

March 14, 2000

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: LASALLE - REQUEST FOR ADDITIONAL INFORMATION (TAC NOS. MA6070
AND MA6071)

Dear Mr. Kingsley:

By letter dated July 14, 1999, Commonwealth Edison Company (ComEd, the licensee) submitted a proposed license amendment for LaSalle County Station, Units 1 and 2, to allow the units to operate at an uprated power level of 3489 MWt. We have reviewed your request and determined that we need additional information to complete our review. The enclosed questions were discussed with your staff during a conference call on February 17, 2000. It was agreed that a response would be provided by March 15, 2000.

If there should be any questions regarding this request, please contact me at (301) 415-1322.

Sincerely,

/RA/

Donna M. Skay, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosure: Request for Additional Information

cc: See next page

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LaSalle County Station
Units 1 and 2

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Commonwealth Edison Company

- 2 -

LaSalle County Station
Units 1 and 2

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REQUEST FOR ADDITIONAL INFORMATION

LASALLE COUNTY STATION, UNITS 1 AND 2

1. What design bases parameters, assumptions or methodologies (other than the source term) were changed in the radiological design basis accident analyses as a result of the proposed change? If there are many changes it would be helpful to compare and contrast them in a table. Also, please provide justification for any changes.
2. NEDC-32701P, Revision 2, Section 8.3.2, states the following: "Consequently, in order to assure that the inventories of the long-lived isotopes are bounded, a limiting fission product activity inventory has been assessed based on a core irradiation of 1300 effective-full-power-days (EFPD) at a bounding power level." Please define the bounding power level for this analysis.
3. NEDC-32701P, Revision 2, Section 9.2, states the following: "The Main Steam Line Break Accident (MSLBA) outside containment for power uprate will release less steam and reactor coolant mass than the design basis modeling in the current Updated Final Safety Analysis Report (UFSAR) evaluation." This assessment of the MSLBA outside of containment does not address any potential increased source term, nor does it provide an assessment of the impact of the proposed change on this analysis. Please provide more justification and the resulting doses due to the proposed change.
4. NEDC-32701P, Revision 2, Section 9.2, discusses five "other" Chapter 15 events: Inadvertent main steam isolation valve (MSIV) Closure, Instrument Line Break, Feedwater Line Break, Radioactive Gas Waste System Leak or Failure, and Postulated Radioactive Releases Due to Liquid Radwaste Tank Failure. This section provides an "expected" increase for each of the accidents, but does not provide a bases for these numbers. Please provide a justification for the "expected" increases and the dose results from these analyses.
5. NUREG/CR - 5009, "Assessment of the Use of Extended Burnup Fuel in Light Water Power Reactors," indicates that for the fuel-handling accident, the release of I-131 could be 20% higher than those assumed in Regulatory Guide 1.25. Has the impact of increased gap fraction as a result of extended burnup fuel been included in the supporting analyses for the proposed change?
6. Table A-1 of Attachment A to the July 14, 1999, submittal provides the loss-of-coolant accident (LOCA) radiological consequences. The second column is labeled "pre-uprate dose" and states that prior approved Amendment No. 125 incorporated the new source terms at uprated conditions. These dose numbers are not included in amendment number 125 to NPF-11. Please clarify the source of these dose numbers.
7. What is the extended burnup source term power level used to calculate the LOCA radiological consequences?

ENCLOSURE