

March 22, 2000

Mr. Michael B. Sellman
Senior Vice President and
Chief Nuclear Officer
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, WI 53201

NRR-058

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENTS RE: CONTROL ROD MOVEMENT EXERCISE FREQUENCY
(TAC NOS. MA7320 AND M7321)

Dear Mr. Sellman:

The Commission has issued the enclosed Amendment No. 195 to Facility Operating License No. DPR-24 and Amendment No. 200 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated November 15, 1999.

This amendment changes the control rod surveillance interval in TS Table 15.4.1-2, Item 10, "Partial movement of all rods," from once "Every 2 weeks" to "Quarterly." This change implements the recommendation of NRC Generic Letter 93-05, "Line Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation."

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/
Beth A. Wetzel, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

- Enclosures: 1. Amendment No. 195 to DPR-24
- 2. Amendment No. 200 to DPR-27
- 3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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3. Safety Evaluation

cc w/encls: See next page

Point Beach Nuclear Plant, Units 1 and 2

cc:

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November 1999



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.195
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated November 15, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

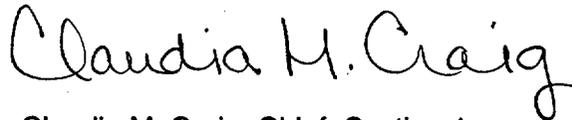
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 195 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: March 22, 2000



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 200
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated November 15, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

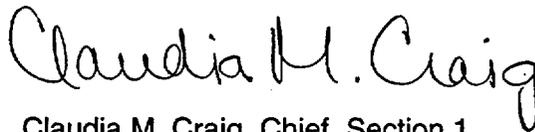
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 200 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: March 22, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 195
TO FACILITY OPERATING LICENSE NO. DPR-24
AND LICENSE AMENDMENT NO. 200
TO FACILITY OPERATING LICENSE NO. DPR-27
DOCKET NOS. 50-266 AND 50-301

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

REMOVE

Table 15.4.1-2 page 2 of 5

INSERT

Table 15.4.1-2 page 2 of 5

TABLE 15.4.1-2 (Continued)

	<u>Test</u>	<u>Frequency</u>
7.	Spent Fuel Pit	
	a) Boron Concentration	Monthly
	b) Water Level Verification	Weekly
8.	Secondary Coolant	
	Gross Beta-gamma Activity or gamma isotopic analysis	Weekly ⁽⁶⁾
	Iodine concentration	Weekly when gross Beta-gamma activity equals or exceeds 1.0 $\mu\text{Ci/g}$ ⁽⁶⁾
9.	Control Rods	
	a) Rod drop times of all full length rods ⁽³⁾	Each refueling or after maintenance that could affect proper functioning ⁽⁴⁾
	b) Rodworth measurement	Following each refueling shutdown prior to commencing power operation
10.	Control Rod	
	Partial movement of all rods	Quarterly ⁽¹⁸⁾
11.	Pressurizer Safety Valves	
	Set point	Every five years ⁽¹¹⁾
12.	Main Steam Safety Valves	
	Set Point	Every five years ⁽¹¹⁾
13.	Containment Isolation Trip	
	Functioning	Each refueling shutdown
14.	Refueling System Interlocks	
	Functioning	Each refueling shutdown
15.	Service Water System	
	Functioning	Each refueling shutdown
16.	Primary System Leakage	
	Evaluate	Monthly ⁽⁶⁾
17.	Diesel Fuel Supply	
	Fuel inventory	Daily
18.	Deleted	
19.	Deleted	
20.	Boric Acid System	
	Storage Tank and piping temperatures \geq temperature required by Table 15.3.2-1	Daily ⁽¹⁹⁾



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 195 TO FACILITY OPERATING LICENSE NO. DPR-24
AND AMENDMENT NO. 200 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By application dated November 15, 1999, the Wisconsin Electric Power Company (the licensee) requested changes to the Technical Specifications (TSs) for Point Beach Nuclear Plant, Units 1 and 2. The proposed amendment would change the control rod surveillance interval in TS Table 15.4.1-2, Item 10, "Partial movement of all rods," from once "Every 2 weeks" to "Quarterly."

2.0 EVALUATION

Point Beach TSs require the performance of a control rod exercise test (partial movement of all rods) when the reactor is critical. The specified frequency of this test is "every 2 weeks." Control rods are important to safety. The Point Beach TSs require the rods to be tested biweekly, at power, to assure that they are not stuck so that if a scram signal opens the trip breakers, the rods will drop into the core. This test is performed to determine if the control rods are movable. The control rods may be immovable either because of an electrical problem in the control rod drive circuitry or because the control rod is mechanically stuck. TSs allow operation with an immovable control rod as long as the control rod can be tripped and the requirements for rod misalignment and rod insertion limits are met.

Operating data show that the control rod exercise test has the potential for causing dropped rods or a reactor trip. Operating data also show that the frequency of stuck control rods is very low and that, when a rod has stuck, the condition of the reactor has remained within the bounds of the accident analysis even with the assumption that the single highest worth control rod does not trip. In addition, a review of the operating data shows that most stuck rods are not found by performing this test, but rather are found either at the beginning of a cycle while withdrawing the control rods before making the reactor critical or during low-power physics testing.

Point Beach-specific operating data for Units 1 and 2 show that the only rod motion failures have been electrical in nature. Therefore, in these cases the control rods remained trippable and continued to be capable of performing their intended safety function.

The licensee's proposed change is to the surveillance frequency only, and does not involve a change to the TS surveillance requirement itself or change the way the surveillance is performed. In addition, industry experience, as reported in NUREG-1366, "Improvements to Technical Specifications Surveillance Requirements," reveals the following:

- The purpose of pressurized water reactor (PWR) control rod movement tests is to detect rods that cannot move.
- Most stuck rods are discovered during plant startup, during initial pulling of the rods, or during drop testing.
- Control rod tests at power cause reactor trips, dropped rods, and unnecessary challenges to safety systems.

The report also recommended changing the PWR control rod movement tests to quarterly. This frequency change is also consistent with the frequency specified in NUREG-1431, "Standard Technical Specifications," Revision 1, and Generic Letter (GL) 93-05, "Line Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation." The licensee's request to increase the interval is further supported by the extremely low testing failure rate identified during past performance of the control rod exercise tests at Point Beach.

The NRC staff has reviewed the licensee's application and concludes that the proposed change is acceptable. The proposed testing period will reduce the potential for causing dropped rods or a reactor trip, and prevent unnecessary challenges to safety systems without diminishing the capability of detecting mechanical binding of control rods. The granting of this amendment is consistent with NUREG-1366, NUREG-1431, and GL 93-05, which suggest a quarterly or 92-day interval for control rod drive mechanism exercising, respectively.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these

amendments involve no significant hazards consideration and there has been no public comment on such finding (64 FR 73103). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Hatchett
B. Wetzel
M. Chatterton

Date: March 22, 2000