

January 3, 2000

Mr. J. V. Parrish  
Chief Executive Officer  
Energy Northwest  
P.O. Box 968 (Mail Drop 1023)  
Richland, WA 99352-0968

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) FOR WNP-2 (TAC NO.  
MA6166)

Dear Mr. Parrish:

By letter dated July 29, 1999, Energy Northwest submitted for staff review an amendment request to revise Technical Specification 3.4.9, Residual Heat Removal Shutdown Cooling - Hot Shutdown. As a result of the review, the staff has determined that additional information is needed to complete the review. The information needed is detailed in the enclosure.

The enclosed request was discussed with Mr. Arbuckle of your staff on November 17, 1999. A mutually agreeable target date of January 15, 2000, was established for responding to the RAI. If circumstances result in the need to revise the target date, please call me at your earliest opportunity at (301) 415-1424.

Sincerely,

***/RA/***

Jack Cushing, Project Manager, Section 2  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-397

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

RESIDUAL HEAT REMOVAL SHUTDOWN COOLING - HOT SHUTDOWN

WNP-2

DOCKET NO. 50-397

In its July 29, 1999 submittal, the licensee stated that the basis for the requested technical specification (TS) change is that the original plant design operating temperature for the residual heat removal (RHR) shutdown cooling (SDC) piping and supports is less than the operational limit currently required by TS Limiting Condition for Operation (LCO) 3.4.9. During a conference call with the staff on November 17, 1999, the licensee stated that in 1988, an evaluation was performed to assess the condition of the RHR SDC piping system because of the potential of exposing the piping system to beyond original design operating temperature. The licensee is requested to provide the details of that 1988 assessment (with respect to thermal stress limit and thermal fatigue cycle limit) and its findings.

Enclosure