



**Northeast
Nuclear Energy**

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Millstone Nuclear Power Station
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B18009

The Northeast Utilities System

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

FEB 29 2000

Subject: Millstone Nuclear Power Station, Unit No. 1, Docket No. 50-245
Correction to Proposed Revision to Technical Specifications

On April 19, 1999, Northeast Nuclear Energy Company (NNECO) submitted a proposed revision to the Millstone Unit No. 1 (MP1) Technical Specifications to the U.S. Nuclear Regulatory Commission (NRC). This submittal requested that the MP1 Technical Specifications be revised to reflect the permanently defueled condition. In addition, the submittal proposed to revise the format of the Technical Specifications to be more consistent with the format of the Improved Standard Technical Specifications. In response to this request, the NRC issued Amendment No. 106 to the MP1 Technical Specifications on November 9, 1999. However, the NRC retained the Limiting Conditions for Operation in the previous format (i.e., Limiting Conditions page 3/4 10-3 of the previous Technical Specifications). This page contains the Technical Specification requirements for fuel storage pool water level, crane operability, and crane travel with a spent fuel cask. The NRC stated that they would act upon the request to revise the format of these requirements at a later date.

Subsequently, during their review of the proposed Limiting Conditions for Operation, the NRC discovered that proposed Surveillance Requirement (SR) 3.2.1 was not consistent with current SR 4.10.D. The last sentence of current SR 4.10.D had not been included in the new SR or its bases. This sentence states: "The above inspections and prelifting procedure shall meet the requirements of ANSI Standard B30.2, 1967." The submittal did not justify the deletion of this requirement. Via this submittal, NNECO is adding the sentence to proposed SR 3.2.1. This resolves the inconsistency, and ensures that the proposed SR is consistent with the current SR.

The following attachments are provided to support this correction to the proposed revision to Technical Specifications:

- Attachment 1 Marked-up Version of the Previously Proposed Technical Specifications; and
- Attachment 2 Typed Technical Specifications.

A001

There are no regulatory commitments contained within this letter. Should you have any questions or comments regarding this submittal, please contact Bryan Ford at (860) 437-5895.

Very truly yours,

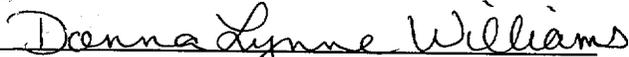
NORTHEAST NUCLEAR ENERGY COMPANY



L. J. Olivier
Senior Vice President and Chief Nuclear Officer-Millstone

Subscribed and sworn to before me

this 29 day of February, 2000



Deanna Lynne Williams
Notary Public

Date Commission Expires: Nov 30, 2001

cc: H. J. Miller, Region I Administrator
L. L. Wheeler, NRC Senior Project Manager, Millstone Unit No. 1
P. C. Cataldo, Resident Inspector, Millstone Unit No. 1

Director
Bureau of Air Management
Monitoring and Radiation Division
Department of Environmental Protection
79 Elm Street
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Attachment 1 to B18009

Millstone Nuclear Power Station Unit No. 1

Marked-Up of Previously Proposed Technical Specifications

3.2 SPENT FUEL HANDLING

3.2.1 Reactor Building Crane Operability

LCO 3.2.1 The Reactor Building crane shall be OPERABLE.

APPLICABILITY When the Reactor Building crane is used for handling of a spent fuel cask.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Reactor Building crane is INOPERABLE.	A.1 Suspend all Spent Fuel Cask handling and place the load in a safe condition.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>← SR 3.2.1 Conduct a visual inspection of crane cables, sheaves, hook, yoke, and cask lifting trunnions. Conduct no-load mechanical and electrical tests to verify proper operation of crane controls, brakes, and lifting speeds. Conduct a load test by lifting the empty cask out of the pivot cradle.</p>	<p>Within 4 days prior to Spent Fuel Cask handling operations and every 4 days thereafter during spent fuel cask handling</p>

The above inspections and prelifting procedure shall meet the requirements of ANSI Standard B30.2, 1967.

Attachment 2 to B18009

Millstone Nuclear Power Station Unit No. 1

Typed Technical Specifications

3.2 SPENT FUEL HANDLING

3.2.1 Reactor Building Crane Operability

LCO 3.2.1 The Reactor Building crane shall be OPERABLE.

APPLICABILITY When the Reactor Building crane is used for handling of a spent fuel cask.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Reactor Building crane is INOPERABLE.	A.1 Suspend all Spent Fuel Cask handling and place the load in a safe condition.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.2.1 Conduct a visual inspection of crane cables, sheaves, hook, yoke, and cask lifting trunnions. Conduct no-load mechanical and electrical tests to verify proper operation of crane controls, brakes, and lifting speeds. Conduct a load test by lifting the empty cask out of the pivot cradle. The above inspections and pre-lifting procedure shall meet the requirements of ANSI Standard B30.2, 1967.	Within 4 days prior to Spent Fuel Cask handling operations and every 4 days thereafter during spent fuel cask handling