

January 19, 2000

Mr. Robert P. Powers, Senior Vice President
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

*Tempo Late
NR R-058*

SUBJECT: ISSUANCE OF AMENDMENTS - DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. MA6885 AND MA6886)

Dear Mr. Powers:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 240 to Facility Operating License No. DPR-58 and Amendment No. 221 to Facility Operating License No. DPR-74 for the Donald C. Cook Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated October 12, 1999.

The amendments would revise TSs Surveillance Requirement 4.6.2.2.d for the spray additive system to relocate the details associated with the acceptance criteria and test parameters to the associated TSs Bases. Additionally, certain administrative text format changes are proposed.

A copy of our related safety evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

John F. Stang, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

- Enclosures: 1. Amendment No. 240 to DPR-58
- 2. Amendment No. 221 to DPR-74
- 3. Safety Evaluation

cc w/encls: See next page

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DATE	12/29/99		12/29/99		01/14/99	1/14/99	1/19/99	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 19, 2000

Mr. Robert P. Powers, Senior Vice President
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

SUBJECT: ISSUANCE OF AMENDMENTS - DONALD C. COOK NUCLEAR PLANT, UNITS 1
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Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: 1. Amendment No. 240 to DPR-58
2. Amendment No. 221 to DPR-74
3. Safety Evaluation

cc w/encls: See next page

Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, IL 60532-4351

Attorney General
Department of Attorney General
525 West Ottawa Street
Lansing, MI 48913

Township Supervisor
Lake Township Hall
P.O. Box 818
Bridgman, MI 49106

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
7700 Red Arrow Highway
Stevensville, MI 49127

David W. Jenkins, Esquire
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

Mayor, City of Bridgman
P.O. Box 366
Bridgman, MI 49106

Special Assistant to the Governor
Room 1 - State Capitol
Lansing, MI 48909

Drinking Water and Radiological
Protection Division
Michigan Department of
Environmental Quality
3423 N. Martin Luther King Jr Blvd
P.O. Box 30630, CPH Mailroom
Lansing, MI 48909-8130

Robert C. Godley
Director, Regulatory Affairs
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

David A. Lochbaum
Union of Concerned Scientists
1616 P Street NW, Suite 310
Washington, DC 20036-1495

A. Christopher Bakken, Site Vice President
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

Michael W. Rencheck
Vice President, Nuclear Engineering
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

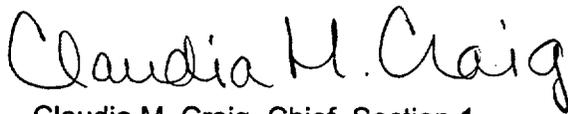
Amendment No. 240
License No. DPR-58

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated October 12, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 240, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.
3. This license amendment is effective as of its date of issuance and shall be implemented at the facility within 30 days. In addition, the licensee shall include the relocated information in the bases of the Technical Specifications as described in the licensee's application dated October 12, 1999, and evaluated in the staff's safety evaluation dated January 19, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: January 19, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 240

TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

3/4 6-13

3/4 6-13

B3/4 6-3

B3/4 6-3

3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS
3/4.6 CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 18 months during shutdown, by verifying that each automatic valve in the flow path actuates to its correct position on a Containment Pressure -- High-High signal.
- d. At least once per 5 years by verifying the flow rate from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation.

3/4 BASES
3/4.6 CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

3/4.6.2.1 CONTAINMENT SPRAY SYSTEM

The OPERABILITY of the containment spray system ensures that containment depressurization and cooling capability will be available in the event of a LOCA. The pressure reduction and resultant lower containment leakage rate are consistent with the assumptions used in the accident analyses.

3/4.6.2.2 SPRAY ADDITIVE SYSTEM

The OPERABILITY of the spray additive system ensures that sufficient NaOH is added to the containment spray in the event of a LOCA. The limits on NaOH minimum volume and concentration, ensure that 1) the iodine removal efficiency of the spray water is maintained because of the increase in pH value, and 2) corrosion effects on components within containment are minimized. These assumptions are consistent with the iodine removal efficiency assumed in the accident analyses.

Surveillance Requirement 4.6.2.2.d is performed by verifying a water flow rate ≥ 20 gpm and ≤ 50 gpm from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation with a pump discharge pressure ≥ 255 psig.

3/4.6.3 CONTAINMENT ISOLATION VALVES

The OPERABILITY of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment. Containment isolation within the time limits specified ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for a LOCA.

The opening of containment purge and exhaust valves and locked or sealed closed containment isolation valves on an intermittent basis under administrative control includes the following considerations: (1) stationing a qualified individual, who is in constant communication with control room, at the valve controls, (2) instructing this individual to close these valves in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valves and that this action will prevent the release of radioactivity outside the containment.

3/4.6.4 COMBUSTIBLE GAS CONTROL

The OPERABILITY of the equipment and systems required for the detection and control of hydrogen gas ensures that this equipment will be available to maintain the hydrogen concentration within containment below its flammable limit during post-LOCA conditions. Either recombiner unit is capable of controlling the expected hydrogen generation associated with: 1) zirconium-water reactions; 2) radiolytic decomposition of water; and 3) corrosion of metals within containment.

The acceptance criterion of 10,000 ohms is based on the test being performed with the heater element at an ambient temperature, but can be conservatively applied when the heater element is at a temperature above ambient.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 221
License No. DPR-74

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated October 12, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

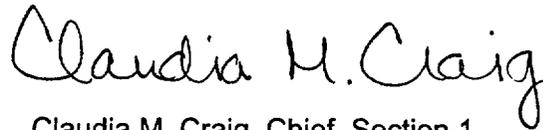
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 221, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented at the facility within 30 days. In addition, the licensee shall include the relocated information in the bases of the Technical Specifications as described in the licensee's application dated October 12, 1999, and evaluated in the staff's safety evaluation dated January 19, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: January 19, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 221

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3/4 6-12

B3/4 6-3

INSERT

3/4 6-12

B3/4 6-3

3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS
3/4.6 CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 18 months during shutdown, by verifying that each automatic valve in the flow path actuates to its correct position on a Containment Pressure--High-High test signal.[†]
- d. At least once per 5 years by verifying the flow rate from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation.

[†]The provisions of Technical Specification 4.0.8 are applicable.

3/4 BASES
3/4.6 CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

3/4.6.2.1 CONTAINMENT SPRAY SYSTEM

The OPERABILITY of the containment spray system ensures that containment depressurization and cooling capability will be available in the event of a LOCA. The pressure reduction and resultant lower containment leakage rate are consistent with the assumptions used in the accident analyses.

3/4.6.2.2 SPRAY ADDITIVE SYSTEM

The OPERABILITY of the spray additive system ensures that sufficient NaOH is added to the containment spray in the event of a LOCA. The limits on NaOH volume and concentration ensure a pH value of between 8.5 and 11.0 for the solution recirculated within containment after a LOCA. This pH band minimizes the evolution of iodine and minimizes the effect of chloride and caustic stress corrosion on mechanical systems and components. These assumptions are consistent with the iodine removal efficiency assumed in the accident analyses.

The contained water volume limit includes an allowance for water not usable because of tank discharge location or other physical characteristics.

Surveillance Requirement 4.6.2.2.d is performed by verifying a water flow rate ≥ 20 gpm and ≤ 50 gpm from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation with a pump discharge pressure ≥ 255 psig.

3/4.6.3 CONTAINMENT ISOLATION VALVES

The OPERABILITY of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment. Containment isolation within the time limits specified ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for a LOCA.

The opening of containment purge and exhaust valves and locked or sealed closed containment isolation valves on an intermittent basis under administrative control includes the following considerations: (1) stationing a qualified individual, who is in constant communication with control room, at the valve controls, (2) instructing this individual to close these valves in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valves and that this action will prevent the release of radioactivity outside the containment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 240 TO FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 221 TO FACILITY OPERATING LICENSE NO. DPR-74
INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By application dated October 12, 1999, the Indiana Michigan Power Company (the licensee) requested amendments to the Technical Specifications (TSs) for the Donald C. Cook Nuclear Plant, Units 1 and 2. The proposed amendments would revise TS Surveillance Requirement (SR) 4.6.2.2.d for the spray additive system to relocate the details associated with the acceptance criteria and test parameters to the associated TSs Bases. Additionally, the proposed amendments will make administrative text format changes to the TSs.

2.0 EVALUATION

2.1 Proposed Changes to Unit 1 and 2 TS Surveillance Requirement 4.6.2.2.d, and TS Bases 3/4 4.6.2.2

Current TSs SR 4.6.2.2.d includes details regarding the required flow rate and test parameters for the spray additive system (details of the acceptance criteria and test parameter for the flow rate verification surveillance). The current Unit 1 and 2 TS SR 4.6.2.2.d states that "At least once per 5 years by verifying a water flow rate of at least 20 gallons per minute (gpm) (greater than or equal to 20 gpm) but not to exceed 50 gpm (less than or equal to 50 gpm) from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation with a pump discharge pressure greater than or equal to 255 psig." The licensee proposes to change the TSs SR to state "At least once per 5 years by verifying the flow rate from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation." The licensee states that the test parameters and the acceptance criteria (i.e., a flow rate between 20 gpm and 50 gpm with a pump discharge pressure of 250 psig) in TSs SR 4.6.2.2.d are proposed to be relocated to the TS Bases 3/4 4.6.2.2. Therefore, the licensee proposes to add the following sentence to Unit 1 and 2 TS Bases 3/4.6.2.2; "SR 4.6.2.2.d is performed by verifying a water flow rate of greater or equal to 20 gpm and less than or equal to 50 gpm from the spray additive tank test line to each containment spray system with the spray pump operating on recirculation with a pump discharge pressure greater or equal to 255 psig." The above details currently contained in SR

4.6.2.2.d are not necessary to ensure the operability of the spray additive system. The current TS requirements contained in the Limiting Condition for Operation (LCO) 3.6.22, "Spray Additive System," and the proposed SR are adequate to ensure the spray additive system is operable and can perform its intended function.

TS 4.6.2.2.d contain the containment spray additive system eductor testing parameters and acceptance limits. The limiting conditions for operation for the Containment Spray Additive System specify the system shall be operable. These testing parameters specify the condition at which the testing is performed and the acceptance limits are the acceptance criteria for the spray eductor testing performed to satisfy the surveillance requirements in TS 4.6.2.2.d. This surveillance ensures that the spray eductor's performance is consistent with the assumption for the safety analyses performed for design basis accidents and transients. The changes involve only the relocation of the details associated with the containment spray additive system's eductor testing parameters and acceptance limits but retain the surveillance requirement to perform containment spray additive system eductor testing. The TS Bases will now contain the eductor testing parameters and acceptance limits for the required spray eductor surveillance.

Although the testing parameters and acceptance limits are relocated from the TS to the TS Bases, the licensee must continue to evaluate any changes to testing requirements in accordance with 10 CFR 50.59. Should the licensee's determination conclude that an unreviewed safety question is involved, due to either (1) an increase in the probability or consequences of accidents or malfunctions of equipment important to safety, (2) the creation of a possibility for an accident or malfunction of a different type than any evaluated previously, or (3) a reduction in the margin of safety, NRC approval and a license amendment would be required prior to implementation of the change.

The staff has reviewed the testing parameters and acceptance limits associated with the containment spray pump proposed to be relocated from the TS to the TS bases against the criteria of 10 CFR 50.36(c)(2)(ii) and determined that none of the criteria applies as discussed below:

Criterion 1: Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.

The surveillance requirement acceptance criteria and the test parameter are not instrumentation and do not affect instrumentation. Therefore, they are not instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.

Criterion 2: A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

The surveillance requirement acceptance criteria and the test parameter are not process variables, design features, or operating restrictions. The surveillance requirement acceptance criteria and the test parameter do not affect process variables, design features, or operating restrictions. Therefore, they are not a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or

transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

Criterion 3: A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

The surveillance requirement acceptance criteria and the test parameter are not structures, systems, or components. The retained surveillance requirement still ensures the containment spray eductors are capable of performing their safety functions to mitigate design basis accidents. Therefore, they are not structures, systems, or components that are part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

Criterion 4: A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

The surveillance requirement acceptance criteria and the test parameter are not structures, systems, or components. Therefore, they are not structures, systems, or components which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Existing TS requirements which fall within or satisfy any of the above criteria must be retained in the TS, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

The design details do not meet the criteria of 10 CFR 50.36(c)(2)(ii) as being required to be included in the TS, and therefore the staff finds the testing parameters and acceptance limits associated with the containment spray pumps may be relocated from the TS to owner controlled documents. These details will be relocated to the TS bases and all changes will be controlled pursuant to 10 CFR 50.59.

The relocation of the above details from the SR to the TS Bases does not eliminate current TS requirements associated with the required testing of the spray additive system, and will not affect the system operability. In addition, the relocation of the details associated with the acceptance criteria and test parameters to the associated TS Bases is consistent with NUREG-1431 "Standard Technical Specifications, Westinghouse Plants, Specifications," Revision 1.

The NRC staff finds that the proposed changes do not constitute a reduction in safety and do not alter the requirement of TS 4.6.6.2.d. The proposed changes are intended to allow for a more complete and accurate testing of the containment spray pump, and timely revision of the parameters upon any future changes in the analyses and calculations associated with the spray additive system. This is consistent with NUREG-1431. Therefore, the staff finds the proposed changes acceptable.

2.2 Proposed Editorial Changes to Unit 1 TS Page 3/4 6-13, Unit 1 TS Bases Page B 3/4 6-3, Unit 2 TS Page 3/4 6-12, and Unit 2 TS Bases Page B 3/4 6-3

The licensee is proposing certain format changes to Unit 1 TS page 3/4 6-13, Unit 1 TS Bases page B 3/4 6-3, Unit 2 TS page 3/4 6-12, and Unit 2 TS Bases page B 3/4 6-3 to correct minor differences in margins and text spacing due to variations in word processing and reprographic technologies. In addition, there are also specific format changes affecting the Unit 2 Bases page B 3/4 6-3. These specific changes include (1) the use of a different font which also results in altered spacing of the text on the page and content for each line of text, (2) the use of horizontal bars to separate the footer and header from the body of the page, (3) the addition of numerical annotation (i.e., 3/4 and 3/4.6 in the header text lines), (4) the removal of underlining from the two lines of header text, (5) the reversal of sequence and deletion of a blank line between the two lines of header text, (6) the removal of spaces immediately preceding and following the hyphen in the footer text, "COOK NUCLEAR PLANT-UNIT 2," (7) the addition of the word "Page" prior to the page number, and (8) the removal of the word "NO." following the word "AMENDMENT" and prior to the historical and current amendment numbers.

The staff finds that the proposed editorial changes do not represent a reduction in safety or alter any requirement. The editorial changes are intended to maintain consistency and enhance usability and clarity of the TS. Therefore, the proposed changes are acceptable.

3.0 SUMMARY

The proposed amendment would revise TS SR 4.6.2.2.d for the spray additive system to relocate the details associated with the acceptance criteria and test parameters to the appropriate TS Bases. Additionally, the proposed amendment allows for administrative text format changes. The proposed amendment does not cause changes to accident initiators or precursors, or to the accident analyses, and does not involve a significant reduction of safety.

Based on the above evaluation, the staff finds that the proposed changes to the TSs are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change the requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (64 FR 59804). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b),

no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Leigh
J. Stang

Date: January 19, 2000