PSEG

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Nuclear Business Unit

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for **March 2000** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is a summary of changes, tests, and experiments that were implemented through **March 2000.**

Sincerely,

for

Mark B. Bezilla
Vice President - Operations

RAR
Attachments

C Distribution

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IE24

The power is in your hands.

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DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 4/10/00
COMPLETED BY: F. Todd
TELEPHONE: (856) 339-1316

Reporting Period **March 2000**

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)	**1067**		
Maximum Dependable Capacity (MWe-Net)	**1031**		
	Month	**Year-to-date**	**Cumulative**
No. of hours reactor was critical	**744**	**2184**	**98911**
No. of hours generator was on line (service hours)	**744**	**2184**	**97331**
Unit reserve shutdown hours	**0**	**0**	**0**
Net Electrical Energy (MWH)	**713411**	**2186975**	**98485574**

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H - Other

(2) Method

1 - Manual
2 - Manual Trip/Scram
3 - Automatic Trip/Scram
4 - Continuation
5 - Other (Explain)

DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 4/10/00
COMPLETED BY: R. Ritzman
TELEPHONE: (856) 339-1445

Summary Of Monthly Operating Experience

- Hope Creek entered the month of March at approximately 100% reactor power.
- On March 10, power was reduced for Hydraulic Control Unit and "C" Feedwater Heater maintenance. Power was restored to approximately 100% on March 23.
- At the end of the month, Hope Creek completed 212 days of continuous power operation.

DOCKET NO.:50-354
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COMPLETED BY:R. Ritzman
TELEPHONE: (856) 339-1445

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION

MONTH March 2000

The following items completed during **March 2000** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

4EC-3658, Package 2, Plant Historian System/Thermal Monitor. This design change installed redundant Plant Historian hardware and software and added thermal monitoring functionality to the Plant Historian system. This design change also added new computer points to provide additional Control Room Indication Display System (CRIDS) indications. The computer points are shown on plant instrumentation and piping drawings; therefore, a safety evaluation was required.

The Plant Historian/Thermal Monitor is not safety related. All connections to safety systems are through optical isolation via CRIDS. The new system does not perform any control functions or impact any existing control functions. This design change does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION – Cont'd

Temporary Modifications Summary of Safety Evaluations

There were no reportable changes in this category implemented during March 2000.

Procedures Summary of Safety Evaluations

HC.OP-SO.AF-0001, Revision 16, Extraction Steam System Operation. This procedure revision allows the automatic control function of the feedwater heater extraction steam isolation valves to be temporarily isolated. The revision also allows the condensate bypass valves to be throttled open to allow warm-up of the feedwater heaters. These changes allow for a gradual heatup of the feedwater heaters prior to placing them in service.

The Extraction Steam system is not safety related and does not affect any safety related systems or components. This procedure revision does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

UFSAR Change Notice HCN 98-027, Operation of the Turbine Building Circulating Water Dewatering Sump with Known Concentrations of Tritium from Associated Supply Ventilation Condensation Drains. This change notice identifies the supply Heating, Ventilation, and Air Conditioning Condensation Drainage Systems as potentially being contaminated by tritium. This change notice also specifies the radioactive sampling and analysis requirements for the circulating water dewatering sump discharges.

The operation of the circulating water dewatering sump with known inputs of tritium does not affect the operation of any safety system, does not affect any accident initiators, and does not affect any safety related equipment or components. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notice HCN 99-044, Operation of Primary Containment Instrument Gas After a Loss of Offsite Power. This change notice allows the compressor suction to be manually transferred from the drywell to the Reactor Building air space if the drywell temperature increases enough to affect Primary Containment Instrument Gas (PCIG) temperatures following a Loss of Offsite Power (LOP).

This change will increase the reliability of PCIG operation after a LOP by avoiding high temperature PCIG trips. This change notice does not affect the operation of any safety

system, does not affect any accident initiators, and does not affect any safety related equipment or components. Therefore this change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notice H99-067, Spent Fuel Water Chemistry Parameters. This design change limits the Spent Fuel Pool water chemistry sampling and analysis requirements to those suggested by industry guidelines and fuel warranty specifications.

This change notice modifies the Spent Fuel Water Chemistry sampling and analysis requirements, but maintains the water chemistry within the appropriate chemistry limits for safe fuel pool operating conditions. This change notice does not affect the operation of any safety system, does not affect any accident initiators, and does not affect any safety related equipment or components. Therefore this change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notice H00-003, Receipt, Inspection, Channeling, and Storage of New Fuel. This change notice revises the methodology used for the receipt of new fuel. This change notice addresses the impact on the entire process from the entry of trucks into the Protected Area to temporary storage in the Spent Fuel Pool pending transport to the reactor core. The most significant change involves the use of a new tool and the use of the New Fuel Vault.

Prior to being placed in the reactor core, new fuel does not perform a safety related function or impact the ability of the plant to safely shutdown. Damage to a fuel assembly cannot cause a significant release of radiation until after it is irradiated in the reactor core. Therefore, this change notice does not affect the operation of any safety system, does not affect any accident initiators, and does not affect any safety related equipment or components. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

UFSAR Change Notice H00-009, Shutdown Margin Demonstration. This change notice clarifies the technical rationale and describes the available methods for satisfying the Shutdown Margin Limiting Condition for Operation during refueling. This change notice explicitly allows for the demonstration of Shutdown Margin using analytical methods for in-vessel fuel movements prior to control rod withdrawals. This change notice also clarifies the usage of the shorting links and clarifies when control rod withdrawal is required to measure shutdown margin.

This change notice provided clarification only and did not change the allowable methods for conducting a Shutdown Margin demonstration. Therefore, this change notice does not affect the operation of any safety system, does not affect any accident initiators, and does not affect any safety related equipment or components. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and does not involve an Unreviewed Safety Question.

Deficiency Reports Summary of Safety Evaluations

Notification 80007888, Residual Heat Removal Heat Exchanger Safety Auxiliaries Cooling System Outlet Valve Rubber Seat Removal. This safety evaluation supports a "use-as-is" disposition of the Residual Heat Removal (RHR) Heat Exchanger Safety Auxiliaries Cooling System (SACS) outlet valve without a rubber seat. The use of this valve without the rubber seat required a safety evaluation to account for the higher leak rate.

The safety evaluation concluded that the removal of the valve seat on the RHR Heat Exchanger SACS Outlet Valve does not affect the operation of the valve, make it more susceptible to failure, or increase the probability or consequences of an accident or malfunction. This "use-as-is" disposition does not reduce the margin of safety and does not involve an Unreviewed Safety Question.