the limiting vessel to determine the bounding type.

### Limiting Composite Plant

At the completion of the loop sensitivity studies, a limiting composite plant configuration, which includes the limiting vessel model along with the limiting loop configuration, will be determined. The choice of limiting configuration will again be based on results, combined with an understanding of the phenomena that led to the results. This composite model will be used to perform a final composite initial transient. This model will form the basis for the remainder of the BE LBLOCA analysis. Other minor differences in plant initial conditions will be addressed in the initial conditions run matrix by ranging the parameters to bound all four units.

### Transition Core Effects

The transition from Framatome to Westinghouse fuel will be addressed with a separate evaluation, similar to that performed for Point Beach. Two additional calculations will be performed to determine the effects of the transition core. One calculation will use a fresh Westinghouse assembly surrounded by once-burned (or more) Framatome assemblies. The second calculation will use a once burned Framatome assembly surrounded by Westinghouse assemblies.