



Nebraska Public Power District

Nebraska's Energy Leader

NLS2000041

April 8, 2000

U.S. Nuclear Regulatory Commission

Attention: Document Control Desk

Washington, D.C. 20555-0001

Gentlemen:

Subject: Maximum Extended Load Line Limit/Increased Core Flow Results for Peak Drywell Temperature - Supplemental Information to NLS2000017
Cooper Nuclear Station, NRC Docket 50-298, DPR-46

- References:**
1. Letter No. NLS2000017 to USNRC Document Control Desk from John H. Swailes dated February 15, 2000, "Maximum Extended Load Line Limit Analysis (MELLLA) Submittal of Final Report"
 2. Letter No. NLS990115 to USNRC Document Control Desk from John H. Swailes dated December 15, 1999, "Proposed Change to CNS Technical Specifications; Change to Average Power Range Monitors (APRMs) Neutron Flux-High (Flow Biased) Allowable Value"

On February 15, 2000, the Nebraska Public Power District submitted the Maximum Extended Load Line Limit/Increased Core Flow (MELLL/ICF) Final Report (Reference 1). The report was submitted to support a requested amendment to Operating License DPR-46 to change the Cooper Nuclear Station (CNS) Technical Specifications (TS) (Reference 2). This proposed TS change would revise the Average Power Range Monitors Neutron Flux-High (Flow Biased) allowable value based on a revised Power to Flow Map. The revised Power to Flow Map extends the current plant operating domain to above the rated rod line, to within an envelope referred to as the Maximum Extended Load Line Limit, and adds the Increased Core Flow region.

Section 5.0 of the MELLL/ICF Final Report evaluates containment (drywell and wetwell) response to a postulated Loss of Coolant Accident while operating in the MELLL/ICF region. Calculated drywell pressure and temperature, and wetwell pressure, at various operating points (percent power and percent core flow) were compared to design values for the containment. The calculated maximum peak drywell temperature was determined to be acceptable as containment structural loads remain within limits and because the calculated peak temperature was bounded by the maximum allowable drywell airspace temperature of 309°F, contained in CNS TS Bases Section B 3.6.1.5.

Cooper Nuclear Station

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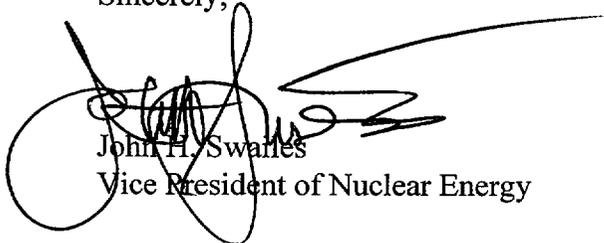
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Upon further investigation it has been determined that the value of 309°F contained in CNS TS Bases may require revision and it is being evaluated within our Corrective Action Program. As such, the purpose of this letter is to notify the Nuclear Regulatory Commission that the airspace acceptance criteria may change, however, this change does not impact the acceptability of the calculated peak drywell airspace temperature submitted in Reference 1 nor the No Significant Hazards evaluation submitted under Reference 2. The revised acceptance criteria is at least equal to worst case (highest) calculated peak drywell temperature in Reference 1. Other acceptance criteria used in the report were also reviewed and confirmed to remain unchanged.

Should you have any questions regarding this matter, please contact Sharon Mahler at (402) 825-5236.

Sincerely,



John H. Swales
Vice President of Nuclear Energy

des

cc: Regional Administrator
USNRC - Region IV

Senior Project Manager
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