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Section 50.90

U S Nuclear Regulatory Commission
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Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Pressure and Temperature Limits Report (PTLR) Revision 1

Attached is an approved copy of Revision 1 to the Pressure and Temperature Limits Report (PTLR) for the Prairie Island Nuclear Generating Plant (PINGP). This revision is submitted in accordance with the provisions of Technical Specification 6.6.F.3.

This revision incorporated the following six changes:

- (1) Revised the discussion of surveillance data credibility.
- (2) References 5.6 and 5.7 were updated to incorporate the findings from the Comprehensive Revised Response to Generic Letter 92-01, which was submitted to the NRC on November 10, 1999.
- (3) Unit 2 values of I, the Initial Reference Temperature, and ART, the Adjusted Reference Temperature, for the Intermediate Shell Forging (C) and the Lower Shell Forging (D) were revised in Table 6.5 to reflect the information incorporated into the updated Reference 5.6 and 5.7 documents.
- (4) The title of the operating limit "Temperature for Disabling both Safety Injection Pumps" was changed to "Safety Injection (SI) Pump Disable Temperature" in preparation for the pending license amendment request to convert to Improved Technical Specifications.
- (5) The titles of Tables 6.1 and 6.2 were changed to match the corresponding titles listed in the PTLR Table of Contents.
- (6) The titles of Figures 6.1 and 6.2 in the PTLR Table of Contents were changed to match the titles on the figures.

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Also attached are copies of the updated PTLR reference 5.6 (WCAP-14637) and reference 5.7 (WCAP-14638) discussed in change (2) above.

If you have any questions related to this submittal, please contact John Stanton at 651-388-1121 x4083.



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Plant Manager
Prairie Island Nuclear Generating Plant

Attachments:

1. Pressure and Temperature Limits Report, Revision 1, March 30, 2000.
 2. WCAP-14637, Prairie Island Unit 2 Heatup and Cooldown Limit Curves Normal Operation, Revision 3, December 1999.
 3. WCAP-14638, Evaluation of Pressurized Thermal Shock for Prairie Island Unit 2, Revision 3, December 1999.
- c: Regional Administrator -- III, NRC
 NRR Project Manager, NRC
 Senior Resident Inspector, NRC
 James Bernstein, State of Minnesota
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