

**Florida  
Power**  
CORPORATION  
Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72

February 2, 2000  
3F0200-08

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555-0001

Subject: Final Safety Analysis Report Revision 26 and 10CFR50.59 Report

Reference: NRC to FPC letter, 3N0899-17, dated August 30, 1999, From Brenda J. Shelton

Dear Sir:

In accordance with 10 CFR 50.71(e), Florida Power Corporation (FPC) hereby submits Revision 26 to the Crystal River Unit 3 (CR-3) Final Safety Analysis Report (FSAR). This submittal incorporates the exception to 10 CFR 50.4(b)(6) as stated in the above referenced letter, in that one original, one paper copy, and four CD-ROM copies are being sent to the Document Control Desk, and one CD-ROM copy to the Regional Administrator. Also, the Senior Resident Inspector will receive a paper copy via FPC's internal controlled document distribution system and one CD-ROM copy.

This revision is provided as a page replacement of the FSAR, dated January 20, 2000. FSAR text changes are indicated by revision bars on the outside border of the page. Using the existing Revision 25 binders, please replace all pages with the Revision 26 pages per the attached instructions. The CD-ROM version includes an Adobe Acrobat Reader to support its use. Adobe Acrobat was upgraded to Version 4.0 for this revision of the FSAR.

This FSAR revision includes material which describes the organization, the modifications, flow diagrams, and changes to CR-3 that have been implemented as of December 31, 1999. As required by 10 CFR 50.71(e), a summary of changes made in FSAR Revision 26 is provided in Attachment A. Additionally, as required by 10 CFR 50.59(b)(2), Attachment B includes a summary of each safety evaluation performed during this submittal period.

*A053 1/2 Hand  
copies 4 CD  
Rams*

There are no new commitments made within this letter. If you have any questions regarding this submittal, please contact Mr. Sidney Powell, Manager, Nuclear Licensing at (352) 563-4883.

Sincerely,



J.J. Holden  
Vice President and Site Director

JJH/pei

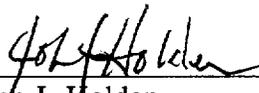
xc: Regional Administrator, Region II - w/Attachments A and B only  
Senior Resident Inspector - w/Attachments A and B only  
NRR Project Manager - w/Attachments A and B only

Attachments: A. FSAR Change Summary  
B. FSAR Safety Evaluation Summaries, 10 CFR 50.59(b)(2)  
C. FSAR Page Replacement Instructions  
D. FSAR Revision 26 Replacement Pages

STATE OF FLORIDA

COUNTY OF CITRUS

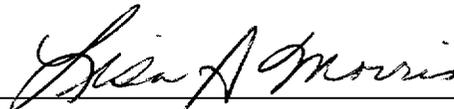
John J. Holden states that he is the Vice President and Site Director for Florida Power Corporation; that he is authorized on the part of said company to sign and file with the Nuclear Regulatory Commission the information attached hereto; and that all such statements made and matters set forth therein are true and correct to the best of his knowledge, information, and belief.

  
\_\_\_\_\_  
John J. Holden  
Vice President and Site Director

Sworn to and subscribed before me this 2<sup>nd</sup> day of February 2000, by  
John J. Holden.



LISA A. MORRIS  
Notary Public, State of Florida  
My Comm. Exp. Oct. 25, 2003  
Comm. No. CC 879691

  
\_\_\_\_\_  
Signature of Notary Public  
State of Florida

LISA A. MORRIS  
\_\_\_\_\_  
(Print, type, or stamp  
Commissioned Name of Notary Public)

Personally Produced  
Known X OR Identification \_\_\_\_\_

**FLORIDA POWER CORPORATION  
CRYSTAL RIVER UNIT 3  
DOCKET NO. 50-302/LICENSE NO. DPR-72**

**ATTACHMENT A**

**FINAL SAFETY ANALYSIS REPORT REVISION 26**

**FSAR Change Summary**

## ATTACHMENT A

### FSAR CHANGE SUMMARY

#### Final Safety Analysis Report Revision 26 Changes

This Final Safety Analysis Report (FSAR) revision reflects License Amendments issued by the NRC along with other plant modifications, information, and analyses that constitute changes to the FSAR since the publication of FSAR Revision 25. This FSAR revision includes the following changes:

A. Changes have been made as a result of the following License Amendments:

1. Amendment No. 168, Organization and Title Changes (Designation of Chief Nuclear Officer) (TAC NO. MA1493)
2. Amendment No. 170, Reactor Coolant Pump Flywheel Inspection Requirements (TAC NO. MA2145)
3. Amendment No. 171, Post-LOCA Boron Dilution Precipitation Prevention (TAC NO. M99892)
4. Amendment No. 172, Steam Generator Tube Surveillance Program (TAC NO. M99721)
5. Amendment No. 173, Makeup System Letdown Line Failure Accident Analysis (TAC NO. M99571)
6. Amendment No. 174, Subcooling Margin Monitoring Using Safety Parameter Display System (TAC NO. MA4147)
7. Amendment No. 175, Methodology Change for Boroflex Degradation (TAC NO. MA4148)
8. Amendment No. 177, Post-Accident Monitoring Instrumentation (TAC NO. MA3612)
9. Amendment No. 178, Reactor Protection System and Engineered Safeguards Actuation System Setpoints and Surveillance Requirements (TAC NO. MA3614)

10. Amendment No. 179, Basis for Reactor Coolant System Leakage Detection Instrumentation (TAC NO. MA3755)
  11. Amendment No. 180, Repair Criteria for Steam Generator Tubing (TAC NO. MA3592)
  12. Amendment No. 181, Basis for Pipe Break Criteria (TAC NO. MA2128)
  13. Amendment No. 182, Addition of a Safety-Related Diesel-Driven Emergency Feedwater Pump (TAC NO. MA3613)
  14. Amendment No. 183, Revised Pressure/Temperature Limits Report and Low Temperature Overpressure Protection Limits (TAC NO. MA4146)
  15. Amendment No. 184, Containment Closure Requirements During Refueling Operations (TAC NO. MA4344)
  16. Amendment No. 185, Control Room Emergency Ventilation System and Ventilation Filter Testing Program (TAC NO. MA0667)
  17. Amendment No. 188, Alternate Repair Criteria for Steam Generator Tubing (TAC NO. MA5395)
- B. Numerous plant modifications and analysis revisions were made. The most notable modifications were the addition of a diesel-driven emergency feedwater pump (SA/USQD 98-0520) and changes to the high pressure injection system flowpath and actuation setpoints (SA/USQD 99-0010). The most notable analysis revision was a change of the analysis of record for loss-of-coolant accidents (LOCAs) from CRAFT to RELAP (SA/USQD 99-0310).
- C. Numerous editorial and clarification changes were made throughout the document. Each change was evaluated for 10 CFR 50.59 applicability utilizing FPC procedures.
- D. Several changes are noteworthy because they involve deletion of information from the FSAR. These changes were evaluated per the guidance of NEI 98-03, Revision 1 and determined to be appropriate. The following Safety Assessment/Unreviewed Safety Question Determinations (SA/USQD) involved the deletion of information from the FSAR:
- SA/USQD 98-0348 involved complete revision of the post-accident hydrogen generation calculation. This change completely rewrote Appendix 14B and deleted the entire Appendix 14C. The revised analysis, contained in Appendix 14B, no

longer utilized the information in Appendix 14C. Therefore, Appendix 14C was deleted because it was obsolete.

- SA/USQD 99-0010 includes deletions of information that was made obsolete by the high pressure injection (HPI) modification. This SA/USQD also supersedes the NRC Safety Evaluation of the Proposed ECCS Modification for Small Break LOCA for Crystal River Unit 3, dated May 29, 1979. The information in this Safety Evaluation is no longer applicable since the new HPI crosstie lines eliminate the need to switch power supplies on the HPI injection valves (MUV-23, MUV-24, MUV-25 and MUV-26) or maintain all HPI header valves (MUV-3, MUV-4, MUV-8 and MUV-9) open.
- SA/USQD 99-0148 revises the analysis for the design basis fuel handling accident and deletes a “realistic case” analysis that is obsolete.
- SA/USQD 99-0193 deleted excessive detail on the aluminum inventory in the reactor building that is more properly maintained in a calculation. The total weight of the aluminum inventory is retained in the FSAR.
- SA/USQD 99-0117 deleted a realistic analysis (Table 14-43) associated with letdown line rupture. This SA/USQD is not included in this submittal since a full unreviewed safety question determination was not required because the change to the letdown line analysis was approved in Amendment 173. This change deleted information that was no longer relevant to the CR-3 licensing and design basis.