Tennessee Valley Authority, Post Office Box 2000, Soddy Daisy, Tennessee 37379-2000

Masoud Bajestani Site Vice President Sequoyah Nuclear Plant

February 25, 2000

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

10 CFR 50.73

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT (SQN)
UNIT 2 - DOCKET NO. 50-328 - FACILITY OPERATING LICENSES DPR-79 LICENSEE EVENT REPORT (LER) 50-328/2000002

The enclosed report provides details concerning a January 26, 2000, condition where the 2B safety injection pump and the 2A safety injection pump were concurrently out of service for approximately 30 minutes. This event is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B), as an operation prohibited by technical specifications, and 10 CFR 50.73(a)(2)(v)(D) as any event or condition that could have prevented the fulfillment of the safety function of systems that are needed to mitigate the consequences of an accident.

Sincerely,

Masoud Bajestani

Enclosure

cc: |\$ee page 2

IE22

Printed on recycled paper

U.S. Nuclear Regulatory Commission Page 2 February 25, 2000

Enclosure

cc (Enclosure):

INPO Records Center Institute of Nuclear Power Operations 700 Galleria Parkway Atlanta, Georgia 30339-5957

Mr. R. W. Hernan, Project Manager U.S. Nuclear Regulatory Commission One White Flint, North 11555 Rockville Pike Rockville, Maryland 20852-2739

NRC Resident Inspector Sequoyah Nuclear Plant 2600 Igou Ferry Road Soddy-Daisy, Tennessee 37379-3624

Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30323-3415

NRC FO (6-1998)	RM 366			U.S. NUCL	EAR RE	GU	ILATORY (	COMMIS	SSION	Estim	nated	VED BY OMB Not burden per reson collection reques	sponse to	comply	with this	s mandatory
LICE	NSEE	EVEN	IT REP	ORT (LER)						intorn incorp Forwa Mana	natio poral ard agen	n collection requested into the licens comments regardent Branch (T-6 F	st: 50 nrs. sing proces ding burde 33), U.S. Nr	Reporters and n estinuclear R	d lessons fed back nate to 1 legulatory	learned are to industry. the Records Commission,
		equired nu for each b								information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Records Management Branch (7-6 F33), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104). Office of Management and Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.						
FACILITY N	VAME (1)									perso DOCH	in is	not required to respond NUMBER (2)	pond to, the	e informa PAGE (3	ation collect	ction.
	• •	clear Pi	iant (SQ	N) UNIT 2						05000328				,,,,,,	" 1 OF	7
TITLE (4)										<u> </u>						
Activity	V		oth Safe	ety Injection Pur	<del></del>	; а —				rror D	)uri					
MONTH	DAY		VEAR	LER NUMBER (6	<del>,</del>	<u> </u>		DAY			OTHER FACILITIES FACILITY NAME					
MONTH	DAT	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBE		MONTH	DAY	YEAR	N/A		NAME		DOCKET NUMBER 05000		
01	26	2000	2000	002	00		02	25	2000		_	NAME		DOCKET NUMBER		
								<u></u>	<u> </u>	NA					0500	)0
OPER/		1	THIS RE	PORT IS SUBMITT	ED PUR	งรบ	IANT TO T	HE REG	JUIREMEN	NTS O	F 1	0 CFR §: (Cher	ok one or r	more)	(11)	
MOD	E (9)		20.27	201(b)		_	20.2203(a	a)(2)(v)			X	50.73(a)(2)(i)		50.73(a)(2)(viii)		
POW		100	II	203(a)(1)			20.2203(a					50.73(a)(2)(ii)		50.73(a)(2)(x)		')(x)
LEVE	_ (10)			203(a)(2)(i)			20.2203(a					50.73(a)(2)(iii)	$\longrightarrow$	73.71		
				203(a)(2)(ii)		—	20.2203(a		<del></del>			50.73(a)(2)(iv)		OTHER Specify in Abstract below or		
			<del>                                     </del>	203(a)(2)(iii)			50.36(c)(1				in NRC For			Form 366	6A	
LICENSE	EE CONT	CACT FO	20.22 R THIS LE	203(a)(2)(iv)			50.36(c)(2	2)				50.73(a)(2)(vii)			<del></del>	
NAME		AUTIO	X IIIIO LL	.R (12)				<del></del>			TEL	EPHONE NUMBER	(Include Are	a Code)		
J. W. F	'roffitt,	Licensi	ng Engir	neer								23) 843-6651	•			
CAUSE	TSYSTE	AL COL	COMPL PONENT	ETE ONE LINE FO	DE BOE	<u> </u>	OMPONEN	IT FAIL!							\ DE	
Unit of	31315	M   CC.	+	MANUFACTURER		EPIX			CAUSE	31316	STEM COMPONENT MANUF		MANUTA	FACTURER REPORTABLE TO EPIX		
	<u> </u>			<u> </u>					ļ							
		Sl	JPPLEME	NTAL REPORT EX	(PECTE	D (1							MONT	Н	DAY	YEAR
YES (If ye		ete EXPE	CTED SU	IBMISSION DATE).	•		X	NO	<u></u>	SUBMISSION DATE (15)						
Abstrac	t (Limi	t to 1400	J paces,	i.e., approximate	ely 15 si	ing	le-spaced	typew	ritten line	es) (1	6)			-		
On Ja	anuar	:y 25	, 200	0, at 202	3 Ea	st	ern s	tand	ard t	ime	ž	(EST), O	perat	ion	ıs en	tered
Limit	ting	Cond	ition	for Opera	ation	n	(LCO)	3.5	.2.a	for	. 1	removing	the	2A-	-A sa	fety
inje	ction	ı pum	p (SI	P) from so	ervi	ce	for i	main	tenan	ice	ac	ctivitie	s. A	ıs p	art	of.
the I	naint	enan.	ce ac	tivity, t	he o	11	for	the	2A-A	SIP	, I	notor wa	s to	be	chan	ıged
				6, 2000, atenance pe												OTD
				personne.												
both	trai	ns o	f eme	rgency co	re c	.00	ling	subs	vster	ıs b	e:	ing inop	erabl	le a	it th	ie
same	time	e. D	uring	the time	of t	th	ne oil	cha	nge o	ut,	k	ooth SIP	s wer	re c	consi	dered
				ause of the												
				There wa												
				ensure that												
				k. The inform self												
	- 4	. L C .	o per	TOTH SCTT	CIIC	C 12	.Tiig,	as w	ETT 0	.S a		-ack or	beer	CITE	CKTII	.g.

U.S. NUCLEAR REGULATORY COMMISSION (4-95)

# LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)
Sequoyah Nuclear Plant (SQN) Unit 2	05000328	YEAR	SEQUENTIAL NUMBER	REVISION	2 OF 7
		2000 -	- 002 -	- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

## I. PLANT CONDITION(S)

Unit 2 was in power operation at approximately 100 percent.

#### II. DESCRIPTION OF EVENT

## A. Event:

On January 25, 2000, at 2023 Eastern standard time (EST), Operations entered Limiting Condition for Operation (LCO) 3.5.2.a for removing the 2A-A safety injection pump (SIP) [EIIS Code BQ] from service for maintenance activities. One of the planned maintenance activities was a work order for changing the oil in the motor bearings. There are two bearings involved with the oil change out. Unknowingly, the Maintenance personnel changed the oil in the 2B-B SIP motor. The oil change takes approximately 15 minutes for each bearing. On January 26, 2000, at approximately 0220 EST, Operations was notified that Maintenance personnel had changed out the oil in the 2B-B SIP motor. Operations personnel entered LCO 3.0.3 (effective at 0039 EST) for both trains emergency core cooling subsystems being inoperable at the same time. During the time of the oil change out, both SIPs were considered inoperable.

# B. <u>Inoperable Structures, Components, or Systems that Contributed to the Even</u>t:

None.

## C. Dates and Approximate Times of Major Occurrences:

January 25, 2000, Operations entered LCO 3.5.2.a for at 2023 EST removing the 2A-A SIP from service for maintenance activities.

January 26, 2000, Maintenance personnel signed onto the at 0039 EST radiation work permit to perform maintenance on the 2A-A SIP.

U.S. NUCLEAR REGULATORY COMMISSION (4-95)

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

TEXT CONTINUATION									
FACILITY NAME (1)	DOCKET	L	ER NUMBER	(6)	PAGE (3)				
Sequoyah Nuclear Plant (SQN) Unit 2	05000328	YEAR	SEQUENTIAL NUMBER	REVISION	3 OF 7				
		2000 -	- 002 -	- 00					
TEXT (If more space is required, use additional copies of NRC Form 366A) (1	7)								
ra	Taintenance personnel entered the adiological controlled area (RCA) to eplace the oil in the 2A-A SIP.								
	nintenance p the oil fr								
th co pe Ma th pe pe	As Maintenance personnel were exiting the RCA, they set off the personnel contamination monitor alarms. Radcon personnel questioned where the Maintenance personnel had performed their activity. The Maintenance personnel stated that they had performed maintenance in the 2A-A SIP room.								
su pe	Radcon personnel went to perform a survey of the 2A-A SIP room. Radcon personnel did not identify any contamination.								
and 0215 EST Ma wh pe to SI	dcon person intenance pare the mai erformed. Took the Rado Proom and	erson ntena he Ma on pe	nel to since activintenance	how the vity wa e perso to the	s nnel 2B-B				
an Ma th SI Op 00 co	Radcon personnel notified Operations and Maintenance management that the Maintenance personnel had notified that they replaced the oil in the 2B SIP instead of the 2A-A SIP. Operations entered LCO 3.0.3 (effect 0039 EST) for both trains emergency core cooling subsystems being out of service at the same time.								

U.S. NUCLEAR REGULATORY COMMISSION (4-95)

# LICENSEE EVENT REPORT (LER)

**TEXT CONTINUATION** 

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)
Sequoyah Nuclear Plant (SQN) Unit 2	05000328	YEAR	SEQUENTIAL NUMBER	REVISION	4 OF 7
		2000 -	- 002 -	- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

at 0255 EST Operations started the 2B-B SIP for the postmaintenance test (PMT) for the oil

replacement.

at 0316 EST

Operations stopped the 2B-B SIP following completion of the PMT run time. Operations declared the 2B SIP operable and exited LCO 3.0.3.

# D. Other Systems or Secondary Functions Affected:

None.

## E. <u>Method of Discovery</u>:

As the Maintenance personnel were exiting the radiological control area, the personnel contamination monitor alarmed. Radcon personnel questioned where the Maintenance personnel had performed their activity and determined that the maintenance had been performed on the wrong equipment.

#### F. Operator Actions:

Control room operators entered LCO 3.0.3 for both trains emergency core cooling subsystems being inoperable at the same time. Operations started the 2B-B SI pump as the PMT for the change out of the oil.

#### G. Safety System Responses:

None.

#### III. CAUSE OF THE EVENT

#### A. Immediate Cause:

The immediate cause of the event was performing a maintenance activity on one train of equipment with the opposite train of equipment out of service.

U.S. NUCLEAR REGULATORY COMMISSION (4-95)

# LICENSEE EVENT REPORT (LER)

**TEXT CONTINUATION** 

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)
Sequoyah Nuclear Plant (SQN) Unit 2	05000328	YEAR	SEQUENTIAL NUMBER	REVISION	5 OF 7
		2000 -	- 002 -	- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

#### B. Root Cause:

The root cause of the oil change on the wrong SIP motor was human performance error. There was a lack of self checking, as well as a lack of peer checking, to ensure that the correct component was identified prior to performance of work.

#### B. Contributing Factors:

Roles and responsibilities were not clearly defined. Two workers were assigned to perform the task. Responsibility for a lead to self check the correct component was not discussed. Responsibility for the coworker to peer check in order to ensure that the correct component was identified was not assigned.

#### IV. ANALYSIS OF THE EVENT

The design basis of the emergency core cooling system (ECCS) is to provide two independent trains of ECCS for accident mitigation. During the time the 2B-B SIP was out of service because of the oil change, the 2A-A SIP was also out of service. The total time both pumps were out of service was approximately 30 minutes.

For a large-break loss-of-coolant accident (LOCA), the reactor coolant system (RCS) will depressurize rapidly. The safety analysis assumes one complete train of ECCS (centrifugal charging pump [CCP], SIP, and residual heat removal [RHR] pump) delivers flow to the reactor coolant system during the reflood phase. Two RHR pumps were available to deliver flow during the period of time that both SIPs were out of service. The flow delivered by two RHR pumps bounds the analysis assumptions.

For a small-break LOCA, a much slower depressurization of the RCS will occur. The safety analysis assumes one complete train of high-head safety injection flow (one CCP and one SIP). With both CCPs available and no SIP, the CCPs will deliver greater than or equal amount of flow to the cold leg than required.

U.S. NUCLEAR REGULATORY COMMISSION (4-95)

# LICENSEE EVENT REPORT (LER)

**TEXT CONTINUATION** 

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)	
Sequoyah Nuclear Plant (SQN) Unit 2	05000328	YEAR	SEQUENTIAL NUMBER	REVISION	6 OF 7	
		2000 -	- 002 -	- 00		

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

In the unlikely event of an accident situation requiring an SIP to actuate during the period of time that both SIPs were inoperable, other ECCS equipment would have been available. Also, it is expected that the operator would start an SIP within a reasonable amount of time after the initiation of the transient, given the early steps within the emergency procedures to verify pump operation. The change in core damage probability is 5.86E-09 with both SIPs out of service for 30 minutes. Therefore, there was minimal actual safety significance that could have arose during this period.

#### V. CORRECTIVE ACTIONS

## A. Immediate Corrective Actions:

A PMT was performed on the 2B-B SIP and the pump was declared operable.

#### B. Corrective Actions to Prevent Recurrence:

The involved individuals were coached and counseled on the failure to perform self checking, as well as a lack of peer checking.

Additionally, the associated Corrective Action Program document contains actions to define expectations for coworkers when more than one person is assigned to perform a task. This should include clear responsibility for conducting peer checks to ensure correct components.

This event and other human performance issues were reviewed with plant personnel by senior site management.

#### VI. ADDITIONAL INFORMATION

#### A. Failed Components:

None.

U.S. NUCLEAR REGULATORY COMMISSION (4-95)

## LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)	
Sequoyah Nuclear Plant (SQN) Unit 2	05000328	YEAR	YEAR SEQUENTIAL NUMBER		7 OF 7	
		2000 -	- 002 -	- 00		

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

# B. <u>Previous LERs on Similar Events</u>:

A review of previous reportable events for the past three years did not identify any events involving maintenance activities on wrong or incorrect equipment.

# C. Additional Information:

None.

## D. Safety System Functional Failure:

This event is considered to be a safety system functional failure in accordance with NEI 99-02.

## VII. COMMITMENTS

None.