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ROBERT C. MECREDY
Vice President
Nuclear Operations

April 5, 2000

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: Corrected Pages Associated with the
Credit for Soluble Boron in Spent Fuel Pool
Rochester Gas and Electric Corporation
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Reference: (a) Letter from Robert C. Mecredy (RG&E) to Guy S. Vissing (NRC), "Application for Amendment to Facility Operating License Credit for Soluble Boron in Spent Fuel Pool", dated March 8, 2000.

Dear Mr. Vissing:

In Reference (a), RG&E submitted a proposed change to the Improved Technical Specifications associated with the spent fuel pool storage requirements. It has been brought to our attention that there are typographical errors associated with the referencing of figures on page 3.7-29 of Attachment V of the referenced submittal. Upon a complete review of Attachment V, one addition minor error was identified on page 4.0-2 (in 4.3.1.1.b, $K_{eff} < 1.00$ should be $K_{eff} < 1.0$). Attached are the corrected replacement pages.

Very truly yours,

Robert C. Mecredy
Vice President
Nuclear Operations Group

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Attachment:

Replacement page 3.7-29
Replacement page 4.0-2

xc: Mr. Guy S. Vissing (Mail Stop 8C2)
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## 3.7 PLANT SYSTEMS

### 3.7.13 Spent Fuel Pool (SFP) Storage

LCO 3.7.13 The combination of initial enrichment and burnup values, with appropriate decay times, of each fuel assembly stored in the spent fuel pool shall be within the acceptable burnup domain of the applicable Figures 3.7.13-1 through 3.7.13-11, based on region and cell type.

APPLICABILITY: Whenever any fuel assembly is stored in the spent fuel pool.

ACTIONS

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|---|---|---|
| A. Requirements of the LCO not met. | A.1 --------NOTE--------- LCO 3.0.3 is not applicable. ---------- Initiate action to move the noncomplying fuel assembly to an acceptable storage location. | Immediately |

SURVEILLANCE REQUIREMENTS

| SURVEILLANCE | | FREQUENCY |
|---|---|---|
| SR 3.7.13.1 | Verify by administrative means the initial enrichment, burnup, and decay time of the fuel assembly is in accordance with the applicable Figures 3.7.13-1 through 3.7.13-11. | Prior to storing, or moving, the fuel assembly in the spent fuel pool |

## 4.0 DESIGN FEATURES

### 4.2 Reactor Core (continued)

#### 4.2.2 Control Rod Assemblies

The reactor core shall contain 29 control rod assemblies. The control material shall be silver indium cadmium.

### 4.3 Fuel Storage

#### 4.3.1 Criticality

4.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

a. Fuel assemblies having a maximum nominal U-235 enrichment of 5.0 weight percent;

b. $k_{eff} < 1.0$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.1 of the UFSAR;

c. $k_{eff} \leq 0.95$ if fully flooded with water borated to $\geq$ 975 ppm, which includes an allowance for uncertainties as described in Section 9.1 of the UFSAR; and

d. Consolidated rod storage canisters may be stored in the spent fuel storage racks provided that the fuel assemblies from which the rods were removed meet all the requirements of LCO 3.7.13 for the region in which the canister is to be stored. The average decay heat of the fuel assembly from which the rods were removed for all consolidated fuel assemblies must also be $\leq$ 2150 BTU/hr.

(continued)