

United States Nuclear Regulatory Commission PLANT ISSUE MATRIX

By Primary Functional Area

Region II
 SUMMER

Date	Source	Functional Area	ID	Type	Template Codes	Item Title Item Description
01/15/2000	1999009	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: Ter:	Year 2000 Preparations The licensee's preparations for the year 2000 (Y2K) transition, including issuance of integrated contingency plans, were appropriate. Precautions taken to avoid unnecessary complications during the Y2K transition (i.e., setting back the integrated plant computer system date) demonstrated conservative management decision making. All systems continued to operate properly with no known Y2K induced errors.
Dockets Discussed: 05000395 Summer						
01/15/2000	1999009	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 2A Ter:	Residual Heat Removal System Walkdown Inspection and review of the residual heat removal system concluded that system readiness, including valve alignments, were consistent with approved procedures and system drawings. Material condition of the system was adequate.
Dockets Discussed: 05000395 Summer						
01/15/2000	1999009	Pri: OPS Sec: ENG	NRC	POS	Pri: 5A Sec: 5B Ter: 5C	Corrective Action Program In the areas of operations, maintenance, and engineering, the corrective action program was adequately addressing equipment, human performance, procedure and program problems. Corrective actions were properly analyzed, technically valid and effectively tracked through completion. The licensee was effective in utilizing quality assurance audits, internal self-assessments, operating experience, and other similar programs, to correct deficiencies.
Dockets Discussed: 05000395 Summer						
12/04/1999	1999008	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 2A Ter: 2B	Cold Weather Preparations The inspectors concluded that the freeze protection and cold weather preparations were properly implemented and heat tracing was available to protect safety-related systems. No significant discrepancies were noted in the licensee's cold weather preparations. The system engineer was knowledgeable of the heat trace system performance and the system was being properly monitored within the licensee's Maintenance Rule program.
Dockets Discussed: 05000395 Summer						
12/04/1999	1999008	Pri: OPS Sec: ENG	NRC	POS	Pri: 1C Sec: 4B Ter:	Change to Emergency Operating Procedures A recent change to emergency operating procedures was appropriate, timely, and properly approved. The inspectors concluded that change to the manual safety injection (SI) actuation criteria should help preclude an unwarranted SI following a reactor trip in a low decay heat situation.
Dockets Discussed: 05000395 Summer						
10/23/1999	1999007	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: Ter:	Safety Conscious Operations Safety conscious operation was evidenced by the site preparations and precautionary staffing of the technical support center for the approach of Hurricane Floyd. An example of a conservative operational decision was management's decision to lower reactor power to support replacement of a reactor coolant loop flow transmitter.
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10/23/1999	1999007	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 2A Ter: 4B	Detailed Service Water System Walkdown Based on a detailed service water (SW) system walkdown and review of numerous documents associated with the SW system, the inspectors concluded the system was properly aligned and operational in accordance with licensee procedures and Technical Specifications. The system engineer was knowledgeable and was properly monitoring the SW system performance under the maintenance rule and licensee programs.
Dockets Discussed: 05000395 Summer						
10/23/1999	1999007	Pri: OPS Sec:	NRC	POS	Pri: 4C Sec: 5B Ter:	Evaluation and Documentation of 10CFR Part 21 issue The licensee properly evaluated and documented a 10 CFR Part 21 issue which could have prevented 480 volt K-line breakers from either tripping or closing.
Dockets Discussed: 05000395 Summer						
10/23/1999	1999007-01	Pri: OPS Sec:	NRC	NCV	Pri: 4C Sec: Ter:	Failure to make a 10CFR 50.72 One-hour Report A non-cited violation was identified for the licensee's failure to make a 10 CFR 50.72 non-emergency one-hour report for the service water pond siphon breakers being outside their design basis.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 2A Ter:	Walkdown of the Control Room Evacuation Panel A walkdown of the Control Room Evacuation Panel (CREP) and related support systems, discussions with a control room shift supervisor on the performance of CREP procedures, and a review of training indicated the licensee is maintaining the capability to shutdown the plant from locations outside the control room in accordance with General Design Criteria 19 of 10 CFR 50, Appendix A.
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07/31/1999	1999005	Pri: OPS Sec:	NRC	POS	Pri: 5A Sec: 4B Ter: 2A	Nuclear Instrumentation N-44 Drift The licensee's actions in response to N-44 power range detector indication exhibiting a current decrease were appropriate. Operations personnel displayed a good questioning attitude by detecting this condition prior to an alarm occurring. The technical assessments, engineering 10 CFR 50.59 screenings, compensatory actions taken and operability assessments were found to be consistent with the guidance of Generic Letter 91-18, Revision 1, for a degraded but operable component
Dockets Discussed: 05000395 Summer						
07/31/1999	1999005	Pri: OPS Sec: MAINT	NRC	POS	Pri: 1B Sec: 3A Ter: 4C	Operator Response to Unexpected Control Rod Motion Prompt operator response to unexpected control rod motion during performance of a calibration minimized any adverse effects on the plant. The rod motion was caused by the loss of the reference temperature signal to the rod control system.
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06/19/1999	1999004	Pri: OPS Sec:	NRC	NEG	Pri: 1B Sec: 3B Ter: 2A	Manual Reactor Trip Due to Main Turbine High Vibration Although, operator performance following a manual reactor trip due to high main turbine vibration was appropriate, the operators responded slowly to decreasing reactor coolant average temperature (Tavg) and delayed the reduction of emergency feedwater flow following the trip. Reactor coolant system temperature decreased approximately eight degrees Fahrenheit below the normal no-load Tavg value. Primary and secondary systems responded as designed to the reactor trip
Dockets Discussed: 05000395 Summer						
06/19/1999	1999004	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 3A Ter:	Startup Observations The reactor startups following the refueling outage and plant trips were performed safely. Reactivity additions were carefully controlled and monitored by operations and reactor engineering personnel. The operators demonstrated good command and control, proper communications and performed the startups in accordance with approved procedures.
Dockets Discussed: 05000395 Summer						
06/19/1999	1999004	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 3A Ter:	Observations of Danger Tagouts The clearance of danger tagouts for a motor driven emergency feedwater pump and the diesel driven fire pump was performed in accordance with procedure requirements. Operators used proper communication, observed safety precautions, and properly conducted independent verification.
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06/19/1999	1999004	Pri: OPS Sec:	NRC	POS	Pri: 1B Sec: 2A Ter: 5B	Automatic Reactor Trip Due to N-43 Spike Operator response to an automatic reactor trip was effective in stabilizing the plant and was in accordance with emergency operating procedures. The trip was caused by caused by spiking on power range instrument N-43 during N-42 power range instrument calibration. Safety-related components functioned as expected. Post trip reviews and troubleshooting effectively isolated the problem to a defective nuclear instrument current meter and appropriate corrective actions were taken.
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06/19/1999	1999004	Pri: OPS Sec:	NRC	POS	Pri: 3B Sec: Ter:	Requalification Annual Operating and Biennial Written Examinations The content of the annual operating tests and biennial written examinations was satisfactory. The written examinations and simulator scenarios provided very good evaluation tools to measure operator knowledge, skills and abilities. This portion of the licensed operator requalification program met the requirements of 10 CFR 55.59, "Requalification."
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06/19/1999	1999004	Pri: OPS Sec:	NRC	POS	Pri: 5A Sec: 3A Ter:	Plant Safety Review Committee and Management Review Board Meetings Observed Plant Safety Review Committee and Management Review Board meetings were comprehensive, properly focused on safety and probing with relevant issues being adequately reviewed. The inspectors noted action items were issued to ensure proper followup and resolution on issues of concern.
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05/08/1999	1999003	Pri: OPS Sec:	NRC	NEG	Pri: 1A Sec: 3B Ter: 3A	Power Reduction and Plant Shutdown/Draindown A negative observation was noted for control board operators not being aware of the cause for several illuminated control room annunciators. An example was the "Source Range Hi Flux at Shutdown Blocked" annunciator being illuminated during fuel reload with the operator being unaware of why it was acceptable to block this alarm function.
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05/08/1999	1999003	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 3A Ter:	Core Offload and Reload The inspectors concluded that core offload, reload and core verification were performed in accordance with established procedures. Fuel handling activities were well controlled.
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05/08/1999	1999003	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 3A Ter: 3C	Power Reduction and Plant Shutdown/Draindown The power reduction, plant cooldown and shutdown operations in preparation for refueling were conducted safely and were well controlled with good communications established between personnel. Operations management appropriately stressed the importance of monitoring and understanding the relationship between reactor vessel level indication and inventory balances to ensure proper reactor coolant system inventory control during shutdown conditions.
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05/08/1999	1999003-01	Pri: OPS Sec:	NRC	NCV	Pri: 1A Sec: 2A Ter: 3A	Failure to Remove Loose Debris from the Reactor Building A non-cited violation was identified for failure to adequately perform a Technical Specification required visual inspection for loose debris in the reactor building. Following completion of the licensee's reactor building closeout inspection, the inspectors found loose debris, including a rubber shoe, a plastic bag, and a cloth booty, in the reactor building. Subsequent evaluation determined that the debris would have had a negligible impact on sump performance.
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03/27/1999	1999002	Pri: OPS Sec:	NRC	NEG	Pri: 5B Sec: 1B Ter:	Management Review Board Meeting Although a Management Review Board (MRB) held to review the plant transient of January 3, 1999, provided valuable insights into the contributing factors and circumstances surrounding the event, both the inspectors and the MRB recognized the need to better understand the circumstances surrounding and contributing factors to this event in a more timely manner.
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03/27/1999	1999002	Pri: OPS Sec:	NRC	POS	Pri: 1A Sec: 2A Ter: 2B	Detailed Walkdown of Emergency Diesel Generators Based on a detailed walkdown the Emergency Diesel Generators (EDGs) were found to be properly aligned and in a standby condition per licensee procedures. Technical specification requirements for fuel oil and surveillance requirements were being met. Several small EDG lube oil leaks were observed. The maintenance rule program properly monitored EDG performance.
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03/27/1999	1999002	Pri: OPS Sec:	NRC	POS	Pri: 3B Sec: 3A Ter:	Licensed Operator Simulator Requalification The licensed operator simulator requalification examination scenarios were challenging and operators' performance met test objectives. Examination critiques were thorough and provided a comprehensive assessment of individual and crew performance.
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02/13/1999	1999001	Pri: OPS Sec:	NRC	POS	Pri: 1B Sec: 2A Ter:	Operator Response to Moisture Separator Pressure Switch Failure Operators promptly responded to a moisture separator pressure switch failure by reducing load. The operators followed the appropriate annunciator response and operating procedures during the transient and prevented a potential loss of feedwater and reactor trip.
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12/04/1999	1999008-01	Pri: MAINT Sec:	NRC	NCV	Pri: 2B Sec: Ter:	Failure to retest an air operated valve following replacement of the air regulator A non-cited violation was identified for the licensee's failure to perform a retest in accordance with their post maintenance testing program. Prior to returning an air operated valve to service, the valve was not tested after the air regulator was replaced.
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10/23/1999	1999007	Pri: MAINT Sec:	NRC	NEG	Pri: 3A Sec: 2B Ter: 5C	Deficient Work Practices Personnel work practices were deficient, in that, while performing maintenance on nearby equipment, a residual heat removal pump breaker manual closing mechanism was struck by an equipment cart. The licensee actions to address the inadvertent pump start should preclude recurrence. In addition, the licensee revised an annunciator response procedure to better aid operators in diagnosing the cause for the alarm which was received when the pump started.
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10/23/1999	1999007-02	Pri: MAINT Sec: ENG	Licensee	NCV	Pri: 2B Sec: 4B Ter:	Inadequate Surveillance Procedure for Control Room Emergency Ventilation A non-cited violation was identified for an inadequate surveillance procedure for measurement of control room emergency ventilation outside makeup airflow. The procedure was corrected and re-performed successfully.
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10/23/1999	1999007-03	Pri: MAINT Sec: ENG	Licensee	NCV	Pri: 2B Sec: 4B Ter:	Inadequate Surveillance Procedure Positioned ECCS valves contrary to TS position A non-cited violation was identified for inadequate surveillance procedures that positioned several emergency core cooling system valves contrary to the positions listed in Technical Specifications.
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09/11/1999	1999006	Pri: MAINT Sec: ENG	NRC	POS	Pri: 2B Sec: 4C Ter: 3C	Modifications to Circulating Water/Service Water Crosstie Pipe Modifications to the circulating water/service water crosstie pipe were well planned and the associated instructions included appropriate precautions to ensure continued fire suppression water was available. The engineer technical work records and 10 CFR 50.59 safety evaluation reviews were detailed and technically adequate. Good oversight of contractor personnel was noted. The equipment setup for the temporary fire service pumps and installation testing were well controlled and implemented in accordance with the work instructions.
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07/31/1999	1999005	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 3A Ter: 3C	Observation of Work Activities Routine maintenance and surveillance activities were satisfactorily performed, i.e., conducted in an appropriate and professional manner in accordance with established procedures. Good communications and supervisor oversight were noted by the inspectors during instrumentation and control surveillance activities.
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07/31/1999	1999005-01	Pri: MAINT Sec:	Self	NCV	Pri: 4C Sec: 2B Ter:	Failure to Establish an Adequate Procedure for Calibration of FW Flow Control Valve A Non-Cited Violation was identified for failure to establish an adequate procedure for the performance of rack calibration of main feedwater to steam generator C flow control valve, IFV00498. The removal of a control system relay card during the calibration resulted in loss of the reference temperature signal to the rod control system and consequent automatic inward motion.
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06/19/1999	1999004	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 3A Ter:	Observation of Work Activities Based on review and observation of eleven surveillance test and maintenance packages, routine maintenance and surveillance activities were satisfactorily performed. Activities were conducted in accordance with written procedure instructions and the procedures provided sufficient detail and guidance. Technicians demonstrated that they were experienced and knowledgeable.
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06/19/1999	1999004-01	Pri: MAINT Sec:	Licensee	NCV	Pri: 2B Sec: 1A Ter: 4C	Missed Surveillance - Turbine Stop Valve Closure Trip Actuating Device Operational Test A non-cited violation was identified for the failure to test the Turbine Trip Actuating Device prior to reactor startup in accordance with Technical Specification Table 4.3-1, Item 17. The surveillance test was performed following the reactor startup.
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05/08/1999	1999003	Pri: MAINT Sec:	NRC	NEG	Pri: 2B Sec: 3A Ter: 1A	Surveillance Observation During preparations for the train A integrated safeguards test, the control room operating crew failed to establish an initial test condition for volume control tank (VCT) level. After the inspectors identified this discrepancy, operators properly established VCT level prior to the start of the test.
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05/08/1999	1999003	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 3A Ter:	Observation of Work Activities The inspectors observed good maintenance practices during refueling outage RF-11. Preventative maintenance and maintenance activities were appropriate and properly implemented in accordance with instructions provided and established work documents. The inspectors concluded that outage maintenance activities were well performed.
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05/08/1999	1999003	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 3A Ter:	Surveillance Observation The observed surveillance activities were successfully completed by knowledgeable personnel. When problems were encountered appropriate corrective actions were implemented and adequate retests were performed. Procedures provided sufficient detail and guidance for the intended surveillance activities. The licensee established good communication and coordination between departments prior to commencement of surveillance tests.
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05/08/1999	1999003	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 3A Ter: 3B	Inservice Inspection (ISI) - Observation of Work Activities Inservice examination and test activities were performed, documented and evaluated in accordance with approved procedures by certified, skilled, and knowledgeable examiners.
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05/08/1999	1999003	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 4C Ter:	Review of Significant Design Modifications / Maintenance Work Requests (MWRs) Selected design modifications and maintenance work requests on the A emergency diesel generator, the service water system, the A reactor coolant pump seal, the station batteries, and a safety injection valve were successfully implemented and satisfactorily tested. Documents generated to support plant changes were thorough and provided sufficient detail to accomplish the design changes.
Dockets Discussed: 05000395 Summer						
05/08/1999	1999003	Pri: MAINT Sec:	NRC	POS	Pri: 5B Sec: 5C Ter: 2A	7.2 kV Breaker Troubleshooting The licensee's troubleshooting plan for failures of General Electric 7.2 kV Magne-Blast breakers to close was effective. Through the use of high speed video cameras the licensee was able to identify the root cause. Corrective actions necessary to prevent recurrence were completed. Additionally, the licensee made an 10 CFR 21 notification for reporting a defect with substantial safety hazards that involved a common mode failure.
Dockets Discussed: 05000395 Summer						
05/08/1999	1999003-02	Pri: MAINT Sec:	Licensee	NCV	Pri: 2B Sec: 4C Ter:	Missed Technical Specification Surveillance Requirement to Vent the Residual Heat Removal Pump Casings A non-cited violation was identified for failure to adequately vent the residual heat removal pump casings as required by Technical Specifications.
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05/08/1999	1999003-03	Pri: MAINT Sec:	Licensee	NCV	Pri: 2B Sec: 4C Ter:	Missed Surveillance Test for Electrical Equipment Protective Devices A non-cited violation was identified for the failure to functionally test portions of breaker control circuits as required by Technical Specifications.
Dockets Discussed: 05000395 Summer						
05/08/1999	1999003-04	Pri: MAINT Sec:	NRC	NCV	Pri: 2B Sec: 3A Ter:	Missed Surveillance on Manipulator Crane Load Cell A non-cited violation was identified for failure to perform a load test on a refueling manipulator crane load cell prior to use as required by Technical Specifications.
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03/27/1999	1999002	Pri: MAINT Sec:	NRC	NEG	Pri: 2A Sec: 2B Ter:	Meteorological Tower Availability Corrective maintenance and corrective actions have been ineffective in preventing an increased unavailability time for the meteorological tower during the last part of 1998 and 1999. The licensee had not established a system to actively track availability time to ensure that the Final Safety Analysis Report annual target of 90% data recovery is achieved.
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03/27/1999	1999002	Pri: MAINT Sec:	NRC	POS	Pri: 2A Sec: 2B Ter:	Review of Maintenance and Test Packages for Emergency Core Cooling System Nine completed surveillance test and preventive maintenance packages demonstrated acceptable test results for emergency core cooling system relief valves and check valves.
Dockets Discussed: 05000395 Summer						
03/27/1999	1999002	Pri: MAINT Sec:	NRC	POS	Pri: 2A Sec: 2B Ter:	Maintenance/Material Condition of Reactor Coolant System (RCS) Pressure Isolation Valves Review of leakage testing data indicated acceptable material condition for Reactor Coolant System (RCS) isolation boundaries. No examples of inadequate maintenance were identified during this review. No problems were identified during the review of equipment history which would indicate an adverse trend or degradation of the material condition of RCS pressure isolation valves.
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03/27/1999	1999002	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: 3A Ter:	Emergency Diesel Generator Surveillance Observations The A emergency diesel generator operability, slave relay and support system leak surveillance tests were performed in accordance with established procedures and demonstrated operability of the equipment in accordance with the Technical Specification surveillance requirements. Personnel conducting the tests demonstrated a good level of knowledge. The pre-job briefing was thorough.
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02/13/1999	1999001	Pri: MAINT Sec:	NRC	POS	Pri: 2B Sec: Ter:	Determination of Moderator Temperature Coefficient Based on a review of test data the inspectors verified that the moderator temperature coefficient met the limits specified in TS 4.1.1.3.b and the Core Operating Limits Report. The licensee performed the test in accordance with procedure requirements.
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: MAINT Sec:	NRC	POS	Pri: 5A Sec: 3A Ter:	Good Questioning Attitude by Electrical Maintenance Technician A particularly noteworthy example of a good questioning attitude by an electrical maintenance technician was noted. The technician questioned the validity of existing electrical schematics versus the installed plant wiring configuration of a component cooling water pump hand switch.
Dockets Discussed: 05000395 Summer						
12/04/1999	1999008	Pri: ENG Sec:	NRC	POS	Pri: 1A Sec: 4B Ter:	Measures to Reduce Likelihood of Draindown while Shutdown The licensee has in place adequate measures to reduce the likelihood of a draindown similar to the Wolf Creek event of September 17, 1994. Enhancements are planned to training documents and one procedure as a result of this inspection.
Dockets Discussed: 05000395 Summer						
10/23/1999	1999007	Pri: ENG Sec:	NRC	POS	Pri: 4A Sec: 5C Ter:	Component Cooling Water Calculations Review of calculations for the component cooling water system led to the conclusion that necessary calculations were on file and that the licensee was adequately addressing weaknesses in these calculations through their design basis document improvement project and internal safety system functional inspections.
Dockets Discussed: 05000395 Summer						
10/23/1999	1999007-04	Pri: ENG Sec:	Licensee	NCV	Pri: 4A Sec: 5C Ter:	Service Water Pond Siphon Breakers not Installed in Accordance with Drawings A non-cited violation was identified for the service water pond siphon breakers not being installed in accordance with plant drawings. The siphon breakers had blind flanges installed such that they would not function. The licensee took prompt corrective actions to resolve the condition once identified.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: ENG Sec:	NRC	POS	Pri: 4B Sec: Ter:	Licensee Actions in Response to Error in Computer Code Westinghouse discovered an error in the computer code (LOCBART) used for the V.C. Summer large break LOCA analysis and provided a re-analysis using an updated computer code. Licensee actions taken in response including the 10 CFR 50.59 Safety Evaluation and Core Operating Limits Report (COLR) revision appeared reasonable, appropriate and timely.
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09/11/1999	1999006	Pri: ENG Sec:	NRC	POS	Pri: 4C Sec: Ter:	Use of BEACON as a Core Power Monitoring System The licensee has implemented a new TS amendment to use BEACON as a core power distribution monitoring system to augment the moveable incore detectors for flux mapping activities. The inspectors concluded that the licensee's use of BEACON is an effective tool and should provide additional insight and understanding of core characteristics, and operational flexibility with less flux maps being required.
Dockets Discussed: 05000395 Summer						
07/31/1999	1999005	Pri: ENG Sec:	NRC	NEG	Pri: 2B Sec: 4B Ter:	Review of Pressurizer Heater Breaker Troubleshooting Activities Although the troubleshooting plan for unexpected tripping of the pressurizer group 2 heater breaker was appropriate and reasonable, the long-term troubleshooting instructions did not ensure that three multimeters installed for troubleshooting would remain continuously operational. This deficiency could have resulted in important troubleshooting information being missed.
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06/19/1999	1999004	Pri: ENG Sec:	NRC	POS	Pri: 4B Sec: Ter:	Control of Core Physics Constants The inspectors verified reactor engineering was entering and maintaining the proper core physics constants in the integrated plant computer system. These constants are used for low power physics testing to verify core performance during startup following refueling.
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05/08/1999	1999003	Pri: ENG Sec:	NRC	POS	Pri: 5B Sec: 5C Ter: 4B	Engineering Evaluation of Fuel Assembly Top Nozzle Defect Based on the results of a Westinghouse safety assessment and the licensee's replacement of 28 fuel assembly top nozzles prior to core reload, the inspectors concluded that the licensee appropriately evaluated and resolved issues associated with fuel assembly top nozzle hold down spring screw failures. The licensee's conclusions were reasonable and there are no safety concerns that would preclude the current Cycle 12 fuel load from meeting the reload safety analysis.
Dockets Discussed: 05000395 Summer						
05/08/1999	1999003-05	Pri: ENG Sec:	Licensee	NCV	Pri: 4A Sec: Ter:	Failure to Comply with 10 CFR 50 Appendix B Criterion III A non-cited violation was identified for failure to correctly translate design requirements into specifications, drawings or procedures. Ten reactor building components, which were required to operate after an accident and which could be submerged during an accident, were not designed or evaluated for submergence.
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03/27/1999	1999002	Pri: ENG Sec:	NRC	POS	Pri: 2A Sec: 4B Ter:	Analysis and Resolution of Battery Degradation Both trains of safety-related batteries have exhibited the early stages of post seal leakage. The licensee made a conservative decision to replace these batteries in the 1999 refuel outage. One non-safety-related battery is approaching end of useful life and will also be replaced.
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03/27/1999	1999002	Pri: ENG Sec:	NRC	POS	Pri: 2B Sec: 5C Ter: 2A	Review of Circuit Breaker Failure The licensee's program for refurbishment of 7.2 kV circuit breakers is being aggressively implemented, and should help preclude failures similar to the circulating water pump breaker failure.
Dockets Discussed: 05000395 Summer						
03/27/1999	1999002	Pri: ENG Sec:	NRC	POS	Pri: 4A Sec: 4C Ter:	Review of Design Basis Document (DBD) Improvement Project Plan The licensee has initiated a design basis document (DBD) improvement project to be completed over a five year period. The licensee plans to prioritize reworking/replacing/initiating calculations, technical reports, etc., using maintenance rule program risk rankings. The emergency feedwater and component cooling water system DBDs were "improved" as trial examples to help define the detailed plan and illustrate the need for the project.
Dockets Discussed: 05000395 Summer						
01/15/2000	1999009	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Radiation Exposure The ALARA (As Low As is Reasonably Achievable) meeting placed an appropriate emphasis and proper perspective on radiological safety and is making a conscience effort to reduce radiation exposure to plant personnel.
Dockets Discussed: 05000395 Summer						
01/15/2000	1999009	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	1999 Post Accident Sampling Drill As compared to a 1998 drill, a marked improvement was observed in the performance of the 1999 post accident sampling system drill. The inspectors concluded that the licensee has in place the necessary equipment, procedures and trained chemistry personnel to collect and analyze post accident samples as required. The licensee appropriately captured in their critique and corrective action process areas for improvement to reduce the time for sampling and analysis.
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01/15/2000	1999009	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Interim Compensatory Measures for Kaowool Fire Barriers The inspectors concluded that the licensee implementation of compensatory measures for one-hour Kaowool fire barriers was appropriate as an interim measure until a final resolution can be determined.
Dockets Discussed: 05000395 Summer						
12/04/1999	1999008	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Radiological Controls and ALARA Program The licensee implemented radiological controls in accordance with the Final Safety Analysis Report, Technical Specifications, license conditions, and 10 CFR Part 20 requirements. The As Low As Is Reasonably Achievable Program activities and initiatives for refueling outage 11 were conducted in accordance with approved procedures. Action items were developed to address work activities for which the actual dose received varied by more than 25 percent from the estimated dose.
Dockets Discussed: 05000395 Summer						

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12/04/1999	1999008	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Emergency Preparedness Drill All six emergency preparedness drill objectives were successfully met. Operators exhibited proper procedure adherence and three-way communications. Staffing and activation of emergency response facilities were timely and were able to support emergency drill activities.
Dockets Discussed: 05000395 Summer						
10/23/1999	1999007	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 5A Ter:	Reactor Building Entry at Power A pre-job briefing conducted prior to an at power reactor building entry was professional and complete. Health Physics personnel provided proper radiological controls during the entry. The entry was successful in identifying the source of a non-reactor coolant system leak into the containment sump. The licensee appropriately identified and entered items discovered during the reactor building entry into their corrective action program for resolution.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Radioactive effluent and waste processing Radioactive effluent and waste processing systems and storage facilities met Final Safety Analysis Report and 10 CFR Part 50 requirements. Radiological controls for radioactive gaseous and liquid effluent storage and processing systems and areas, and for radioactive material/waste storage areas met technical specification (TS), and 10 CFR Part 20 requirements.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Radiological Environmental Monitoring Program The radiological environmental monitoring program for airborne radionuclides and leafy vegetation, and monitoring of direct radiation was implemented in accordance with the Offsite Dose Calculation Manual.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Counting Room Quality Control Activities Counting-room quality control activities were implemented appropriately and verified the accuracy of radionuclide analytical measurement instrumentation.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Releases met Established Regulatory Limits The 1998 Annual Effluent Release Report and Annual Radiological Environmental Monitoring Report were submitted in accordance with TS and documented results demonstrated gaseous and liquid effluent processing and subsequent releases met established regulatory limits and Appendix I to 10 CFR 50 design objectives.
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09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 3B Ter:	Liquid Effluent Releases Chemistry and operations personnel demonstrated appropriate knowledge of procedural requirements, and proficiency in initiating and conducting liquid effluent releases. Licensee programs to control effluent releases were implemented effectively with effluent radionuclide concentrations and resultant projected offsite doses within established regulatory limits and design objectives.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 3B Ter:	Reactor Coolant System Chemistry Routine chemistry sampling of the reactor coolant system was conducted in accordance with written procedures with satisfactory results. The chemistry technician was experienced and knowledgeable of his assigned task and used appropriate safety precautions and radiation control measures.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 5A Ter:	Annual Fire Drill Utilizing Off Site Assistance An annual fire drill utilizing off site fire fighting assistance was performed satisfactorily. The drill was adequately monitored and controlled. The inspectors identified a concern with the thoroughness of the original critique process not capturing lessons learned from the annual fire drill. The inspectors' concerns were captured in the critique documentation for follow-up action following discussions with the licensee.
Dockets Discussed: 05000395 Summer						
09/11/1999	1999006-03	Pri: PLTSUP Sec:	NRC	NCV	Pri: 1C Sec: 2B Ter:	Failure to Properly Calibrate Liquid Effluent Discharge Monitoring A non-cited violation was identified for failure to calibrate liquid radiation monitoring systems in accordance with vendor or accepted industry practices as required by 10 CFR Part 20.1501(b) requirements. General area radiation monitoring system equipment was calibrated properly.
Dockets Discussed: 05000395 Summer						
07/31/1999	1999005	Pri: PLTSUP Sec:	NRC	NEG	Pri: 1C Sec: 1B Ter:	Evaluation of Exercises for Power Reactors Although command and control in each of the emergency response facilities was effective, there was room for improvement in performing briefings and maintaining the plant status priority board in the Operations Support Center
Dockets Discussed: 05000395 Summer						
07/31/1999	1999005	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Review of Exercise Objectives and Scenarios for Power Reactors The licensee's submittals of the scope and objectives, as well as, the scenario package were timely and appropriate for the biennial emergency preparedness exercise. The exercise scenario was sufficiently detailed and challenging.
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07/31/1999	1999005	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 1B Ter:	Evaluation of Exercises for Power Reactors The licensee's overall performance in response to a simulated emergency was satisfactory. The inspectors concluded that the exercise was a successful demonstration of the licensee's emergency response capabilities. The Alert, Site Area Emergency, and General Emergency declarations were timely and correct, and all offsite notifications were completed within 15 minutes.
Dockets Discussed: 05000395 Summer						
07/31/1999	1999005	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 3B Ter:	Review of Fire Brigade Drill and Qualifications A fire brigade drill was performed satisfactorily and met established criteria. The critique conducted was thorough. Areas needing improvement were captured in the drill critique and will be incorporated in quarterly training for fire brigade team members. No concerns were identified with the protected area fire brigade team member qualifications.
Dockets Discussed: 05000395 Summer						
07/31/1999	1999005	Pri: PLTSUP Sec: ENG	NRC	NEG	Pri: 1C Sec: 4B Ter: 3A	Evaluation of Exercises for Power Reactors The Technical Support Engineering (TSE) team recommendations were not based on thorough engineering evaluations in its support of several off-normal actions taken by the Technical Support Center (TSC). In addition, poor communications were noted between the TSC main room personnel and TSE team members concerning the status of plant and equipment conditions.
Dockets Discussed: 05000395 Summer						
06/19/1999	1999004	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Implementation of the Emergency Preparedness Program The emergency preparedness program was being maintained in a state of operational readiness. Changes made to the Emergency Preparedness program since the last inspection met NRC requirements and did not adversely affect the overall state of emergency preparedness.
Dockets Discussed: 05000395 Summer						
06/19/1999	1999004	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 3B Ter:	New Handgun Training and Qualification Security force handgun training and testing was effective, well controlled, with appropriate emphasis on safety and conducted in accordance with the Security Plan Procedures.
Dockets Discussed: 05000395 Summer						
05/08/1999	1999003	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Tour of Radiological Protected Areas Radiological conditions in radioactive material storage areas, health physics facilities, and waste storage buildings were appropriate, areas were properly posted and material was properly labeled. Personnel dosimetry devices were appropriately worn. Radiation worker doses were being maintained well below regulatory limits and the licensee was maintaining personnel exposure as low as is reasonably achievable.
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Date	Source	Functional Area	ID	Type	Template Codes	Item Title Item Description
05/08/1999	1999003-06	Pri: PLTSUP Sec:	Licensee	NCV	Pri: 1C Sec: 2B Ter:	Failure to Properly Control Access to a High Radiation Area A non-cited violation was identified concerning failure to properly control access to a high radiation area in the spent fuel pool building. Movement of spent fuel assemblies past a drained spent fuel cask loading pit resulted in the high radiation area. A contributing factor to this event was that the licensee elected to not remove scaffolding in the spent fuel pit after completion of maintenance and therefore did not fill the pit with water prior to moving spent fuel assemblies.
Dockets Discussed: 05000395 Summer						
03/27/1999	1999002-01	Pri: PLTSUP Sec:	NRC	NCV	Pri: 1C Sec: 3A Ter:	Improperly Escorted Visitor Outside the Diesel Generator Building A non-cited violation was identified concerning failure to properly control an escorted visitor in the protected area. A contributing factor was an informal turnover of escort responsibilities prior to the occurrence.
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Security Plan Changes and Security Procedures Security plan changes and security procedures were thorough, well documented, and consistent with the Physical Security Plan commitments and 10 CFR Part 50.54.
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Logging of Safeguards Events The licensee's safeguards events were logged according to the Physical Security Plan commitments. The licensee's process of tracking, trending, analyzing, and resolving these events was noteworthy.
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Security Organization Response Capability The inspector verified that responses by the security organization to security threats, contingencies, and routine response situations were consistent with the security procedures, the Physical Security Plan and Security Contingency Plan. Appropriate procedural guidance was developed in response to NRC Information Notice 98-35, "Threat Assessments and Consideration of Heightened Physical Protection Measures."
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: Ter:	Security Force Training and Requalification The security force was effectively trained and requalified according to the Training and Qualification Plan and regulatory requirements. Training records were properly maintained and reflected current qualifications according to the training program commitments.
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02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 2A Ter:	Vehicle Barrier System The vehicle barrier system was functional, well maintained, and effective in its intended purpose. The vehicle barrier system met the Physical Security Plan commitments and regulatory requirements.
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 2B Ter:	Security Compensatory Measures Program The security compensatory measures program was effective and functional for failed or impaired security equipment and met Physical Security Plan commitments and regulatory requirements.
Dockets Discussed: 05000395 Summer						
02/13/1999	1999001	Pri: PLTSUP Sec:	NRC	POS	Pri: 1C Sec: 3B Ter: 2B	Operability and Readiness of Security Contraband Detection System The observed tests effectively provided assurance of the operability and readiness of the security contraband detection system. Security maintenance personnel performing the tests demonstrated a good level of knowledge and familiarity with security equipment.
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Legend

Type Codes:

BU	Bulletin
CDR	Construction
DEV	Deviation
EEI	Escalated Enforcement Item
IFI	Inspector follow-up item
LER	Licensee Event Report
LIC	Licensing Issue
MISC	Miscellaneous
MV	Minor Violation
NCV	NonCited Violation
NEG	Negative
NOED	Notice of Enforcement Discretion
NON	Notice of Non-Conformance
OTHR	Other
P21	Part 21
POS	Positive
SGI	Safeguard Event Report
STR	Strength
URI	Unresolved item
VIO	Violation
WK	Weakness

Template Codes:

1A	Normal Operations
1B	Operations During Transients
1C	Programs and Processes
2A	Equipment Condition
2B	Programs and Processes
3A	Work Performance
3B	KSA
3C	Work Environment
4A	Design
4B	Engineering Support
4C	Programs and Processes
5A	Identification
5B	Analysis
5C	Resolution

ID Codes:

NRC	NRC
Self	Self-Revealed
Licensee	Licensee

Functional Areas:

OPS	Operations
MAINT	Maintenance
ENG	Engineering
PLTSUP	Plant Support
OTHER	Other

EEIs are apparent violations of NRC Requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUREG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entries may be modified when the final decisions are made.

URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. A URI may also be a potential violation that is not likely to be considered for escalated enforcement action. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.