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March 31, 2000

Re: Indian Point Unit No. 2  
Docket No. 50-247

Document Control Desk  
US Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, DC 20555-0001

Subject: Correction of a Typographical Error License Amendment Nos. 206 and 207

- References:
- 1) USNRC Letter to A. A. Blind from J. F. Harold, "Indian Point Nuclear Generating Unit No. 2 - Re: Issuance of Amendment Allowing Relocation of Administrative Controls Related to the Quality Assurance program Description (TAC No. MA5724)," dated February 25, 2000.
  - 2) USNRC Letter to A. A. Blind from J. F. Harold, "Indian Point Nuclear Generating Unit No. 2 - Re: Issuance to Modify Operations Management Requirements and Correct Editorial Errors (TAC No. MA4654)," dated February 29, 2000.
  - 3) Con Edison Letter to the USNRC Document Control Desk from A. A. Blind, "Proposed Technical Specification Amendment Regarding the Relocation of Administrative Controls Related to Quality Assurance," dated June 2, 1999.
  - 4) Con Edison Letter to the USNRC Document Control Desk from A. A. Blind, "Proposed Amendment Consisting of Administrative Changes to Technical Specifications," dated January 22, 1999.
  - 5) USNRC Policy Issue SECY-96-238, "Guidance for Correction of Technical Specification Typographical Errors," dated November 19, 1996.

Recently the US Nuclear Regulatory Commission (NRC), issued to Consolidated Edison Company of New York, Inc. (Con Edison) License Amendments 206 (Reference 1) and 207 (Reference 2) to the Indian Point Unit No. 2 Technical Specifications. However, there is a typographical error on page 6-3 of Amendment 207. The Con Edison letters that provided the retyped versions of page 6-3 were References 3 (for Amendment 206) and Reference 4 (for Amendment 207).

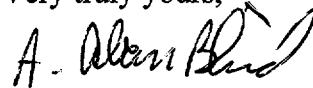
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Prior to the issuance of Amendments 206 and 207, page 6-3 had last been revised in Amendment 198. It was this amendment that provided the template for page 6-3 in References 3 and 4. Since Amendment 206 should serve as the template for Amendment 207, please find attached the corrected page 6-3 for Amendment 207.

Therefore, Con Edison is now requesting, in accordance with SECY-96-238 (Reference 5), that the attached page be used as the corrected page for Amendment 207. In accordance with 10 CFR 50.91, a copy of this letter is being submitted to the designated New York State official.

Should you or your staff have any questions regarding this submittal, please contact Mr. John F. McCann, Manager, Nuclear Safety and Licensing.

Very truly yours,



Attachment

cc: Mr. Hubert J. Miller  
Regional Administrator-Region I  
US Nuclear Regulatory Commission  
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Mr. Jefferey F. Harold, Project Manager  
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ATTACHMENT

CORRECTED PAGE 6-3 OF AMENDMENT 207

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.  
INDIAN POINT UNIT NO. 2  
DOCKET NO. 50-247  
MARCH 2000

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Operation Manager's and the Assistant Operation Manager's SRO license requirement which shall be in accordance with Technical Specification 6.2.2.h, and, (2) the Radiation Protection Manager who shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.

6.3.2 The Plant Manager shall meet or exceed the minimum qualifications specified for Plant Manager in ANSI N18.1-1971.

6.3.3 The Watch Engineer shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.

6.4.2 DELETED

### 6.5 REVIEW AND AUDIT

6.5.1 The review and audit functions of the Station Nuclear Safety Committee (SNSC) and the Nuclear Facilities Safety Committee (NFSC) are described in the Quality Assurance Program Description (QAPD).

### 6.6 REPORTABLE EVENT ACTION

6.6.0 A Reportable Event is defined as any of the conditions specified in 10 CFR 50.73a(2).

6.6.1 The following actions shall be taken in the event of a Reportable Event: