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PROCEEDINGS OF THE WORKSHOP ON DIGITAL TWIN APPLICATIONS FOR ADVANCED NUCLEAR TECHNOLOGIES

Virtual Workshop December 1–4, 2020

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Prepared by:

- V. Yadav
- E. Sanchez
- A. Gribok
- C. Chwasz
- R. Hays
- H. Zhang
- N. Lybeck

Idaho National Laboratory

- R. Gascot
- D. Eskins
- D. Ju
- R. lyengar

U.S. Nuclear Regulatory Commission

Research Information Letter Office of Nuclear Regulatory Research

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EXECUTIVE SUMMARY

The Office of Nuclear Regulatory Research (RES) at the U.S. Nuclear Regulatory Commission (NRC) has initiated a future focused research project to assess the regulatory viability of digital twins for nuclear power plants. The objectives of this project are to:

- Understand the current state of the technology and potential applications for the nuclear industry.
- Identify and evaluate technical issues that could benefit from regulatory guidance, and
- Develop infrastructure to support regulatory decisions associated with digital twins.

As part of the project, the Office of Nuclear Regulatory Research (RES) sponsored the Virtual Workshop on Digital Twin Applications for Advanced Nuclear Technologies. The workshop was hosted by Idaho National Laboratory in collaboration with Oak Ridge National Laboratory and was held on December 1-4, 2020.

The 4 days of the workshop comprised 13 technical and panel sessions with over 50 presenters from a wide range of national and international organizations, including universities, national laboratories, government agencies, nuclear vendors, advanced reactor vendors, and digital twin vendors. With over 400 participants across the globe, the workshop provided a forum for nuclear industry and digital twin stakeholders to discuss the state of knowledge and research activities related to digital twins and their application in the nuclear industry.

The workshop had two main purposes: (1) to review and exchange information on the current understanding of digital twin technologies, and (2) to identify the potential benefits, opportunities, and challenges of applying digital twin technologies to nuclear reactors. The workshop covered specific topics such as applications to advanced reactors, nonnuclear applications, cybersecurity, and regulatory impacts.

In the opening session, on Tuesday, December 1, Ms. Stephanie Coffin, Deputy Director of RES, made introductory remarks. Mr. Jeremy Bowen, Deputy Director of the RES Division of Engineering, moderated the opening plenary session, entitled "Reactor Digital Twins—Shifting the Paradigm." Technical sessions on specific topics took place Tuesday through Friday. Each session consisted of technical presentations followed by question -and -answer periods. On Friday morning, Mr. Ray Furstenau, Director of RES, moderated the closing plenary session entitled "Digital Twin – Regulatory Discussion".

The following are some major takeaways from the workshop:

- There is significant interest in digital twin technology because it affects many industries. Applications of digital twin technology in nuclear reactors are expected to increase; thus, early engagement with regulators is important.
- Digital twin technology is improving rapidly, with intense activity from a variety of stakeholders. Current efforts aim to establish proof -of -concept use cases, build consensus on definitions, develop advanced sensors to enable digital twin technology, establish best practices and standards, and define cybersecurity requirements.

- Digital twin technology holds promise of benefits such as improved design, reduced uncertainty (including regulatory uncertainty), reduced risk, and improved prognostics and diagnostics.
- Increased collaboration and coordination would allow digital twin stakeholders to share information, develop common solutions to shared challenges, and establish a community of practice for applications in advanced nuclear technologies. Collaborative research would help address unresolved issues.
- Digital twin technology could be a novel source of trusted information on plant design, performance monitoring and prediction, process optimization, and regulation. The technology serves as a tool for general integrated data -sharing among vendors, licensees, regulators, and the public. Such an information source could both build public trust and improve regulatory efficiency.
- Planned collaborative and informational activities include follow-on- workshops focused on technical issues, technological advances, industry plans, and regulatory topics.
- The NRC plans to issue several technical reports detailing the following:
 - the state of technology for applications of digital twins,
 - the state of the art, technical challenges, and gaps for using digital twins in data analytics, machine learning (ML), artificial intelligence (AI), and multiphysics models,
 - regulatory readiness levels and gaps in applying digital twins for nuclear reactor applications, and
 - a summary report of technical and regulatory gaps
- Workshop participants identified several topics related to digital twin technology and safety that would be of interest for collaborative research:
 - development of a common language or taxonomy for digital twin stakeholders to use in communication, collaboration, and research,
 - development of advanced sensors and an approach for sufficient instrumentation of nuclear plants to enable digital twin use,
 - development of holistic nuclear life cycle models that could be integrated within a digital twin, covering requirements, design, testing, implementation, and change management,
 - development of data types, aggregation methods, and abstractions needed to implement digital twin technology,
 - development of data driven- model technologies to allow continuous model updates in response to real-time- plant data,
 - development of tools to characterize the interface between digital twin technology and human operators, including possible effects on cost, efficiency, and plant processes,
 - establishment of a Community of Practice with specific focus on digital twin applications for advanced nuclear technologies, and
 - establishment of a crowdsourcing platform for sharing of models, algorithms, and best practices

All presentation slides from this workshop are available in the NRC's Agencywide Documents Access and Management System, under Accession Nos. ML20356A234, ML20356A235, ML20356A236, and ML20356A237.

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TABLE OF CONTENTS

1 Day 1 Presentations 1 1.1 Session 1: Opening Plenary Session Summary. 1 1.1.1 Nuclear Digital Twins 1 1.1.2 IBM Digital Twins 2 1.1.3 ORNL Resources to Support Digital Twin Applications for Nuclear Systems. 2 1.1.4 National Reactor Innovation Center Digital Engineering 2 1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader Opportunities. 2 1.2 Session 2: Advanced Reactors. 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley. 4 1.3 Session 3: Nonuclear Applications of Digital Twins Overview. 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications. 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.4 ARPA-E GEMINA Progrets 8	E	EXECUTIVE SUMMARYiii							
1.1 Session 1: Opening Plenary Session Summary	1 Day 1 Presentations								
1.1.1 Nuclear Digital Twins 1 1.1.2 IBM Digital Twin 2 1.1.3 ORNL Resources to Support Digital Twin Applications for Nuclear Systems 2 1.1.3 ORNL Resources to Support Digital Twin Applications for Nuclear Systems 2 1.1.4 National Reactor Innovation Center Digital Engineering 2 1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader Opportunities 2 1.2 Session 2: Advanced Reactors 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley. 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1									
1.1.2 IBM Digital Twin. 2 1.1.3 ORNL Resources to Support Digital Twin Applications for Nuclear Systems. 2 1.1.4 National Reactor Innovation Center Digital Engineering 2 1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader Opportunities. 2 1.2.5 Session 2: Advanced Reactors 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley. 4 1.2.4 Revolutionary Reactor Designs for a Changing and Challenged World 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview 4 1.3.1 Industrial Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin 5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications. 6 2 Day 2 Presentations 7 2.1 ARPA-E									
1.1.3 ORNL Resources to Support Digital Twin Applications for Nuclear Systems 2 1.1.4 National Reactor Innovation Center Digital Engineering 2 1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader Opportunities 2 1.2 Session 2: Advanced Reactors 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design for a Changing and Challenged World 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 2.1 2.1.1 ARPA-E GEMINA Summary 7 2.1.1 ARPA-E GEMINA Summary 7 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors			1.1.2						
Systems 2 1.1.4 National Reactor Innovation Center Digital Engineering 2 1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader 2 0pportunities 2 1.2 Session 2: Advanced Reactors. 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to 3 1.2.3 Advanced Reactor Design Meets Silicon Valley. 4 1.2.4 Revolutionary Reactor Designs for a Changing and Challenged World 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview. 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System 5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital T			1.1.3						
1.1.4 National Reactor Innovation Center Digital Engineering 2 1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader 2 0pportunities 2 1.2 Session 2: Advanced Reactors 2 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley 4 1.3.4 Revolutionary Reactor Designs for a Changing and Challenged World 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital					2				
1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader Opportunities. 2 1.2 Session 2: Advanced Reactors. 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley 4 1.2.4 Revolutionary Reactor Designs for a Changing and Challenged World 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview. 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 7 2.1.4 A Digital I Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 7 2.1.1 ARPA-E GEMINA Summary 7 2.1.1 ARPA-E GEMINA Summary 7 2.1.1 ARPA-E GEMINA Program Overview 8 2.1.2 Artificial Intelligence -Enabled Predictive Maintenance Digital Twins for Adv			1.1.4						
Opportunities. 2 1.2 Session 2: Advanced Reactors. 3 1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley. 4 1.2.4 Revolutionary Reactor Designs for a Changing and Challenged World 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 7 7.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 8 7 2.1.4 ARPA-E GEMINA Program Overview 8 7 2.1.5 Nynopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 <th></th> <th></th> <th>1.1.5</th> <th></th> <th></th>			1.1.5						
1.2.1 Xe-100 Digital Technologies Overview 3 1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley 4 1.2.4 Revolutionary Reactor Designs for a Changing and Challenged World 4 1.3 Session 3: Nonnuclear Applications of Digital Twins Overview 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin 5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 7 2.1 ARPA-E GEMINA Summary 7 2.1.1 ARPA-E GEMINA Summary 7 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Program Overview 9 2.2.1 Synopsis of Westinghouse					2				
1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value		1.2	Session	2: Advanced Reactors	3				
1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value			1.2.1	Xe-100 Digital Technologies Overview	3				
Market, Increasing Safety Margins, Maximizing Value 3 1.2.3 Advanced Reactor Design Meets Silicon Valley			1.2.2	Digital Twin Development for Advanced Reactors, Accelerating Time to					
1.2.3 Advanced Reactor Design Meets Silicon Valley				Market, Increasing Safety Margins, Maximizing Value	3				
1.3 Session 3: Nonnuclear Applications of Digital Twins Overview 4 1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors. 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital Twin			1.2.3						
1.3.1 Industrial Digital Twins: GE Experience and Perspectives 5 1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict. 5 1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin 5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 2.2.2 Digital Twin or the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applic			1.2.4	Revolutionary Reactor Designs for a Changing and Challenged World	4				
1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict		1.3	Session	3: Nonnuclear Applications of Digital Twins Overview	4				
Predict			1.3.1	Industrial Digital Twins: GE Experience and Perspectives	5				
1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System .5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing .5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing .6 2 Day 2 Presentations .6 2 Day 2 Presentations .7 2.1 Panel Session: ARPA-E GEMINA Summary .7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations .8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for .8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview .8 2.1.4 ARPA-E GEMINA Projects: .8 2.2 Session 2: Industry Vision Overview .9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications. .9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components. .10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry .10 2.2.4 Digital Twins for Advanced Reactor Applications. .10			1.3.2	Overcoming Digital Twin Data Scarcity and Accelerating the Journey To					
Digital Twin 5 1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10					5				
1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10			1.3.3						
Toward Smart Component Fabrications 6 2 Day 2 Presentations 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10				Digital Twin	5				
2 Day 2 Presentations 7 2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10			1.3.4	A Digital Twin Approach to Study Sensor Fused Additive Manufacturing					
2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively 9 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10				Toward Smart Component Fabrications	6				
2.1 Panel Session: ARPA-E GEMINA Summary 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations 7 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively 9 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10	2	Dav	2 Prese	ntations	7				
 2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 2.1.4 ARPA-E GEMINA Projects: 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 	-								
and Maintenance Costs 8 2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10		2							
2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors. 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview. 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview. 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications. 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components. 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10				and Maintenance Costs	8				
Advanced Nuclear Reactors. 8 2.1.3 Xe-100 ARPA-E GEMINA Program Overview. 8 2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview. 9 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications. 9 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components. 10 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry 10 2.2.4 Digital Twins for Advanced Reactor Applications 10			2.1.2						
 2.1.3 Xe-100 ARPA-E GEMINA Program Overview					8				
2.1.4 ARPA-E GEMINA Projects: 8 2.2 Session 2: Industry Vision Overview			2.1.3						
 2.2 Session 2: Industry Vision Overview			-						
 2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications		2.2	Session						
Digital Twin Developments for Nuclear Power Applications92.2.2Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components									
 2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components				Digital Twin Developments for Nuclear Power Applications	9				
Manufactured Nuclear Components102.2.3EPRI's Digital -Twin -Related Activities for the Nuclear Industry102.2.4Digital Twins for Advanced Reactor Applications10			2.2.2	•	-				
 2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry					10				
2.2.4 Digital Twins for Advanced Reactor Applications			2.2.3						
2.5 Session 5. Applications of Auvaliced Technologies—Fait Toverview		2.3		•					
2.3.1 On Artificial Intelligence Research at ORNL and Its Application at the									
Spallation Neutron Source					11				
2.3.2 Overview of Digital Twin Work at Argonne National Laboratory			2.3.2						
2.3.3 Extending a Digital Engineering Framework through Operations									
2.3.4 Digital Platform for the Transformational Challenge Reactor									
2.4 Session 4: Applications of Advanced Technologies—Part II Overview		2.4							

		2.4.1	Digital -Twin -Based Asset Performance and Reliability Diagnosis for the HTGR Reactor Cavity Cooling System Using Metroscope	13
		2.4.2	Data -Driven Optimization of Moisture Carryover in an Operating	
			Boiling-Water Reactor	13
		2.4.3	Role and Status of Virtual Reality, Augmented Reality, and Mixed	
		2.1.0	Reality in Digital Twins in the Nuclear Industry	13
		2.4.4	Online Artificial Intelligence/Machine Learning and	
		2.7.7	Computational-Mechanics-Based Predictive Tools for a Digital	
			Twin Framework	13
				10
3	Dav	/ 3 Prese	entations	15
•			ional Activities in Digital Twins Session Summary	
	••••	3.1.1	Qualification of the Pickering a Test Facility	
		3.1.2	The United Kingdom's Nuclear Virtual Engineering Capability	
		3.1.3	Benefits of Digitalizing and Employing Simulation to Increase Plant	
		0.1.0	System Performance and Ensure Compliance with Technical	
			Specifications	16
		3.1.4	Euratom Research and Training Programme—Fission Research	
		3.1.4	European Research, Development, and Innovation Towards Digital	10
		5.1.5	Twins	17
	2 2	Cybored	ecurity Session Summary	
	3.Z	3.2.1		
		3.2.1	Digital Twins and Cybersecurity Cybersecurity for Digital Twins	
		3.2.2	The Asherah Nuclear Power Plant Simulator in a Closed-Loop Digital	10
		3.2.3		10
	<u> </u>	رما مناز ال	Twin Environment	
	ა.ა	3.3.1	ysics Modeling Session Summary	19
		3.3.1	Advanced Modeling and Simulation and Its Future Role in Nuclear	40
			Systems Digital Twin Technology	
		3.3.2	Modeling and Simulation to Support Digital Twins	20
		3.3.3	Multiphysics Modeling for Advanced Reactor Safety and Digital Twin	00
		004	Development	20
		3.3.4	Hybrid Physics-Informed Neural Networks, Cumulative Damage Models,	~~~
	~ .	D .	and Digital Twins.	
	3.4		stics, Prognostics, and Condition Monitoring Session Summary	20
		3.4.1	A Quantitative Framework to Assess Tradeoffs in Alternative Models	
			and Algorithms for Prognostics and Health Management	21
		3.4.2	Digital Twins in a Nearly Autonomous Management and Control System	~~~
			for Advanced Reactors	22
		3.4.3	Digital Twins for Prognostic Health Management in Nuclear Energy:	
			Opportunities and Challenges	22
	_			~~
4			entations	
	4.1	0	Plenary Session Summary	23
		4.1.1	Managing Regulated Change: An Enterprise-Level Digital Twin for the	~~~
			Nuclear Industry	
		4.1.2	Including Risk in Digital Twins	
		4.1.3	Towards a Digital Twin to Detect Nuclear Proliferation Activities	
	4.2		Win Regulatory Discussion Panel	
		4.2.1	Bret Kugelmass, Managing Director	
		4.2.2	Neil Olivier, Director of Corporate Services	
		4.2.3	Pat Everett, Director of Thermal Engineering	25

	4.2.4	Gregory A. Banyay, Modeling and Simulation Hub Technical Lead (Principal Engineer)	25
APPE	ENDIX A	Speakers BIOS	
APPI	ENDIX B	Workshop Attendees	B-1
APPE		Presentation Slides	C-1

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1 DAY 1 PRESENTATIONS

1.1 Session 1: Opening Plenary Session Summary

This session served as an introduction to the digital twin concept, as well as to potential applications in advanced nuclear technologies and regulatory viability. The speakers represented a wide spectrum of stakeholders: the NRC, universities, national laboratories, and digital twin developers. Specific applications discussed included PLM, operational improvement in asset intensive- businesses, additive manufacturing, digital engineering approaches, and predictive maintenance. Digital twins are by their nature diverse, as different applications call for different information. However, they all share the trait of using a digital representation of a physical thing to accomplish desired goals.

Participants in this session identified the following challenges:

- technology
- risk versus uncertainty
- cultural inertia
- needed tools and data

Participants in this session identified the following key takeaways:

- Recent advances in computing capabilities have enabled M&S and the use of AI.
- Standards are currently fragmented.
- Validation is critical.
- Digital twin technology offers the opportunity to shorten learning curves.

The presentations slides for Day 1 can be found <u>here</u> and in the Agency Documents Access and Management System (ADAMS) under <u>ML20356A237</u>.

Presentations

1.1.1 Nuclear Digital Twins

Michael Grieves, Chief Scientist of Advanced Manufacturing, Executive Vice President of Operations Florida Institute of Technology

<u>Presentation Overview</u>: The first presentation of the opening plenary presents the origin, definition, application, and the recent propagation of digital twin technologies. Traditional digital twins as originated from the concept of product lifecycle management have enabled design, manufacturing, testing and operation of products or system in virtual space. The enabling technologies of digital twins comprise of digital instrumentation, computational hardware, data analytics, machine learning and artificial intelligence and physics-based simulation. As these technologies continue to mature, the digital twins from the past continue to evolve from passive, offline, goal-given and predictive into the digital twins of the future that are active, online, real-time, goal-seeking, and anticipatory.

1.1.2 IBM Digital Twin

Joseph Berti, Vice President of Offering Management IBM

<u>Presentation Overview</u>: IBM and Rotterdam developed a HydroMeteo digital twin prototype. This twin fuse sensor, weather, and water conditions predicted sensor and water conditions (predictive digital twin), and anomaly detection for sea vessel traffic management (simulated digital twin). These sensors continuously capture data on air temperature, wind speed, relative humidity, water salinity, water flow/levels, and tides/currents. This data is used to predict optimal moor and departure times at Rotterdam. This real-time twin enables Rotterdam to facilitate cost-effective vessel management and can help ensure cargo arrives safely.

1.1.3 ORNL Resources to Support Digital Twin Applications for Nuclear Systems

Jeremy Busby, Division Director, Reactor and Nuclear Systems Division Oak Ridge National Laboratory

<u>Presentation Overview</u>: The major science and technology initiatives or ORNL include materials and chemical processes, neutron science, computer and data analytics, nuclear fission and fusion, isotope research and production, biological and environmental systems and natural security. This presentation highlights the potential research and development in digital twin technologies that address several of the ORNL initiatives. The transformational challenge reactor (TCR) program is harnessing advances in additive manufacturing, materials and computational sciences, data analytics and machine learning to enhance efficiency and safety in advanced reactors.

1.1.4 National Reactor Innovation Center Digital Engineering

Ashley Finan, Director National Reactor Innovation Center

<u>Presentation Overview</u>: The National Reactor Innovation Center at INL provides resources for testing, demonstration, and performance assessment to accelerate the deployment of new advanced nuclear technology concepts. In 2020, NRIC started transforming the traditional engineering design ecosystem from a document-centric paradigm to a digital engineering framework to increase collaboration and efficiency. NRIC has focused on MBSE tools, data models, and processes for the design of the facilities, components, and contractor interfaces to enable GenIV microreactors. NRIC utilizes the Deep Lynx framework from VTR to facilitate lifecycle communication between conceptual, functional, and detailed design data and information. This paradigm is currently being applied for the system-level artifacts for both the EBR-II and Zero Power Physics Reactor (ZPPR) demonstration testbeds at NRIC.

1.1.5 The GEMINA Program: What ARPA-E Is Doing and Broader Opportunities

Jenifer Shafer, Special Government Employee Consultant Advanced Research Projects Agency–Energy <u>Presentation Overview</u>: This presentation provides an overview of the ARPA-E programs of MEITNER, MEITNER-RT, LISE, OPEN-Fission, and GEMINA. The ARPA-E GEMINA program is focused on leveraging advanced technologies such as autonomy and machine learning so that advanced reactors can achieve operating and maintenance costs comparable to a natural gas combine cycle plant. The outcome of ARPA-E GEMINA are intended to be tools that can be used by the industry to predict and prevent failure, avoid plant trip, reduce maintenance and insurance cost, improve reliability and utilization without increasing risk. The FY21 GEMINA awardees are interdisciplinary teams that will build digital twins for advanced reactor systems, build cyber-physical hardware in the loop system, assess the needed signals and sensor modalities, gain data for validation of software for nuclear application, practice control operations for scenarios such as startup, shutdown and transient, and define standard approaches to handle uncertainty, fidelity and interface etc.

1.2 <u>Session 2: Advanced Reactors</u>

Representatives from several companies developing advanced reactors discussed their views on the appropriate uses for digital twins in the design stages for both reactor modeling and O&M.

Participants in this session identified the following challenges:

- identification of the most appropriate applications for twins in the design process
- verification and validation of twins for as-yet-unbuilt designs

Participants in this session identified the following key takeaways:

- Twins can be used for many purposes, each of which may call for different twins and different approaches, depending on the scope of the analyses required.
- Several applications have used machine learning algorithms with good results.

Presentations

1.2.1 Xe-100 Digital Technologies Overview

Ian Davis, Senior Digital Twin System Engineer X-Energy LLC

<u>Presentation Overview</u>: X-Energy LLC (X-energy) is transforming the nuclear energy marketplace through the development of the Xe-100 advanced reactor, which is a Generation IV high temperature gas- -cooled reactor (HTGR). X-energy sees innovative digital technologies, especially the digital twin, as an integral part of that transformation. Unlike digital twin use cases in today's existing nuclear fleet, the Xe-100 digital twin will provide invaluable feedback for the design process. It will help shape our systems design, control strategy, operations and maintenance (O&M) programs, and much more.

1.2.2 Digital Twin Development for Advanced Reactors, Accelerating Time to Market, Increasing Safety Margins, Maximizing Value

Anthonie Cilliers, Senior Manager, Instrumentation, Controls, and Electrical

Kairos Power

<u>Presentation Overview</u>: The development of advanced reactors today coincides with the fast maturing of digital modeling tools, virtual reality, machine learning, and the ever -increasing computing power becoming available to reactor designers and operators. As advanced reactor developers, we have a unique opportunity to incorporate a number of digital twin use cases from the conception of each project. This discussion explores the fundamental use cases of digital twins and how they can impact and support advanced reactor designs.

1.2.3 Advanced Reactor Design Meets Silicon Valley

Clyde Huibregtse, Reactor/Software Engineer Oklo, Inc.

<u>Presentation Overview</u>: Contrary to the conventional notion of a digital twin as a tool exclusively for simulating system dynamics, Oklo has adopted a methodology known as a surrogate model in the early stages of reactor design. Leveraging the containerization capabilities of Docker Engine, Oklo has constructed a virtualized analysis pipeline through which we can feed a large number of permutations of our nominal design, each of which outputs a scalar -valued performance metric. We train a surrogate model to map a vector of input dimensions to a performance value. With it, we can efficiently compute gradients of our performance function with respect to input parameters, allowing us to optimize our design's performance.

1.2.4 Revolutionary Reactor Designs for a Changing and Challenged World

Charles R. "Chip" Martin, Scientific/Technical Advisor HolosGen, LLC

<u>Presentation Overview</u>: This presentation introduces microreactor designs that eliminate the traditional "balance of plant" through the integration of the power conversion components with the nuclear core. The designs use modern high-speed motors and generators to convert thermal energy from the nuclear core to load -following electricity, with simplifications that make them competitive with nonnuclear electricity -producing technologies. The Holos Quad design is modeled using high -fidelity simulators from the national laboratories and academia and a subscale helium closed -loop simulator. The further simplified Risk Reduction Demonstration Monolithic-Holos (M-Holos) design facilitates the "virtual build" through digital twins, leveraging national laboratory and industry expertise, to reduce risks and accelerate deployment.

1.3 <u>Session 3: Nonnuclear Applications of Digital Twins Overview</u>

There is a wide array of digital twin applications in other advanced fields, such as aviation, conventional power generation, and healthcare. These twins help optimize uptime, reduce unplanned disruptions, and optimize lean supply chains. Digital twins have also found application in the monitoring and improving of human performance.

Participants in this session identified the following challenges:

• maximizing value from scarce data

• verification to the level of accuracy required for application

Participants in this session identified the following key takeaways:

- Digital twins offer considerable savings and performance improvements in some applications.
- Advanced sensor technology presents new avenues for verifying digital twin accuracy and performance.

Presentations

1.3.1 Industrial Digital Twins: GE Experience and Perspectives

Abhinav Saxena, Senior Scientist, Machine Learning, GE Research General Electric

<u>Presentation Overview</u>: This talk presents GE's definition of digital twins for industrial assets, with specific examples of applications in several industrial domains, such as aviation, healthcare, power, and transportation. Given GE's digital twin experience in the field, the talk also discusses current challenges and research directions.

1.3.2 Overcoming Digital Twin Data Scarcity and Accelerating the Journey To Predict

Chandler Maskal, Offering Manager, IBM AI Applications IBM

<u>Presentation Overview</u>: This presentation discusses two of the greatest technical challenges in adopting digital twin technology and IBM's strategy for overcoming these challenges. This session introduces the challenges seen across many industries and discusses the technical steps that IBM is taking to drive the adoption of digital twin technology for equipment operations.

1.3.3 Pacing Optimization Enabled by a Human Thermoregulatory System Digital Twin

Mark Buller, Principal Investigator, Biophysics and Biomedical Modeling Division U.S. Army Research Institute of Environmental Medicine

<u>Presentation Overview</u>: This presentation describes how physiological -feedback pacing enabled by a human thermoregulatory system digital twin can optimize both performance and safety for relevant military tasks. It shows that using a digital twin enables a simple Markov decision process (MDP) representation of military pacing problems. By solving these MDPs, we can construct pacing policies to optimize human physiological resources while minimizing thermal -work strain safety risks. What had been identified in the literature as expert "black box" pacing templates were successfully enumerated by the use of a digital twin and can now be applied by novices on new and novel tasks.

1.3.4 A Digital Twin Approach to Study Sensor Fused Additive Manufacturing Toward Smart Component Fabrications

Kevin P. Chen, Professor University of Pittsburgh

<u>Presentation Overview</u>: One of the major challenges for metal -powder -based additive manufacturing is to design an optimized manufacturing strategy to mitigate the residual strain induced during the manufacturing processes. This talk discusses distributed fiber optic sensors embedded in Inconel alloy components as experimental means to validate numerical models of additive manufacturing processes. Using high -spatial -resolution data harnessed by distributed fiber sensors, digital twin models can accurately model the manufacturing process, leading to design and manufacturing optimization.

2 DAY 2 PRESENTATIONS

2.1 Panel Session: ARPA-E GEMINA Summary

The recent ARPA-E programs Modeling -Enhanced Innovations Trailblazing Nuclear Energy Reinvigoration (MEITNER), Leveraging Innovations Supporting Nuclear Energy (LISE), and GEMINA aim to apply data, physics -based models, and algorithms to increase operational efficiencies and reduce construction and O&M costs for current and future nuclear power plants. This panel discussion focused on the recently awarded R&D efforts within the GEMINA program. It included an overview of programs funded by ARPA-E and project -specific presentations by three of the awardees, followed by a question -and -answer session.

Participants in this session identified the following challenges:

- Verification and validation of AI and machine learning algorithms is a major challenge, especially in light of the lack or absence of data.
- Use of digital twins to reduce security, maintenance, and operational personnel would potentially require addressing major regulatory constraints.

Participants in this session identified the following key takeaways:

- ARPA-E envisions a reduction of O&M costs to \$3 per megawatt -hour (a cost profile comparable to that of a natural gas combined -cycle plant).
- A GE project funded under GEMINA focuses on AI -enabled predictive maintenance of digital twins for advanced nuclear reactors.
- The X-energy GEMINA project focuses on reducing fixed O&M costs in Xe-100, targeting three labor -intensive areas: operators, maintenance personnel, and security personnel.
- The Kairos SAFARI project aims to deliver a capability enabling smart functionalities in advanced reactor systems, such as autonomous operations, flexible operations, and predictive maintenance.
- The Kairos MARS project aims to develop advanced distributed sensing and data generation techniques to characterize critical components and systems; increase sensor diversity and develop multifunctional sensors that measure several process variables simultaneously; automate maintenance tasks through machine -learning -enabled fault detection and diagnostics; and inform intelligent sensor placement to achieve autonomous operation.

The presentations slides for Day 2 can be found <u>here</u> and in the Agency Documents Access and Management System (ADAMS) under <u>ML20356A234</u>.

Presentations

2.1.1 ARPA-E Perspective: Digital Twins as an Enabler of Low Operations and Maintenance Costs

Joel Fetter, Lead Associate ARPA-E

<u>Presentation Overview</u>: This presentation describes several examples on how the inclusion of digital twin technology can reduce operation and maintenance costs. In addition, a portfolio at a glance of currently digital twin funded programs are discussed.

2.1.2 Artificial -Intelligence -Enabled Predictive Maintenance Digital Twins for Advanced Nuclear Reactors

Abhinav Saxena, Senior Scientist, Machine Learning, GE Research General Electric

<u>Presentation Overview</u>: This presentation gives a brief overview of GE's GEMINA project on AI -based digital twins for reducing O&M costs for advanced reactors. The presentation describes key project goals and technology R&D towards achieving O&M cost reductions.

2.1.3 Xe-100 ARPA-E GEMINA Program Overview

Yvotte Brits, Supply Chain Manager and Operator Training Simulator Program Manager X-Energy LLC

<u>Presentation Overview</u>: X-energy is transforming the nuclear energy marketplace through the development of the Xe-100 advanced reactor, which is a Generation IV HTGR. Levelized fixed O&M costs of conventional energy -generating technologies, such as coal and gas, are lower than those of nuclear energy. The regulatory framework of traditional nuclear power requires a number of operators, security, and maintenance personnel, resulting in high levelized fixed O&M costs of approximately \$14.5 per megawatt -hour for the Xe-100 plant. The presentation demonstrates the digital twin's ability to reduce levelized fixed O&M costs to a target of \$2 per megawatt hour in the Xe-100 plant. The Xe-100's intrinsic passive safety features make it ideal to showcase the abilities of the digital twin.

2.1.4 ARPA-E GEMINA Projects:

Project SAFARI—Secure Automation for Advanced Reactor Innovation Project MARS—Maintenance of Advanced Reactor Sensors and Components Anthonie Cilliers, Senior Manager, Instrumentation, Controls, and Electrical Kairos Power

Presentation Overviews:

<u>SAFARI</u>: This project will deliver a capability enabling smart functionalities in advanced reactor systems, such as autonomous operations, flexible operations, and predictive maintenance. This has the potential to dramatically lower O&M costs compared to those of currently operating LWRs.

<u>MARS</u>: This project will develop advanced distributed sensing and data generation techniques to characterize critical components and systems; increase sensor diversity and develop multifunctional sensors that measure several process variables simultaneously; and automate maintenance tasks through machine-learning-enabled fault detection and diagnostics and intelligent sensor placement to achieve autonomous operation.

2.2 Session 2: Industry Vision Overview

This session presented the industry vision of digital twin technology R&D, with a focus on current and future applications. Participants discussed the current state of the technology and its potential use in the nuclear industry, along with the industry's first impressions of the techniques. The session highlights included overviews from EPRI, BWX Technologies, Westinghouse, and Analysis and Measurement Services Corporation (AMS).

Participants in this session identified the following challenges:

- Use of a digital twin as a repository for the subject matter expert knowledge base is a challenge.
- Porosity is a challenge in metal powder bed additive manufacturing, and traditionally, the only nondestructive way to detect it is through expensive CT scanning. The digital twin approach could be much cheaper than these scans.
- Prohibitively large and unforeseen increases in construction cost are one of the greatest challenges for future nuclear reactors.

Participants in this session identified the following key takeaways:

- Westinghouse's current efforts in digital twin applications include eliminating destructive testing in fabrication processes, coupling machine learning with neutronics models, replacing reactor internal components, and identifying cracks in concrete structures.
- At BWX Technologies, digital twin efforts focus on the inspection of additively manufactured nuclear components to detect porosity.
- EPRI digital twin efforts aim to transition the nuclear industry from the current approach of "maintain and repair" to "replace and refurbish" life cycle management, and to lower construction costs.
- One of the current efforts at AMS is online monitoring to support the autonomous remote operation of advanced reactors.

Presentations

2.2.1 Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications

Brian Golchert, Principal Engineer Westinghouse Electric Company <u>Presentation Overview</u>: This presentation gives an overview of digital-twin-related activities at Westinghouse, emphasizing current and future applications of digital twins.

2.2.2 Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components

Matthew LeVasseur, Director of Research BWX Technologies

Ryan Kitchen, Research and Development Data Scientist BWX Technologies

<u>Presentation Overview</u>: This presentation shows a quality grade platform for digital twin inspection during build, how it works and the benefit of implement it. In addition, a quality assessment for digital twin is presented.

2.2.3 EPRI's Digital -Twin -Related Activities for the Nuclear Industry

Hasan Charkas, Principal Technical Leader Electric Power Research Institute

<u>Presentation Overview</u>: This presentation gives an overview of EPRI's recent activities and research projects in digital twin technology.

2.2.4 Digital Twins for Advanced Reactor Applications

Hash Hashemian, President and CEO Analysis and Measurement Services Corporation

Jacob Houser, Senior Research Engineer Analysis and Measurement Services Corporation

<u>Presentation Overview</u>: This presentation covers the adaptation of digital twins for improved instrumentation and control maintenance in current and next -generation nuclear reactors. We have integrated process data from measurements collected in nuclear power plants with empirical and physical modeling to produce reliable predictions for process and sensor behavior, which can be used for anomaly detection, diagnostics, and prognostics.

2.3 <u>Session 3: Applications of Advanced Technologies—Part I Overview</u>

Individuals from ORNL, INL, and ANL summarized their use of digital twins in the development and management of research facilities (the Versatile Test Reactor, Transformational Challenge Reactor, and Spallation Neutron Source) and in the specific research areas of additive manufacturing and digital twin sensor deployment optimization.

Participants in this session identified the following challenges:

- collecting good data—sensor selection and placement
- uncertainty quantification
- obtaining explainable and trustworthy results
- validation and verification
- cybersecurity

Participants in this session identified the following key takeaways:

- We are at a tipping point for AI, and the future will see increasing use of AI in science and engineering.
- Digital twin technology may be able to dramatically reduce O&M costs through maintenance scheduling and the elevation of humans to oversight roles.
- Digital twins allow the integration of megaprojects to increase communication, document control, and scheduling to avert "failures."
- Digital twins can accelerate the manufacturing process by creating digital threads to track quality during component development and creation.

Presentations

2.3.1 On Artificial Intelligence Research at ORNL and Its Application at the Spallation Neutron Source

David E. Womble, Director of Artificial Intelligence Programs Oak Ridge National Laboratory

<u>Presentation Overview</u>: This presentation describes the biggest challenges in the implementation of artificial intelligence (AI)/ machine learning (ML) and the strategic directions of ORNL regarding the topic. In addition, introduce "easy steps" for the anomaly detection in AI/ML systems.

2.3.2 Overview of Digital Twin Work at Argonne National Laboratory

Richard Vilim, Senior Nuclear Engineer; Manager, Plant Analysis and Control and Sensors Department, Nuclear Science and Engineering Division Argonne National Laboratory

<u>Presentation Overview</u>: This presentation defines what is a digital twin and why is the interest in the topic. Introduces AI/ML as an enabler of autonomous operation and describe several examples of ANL digital twin projects.

2.3.3 Extending a Digital Engineering Framework through Operations

Christopher Ritter, Director, Digital Innovation Center of Excellence Idaho National Laboratory <u>Presentation Overview</u>: This presentation provides an explanation of the National Reactor Innovation Center (NRIC) at Idaho National Laboratory (INL) and discuss several examples of projects and applications that the center is currently working.

2.3.4 Digital Platform for the Transformational Challenge Reactor

Ben Betzler, Nuclear Engineer Oak Ridge National Laboratory

Vincent Paquit, Senior Research Scientist, Electrical and Electronics Systems Research Oak Ridge National Laboratory

<u>Presentation Overview</u>: This ORNL presentation gives an overview of how the Transformational Challenge Reactor (TCR) uses additive manufacturing and artificial intelligence to provide high performance materials and assess the quality of the component during the manufacturing process. ORNL is developing new certification methodologies for manufacturing technologies. The TCR Digital Platform uses data analytics for prototyping and evaluating the products to optimize the manufacturing process. ORNL continues to research in areas of data management, in-situ quality control, and sensor development.

2.4 <u>Session 4: Applications of Advanced Technologies—Part II Overview</u>

Experts from Framatome, ANL, and the University of Illinois detailed specific applications of digital twins in the nuclear field. The Framatome representative discussed the use of digital twins and AI in the development of reactor cooling models for HTGRs. The ANL representatives detailed digital twin use in the optimization of moisture carryover in boiling water reactors, as well as work on developing AI and machine learning tools as predictive tools for computational mechanics. The presenter from the University of Illinois proposed the use of digital twins as training tools in the nuclear field.

Participants in this session identified the following challenges:

- addressing more than a single steady -state system mode in the modeling of a reactor
- sensor identification
- generating meaningful data to validate failure modes
- obtaining data from actual reactor components

Participants in this session identified the following key takeaways:

- Digital twin technology can provide value in failure detection, regulatory basis verification, and cost savings.
- Digital twins and neural networks efficiently analyze parameters and evaluate models.

• Digital twin and virtual reality training tools can be valuable for dose reduction and operational efficiency.

Presentations

2.4.1 Digital -Twin -Based Asset Performance and Reliability Diagnosis for the HTGR Reactor Cavity Cooling System Using Metroscope

Eric Helm, Product Manager—Metroscope Framatome

<u>Presentation Overview</u>: This presentation provides an overview of the technical and commercial challenges in using digital twins for system diagnostics, along with savings and regulatory basis considerations. It also discusses the initial project approach meant to address those challenges.

2.4.2 Data -Driven Optimization of Moisture Carryover in an Operating Boiling-Water Reactor

Richard Vilim, Senior Nuclear Engineer; Manager, Plant Analysis and Control and Sensors Department, Nuclear Science and Engineering Division Argonne National Laboratory

<u>Presentation Overview</u>: Argonne National Laboratory (ANL) is exploring how historical operating data for moisture carryover gives data-driven model to guide and manage reactor operating conditions, so moisture carryover is acceptable. ANL uses machine learning model with applicable algorithms to perform an analysis of different functional forms including actual inputs and a neural network model. These models provide predictive capabilities to changes power up rates and new core loading patterns in a BWR.

2.4.3 Role and Status of Virtual Reality, Augmented Reality, and Mixed Reality in Digital Twins in the Nuclear Industry

Rizwan Uddin, Professor and Head of the Department of Nuclear, Plasma, and Radiological Engineering

University of Illinois at Urbana--Champaign

<u>Presentation Overview</u>: This presentation provides an overview of the current state of technology and future vision for application of virtual reality, augmented reality and mixed reality for potential application in the nuclear industry.

2.4.4 Online Artificial Intelligence/Machine Learning and Computational-Mechanics-Based Predictive Tools for a Digital Twin Framework

Subhasish Mohanty, Principal Research and Development Engineer Argonne National Laboratory <u>Presentation Overview</u>: The presentation focuses on various aspects of digital twins, with some example results related to laboratory-scale testing, three-dimensional finite-element modeling, online state estimation based on heterogeneous sensor measurements, and online condition-based state forecasting and remaining life estimations.

3 DAY 3 PRESENTATIONS

3.1 International Activities in Digital Twins Session Summary

This session focused on international activities in digital twin R&D, with four presentations on digital twin R&D in Canada, the United Kingdom, and Spain and at Euratom. The presentations covered diverse topics in digital twin research, including software qualification, the economic impact of introducing digital twins in the nuclear industry, the benefits of using digital twins for maintenance and surveillance in the nuclear industry, and Euratom activities on digitization in the nuclear industry.

Participants in this session identified the following challenges:

- Software qualification (i.e., demonstration that software tools will not fail) is a challenge.
- Quantifying the economic impact of introducing digital twins is a challenge.
- Developing digital twins for plant maintenance and surveillance is challenging because there are no high-fidelity models for many equipment degradation mechanisms.
- The models available for use in digital twins are often based on empirical dependencies, with parameters inferred from the data.
- The limited availability of experimental and plant data for digital twin development is a challenge.
- Information sharing, innovation sharing, and cost management are challenges in developing digital twins.

Participants in this session identified the following key takeaways:

- Some nuclear research and industry projects have used prototypes of digital twins.
- Efforts are in progress to develop digital environments to support the nuclear life cycle.
- An integrated approach for design modifications is being developed.
- International collaboration on digital twin development is underway in the European Union.
- The use of digital twins in the nuclear industry often depends on simulated data.
- Highly useful applications exist for small modular reactors and existing facilities; there are also links to the construction sector.
- Response.

The presentations slides for Day 3 can be found <u>here</u> and in the Agency Documents Access and Management System (ADAMS) under <u>ML20356A235</u>

Presentations

3.1.1 Qualification of the Pickering a Test Facility

Richard Henry, Section Manager, Computers and Control Design, Central Engineering Ontario Power Generation

<u>Presentation Overview</u>: A software-based test facility can be used for the verification and validation testing of nuclear control computer software modifications. Ontario Power Generation successfully implemented this digital twin application at one of its plants. This presentation discusses regulatory requirements and software qualifications and gives an overview of the test facility.

3.1.2 The United Kingdom's Nuclear Virtual Engineering Capability

Albrecht Kyrieleis, Senior Consultant Jacobs

<u>Presentation Overview</u>: This presentation gives an overview how the NVEC plans to deliver net zero carbon emissions by 2050. The NVEC plans to achieve this goal by developing a digital environment for the nuclear sector. The NVEC is in phase 2 of their plan in developing standards, guidance, and models. The NVEC will establish validation of the benefits of operation of a digital twin using case studies and further collaboration with the manufacturing and supply chain to reduce economic costs, improve accuracy, safety, and reliability.

3.1.3 Benefits of Digitalizing and Employing Simulation to Increase Plant System Performance and Ensure Compliance with Technical Specifications

Susana López Lumbierres

<u>Presentation Overview</u>: Tecnatom designed, implemented, and tested a design modification for the online monitoring of the essential services water system in a boiling water reactor nuclear power plant by integrating it into the existing digital control system (DCS). Key system parameters were acquired in real time to be displayed in the human system interface and used in performing calculations, and their historical evolution was stored. The objective was to optimize the monitoring and surveillance of this system. Furthermore, Tecnatom developed an engineering simulator (a "what if" simulator) consisting of a hydraulic model of the system. This simulator takes as inputs the heat exchanger performance parameters from the plant DCS, allows the user to change the essential services water system configuration (valve positions, uniform hazard spectrum level), and calculates theoretical process values predicting the system's real behavior. In a second stage, more system instruments were wired to the DCS to verify compliance with technical specifications and provide automatic surveillance of opening and closing times of system valves.

3.1.4 Euratom Research and Training Programme—Fission Research

Panagiotis Manolatos

<u>Presentation Overview</u>: This presentation covers the modus operandi of the European Atomic Energy Community (Euratom) Research and Training Programme, examples of currently

funded research projects in nuclear safety, the status of preparation of the next Framework Programme 2021–2027 (Horizon Europe), and opportunities for international cooperation.

3.1.5 European Research, Development, and Innovation Towards Digital Twins

Abderrahim Al Mazouzi

<u>Presentation Overview</u>: Within the Sustainable Nuclear Energy Technology Platform (SNETP), many collaborative technological and scientific projects (most of them sponsored by Euratom) are helping association members to progress toward building digital twins, from critical components up to the entire reactor. This presentation gives the flavor of some ongoing projects and highlights some examples considered by SNETP members.

3.2 Cybersecurity Session Summary

This session described the cybersecurity challenges that can arise when implementing digital twin techniques. It highlighted regulatory considerations for the development phase of digital twin technology and emphasized the importance of understanding the technology before procuring or using it.

Participants in this session identified the following challenges:

- Cybersecurity considerations could exponentially complicate any digital twin implementation; addressing cybersecurity issues before implementation is a significant challenge.
- Analysis of vulnerability to cyberattacks in nuclear power plants is a major challenge.
- Understanding the levels of granularity for cybersecurity modeling is a challenge.

Participants in this session identified the following key takeaways:

- Al holds potential for modeling cyberattacks in digital twins.
- An infrastructure developed to test against an AI attack could prove valuable for digital twins as well.
- Data exfiltration and supply chain threats are standard cybersecurity concerns, as is the potential for an attacker to contaminate the supply chain by altering a digital twin used for design.
- Digital twins and AI allow for automated vulnerability analysis.
- The selection of appropriate levels of granularity depends on the exploit and the hardware

Presentations

3.2.1 Digital Twins and Cybersecurity

Christopher Spirito, Nuclear Cybersecurity Consultant Idaho National Laboratory

<u>Presentation Overview</u>: As digital twins are integrated into the systems used to control nuclear reactors and supporting systems, it is necessary to ensure that they are not vulnerable to manipulation by cyber means, but there is also an opportunity to use digital twins to support cybersecurity goals and objectives. This presentation gives a historical account of each of these problems, from how these systems have been envisioned through how they have been implemented. We also discuss how we believe they could be used in the future.

3.2.2 Cybersecurity for Digital Twins

Cynthia DeBisschop, Senior Cybersecurity Analyst Oasis Systems, LLC (NRC contractors)

<u>Presentation Overview</u>: This presentation offers a regulatory perspective on cybersecurity considerations while digital twin technology is in development. Before the procurement or use of technology, the attack surfaces and environments associated with digital assets should be understood. Throughout the life cycle of digital assets, plant operators must maintain a defensive security architecture to address all attack surfaces and environments, as well as multiple layers of cybersecurity protections to establish sufficient defense in depth. Defense-in-depth protective strategies ensure the capability to detect, respond to, and recover from cyberattacks. Their effectiveness depends on thorough understanding and careful consideration of the technology before procurement or use.

3.2.3 The Asherah Nuclear Power Plant Simulator in a Closed-Loop Digital Twin Environment

Rodney Busquim e Silva, Computer Security Officer International Atomic Energy Agency

<u>Presentation Overview:</u> Nuclear power plants consist of several complex industrial processes with a large number of information technology and automation systems, implementing process control, safety, and security functions. The need to understand the impacts of cyberattacks— and how they propagate—led to the development of a specific simulator, the Asherah Nuclear Power Plant Simulator, for an IAEA Coordinated Research Project. Digital twins open new possibilities for simulating, monitoring, estimating, and optimizing the state of nuclear energy systems. Within this scope, digital twins can be leveraged for computer security purposes when integrated into simulators like the Asherah Nuclear Power Plant Simulator.

3.3 <u>Multiphysics Modeling Session Summary</u>

This session addressed the role of advanced multiphysics M&S in the digital twin context, discussing M&S tools and frameworks for both LWR and non-LWR advanced reactor designs. M&S plays a pivotal role in the development and operation of digital twins: it fills gaps in knowledge or data and provides access to unmeasurable quantities.

Participants in this session identified the following challenges:

- Uncertainties from multiphysics M&S, data generation, and the training of machine learning algorithms or surrogate models are nonlinear and are amplified when we apply these tools in developing digital twins. How can we quantify such uncertainties?
- How can we efficiently perform predictions using digital twins?

Participants in this session identified the following key takeaways:

- Integrating high-fidelity, high-resolution M&S with advanced sensors yields unprecedented details of reactor behavior.
- The role of M&S is to provide access to a wealth of data. However, results must be obtainable and meaningful.
- We need to establish credibility and applicability of the M&S evaluation model in order to account for uncertainty and verify the trustworthiness of the tools.
- Digital twins must bridge the gap between model predictions (understanding) and observations (reality).

Presentations

3.3.1 Advanced Modeling and Simulation and Its Future Role in Nuclear Systems Digital Twin Technology

Dave Kropaczek, Director, Consortium for Advanced Simulation of Light Water Reactors (a DOE Energy Innovation Hub) Oak Ridge National Laboratory

<u>Presentation Overview</u>: This presentation demonstrates how a digital twin virtual simulator gives reliable predictive capabilities for reactor quantities of interest based on multiphysics modeling. Formal calibration methods can address the uncertainties of the input parameters and closure relations. This presentation goes into how the Virtual Environment for Reactor Applications (VERA) give unprecedented reactor analysis. Through the integration of high fidelity, high resolution simulation along with advanced sensors results in accurate reactor behavior.

3.3.2 Modeling and Simulation to Support Digital Twins

Jeffrey W. Lane, Chief Engineer and Principal Consultant Zachry Nuclear Engineering

<u>Presentation Overview</u>: This presentation discusses the role of advanced M&S in digital twin development and applications. It focuses on what advanced M&S can provide, and on the required attributes of an advanced M&S tool to support digital twin applications. This presentation also discusses challenges related to data assessment and credibility.

3.3.3 Multiphysics Modeling for Advanced Reactor Safety and Digital Twin Development

Rui Hu, Manager, Plant System Analysis Group Argonne National Laboratory

<u>Presentation Overview</u>: Inherent safety is a key characteristic for various advanced reactor concepts; it requires an improved understanding of multiphysics phenomena and M&S capabilities. This talk gives an overview of the safety characteristics and the needs of multiscale multiphysics simulation. It provides an example of coupled multiphysics simulation of a heat-pipe-cooled microreactor and presents some thoughts on leveraging multiphysics simulations in digital twin development.

3.3.4 Hybrid Physics-Informed Neural Networks, Cumulative Damage Models, and Digital Twins

Felipe A. C. Viana, Assistant Professor University of Central Florida

<u>Presentation Overview</u>: This presentation challenges the myth that building digital twins with machine learning requires large datasets. First, it addresses how physics-driven and data-driven kernels combine within deep neural networks. This framework, pioneered in the Probabilistic Mechanics Lab, allows for a neural network to directly implement differential equations while accounting for uncertainty in the model form as well as in observations. The presentation also gives an overview of the theoretical aspects and show engineering applications in digital twins for the failure prognosis of main bearings of wind turbines, aircraft fuselage panels, and batteries used to power electric vehicles

3.4 Diagnostics, Prognostics, and Condition Monitoring Session Summary

This session discussed diagnostics, prognostics, and condition monitoring using digital twins. Digital twins are an important tool in these applications, as they allow the modeling of different degradation and failure mechanisms without resorting to physical modeling. Also, digital twins can forecast the development of various failures and mitigate their consequences.

Participants in this session identified the following challenges:

• There are no high-fidelity models for many degradation and failure mechanisms present in nuclear equipment.

- Many existing degradation models are empirical and have little first principles support.
- Diagnostics is an ill posed problem, as similar symptoms (effects) can have different causes.
- Model comparison and validation are challenges for diagnostics and prognostics.
- In general, prognostics and diagnostics models need to be time dependent and adaptable.

Participants in this session identified the following key takeaways:

- Active and passive component models require different approaches for use in digital twins.
- It may be necessary to frequently update or even change models based on component condition. Digital twins are time dependent.
- Digital twin development should follow a data driven physics inspired approach.
- There is a tradeoff between early maintenance and failures due to a lack of maintenance.
- A useful concept is that of prognostic distance. Maintenance should be performed when remaining useful life is less than prognostic distance.
- Digital twins can contribute to information driven asset management.

Presentations

3.4.1 A Quantitative Framework to Assess Tradeoffs in Alternative Models and Algorithms for Prognostics and Health Management

Lance Fiondella, Associate Professor of Electrical and Computer Engineering University of Massachusetts at Dartmouth

Presentation Overview: The field of PHM is transforming reliability engineering by pinpointing which components or subsystems require maintenance, as well as precisely predicting when maintenance actions should occur. While there are several metrics to quantitatively assess the accuracy of remaining-useful-life predictions, few studies have explicitly modeled the economic benefits of implementing PHM, such as return on investment, life cycle cost reduction, and average total cost over a period. Although simulation and probabilistic techniques have been developed to select a time horizon for use based on remaining-useful-life predictions, in order to guide maintenance decisions that minimize cost, these past techniques do not consider additional factors of interest. To overcome this limitation, we develop data-driven analogs to metrics from renewal theory, including average cost per unit time, utilization, safety, and availability, which are suitable for application in the context of PHM methods. Simultaneous consideration of multiple metrics leads to a multiobjective generalization of the cost minimization problem, necessitating a framework to compare alternative PHM methods. Therefore, we also explicitly decouple degradation models from the algorithms that iteratively update estimates of a model's parameters. This decoupling enables the direct comparison of alternative combinations of models and algorithms, as well as provides a method to select a time horizon that balances tradeoffs

between multiple competing metrics according to stakeholder preference. We apply this approach to lithium-ion batteries. The results indicate that the approach can be used to select a combination of model and algorithm that balances tradeoffs between competing objectives, such as cost and utilization. Moreover, the framework is general and accommodates both existing and future degradation models and algorithms.

3.4.2 Digital Twins in a Nearly Autonomous Management and Control System for Advanced Reactors

Linyu Lin, Postdoctoral Research Scholar, Department of Nuclear Engineering North Carolina State University

<u>Presentation Overview</u>: This presentation introduces the implementation of a nearly autonomous management and control system with digital twins and machine learning algorithms. The presentation discusses four design principles for nearly autonomous management and control: three-layer architecture, modular frameworks, digital twin development and assessment processes, and digital twin trustworthiness assessment.

3.4.3 Digital Twins for Prognostic Health Management in Nuclear Energy: Opportunities and Challenges

Pradeep Ramuhalli, Distinguished Scientist Oak Ridge National Laboratory

<u>Presentation Overview</u>: Oak Ridge National Laboratory (ORNL) is a key player for providing diagnostics, prognostics, and decision making using an intelligent digital twin. ORNL is leading in research in sensors, modeling, simulation, data analytics, and advanced manufacturing and communication technologies to make risk informed operational and maintenance decisions. Data driven with physics models allow diagnostics and predictive maintenance. The Bayesian method integrated with failure physics information minimizes uncertainties. Resulting technologies enable sustainable nuclear power by improving the reliability of nuclear plants.

4 DAY 4 PRESENTATIONS

4.1 <u>Closing Plenary Session Summary</u>

The closing plenary comprised technical presentations covering a wide variety of topics from a regulatory perspective, including risk in digital twins, regulations, and nonproliferation. A panel session followed with industry representatives, advanced reactor designers, and others. Dr. Raj lyengar of the NRC presented the closing remarks.

Participants in this session identified the following challenges:

- Implementation of digital twins in the nuclear setting is increasingly complex, as nuclear power plant operational models are difficult to change.
- Real-time data management to support digital twin implementation is a challenge.
- It is a challenge to identify the areas in which digital twins can contribute to optimizing regulatory oversight.

Participants in this session identified the following key takeaways:

- Digital twin technology may be able to reduce the scope and cost of regulatory oversight.
- Digital twins can help identify the components that really matter for safety.
- It will be helpful if organizations turn over digital twins to the NRC to increase shared information and system knowledge. Sharing models directly with the NRC staff has already been fruitful. It has saved hundreds of hours by providing a platform for direct interactions with the digital twin finite-element models, allowing quick responses to questions.
- The IAEA is working on a plant taxonomy that may eventually support digital twins.
- It is important to learn from organizations in other regulated industries that use digital twins, such as self-driving car manufacturers and the Federal Aviation Administration. For example, the Food and Drug Administration uses risk-based information to determine how extensively to test a new drug.

The presentations slides for Day 4 can be found <u>here</u> and in the Agency Documents Access and Management System (ADAMS) under <u>ML20356A236</u>.

Presentations

4.1.1 Managing Regulated Change: An Enterprise-Level Digital Twin for the Nuclear Industry

Michael Mazzola, Executive Director, Energy Production and Infrastructure Center University of North Carolina at Charlotte

<u>Presentation Overview</u>: An enterprise level-digital twin integrates all business, technical, and regulatory compliance on an enterprise wide-digital platform. Such a system would allow the NRC to participate more collaboratively in the process of making changes while maintaining the intent of the approved design control document. With adequate provision for the independence of the NRC's oversight, a certified enterprise-level digital twin would allow both the enterprise and the NRC to carry out efficient assessment and approval of changes while maintaining the as constructed- plant's performance to license.

4.1.2 Including Risk in Digital Twins

Michael Calley, Department Manager, Regulatory Support, Nuclear Safety and Regulatory Research Division Idaho National Laboratory

<u>Presentation Overview</u>: This presentation looks at including risk in digital twin applications. Digital twin technology will be part of next generation-reactors. Risk in terms of performance shortfalls is a powerful way to characterize and understand complex systems, and public health frequency consequence- is a key part of a risk informed-- approach. Completeness in design and operation must account for uncertainties. If risk elements are taken into consideration, a digital twin approach can yield major efficiency improvements in the design, operation, and licensing of advanced reactors.

4.1.3 Towards a Digital Twin to Detect Nuclear Proliferation Activities

Christopher Ritter, Director, Digital Innovation Center of Excellence Idaho National Laboratory

<u>Presentation Overview</u>: This project will develop technologies to enable digital engineering and digital twinning to assist in diversion pathway analysis and apply safeguards by design concept for advanced reactors and power plants. Digital twinning and digital engineering have produced significant performance improvements and schedule reduction in the aerospace, automotive, and construction industries. This integrated modeling approach has not been fully applied to nuclear safeguards programs in the past. Digital twinning combined with AI technologies can lead to innovations in process monitoring detection, particularly in event classification and data tampering.

4.2 Digital Twin Regulatory Discussion Panel

4.2.1 Bret Kugelmass, Managing Director

Energy Impact Center

4.2.2 Neil Olivier, Director of Corporate Services

NuScale Power

- 4.2.3 Pat Everett, Director of Thermal Engineering Oklo, Inc.
- 4.2.4 Gregory A. Banyay, Modeling and Simulation Hub Technical Lead (Principal Engineer) Westinghouse Electric Company

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APPENDIX A SPEAKERS BIOS

A.1 Day 1 Speaker Bios

Michael Grieves, Chief Scientist of Advanced Manufacturing, Executive Vice President of Operations

Florida Institute of Technology

<u>Education/Experience</u>: Dr. Michael Grieves is currently at the Florida Institute of Technology in Melbourne, FL, where he helped form the Center for Advanced Manufacturing and Innovative Design. He is an internationally renowned expert in product life cycle management (PLM) and originated the concept of the digital twin. His focus is on virtual product development and engineering; systems engineering and complex systems; manufacturing, especially additive manufacturing; and operational sustainment. Dr. Grieves wrote the seminal books on PLM, *Product Lifecycle Management* and *Virtually Perfect: Driving Innovative and Lean Products through PLM*. He has consulted with and done research at top global organizations, including the National Aeronautics and Space Administration (NASA), Boeing, Newport News Shipbuilding, and General Motors. Dr. Grieves has presented at industry and academic conferences worldwide.

Joseph Berti, Vice President of Offering Management IBM

<u>Education/Experience</u>: Mr. Joseph Berti provides detailed direction on key technical and marketing tasks to launch offering features, offerings, or collections of offerings. He is accountable for key outcome metrics, including customer satisfaction, accessibility, and the revenue and profitability of assigned offerings. Mr. Berti also demonstrates offering capabilities and value propositions to external stakeholders (customers, partners, and analysts), analyzing feedback to identify potential gaps or opportunities, and recommending solutions. Mr. Berti has over 25 years of leadership experience in software and services in more than eight industries, with a focus on transforming industries using optimization technologies. As an experienced innovator, Mr. Berti has launched more than 10 products and services, transforming entire industries. Before joining IBM, Mr. Berti led the company Oniqua into a rapid -growth cycle while launching new products and achieving new levels of innovation. Since IBM acquired Oniqua, Mr. Berti has led the IBM Maximo[®] Inventory Optimization product offering. Mr. Berti received a bachelor's degree in finance and management information systems from Ohio State University.

Jeremy Busby, Division Director, Reactor and Nuclear Systems Division Oak Ridge National Laboratory

<u>Education/Experience</u>: Dr. Jeremy Busby's research focuses on materials performance and the development of materials for nuclear reactor applications. While at Oak Ridge National Laboratory (ORNL), Dr. Busby has participated in materials research efforts for space reactors, fusion machines, advanced fast reactors, and light -water reactors (LWRs). Ultimately, the results of this diverse research will enable the development of operating criteria for structural materials in a variety of adverse environments, which will allow the design and operation of safe, reliable, and cost -effective nuclear systems.

From 2009 to 2015, Dr. Busby led the Materials Aging and Degradation Pathway for the Light Water Reactor Sustainability research and development (R&D) program at the U.S. Department of Energy's (DOE's) Office of Nuclear Energy. He also led the Materials Cross--Cut effort of the Nuclear Energy Enabling Technologies program, in addition to participating in several research

tasks sponsored by the nuclear industry. As principal investigator for the DOE Office of Science ITER program, he led an investigation into the feasibility of using an innovative cast austenitic stainless steel for the first wall structure of the international ITER project. In 2010, following this effort, Dr. Busby received the Presidential Early Career Award for Science and Engineering for "excellence in research leading to the development of high -performance cast stainless steels, a critical part of the U.S. Contributions to ITER project, and for his mentoring of students both as an Adjunct Assistant Professor at the University of Michigan and at ORNL." In 2011, he received the Secretary of Energy Achievement Award for contributions to the DOE's response to the Fukushima Dai-ichi nuclear accident. The American Nuclear Society presented Dr. Busby with the Landis Young Member Achievement award in 2006, and in 2007 he received the ORNL Early Career Award for Engineering Accomplishment for his leadership in the cast stainless steel effort.

Ashley Finan, Director National Reactor Innovation Center

<u>Education/Experience</u>: Dr. Ashley Finan is the director of the National Reactor Innovation Center. In this role, she oversees initiatives to provide reactor innovators with resources to test, demonstrate, and conduct performance assessments to accelerate the deployment of advanced nuclear technology concepts. Dr. Finan holds a bachelor's degree in physics as well as bachelor's and master's degrees in nuclear science and engineering from the Massachusetts Institute of Technology.

Dr. Finan earned her doctoral degree in nuclear science and engineering at the Massachusetts Institute of Technology. Her doctoral work focused on energy innovation investment and policy optimization, in both nuclear and renewable energy technologies. She has played a key role in studies of the use of advanced nuclear energy to reduce greenhouse gas emissions in several applications, including hydrogen production, coal -to -liquids processes, and oil production methods. She has worked as a strategy and engineering consultant, primarily on nuclear energy applications. She has also contributed to analyses of the technoeconomic potential of energy efficiency improvements in the residential and commercial sectors, and several related topics.

Jenifer Shafer, Special Government Employee Consultant Advanced Research Projects Agency–Energy

<u>Education/Experience</u>: Dr. Jenifer Shafer currently serves as a special government employee consultant at the Advanced Research Projects Agency–Energy (ARPA-E). At ARPA-E, she focuses on developing innovative and proliferation -resistant technologies to manage nuclear waste and used nuclear fuel. She is an expert in nuclear separations, nuclear forensics, and the fundamentals of actinide chemistry. Before joining ARPA-E, Dr. Shafer served on the faculty at the Colorado School of Mines as an Associate Professor in the Chemistry Department and the Nuclear Science and Engineering Program. Before that, she worked for 2 years at Pacific Northwest National Laboratory.

Dr. Shafer received a bachelor's degree from Colorado State University in 2005 and a doctoral degree from Washington State University in 2010. She was a DOE Early Career Award winner and currently serves on the American Chemical Society's Committee on Science. She is the coauthor of several book chapters and nearly 60 technical manuscripts, and she has led or collaborated on several projects for the DOE, the U.S. Department of Homeland Security, and the U.S. Department of Defense, as well as the National Science Foundation.

Ian Davis, Senior Digital Twin System Engineer X-Energy LLC

<u>Education/Experience</u>: Mr. Ian Davis holds both a bachelor's and a master's degree in nuclear engineering from the Pennsylvania State University. He is a nuclear engineer with over 6 years of experience in the nuclear power generation industry, specializing in the simulation of thermal hydraulics and neutronics, software programming, and data science.

Anthonie Cilliers, Senior Manager, Instrumentation, Controls, and Electrical Kairos Power

<u>Education/Experience</u>: Dr. Anthonie Cilliers holds a doctoral degree in nuclear engineering and both a master's and a bachelor's degree in computer and electronic engineering. He has over 12 years of experience in nuclear -specific plant control and protection systems, specializing in model reference plant control and diagnostics and control system architecture.

Clyde Huibregtse, Reactor/Software Engineer Oklo, Inc.

<u>Education/Experience</u>: Mr. Clyde Huibregtse holds a bachelor's degree in both mathematics for computer science and physics from the Massachusetts Institute of Technology. He has been with Oklo in some capacity for almost 3 years.

Charles R. "Chip" Martin, Scientific/Technical Advisor HolosGen, LLC

<u>Education/Experience</u>: Dr. Charles R. Martin holds a doctoral degree in nuclear engineering from the U.S. Air Force Institute of Technology and a bachelor's degree in nuclear engineering from North Carolina State University. He is currently an executive consultant with Longenecker and Associates, but he has held many interesting positions over the years. In 2018, he was the Glenn T. Seaborg Science and Technology Policy Fellow for the American Nuclear Society and the American Association for the Advancement of Science. In this role, he served as a staffer in the U.S. House of Representatives. Before that, he was the chief nuclear officer for the Nevada National Security Site. He has served on the faculties of the University of Nevada, Las Vegas; the University of Maryland; and the U.S. Air Force Academy. He served as a technical specialist at the U.S. Defense Nuclear Facilities Safety Board; he was a nuclear research officer in the Office of the Secretary of the Air Force; he managed the U.S. Advanced Space Reactor Program at the DOE; and he served as technical director for three underground nuclear weapon tests.

Abhinav Saxena, Senior Scientist, Machine Learning, GE Research General Electric

Education/Experience: Dr. Abhinav Saxena is a senior scientist in AI and learning systems at General Electric (GE) Research and the principal investigator for the GE -led Generating Electricity Managed by Intelligent Nuclear Assets (GEMINA) Award 2174-1511. Dr. Saxena is developing prognostics and health management (PHM) solutions based on machine learning and AI for various industrial systems at GE (aviation, nuclear, power, and healthcare) and is driving the integration of AI -based PHM analytics in GE's industrial systems. Before joining GE, Dr. Saxena worked as a research scientist at NASA Ames Research Center, carrying out fundamental research on prognostics methods and evaluation. Dr. Saxena has over 15 years of experience in developing predictive maintenance methods and technologies. He is also an adjunct professor in the Division of Operation and Maintenance Engineering at Luleå University

of Technology, Sweden. His interests lie in developing PHM methods and algorithms with special emphasis on deep learning and data -driven methods in general for practical prognostics. Dr. Saxena has published over 100 peer -reviewed technical papers and has coauthored a seminal book on prognostics. He is a fellow of the PHM Society and actively participates in several SAE standards committees, the Institute of Electrical and Electronics Engineers (IEEE) prognostics standards committee, and various PHM Society educational activities. He has served as the chief editor of the *International Journal of Prognostics and Health Management* since 2011 and actively participates in organizing PHM Society conferences.

Chandler Maskal, Offering Manager, IBM AI Applications IBM

<u>Education/Experience</u>: Ms. Chandler Maskal graduated from Rensselaer Polytechnic Institute in 2018 with bachelor's and master's degrees in information technology and Web science. She has worked for IBM for 2 years as an offering manager on multiple products in the enterprise asset management space. She most recently led the launch of IBM's digital twin initiative in May 2020.

Mark Buller, Principal Investigator, Biophysics and Biomedical Modeling Division U.S. Army Research Institute of Environmental Medicine

<u>Education/Experience</u>: Dr. Mark Buller gained his doctorate in computer science from Brown University in the area of computational physiology. Dr. Buller has over 20 years of experience in designing and fielding ambulatory physiological monitoring systems for warfighters. Dr. Buller's current research interests are real--time algorithms that determine health state from wearable sensors and performance optimization from physiological feedback. Dr. Buller is currently the principal investigator of a multi--institute research study to identify noninvasive markers of exertional heat stroke. Dr. Buller has authored more than 100 publications and is currently serving as a chair for the North Atlantic Treaty Organization (NATO) working group "Development of a NATO STANREC for Physiological Status Monitoring to Mitigate Exertional Heat Illness."

Kevin P. Chen, Professor University of Pittsburgh

Education/Experience: Dr. Kevin Chen received a doctoral degree in 2002 from the University of Toronto.

A.2 Day 2 Speaker Bios

Joel Fetter, ARPA-E, Lead Associate Booz Allen Hamilton

<u>Education/Experience</u>: Over the past 9 years, Mr. Joel Fetter has focused primarily on the establishment of ARPA-E's Technology -to -Market capability, where he advises on the development and implementation of program structures that translate science into business concepts. To date, ARPA-E portfolios have accrued many billions of dollars in follow -on funding, extensive patent activity, and seminal research that has created new learning curves for advanced energy technologies. Most recently, Mr. Fetter advised on the creation of ARPA-E's initial suite of investments into advanced nuclear energy systems, which comprised nearly \$100 million in enabling technologies, microreactors, and improved O&M technologies.

Before his engagement with ARPA-E, Mr. Fetter consulted to public, private, and nonprofit organizations across the energy landscape. He earned a master's degree in law and diplomacy from the Fletcher School at Tufts University and a bachelor's degree, summa cum laude, in international affairs from the University of Colorado at Boulder.

Abhinav Saxena, Senior Scientist, Machine Learning, GE Research General Electric

Education/Experience Summary: Dr. Abhinav Saxena is a senior scientist in AI and learning systems at GE Research and the principal investigator for the GE -led GEMINA Award 2174-1511. Dr. Saxena is developing PHM solutions based on machine learning and AI for various industrial systems at GE (aviation, nuclear, power, and healthcare) and is driving the integration of AI -based PHM analytics in GE's industrial systems. Before joining GE, Dr. Saxena worked as a research scientist at NASA Ames Research Center, carrying out fundamental research on prognostics methods and evaluation. He has over 15 years of experience in developing predictive maintenance methods and technologies. He is also an adjunct professor in the Division of Operation and Maintenance Engineering at Luleå University of Technology, Sweden.

Yvotte Brits, Supply Chain Manager and Operator Training Simulator Program Manager X-Energy LLC

<u>Education/Experience</u>: Mr. Yvotte Brits holds a master's degree in nuclear engineering and electric and electronic engineering. Mr. Brits is a nuclear engineer with 13 years of vital experience in the international nuclear industry, specializing in supply chain management, operator training simulator program management, energy plant transient analyses, instrumentation and control design, cost modeling, and plant system design for power plants.

Anthonie Cilliers, Senior Manager, Instrumentation, Controls, and Electrical Kairos Power

<u>Education/Experience</u>: Dr. Anthonie Cilliers holds a doctoral degree in nuclear engineering and master's and bachelor's degrees in computer and electronic engineering. Dr. Cilliers has over 12 years of experience in nuclear -specific plant control and protection systems; he specializes in model reference plant control and diagnostics and control system architecture.

Brian Golchert, Principal Engineer Westinghouse Electric Company

<u>Education/Experience</u>: Dr. Brian Golchert holds a doctorate in nuclear engineering from the University of Illinois. He has engineering work experience from Argonne National Laboratory (ANL), Fluent, GE Nuclear, and Westinghouse, as well as teaching experience at DePaul University (mathematics and statistics) and Purdue Calumet (engineering).

Matthew LeVasseur, Director of Research BWX Technologies

Education/Experience: Mr. Matthew LeVasseur has been with BWX Technologies over 21 years; he previously spent 10 years with the U.S. Marine Corps as an aerospace officer. He holds degrees and qualifications from Duke University (Global Executive Management M.B.A., with honors, 2007), the University of Michigan (M.S. in aerospace science, 1995, and B.S. in astronomy, 1989); Six Sigma Qualtec (Master Black Belt, Process Analytics and Data Methods,

2006); and the U.S. Space Command (Space Control Qualification, Data Modeling/Infrastructure, 1997).

Ryan Kitchen, Research and Development Data Scientist BWX Technologies

<u>Education/Experience</u>: Mr. Ryan Kitchen is the lead data scientist and innovator for BWX Technologies on a shared project with ORNL to develop digital twin technology for electron beam melt additive manufacturing as well as additional R&D for nuclear manufacturing. Mr. Kitchen brings expertise in high performance computing, GPU computing, biocomputing, and machine vision and instrumentation for integration into manufacturing systems. He received a bachelor's degree in computer science from Oregon State University in 2018.

Hasan Charkas, Principal Technical Leader Electric Power Research Institute

<u>Education/Experience</u>: Dr. Hasan Charkas holds a doctoral degree in structural engineering and engineering mechanics. Dr. Charkas has been with the Electric Power Research Institute (EPRI) for almost 5 years. Previously, he worked for Areva/Framatome as an engineering supervisor in the component analysis and fracture mechanics group (specializing in stress analysis for nuclear steam supply system components and reactor vessel internals). Before working at Areva/Framatome, Dr. Charkas was a design engineer for a structural group (specializing in strengthening of deficient structures).

Hash Hashemian, President and CEO Analysis and Measurement Services Corporation

<u>Education/Experience</u>: Dr. Hash Hashemian obtained a D.E. degree in electrical engineering from Lamar University, Beaumont, in 2009; a doctoral degree in nuclear engineering from Chalmers University, Gothenburg, in 2010; and a doctoral degree in computer engineering from Western University, London, in 2011.

Jacob Houser, Senior Research Engineer Analysis and Measurement Services Corporation

<u>Education/Experience</u>: Dr. Jacob Houser holds doctoral and master's degrees in mechanical engineering from the University of Tennessee, Knoxville, and a bachelor's degree in mechanical engineering and management from the Rensselaer Polytechnic Institute.

David E. Womble, Director of Artificial Intelligence Programs Oak Ridge National Laboratory

<u>Education/Experience</u>: Dr. David E. Womble received his doctoral degree in applied mathematics from Georgia Tech in 1986. Before joining ORNL in 2017, Dr. Womble served as the program deputy for Advanced Simulation and Computing at Sandia National Laboratories, responsible for developing and deploying modeling and simulation (M&S) capabilities. He also served as the senior manager for the Computational Simulation Group and for the Computer Science and Mathematics Group. His recognitions include two R&D 100 Awards and the Gordon Bell Award. Dr. Womble's research interests include numerical algorithms and methods for machine learning and high -performance computing, including the solution of linear and nonlinear systems, multigrid and multiscale algorithms, time-series analysis, and scalable algorithms in high -performance computing. Dr. Womble has also worked across several

application domains, including seismic imaging, semiconductor device simulation, computational mechanics, and wind energy.

Richard Vilim, Senior Nuclear Engineer; Manager, Plant Analysis and Control and Sensors Department, Nuclear Science and Engineering Division Argonne National Laboratory

<u>Education/Experience</u>: Dr. Richard Vilim has over 30 years of professional experience in the design and safety analysis of nuclear reactors, with ongoing research projects involving control system design, M&S of nuclear systems, the operation of advanced nuclear reactors employing load -following and load -leveling using energy storage, and AI and machine learning for plant performance improvement. He is an author on over 300 reports and publications and nine U.S. patents.

Christopher Ritter, Director, Digital Innovation Center of Excellence Idaho National Laboratory

<u>Education/Experience</u>: With a bachelor's degree in computer science from Virginia Polytechnic Institute and State University, Mr. Christopher Ritter is a group lead with the Digital and Software Engineering Group at Idaho National Laboratory (INL). His expertise is in software engineering, software development, leading software teams, systems engineering software integration, and database management. Before coming to INL, he was director of software development at SPEC Innovations, where he was the chief architect of Innoslate. He also architected the software system and consulted on the data ontology for a centralized mission risk management system for the Joint Staff at the Pentagon and supported the Marine Corps business process reengineering for its Capability Portfolio Management processes. In addition, he has served as a computer programming teacher at St. Michael's Academy in Warrenton, VA.

Ben Betzler, Nuclear Engineer Oak Ridge National Laboratory

<u>Education/Experience</u>: Dr. Benjamin R. Betzler is an outcome -focused reactor physics nuclear engineer with demonstrated experience and performance on R&D programs for a variety of sponsors; his experience includes leading diverse multiorganization teams. He has recognized expertise in both reactor analysis and methods development, with specialized knowledge of advanced reactor systems (e.g., molten -salt reactors, microreactors, HTGRs, and space propulsion systems) and Monte Carlo radiation transport methods (alpha -eigenvalue methods, time -dependent problems, and matrix methods and applications of Markov processes). Dr. Betzler received his doctoral degree in nuclear engineering and radiological sciences from the University of Michigan in 2014.

Vincent Paquit, Senior Research Scientist, Electrical and Electronics Systems Research Oak Ridge National Laboratory

<u>Education/Experience</u>: Before joining ORNL, Dr. Vincent Paquit worked at the University of Burgundy (France), in the Laboratoire d'Électronique et d'Informatique de l'Image, as an engineer in technology transfer for all commercial and technical applications in the fields of electronics, computer science, and signal processing. Since then, Dr. Paquit has been an active member of the Imaging, Signals, and Machine Learning Group at ORNL, working on multiple projects and programs supporting two core missions of the DOE: energy sustainability and national security. He is contributing to ORNL's scientific endeavors by conceiving, designing, and implementing complex computer vision and multidimensional imaging systems—combining hardware and software development—to perform quantitative analysis of complex

datasets and to make quantitative measurements of various objects. Currently, Dr. Paquit is the data analytics lead for the Manufacturing Demonstration Facility. His team is developing a data analytics framework aimed at better understanding of additive manufacturing processes for the purpose of process certification and control. His research interests include applied signal and image processing, algorithm development on GPU platforms, two- and three -dimensional image segmentation, multispectral and hyperspectral imaging, biomedical imaging, pattern recognition, remote sensing data understanding, and machine learning. He has published numerous peer -reviewed articles and one book chapter, submitted multiple invention disclosures, and served on program committees of several international conferences.

Eric Helm, Product Manager—Metroscope Framatome

<u>Education/Experience</u>: Mr. Eric Helm holds a bachelor's degree in mechanical engineering and a master's in systems engineering. He has 5 years of experience in the automotive manufacturing industry and 15 years of experience at Framatome in a variety of engineering roles, including fuel fabrication, large projects, systems engineering methods, field service, equipment analytics, and advanced diagnostics with Metroscope.

Richard Vilim, Senior Nuclear Engineer; Manager, Plant Analysis and Control and Sensors Department, Nuclear Science and Engineering Division Argonne National Laboratory

<u>Education/Experience</u>: Dr. Richard Vilim has over 30 years of professional experience in the design and safety analysis of nuclear reactors, with ongoing research projects involving control system design, M&S of nuclear systems, the operation of advanced nuclear reactors employing load-following and load-leveling using energy storage, and AI and machine learning for plant performance improvement. He is an author on over 300 reports and publications and nine U.S. patents.

Rizwan Uddin, Professor and Head of the Department of Nuclear, Plasma, and Radiological Engineering University of Illinois at Urbana-Champaign

<u>Education/Experience</u>: Professor Rizwan Uddin is a fellow of the American Nuclear Society. He directs the Virtual Education and Research Lab and the Master of Engineering in Energy Systems program at the University of Illinois at UrbanaChampaign (UIUC). He received the American Society of Engineering Education's Glenn Murphy Award in 2015, the American Nuclear Society's Arthur Holy Compton Award for his teaching and research accomplishments in 2016, and UIUC's Campus Award for Excellence in Guiding Undergraduate Research in 2017.

Subhasish Mohanty, Principal Research and Development Engineer Argonne National Laboratory

<u>Education/Experience</u>: Dr. Subhasish Mohanty is currently working as a principal R&D engineer at the Nuclear Science and Engineering division of ANL. Dr. Mohanty began working at ANL in 2010 after finishing his doctoral degree in aerospace engineering from Arizona State University. Dr. Mohanty also has 4 years of experience in the aerospace industry. His experience and interests primarily focus on structural mechanics and digital twins of nuclear reactor and aerospace systems; machine learning, AI, and data analytics techniques; and Internet -of -things concepts.

A.3 Day 3 Speaker Bios

Richard Henry, Section Manager, Computers and Control Design, Central Engineering Ontario Power Generation

<u>Education/Experience</u>: Mr. Richard Henry holds a bachelor's degree in electrical engineering from McMaster University.

John Sladek, Specialist, Systems Engineering Division, Directorate of Assessment and Analysis Canadian Nuclear Safety Commission

<u>Education/Experience</u>: Mr. John Sladek holds a bachelor's degree in electrical engineering from Queen's University, Kingston.

Albrecht Kyrieleis, Senior Consultant, Jacobs

<u>Education/Experience</u>: With more than 10 years of experience in the nuclear industry and a background in physics, Dr. Albrecht Kyrieleis has worked on a broad range of projects in the areas of simulation software development, radiation shielding and protection, and nuclear physics and criticality. Involved in fission as well as fusion, he has led various R&D and application projects and is the technical lead for the United Kingdom's Nuclear Virtual Engineering Capability project, responsible for the overall technical program.

Susana López Lumbierres, Senior Project Manager, Tecnatom

Education/Experience: Ms. Susana López Lumbierres is an industrial and simulation Engineer.

Panagiotis Manolatos, Project Officer, European Atomic Energy Community

<u>Education/Experience</u>: Dr. Panagiotis Manolatos is an engineer who holds a doctoral degree in materials sciences from the École des Mines in France. His experience includes 10 years in laboratory research on the behavior of materials and components at various European national laboratories (in France and the Netherlands), 5 years tutoring at the École Centrale de Paris in France, and 20 years coordinating research in nuclear safety at the European Commission's Directorate General for Research and Innovation.

Abderrahim Al Mazouzi, Expert Group in charge of the European affairs of the research and development program on energy production EDF France

<u>Education/Experience</u>: Dr. Abderrahim Al Mazouzi acted as the general secretariat and as a member of the executive committee of the Nuclear Generation II & III Alliance (NUGENIA) for 7 years. After receiving his doctoral degree in materials science in 1989, he spent 3 years as a postdoctoral researcher at the Hahn Meitner Institute in Berlin, Germany, then held a position as visiting scientist at Kyoto University, Japan, from 1993 to 1995.

In 1995, he joined the Centre de Recherches en Physique des Plasmas at the École Polytechnique Fédérale de Lausanne (Switzerland) to work on fusion technology. He then moved to the Paul Sherrer Institute (Switzerland), where he acted as project manager at the hot-lab facility from 1999 to 2001. From 2002 until 2009, he served as a senior scientist and then group leader at SCK CEN, Belgium, before joining Électricité de France (EDF) R&D.

Christopher Spirito, Nuclear Cybersecurity Consultant, Idaho National Laboratory

<u>Education/Experience</u>: Mr. Christopher Spirito is a nuclear cybersecurity consultant with INL. He supports domestic and international programs with the DOE and the International Atomic Energy Agency (IAEA). For the past 7 years he worked closely with the Korea Atomic Energy Research Institute and the Korea Institute of Nuclear Nonproliferation and Control through the U.S.– Republic of Korea bilateral relationship, as well as on international research projects through the IAEA. Mr. Spirito is also a visiting professor in the Faculty of Law at the University of Tartu, Estonia, and a board member for WiRED International, a global health NGO providing support to underserved regions of the world. Mr. Spirito graduated from Boston College with a degree in mathematics and attended graduate school at Worcester Polytechnic Institute and the Harvard School of Public Health. Before joining INL, Mr. Spirito was the International Cyber Lead for The MITRE Corporation.

Cynthia DeBisschop, Senior Cybersecurity Analyst Oasis Systems, LLC (NRC contractors)

Education/Experience: Dr. Cynthia DeBisschop holds a bachelor's degree in chemical engineering from Drexel University and master's and doctoral degrees in engineering sciences and applied mathematics from Northwestern University. Inspired by computational mentors at the Mobil Research and Development Corporation, where she worked as a cooperative education student early in her career, she pursued graduate research focused on the mathematical modeling of physical processes at Northwestern as a National Science Foundation Graduate Research Fellow. She engaged in computational interdisciplinary research as a postdoctoral researcher in the Department of Mathematics and Statistics at Old Dominion University. In 2009, Dr. DeBisschop began work as a research analyst for CNA's Institute for Public Research under contract to the Federal Aviation Administration, where she conducted research in data and information management and in systems engineering for the NextGen modernization effort. While at CNA, she coauthored a paper that won two David Lubkowski Memorial Best Paper Awards in 2011.

Since 2017, Dr. DeBisschop has supported the cybersecurity program at the U.S. Nuclear Regulatory Commission (NRC) and has conducted and supported 18 cybersecurity inspections of nuclear power plants. She assisted in the presentation of the Advanced Cyber Security Inspection training course at the NRC Technical Training Center in Chattanooga, TN. More recently, she assisted in the development of NRC regulatory guidance.

Rodney Busquim e Silva, Computer Security Officer, International Atomic Energy Agency

Dave Kropaczek, Director, Consortium for Advanced Simulation of Light Water Reactors (a DOE Energy Innovation Hub) Oak Ridge National Laboratory

<u>Education/Experience</u>: Former president and CEO of Studsvik Scanpower, the nuclear software division of Studsvik AB, Dr. Dave Kropaczek holds a bachelor's degree in engineering science from the New Jersey Institute of Technology and master's and doctoral degrees in nuclear engineering from North Carolina State University. He has over 27 years of experience in the nuclear industry, with areas of expertise including fuel cycle and plant optimization, computational reactor physics and thermal hydraulics, and numerical algorithm development. Previous experience includes positions in research, product development, and management, including 9 years with GE Global Nuclear Fuel, developing methods and software for boiling water reactor

fuel technology; 12 years with Studsvik Scanpower, developing methods for real-time kinetics simulation and multicycle optimization; and 3 years with Westinghouse Fuels, focusing on core design and monitoring applications. In addition, Dr. Kropaczek spent 3 years as a research assistant professor at North Carolina State University, working with students and on R&D projects sponsored through the Electric Power Research Center. Dr. Kropaczek serves as the American Nuclear Society Reactor Physics Division Chair for the Advances in Nuclear Fuel Management topical meetings, and as technical reviewer for several journals, including *Nuclear Technology* and *Nuclear Science and Technology*.

Jeffrey W. Lane, Chief Engineer and Principal Consultant, Zachry Nuclear Engineering

<u>Education/Experience</u>: Dr. Jeffrey W. Lane has 15 years of software development experience in computational thermal hydraulics and reactor safety analysis for existing LWRs, as well as for next generation small modular reactor and non-LWR concepts. His expertise is in multiphysics and multiscale methods; verification, validation, and uncertainty quantification; and software quality assurance. Dr. Lane has also taken part in digital twin development, autonomous control, and data driven modeling. Currently, he is the technical lead and program manager for the GOTHIC coarse grid computational fluid dynamics software. Dr. Lane worked for the Bettis Atomic Power Laboratory, where he was responsible for advancing simulation capabilities to support existing and future applications in the Naval Nuclear Propulsion Program, including the safety analysis for the Ford class aircraft carrier, multiphysics methods development, and integrated plant analysis development. Dr. Lane received his doctoral degree from the Pennsylvania State University, where he studied under the Rickover Fellowship Program in nuclear engineering.

Rui Hu, Manager, Plant System Analysis Group, Argonne National Laboratory

<u>Education/Experience</u>: Dr. Rui Hu holds a doctoral degree in nuclear engineering from the Massachusetts Institute of Technology.

Felipe A. C. Viana, Assistant Professor, University of Central Florida

<u>Education/Experience</u>: Before joining the University of Central Florida, Dr. Felipe A.C. Viana was a senior scientist at GE Renewable Energy, where he led the development of computational methods for improving wind turbine performance and reliability. Before that role at GE, he spent 5 years at GE Global Research, where he led and conducted research on design and optimization under uncertainty, probabilistic analysis of engineering systems, and services engineering. Dr. Viana holds a doctoral degree in aerospace engineering from the University of Florida and both a doctoral and a master's degree in mechanical engineering from the Federal University of Uberlândia (Brazil).

Lance Fiondella, Associate Professor of Electrical and Computer Engineering at University of Massachusetts in Dartmouth

<u>Education/Experience</u>: Dr. Lance Fiondella holds a doctoral degree in computer science and engineering from the University of Connecticut.

Linyu Lin, Postdoctoral Research Scholar, Department of Nuclear Engineering of North Carolina State University

Education/Experience: Dr. Linyu Lin holds a doctoral degree in nuclear engineering.

Pradeep Ramuhalli, Distinguished Scientist at Oak Ridge National Laboratory

Education/Experience: Over the last 18 years, Dr. Pradeep Ramuhalli has led and contributed to advances in systems resilience and reliability, with current research focused on developing technologies that enable robust digital twins, and on applying these technologies to improve the economics of nuclear power, enhance the reliability of renewable energy systems, and support cybersecurity and international safeguards. Relevant technology areas include sensors and algorithms for the continuous online monitoring of stressors and systems for degradation detection and characterization, physics informed machine learning algorithms for prognostic assessment of system and component remaining useful life, and risk informed methodologies to ensure the reliability of measurements and resilience of degraded systems. Dr. Ramuhalli has coedited a book on integrated vision and imaging techniques for industrial inspection and has authored or coauthored four book chapters, over 175 technical publications in peer reviewed journals and conferences (including over 35 peer reviewed journal publications), and over 90 technical research reports. He is a senior member of IEEE and a member of the American Nuclear Society.

A.4 Day 4 Speaker bios

Michael Mazzola, Executive Director, Energy Production and Infrastructure Center, University of North Carolina at Charlotte

<u>Education/Experience</u>: Dr. Michael Mazzola attended the University of North Carolina at Charlotte; he received a doctoral degree in electrical engineering from Old Dominion University in 1990. From 1990 to 1993, Dr. Mazzola was employed by the U.S. Navy at the Naval Surface Warfare Center Dahlgren. From 1993 to 2017, he served on the faculty in the Electrical and Computer Engineering Department at Mississippi State University. In 2009, he was appointed Associate Director for Advanced Vehicle Systems at the Center for Advanced Vehicular Systems, a unit of the High-Performance Computing Collaboratory at Mississippi State University. In July 2017, Dr. Mazzola was appointed the executive director of the Energy Production and Infrastructure Center, as well as the Duke Energy Distinguished Chair in Power Engineering Systems, at the University of North Carolina at Charlotte.

Michael Calley, Department Manager, Regulatory Support, Nuclear Safety and Regulatory Research Division Idaho National Laboratory

<u>Education/Experience</u>: Mr. Michael Calley has over 31 years of experience in probabilistic risk assessment (PRA), safety evaluations, and hazards assessments, including project management. His experience includes performing PRAs for both commercial nuclear power plants and nuclear research and test reactors, supporting the NRC on inspections at commercial nuclear power plants, and providing PRA technology transfers both domestically and internationally. He has knowledge of preparing hazards assessments, developing guidelines for the preparation of safety analysis reports, and resolving concerns about the adequacy of safety analysis reports. Mr. Calley's background also includes the comprehensive use of PRA software. He holds a master's degree in nuclear science and engineering and a bachelor's degree in general engineering from Idaho State University.

Christopher Ritter, Director, Digital Innovation Center of Excellence of Idaho National Laboratory

<u>Education/Experience:</u> Mr. Christopher S. Ritter is a group lead with the Digital and Software Engineering Group at INL. His expertise is in software engineering, software development, leading software teams, systems engineering software integration, and database management. Before coming to INL, he directed software development at SPEC Innovations, in Manassas, VA. He served as the chief architect of Innoslate, a popular system engineering tool that leverages elastic cloud technologies and AI and neurolinguistic programming for high scalability and advanced analytics. Mr. Ritter architected the software system and consulted on the data ontology for a centralized mission risk management system for the Joint Staff at the Pentagon and supported the Marine Corps business process reengineering for its Capability Portfolio Management processes. He was also a computer programming teacher at St. Michael's Academy in Warrenton, VA, and developed an elementary school computer programming curriculum. He holds a bachelor's degree in computer science from Virginia Polytechnic Institute and State University.

Bret Kugelmass, Managing Director of Energy Impact Center

<u>Education/Experience</u>: Mr. Bret Kugelmass holds a master's degree in mechanical engineering from Stanford University and is a former robotics entrepreneur.

Neil Olivier, Director of Corporate Services at NuScale Power

<u>Education/Experience</u>: Mr. Neil Olivier, who has over 25 years of experience, began his career as a nuclear submarine mechanic in the U.S. Navy, and then went on to work as an operator of multiple commercial pressurized water and boiling water reactors. Mr. Olivier has an NRC reactor operator license at Columbia Nuclear Generating Station, an NRC senior reactor operator license at Limerick Nuclear Generating Station, a bachelor's degree in nuclear engineering technology, and a master's degree in business administration.

In his current position, Mr. Olivier leads the Document Control and Records Management Group, the Engineering Support Group, Facilities Management, and the Performance Improvement Group at NuScale Power. These groups administer multiple programs for compliance with Nuclear Quality Assurance1, including Engineering Design Control, Document Control and Records Management, and the Corrective Action Program. Mr. Olivier also currently heads NuScale's new PLM implementation, which will enable the digital twin and digital thread.

Pat Everett, Director of Thermal Engineering Oklo, Inc.

<u>Education/Experience</u>: Mr. Pat Everett holds a bachelor's degree in nuclear and mechanical engineering from the Massachusetts Institute of Technology. He leads the technical design of Oklo's advanced reactor systems and is an active developer of Oklo's advanced reactor analysis infrastructure. He led the safety analysis of the Aurora, as described in the Aurora combined license application to the NRC, which is the first and only non-LWR and microreactor combined license application submitted to and accepted for review by the NRC. He is actively supporting the NRC's technical review of the Aurora.

Gregory A. Banyay, Modeling and Simulation Hub Technical Lead (Principal Engineer)

<u>Education/Experience</u>: Dr. Gregory A. Banyay received a doctoral degree in civil and environmental engineering from the University of Pittsburgh in 2019; he also holds master's and

bachelor's degrees in mechanical engineering from Ohio University. Since 2010, he has worked at Westinghouse Electric Company as a computational mechanics analyst with an emphasis on flow induced vibration, acoustics, and probabilistic analysis. From 2006 to 2010, Dr. Banyay worked at Parker Hannifin as a design engineer for aerospace fuel pumps and pneumatic valves. Currently, Dr. Banyay focuses on the intersection of data driven and physics-based modeling for objectives related to structural health monitoring and PHM for nuclear power plants.

APPENDIX B WORKSHOP ATTENDEES

First Name	Last Name	E-Mail	Organization
Mohammad	Abdo	mohammad.abdo@inl.gov	INL
Andrea	Alfonsi	andrea.alfonsi@inl.gov	INL
Hany	Abdel-Khalik	abdelkhalik@purdue.edu	PURDUE
Abi	Ade	adeniyiai@ornl.gov	ORNL
Sunday	Aduloju	adulojusc@ornl.gov	ORNL
Vivek	Agarwal	vivek.agarwal@inl.gov	INL
Abderrahim	Al Mazouzi	abderrahim.al-mazouzi@edf.fr	EDF France
Ahmad	Al Rashdan	ahmad.alrashdan@inl.gov	INL
Tony	Alberti	anthony.alberti@oregonstate.edu	OREGON STATE
Lucas	Albright	lialbri@sandia.gov	SANDIA NATIONAL LABORATORIES
Tunc	Aldemir	aldemir.1@osu.edu	OSU
Rasool	Anooshehpoor	rasool.anooshehpoor@nrc.gov	NRC
Todd	Anselmi	todd.anselmi@inl.gov	INL
Abhinav	Anup	aanup@sgh.com	SIMPSON GUMPERTZ & HEGER
Robert	Armstrong	robert.armstrong@inl.gov	INL
John	Atchison	jatchison@islinc.com	PENN STATE NUCLEAR ENGINEERING
Paridhi	Athe	pathe@ncsu.edu	NCSU
Ronke	Ayo-Imoru	ronmonicks@yahoo.com	NIGERIA ATOMIC ENERGY COMMISSION
Vittorio	Badalassi	badalassiv@ornl.gov	ORNL
Emilio	Baglietto	emiliob@mit.edu	MIT
Stephen	Bajorek	stephen.bajorek@nrc.gov	NRC
Han	Bao	han.bao@inl.gov	INL
Jonathan	Barr	jonathan.barr@nrc.gov	NRC
Sergiu	Basturescu	sergiu.basturescu@nrc.gov	NRC
Phill	Bates	phillip.bates@mottmac.com	MOTT MACDONALD
Rob	Beason	robert.beason@inl.gov	INL
Geoffrey	Beausoleil	geoffrey.beausoleil@inl.gov	INL
Randy	Belles	bellesrj@ornl.gov	ORNL
Michael	Beran	michael.beran@inl.gov	INL
Joe	Berti	joseph.berti@ibm.com	IBM
Benjamin	Betzler	betzlerbr@ornl.gov	ORNL
Bruce	Bevard	bevardbb@ornl.gov	ORNL
Ryan	Bills	ryan.bills@inl.gov	INL
Tammy	Bloomer	tamara.bloomer@nrc.gov	NRC
R. Stuart	Bondurant	richard.bondurant@inl.gov	INL
Cody	Boriboun	cody.boriboun@inl.gov	INL
Jeffrey	Borkowski	jeffrey.borkowski@studsvik.com	STUDSVIK
Jeremy	Bowen	jeremy.bowen@nrc.gov	NRC

First Name	Last Name	E-Mail	Organization
Thomas	Braudt	tom.braudt@gmail.com	NRC
Robert	Braun	rbraun@arcnuclear.com	ARC NUCLEAR
Yvotte	Brits	ybrits@x-energy.com	X-ENERGY
Hayden	Brundage	hayden.brundage@nrc.gov	NRC
Mark	Buller	mark_buller@alumni.brown.edu	BROWN UNIVERSITY
Troy	Burnett	troy.burnett@inl.gov	INL
Jonathon	Burstein	jdburste@bechtel.com	BECHTEL
Jeremy	Busby	busbyjt@ornl.gov	ORNL
Stephanie	Bush-Goddard	spb@nrc.gov	NRC
Rodney	Busquim e Silva	r.busquim@iaea.org	IAEA
Xavier	Bussenault	xavier.bussenault@3ds.com	3DS
Scott	Bussey	scott.bussey@nrc.gov	NRC
Pattrick	Calderoni	pattrick.calderoni@inl.gov	INL
Mike	Calley	michael.calley@inl.gov	INL
Brent	Capell	bcapell@epri.com	EPRI
Lane	Carasik	lbcarasik@vcu.edu	VCU
Michael	Case	michael.case@nrc.gov	NRC
Sacit	Cetiner	<u>cetinerms@ornl.gov</u>	ORNL
David	Chandler	<u>chandlerd@ornl.gov</u>	ORNL
Hasan	Charkas	hcharkas@epri.com	EPRI
Stylianos	Chatzidakis	<u>chatzidakiss@ornl.gov</u>	ORNL
Yifeng	Che	yfche@mit.edu	MIT
Edward	Chen	echen2@ncsu.edu	NCSU
Yiren	Chen	yiren_chen@anl.gov	ANL
Minghui	Chen	mnu@unm.edu	UNM
Hangbok	Choi	hangbok.choi@ga.com	GA
Robby	Christian	robby.christian@inl.gov	INL
Pong	Chung	pong.chung@nrc.gov	NRC
Christopher	Chwasz	christopher.chwasz@inl.gov	INL
Anthonie	Cilliers	cilliers@kairospower.com	KAIROS POWER
Sheldon	Clark	sheldon.clark@nrc.gov	NRC
Alyson	Coates	coatesal@ornl.gov	ORNL
Jamie	Coble	jamie@utk.edu	UTK
Stephanie	Coffin	stephanie.coffin@nrc.gov	NRC
Thad	Cole	thad@ams-corp.com	AMS CORP
Robert	Cox	robert.cox@uncc.edu	UNCC
Joseph	Cristiano	joseph.cristiano@nrc.gov	NRC
Gordon	Curran	gordon.curran@nrc.gov	NRC
Jason	D'Haene	jasondd@gmail.com	MSU
Dayna	Daubaras	dayna.daubaras@inl.gov	INL
Eva	Davidson	davidsonee@ornl.gov	ORNL
lan	Davis	idavis@x-energy.com	X-ENERGY
Cynthia	DeBisschop	cynthia.debisschop@oasissystems.com	OASIS SYSTEMS
Marc Olivier	Delchini	delchinimg@ornl.gov	ORNL
Gregory	Delipei	gkdelipe@ncsu.edu	NCSU

First Name	Last Name	E-Mail	Organization
David	Desaulniers	david.desaulniers@nrc.gov	NRC
Kevin	Deyette	kdeyette@nuscalepower.com	NUSCALE POWER
Mihai	Diaconeasa	madiacon@ncsu.edu	NCSU
Mark	Dietrich	mark.dietrich@3ds.com	3DS
Nam	Dinh	ntdinh@ncsu.edu	NCSU
Bernard	Dittman	bernard.dittman@nrc.gov	NRC
Abdul	Dulloo	abdul.dulloo@inl.gov	INL
Ray	Duthu	ray.duthu@qs-2.com	QS-2
Kimberlee	Edwards	kimberlee.edwards@nrc.gov	NRC
Shannon	Eggers	shannon.eggers@inl.gov	INL
János	Eiler	eilerj1@outlook.com	IAEA
Doug	Eskins	doug.eskins@nrc.gov	NRC
Thomas	Evans	evanstm@ornl.gov	ORNL
Patrick	Everett	pat@oklo.com	OKLO
Julie	Ezell	julie.ezell@nrc.gov	NRC
Nathan	Faith	nathan.faith@exeloncorp.com	EXELON CORP
Mario	Fernandez	mario.fernandez@nrc.gov	NRC
Emerson	Ferreira	ferreira.eam@gmail.com	FEDERAL U OF BAHIA
William	Ferrell	will@ams-corp.com	AMS CORP
Kurt	Fielding	kurt.fielding@inl.gov	INL
Leo	Fifield	leo.fifield@pnnl.gov	PNNL
Claudio	Filippone	claudio@holosgen.com	HOLOSGEN
Ashley	Finan	ashley.finan@inl.gov	INL
James D.	Freels	freelsjd@gmail.com	ORNL/U OF TENNESSEE
Konor	Frick	konor.frick@inl.gov	INL
Steve	Frost	steve.frost@onr.gov.uk	ONR
Raymond	Furstenau	raymond.furstenau@nrc.gov	NRC
Marisa	Garcia Heras	mheras@tecnatom.es	TECNATOM
Elvira	Gayton Gallardo	elvira.gaytan@inin.gob.mx	ININ
Alejandro	Galeano	agaleano@cuatrocubos.com	CUATRO CUBOS
Humberto	Garcia	humberto.garcia@inl.gov	INL
Ramon	Gascot	ramon.gascot@nrc.gov	NRC
Fred	Gelbard	fgelbar@comcast.net	SANDIA
Cole	Gentry	gentryca@ornl.gov	ORNL
Sandra	Geupel	s.geupel@iaea.org	IAEA
Anders	Gilbertson	anders.gilbertson@nrc.gov	NRC
Paul	Gilbreath	paul.gilbreath@inl.gov	INL
Chester	Gingrich	chester.gingrich@nrc.gov	NRC
Kellen	Giraud	kellen.giraud@inl.gov	INL
Robert	Gladney	robert.gladney@nrc.gov	NRC
Brian	Golchert	golchebm@westinghouse.com	WESTINGHOUSE
Russ	Gold	russell.gold@inl.gov	INL
Les	Goldberg	les.goldberg@3ds.com	3DS
Nolan	Goth	gothne@ornl.gov	ORNL
Brian	Green	brian.green@nrc.gov	NRC

First Name	Last Name	E-Mail	Organization
Scott	Greenwood	greenwoodms@ornl.gov	ORNL
Andrei	Gribok	andrei.gribok@inl.gov	INL
Michael	Grieves	mgrieves@fit.edu	FIT
Cody	Griffith	cody.griffith@hii-nns.com	HII-NNS
Robert	Grove	grovere@ornl.gov	ORNL
Sofia	Guerra	aslg@adelard.com	ADELARD
Donna	Guillen	donna.guillen@inl.gov	INL
Anil	Gurgen	agurgen@ncsu.edu	NCSU
Izabela	Gutowska	gutowski@oregonstate.edu	OREGON STATE
Alison	Hahn	alison.hahn@nuclear.energy.gov	DOE
Mostafa	Hamza	mmhamza@ncsu.edu	NCSU
Botros	Hanna	bn@nmsu.edu	NMSU
Alex	Hashemian	alex@ams-corp.com	AMS CORP
Hash	Hashemian	hash@ams-corp.com	AMS CORP
Alfred	Hathaway	alfred.hathaway@nrc.gov	NRC
Jeffery	Hawkins	jeffhawkins@jhawkconsulting.com	JHAWK CONSULTING
Robert	Hayes	rbhayes@ncsu.edu	NCSU
Michelle	Hayes	mwh2@nrc.gov	NRC
Ross	Hays	ross.hays@gmail.com	
Stephen	Heagy	steveheagy@fpolisolutions.com	FPOLISOLUTIONS
Eric	Helm	eric.helm@framatome.com	FRAMATOME
David	Henderson	david.henderson@nuclear.energy.gov	DOE
Raul	Hernandez	raul.hernandez@nrc.gov	NRC
Michael	Hilliard	hilliardmr@ornl.gov	ORNL
Bob	Hirmanpour	bhirmanpour@nuscalepower.com	NUSCALE POWER
Faramarz	Hojati	f hojati iust@yahoo.com	
Kim	Holloway	kim.holloway@nrc.gov	NRC
Lindsey	Holmes	lindsey.m.holmes@ama-inc.com	AMA INC
Philip	Honnold	phonnol@sandia.gov	SANDIA
Jacob	Houser	jakeh@ams-corp.com	AMS CORP
Rui	Hu	<u>rhu@anl.gov</u>	ANL
Nathanael	Hudson	nathanael.hudson@nrc.gov	NRC
Clyde	Huibregtse	huibregc@oklo.com	OKLO
Allison	Hummel	allison.hummel@inl.gov	INL
Alexander	Huning	huninghj@ornl.gov	ORNL
Chris	Hussey	chussey@gafcon.com	GAFCON
Eric	Isaacson	eisaacso@bechtel.com	BECHTEL
Eman	Ibrahim	eman.ibrahim@canada.ca	CANADIAN NUCLEAR SAFETY COMMISSION
Mesfin Seid	Ibrahim	mesfin.ibrahim@connect.polyu.hk	POLYU
Daniel	Iglesias	daniel.iglesias@iter.org	ITER
Daniel	lliescu	daniel.iliescu@ansys.com	ANSYS
Raj	lyengar	raj.iyengar@nrc.gov	NRC
Prashant	Jain	jainpk@ornl.gov	ORNL
Juris	Jauntirans	juris.jauntirans@nrc.gov	NRC

First Name	Last Name	E-Mail	Organization
Stuart	Jensen	stuart.jensen@inl.gov	INL
Colby	Jensen	colby.jensen@inl.gov	INL
Yeongshin	Jeong	jeongys@mit.edu	MIT
Bob	Jewart	robert.jewart@inl.gov	INL
Jing	Jiang	jjiang@eng.uwo.ca	UNIVERSITY OF WESTERN ONTARIO
Yue	Jin	yuejin@mit.edu	MIT
William	Johns	william.johns@nrc.gov	NRC
Kelly	Johnson	kelly.johnson@inl.gov	INL
Justin	Johnson	justin.johnson@inl.gov	INL
Robby	Joseph	josephra@ornl.gov	ORNL
Branko	Jovanovski	b.jovanovski@iaea.org	IAEA
Daniel	Ju	daniel.ju@nrc.gov	NRC
Chul Hwan	Jung	chulhwan.jung@canada.ca	CANADIAN NUCLEAR SAFETY COMMISSION
Joshua	Kaizer	joshua.kaizer@nrc.gov	NRC
Hyun	Kang	kangh6@rpi.edu	RPI
Min-Tsung	Kao	kaom@ornl.gov	ORNL
Sri Rameswar Pramod	Kasturi	pramodkasturi@gmail.com	U OF FLORIDA
Maxine	Keefe	maxine.segarnick@nrc.gov	NRC
Robert	Kepler	r.kepler@inl.gov	INL
Timothy	Kernicky	tkernick@uncc.edu	UNCC
Rajiv	Khadka	rajiv.khadka@inl.gov	INL
Anya	Kim	anya.kim@nrc.gov	NRC
Eric	Knox	pugnaxx@gmail.com	ICANN
Brendan	Kochunas	bkochuna@umich.edu	UMICH
Alan	Konkal	alan.konkal@nrc.gov	NRC
Robert	Krawczak	krawczrk@westinghouse.com	WESTINGHOUSE
David	Kropaczek	kropaczekdj@ornl.gov	ORNL
Vineet	Kumar	kumarv@ornl.gov	ORNL
Meimei	Li	mli@anl.gov	ANL
Chris	Lamb	cclamb@sandia.gov	SANDIA
Hilary	Lane	hml@nei.org	NEI
Jeffrey	Lane	lanejw@zachrynuclear.com	ZACHRY NUCLEAR
Daniel	Lau	dllau@uky.edu	UKY
Svetlana	Lawrence	svetlana.lawrence@inl.gov	INL
Kyoung	Lee	leeko@ornl.gov	ORNL
John	Lee	jcl@umich.edu	UMICH
Roger	Lew	rogerlew@uidaho.edu	UIDAHO
Ruixuan	Li	ruixuan.li@inl.gov	INL
Jun	Liao	liaoj@westinghouse.com	WESTINGHOUSE
Linyu	Lin	llin7@ncsu.edu	NCSU
Bruce	Lin	bruce.lin@nrc.gov	NRC
Yuxuan	Liu	yuxuanl@umich.edu	UMICH
Yong Chang	Liu	yongchang.liu@canada.ca	CANADIAN NUCLEAR SAFETY COMMISSION

First Name	Last Name	E-Mail	Organization
Yang	Liu	yang.liu@anl.gov	ANL
Don	Lore	lored@westinghouse.com	WESTINGHOUSE
Leonard	Lucas	leonard.lucas@unnpp.gov	UNNPP
Louise	Lund	louise.lund@nrc.gov	NRC
Nancy	Lybeck	nancy.lybeck@inl.gov	INL
Susana	López	slopez@tecnatom.es	TECNATOM
Shah	Malik	shah.malik@nrc.gov	NRC
Natasha	Mallette	natasha.mallette@oregonstate.edu	OREGON STATE
Diego	Mandelli	diego.mandelli@inl.gov	INL
Annalisa	Manera	manera@umich.edu	UMICH
Panagiotis	Manolatos	panagiotis.manolatos@ec.europa.eu	EURATOM
Ricardo	Marques	rpm@lac.usp.br	U OF SAO PAULO
Timothy	Marshall	timothy.marshall@oasissystems.com	OASIS SYSTEMS
Nicolas	Martin	nicolas.martin@inl.gov	INL
Paul	Martyak	pmartyak@epri.com	EPRI
Chandler	Maskal	<u>chandler.maskal@ibm.com</u>	IBM
Michael	Mazzola	mmazzola@uncc.edu	UNCC
Bruce	McDowell	bruce.mcdowell@pnnl.gov	PNNL
Michael	McGregor	michael.mcgregor@inl.gov	INL
Murray	Medlock	mmedlock@southernco.com	SOUTHERN CO
Craig	Miller	craig.miller@ansys.com	ANSYS
Samuel	Miller	samiller@terrapower.com	TERRAPOWER
Wayne	Мое	wayne.moe@inl.gov	INL
Subhasish	Mohanty	smohanty@anl.gov	ANL
Carol	Moyer	carol.moyer@nrc.gov	NRC
Mike	Muhlheim	muhlheimmd@ornl.gov	ORNL
Lynn	Munday	lynn.munday@inl.gov	INL
Curt	Nehrkorn	nehrkorn_curt@bah.com	ВАН
Scott	Nelson	nelsonsw@ornl.gov	ORNL
Jinsuo	Nie	jinsuo.nie@nrc.gov	NRC
Matt	O'Connor	mcoconnor@epri.com	EPRI
Daniel	O'Grady	dogrady@anl.gov	ANL
Rebecca	Onuschak	rebecca.onuschak@nuclear.energy.gov	DOE
Courtney	Otani	courtney.otani@inl.gov	INL
Chad	Painter	chad.painter@pnnl.gov	PNNL
Scott	Palmtag	sppalmta@ncsu.edu	NCSU
Charity	Pantalo	charity.pantalo@nrc.gov	NRC
Vincent	Paquit	paquitvc@ornl.gov	ORNL
Japan	Patel	jakpatel@umich.edu	UMICH
Tyler	Pawley	tyler.pawley@siemens.com	SIEMENS
Eternity	Perry	eternity@ams-corp.com	AMS CORP
Angelica	Petrovic	angelica.petrovic@inl.gov	INL
Loc	Pham	locpham@nmsu.edu	NMSU
Birdy	Phathanapirom	birdy@ornl.gov	ORNL
Jeffrey	Phillips	jeffrey.phillips@inl.gov	INL

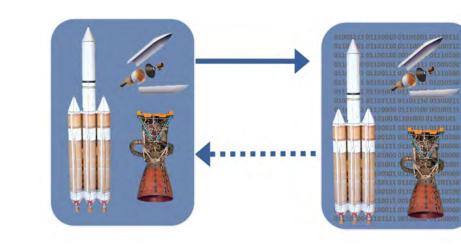
First Name	Last Name	E-Mail	Organization
Brandon	Pinson	brandon.pinson@nrc.gov	NRC
Stephanie	Pitts	stephanie.pitts@inl.gov	INL
Robert	Podgorney	robert.podgorney@inl.gov	INL
Mike	Poore	poorewpiii@ornl.gov	ORNL
Steven	Pope	spope@islinc.com	ISLINC
Steven	Prescott	steven.prescott@inl.gov	INL
Craig	Primer	craig.primer@inl.gov	INL
Dimitrios	Pylorof	dimitrios.parsinaspylorof@inl.gov	INL
Jean	Ragusa	jean.ragusa@tamu.edu	TAMU
Pradeep	Ramuhalli	ramuhallip@ornl.gov	ORNL
Samantha	Rawlins	samantha.b.rawlins@ama-inc.com	AMA INC
Paul	Rebstock	paul.rebstock@nrc.gov	NRC
Wendy	Reed	wendy.reed@nrc.gov	NRC
Mehdi	Reisi Fard	mehdi.reisifard@nrc.gov	NRC
Gustavo	Reyes	gustavo.reyes@inl.gov	INL
Chris	Ritter	christopher.ritter@inl.gov	INL
Andy	Rivas	arivas2@ncsu.edu	NCSU
Robert	Roche-Rivera	robert.roche-rivera@nrc.gov	NRC
Bill	Rosko	william.rosko@rolls-royce.com	ROLLS-ROYCE
Michael	Russell	russellms@ornl.gov	ORNL
Amanda	Rynes	amanda.rynes@inl.gov	INL
Frederick	Sock	frederick.sock@nrc.gov	NRC
Masoumeh	Salimi	masoumeh.salimi@edu.turkuamk.fi	TURKUAMK
Michele	Sampson	michele.sampson@nrc.gov	NRC
Erica	Sanchez	erica.sanchez@inl.gov	INL
Abhinav	Saxena	asaxena@ge.com	GE
Andreas	Scheibe	andreas.scheibe@inl.gov	INL
Suibel	Schuppner	suibel.schuppner@nuclear.energy.gov	DOE
Dogan	Seber	dogan.seber@nrc.gov	NRC
Jenifer	Shafer	jenifer.shafer@hq.doe.gov	DOE
Sam	Sham	ssham@anl.gov	ANL
Мо	Shams	mohamed.shams@nrc.gov	NRC
Koroush	Shirvan	kshirvan@mit.edu	MIT
Prakash	Shrivastava	prakash.shrivastava@utdallas.edu	UT DALLAS
Brent	Shumaker	bshumaker@ams-corp.com	AMS CORP
Pawel	Siembab	pawel.siembab@3ds.com	3DS
Sara	Smith	sara.smith@inl.gov	INL
Curtis	Smith	curtis.smith@inl.gov	INL
Michael	Spencer	michael.spencer@nrc.gov	NRC
Chris	Spirito	christopher.spirito@inl.gov	INL
Shawn	St. Germain	shawn.stgermain@inl.gov	INL
John	Stairmand	john.stairmand@jacobs.com	JACOBS
lan	Stevenson	ian.stevenson@3ds.com	3DS
Ryan	Stewart	ryan.stewart@inl.gov	INL
Xiaodong	Sun	xdsun@umich.edu	UMICH

First Name	Last Name	E-Mail	Organization
Daniel	Sweeney	sweeneydc@ornl.gov	ORNL
Rob	Sweeney	rob.sweeney@ibexesi.com	IBEX ESI
Dylan	Sylvester	dylan.e.sylvester@xcelenergy.com	XCEL-ENERGY
Emre	Tatli	tatlie@westinghouse.com	WESTINGHOUSE
Tina	Taylor	ttaylor2@epri.com	EPRI
Ken	Thomas	kenneth.thomas@inl.gov	INL
Dennis	Thompson	dennis.1101@outlook.com	OH-TECH
Kirk	Tien	kirk.tien@nrc.gov	NRC
Tim	Trask	timctrask@gmail.com	
Rob	Tregoning	robert.tregoning@nrc.gov	NRC
Shawn	Tyler	shawn@ams-corp.com	AMS CORP
Rizwan	Uddin	rizwan@illinois.edu	UIUC
Thomas	Ulrich	thomas.ulrich@inl.gov	INL
Troy	Unruh	troy.unruh@inl.gov	INL
Athi	Varuttamaseni	avarutta@bnl.gov	BNL
Justin	Vazquez	justin.vazquez@nrc.gov	NRC
Andrea	Veil	andrea.veil@nrc.gov	NRC
Felipe	Viana	viana@ucf.edu	UCF
Richard	Vilim	rvilim@anl.gov	ANL
Susan	Vrahoretis	susan.vrahoretis@nrc.gov	NRC
Cody	Walker	cody.walker@inl.gov	INL
Shakur	Walker	shakur.walker@nrc.gov	NRC
Jay	Wallace	jay.wallace@nrc.gov	NRC
Congjian	Wang	congjian.wang@inl.gov	INL
Jun	Wang	jwang564@wisc.edu	UNIVERSITY OF WISCONSIN-MADISON
Hong	Wang	wangh6@ornl.gov	ORNL
Justin	Weinmeister	weinmeistejr@ornl.gov	ORNL
Robert	Weisman	robert.weisman@nrc.gov	NRC
Eric	Whiting	eric.whiting@inl.gov	INL
Dan	Widrevitz	dxw7@nrc.gov	NRC
Katherine	Wilsdon	wilskat7@isu.edu	ISU
Nathan	Wiltbank	wiltbann@oregonstate.edu	OREGON STATE
Brian	Wittick	brian.wittick@nrc.gov	NRC
Oscar	Wiygul	owiygul@gmail.com	WEBA
David	Womble	womblede@ornl.gov	ORNL
Emma	Wong	ewong@epri.com	EPRI
Andrew	Worrall	worralla@ornl.gov	ORNL
Xu	Wu	xwu27@ncsu.edu	NCSU
Zhiwen	Xu	zhiwen.xu@inl.gov	INL
Vaibhav	Yadav	vaibhav.yadav@inl.gov	INL
Xingyue	Yang	xingyue.yang@inl.gov	INL
SuJong	Yoon	sujong.yoon@inl.gov	INL
Bob	Youngblood	robert.youngblood@inl.gov	INL
Jianguo	Yu	jianguo.yu@inl.gov	INL

First Name	Last Name	E-Mail	Organization
Akos	Zakar	zakar.akos@paks2.hu	PAKS II
Kai	Zhang	zhnkai@gmail.com	SAUDER
Sai	Zhang	sai.zhang@inl.gov	INL
Hongbin	Zhang	hongbin.zhang@inl.gov	INL
Fan	Zhang	fan@utk.edu	UTK
Xingang	Zhao	zhaox2@ornl.gov	ORNL
Jack	Zhao	jack.zhao@nrc.gov	NRC
Dirk	Cairns-Gallimore	dirk.cairns-gallimore@nuclear.energy.gov	DOE
Kevin	Chen	pec9@pitt.edu	PITT
Lon	Dawson	ladawso@sandia.gov	SANDIA
Fleurdeliza	dePeralta	fleurdeliza.deperalta@pnnl.gov	PNNL
Michael	Fluss	mjfluss@comcast.net	ANS
Adakou	Foli	adakou.foli@nrc.gov	NRC
Daniel	Luke	daniel.b.luke@hii-nns.com	HII-NNS
Andrea	Nicolas	anicolas@anl.gov	ANL
Sheila	Ray	sheila.ray@nrc.gov	NRC
Walter	Schwarz	walter.schwarz@ansys.com	ANSYS
Mark	Thaggard	mark.thaggard@nrc.gov	NRC
Rattehalli	Vijay	rattehalli.vijay@unnpp.gov	UNNPP
Chris	Adolfson	chris.adolfson@inl.gov	INL
Dave	Anthony	david.anthony@ucalgary.ca	UCALGARY
Jay	Appanam Karakkad	appanamkaraj@ornl.gov	ORNL
Daowei	Ві	bidaowei@snerdi.com.cn	SNERDI
Eric	Benner	eric.benner@nrc.gov	NRC
Nicoleta	Bocaneala	nicolebocaneala@gmail.com	BCU UK
Ronald	Boring	ronald.boring@inl.gov	INL
Yung Hsien	Chang	james.chang@nrc.gov	NRC
Travis	Chapman	tchapman@x-energy.com	X-ENERGY
Michael	Cheok	michael.cheok@nrc.gov	NRC
Helene	Chini	chinih@westinghouse.com	WESTINGHOUSE
Joey	Church	wch@3ds.com	3DS
Kevin	Clarno	<u>clarno@utexas.edu</u>	UTEXAS
Rachael	Collins	rachael.a.collins@ama-inc.com	AMA INC
Jason	D'Haene	jason.dhaene@unnpp.gov	UNNPP
Greg	Davidson	davidsongg@ornl.gov	ORNL
Som	Dhulipala	som.dhulipala@inl.gov	INL
Elvis	Domínguez	dominguezoee@ornl.gov	ORNL
Janos	Eiler	j.eiler@iaea.org	IAEA
Patrick	Ellis	patricke@ams-corp.com	AMS CORP
Austin	Fleming	austin.fleming@inl.gov	INL
William	Freebairn	william.freebairn@spglobal.com	S&P GLOBAL
Anne	Gaffney	anne.gaffney@inl.gov	INL
Tyler	Gavin	tyler@ams-corp.com	AMS CORP
Askin	Guler Yigitoglu	<u>yigitoglua@ornl.gov</u>	ORNL

First Name	Last Name	E-Mail	Organization
Steven	Hamilton	hamiltonsp@ornl.gov	ORNL
Mike	Hancock	mike.hancock@ansys.com	ANSYS
Brooks	Holland	brooks.holland@inl.gov	INL
Hao	Huang	hao.huang1@ge.com	GE
Ronaldo	Jenkins	ronaldo.jenkins@nrc.gov	NRC
Yuchih	Ко	yko@terrapower.com	TERRAPOWER
Joseph	Kanney	joseph.kanney@nrc.gov	NRC
Anil	Kapahi	akapahi@jensenhughes.com	JENSEN HUGHES
Adam	Kecskes	kecskesa@paks2.hu	PAKS II
Jim	Kinsey	jim.kinsey@inl.gov	INL
Albrecht	Kyrieleis	albrecht.kyrieleis1@jacobs.com	JACOBS
Matthew	LeVasseur	mplevasseur@bwxt.com	BWXT
David	Luxat	dlluxat@sandia.gov	SANDIA
Andrea	Mack	andrea.mack@inl.gov	INL
Charles	Martin	chiprmartin@gmail.com	ANS-NEVADA
Caleb	Massey	masseycp@ornl.gov	ORNL
Gurcharan	Matharu	gurcharan.matharu@nrc.gov	NRC
Marsha	McDaniel	marsha.mcdaniel@inl.gov	INL
Tim	McJunkin	timothy.mcjunkin@inl.gov	INL
Thanh	Nguyen	thanhnh@nmsu.edu	NMSU
Joshua	Peterson-Droog h	joshua.petersondroogh@inl.gov	INL
Ed	Pheil	e.pheil@elysium-v.com	ELYSIUM-V
Dave	Pointer	pointerwd@ornl.gov	ORNL
Bayan	Rafeh	bayan.rafeh@framatome.com	FRAMATOME
Fayaz	Rasheed	rasheedf@ornl.gov	ORNL
Andrea	Rovinelli	arovinelli@anl.gov	ANL
Sergey	Sadakov	sergey.sadakov@iter.org	ITER
Ali	Sharif	ali.sharif@pulseconsult.co.uk	PULSE CONSULT
Laura	Smith	laura.smith@nrc.gov	NRC
Minseop	Song	minseop.song@inl.gov	INL
Theresa	Sutter	tsutter@curtisswright.com	CURTISS-WRIGHT
Zeechung	Wang	zeechung.wang@nrc.gov	NRC
Guanyi	Wang	guanyi.wang@anl.gov	ANL
Bradley	Williams	bradley_williams@epw.senate.gov	EPW
Dale	Yeilding	dale.yeilding@nrc.gov	NRC
Candace	de Messieres	candace.demessieres@nrc.gov	NRC
Yanjun	Wang	13801256033@139.com	

APPENDIX C PRESENTATION SLIDES



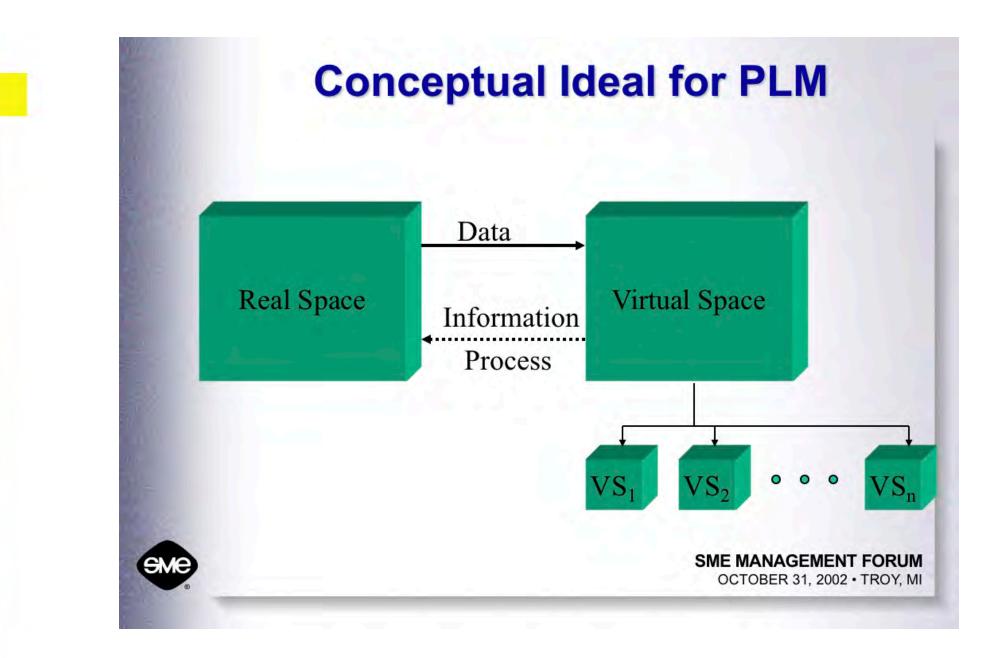
Nuclear Digital Twins

Dr. Michael Grieves

Chief Scientist of Advanced Manufacturing

Florida Institute of Technology

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FLÓRIDA TECH

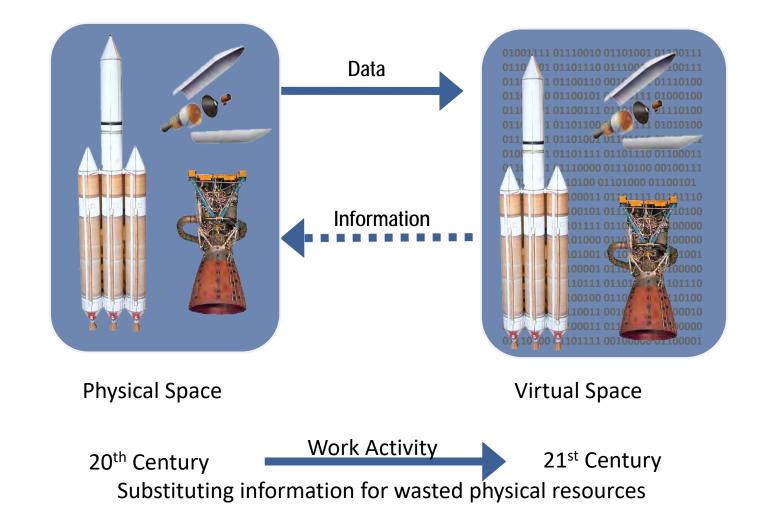
PRODUCT LIFECYCLE MANAGEMENT

> DIMAGNIC NACE ANNALSING DI LLAR NAMENC

Virtually Perfect

the second lines, so the second

Digital Twin Model



FLORIDA LECH PRODUCT LIFECYCLE MANAGEMENT DESCRIPTION CONTACT CONTACT VICTUALING VICTUALING VICTUALING VICTUALING

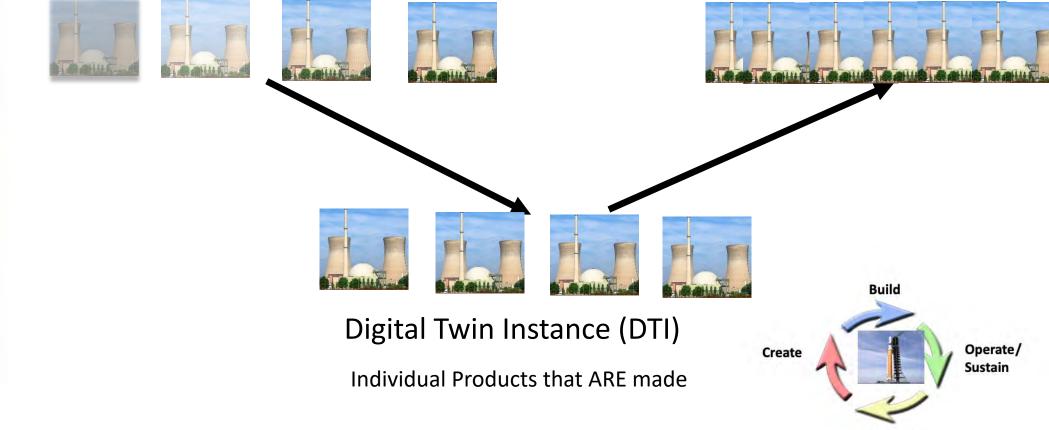
Digital Twin Types (DT)

Digital Twin Prototype (DTP)

All Products that CAN BE made

Digital Twin Aggregate (DTA) All Products that HAVE BEEN made

Dispose



FC

PRODUCT LIFECYCLE

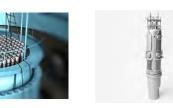
> Virtually Perfect



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Physical Twin (PT)



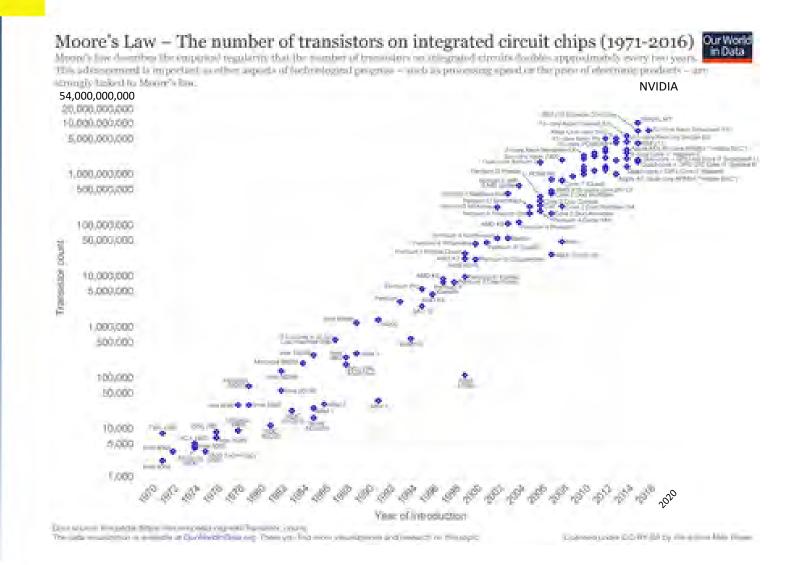




- Smart
- Sensing
- Translating
- Comparing
- Reacting
- Smart, Connected
 - Communicating, Assessing, Response
 - Protecting

Grieves, M., *Virtually Intelligent Product Systems: Digital and Physical Twins*, in *Complex Systems Engineering: Theory and Practice*, S. Flumerfelt, et al., Editors. 2019, American Institute of Aeronautics and Astronautics. p. 175-200.

Computing Capability Enabling M&S and AI



2040 – 885T - 16,000x Increase 2030 – 7T - 128x Increase

EC

PRODUCT

LIFECYCLE

MANAGEMENT

Virtually Perfect



Modeling and Simulation M&S

- Increase in granularity
- Increase in fidelity
- Increase in cohesion (integration)
- Improved physics modeling

Artificial Intelligence (AI)

- On a spectrum Expert Systems to Singularity
- Intelligence Goal seeking while minimizing scarce resources
- Dramatically different hardware
- Human assistance (cued availability) vs. human replacement



PRODUC



Digital Twin vs. Intelligent Digital Twin

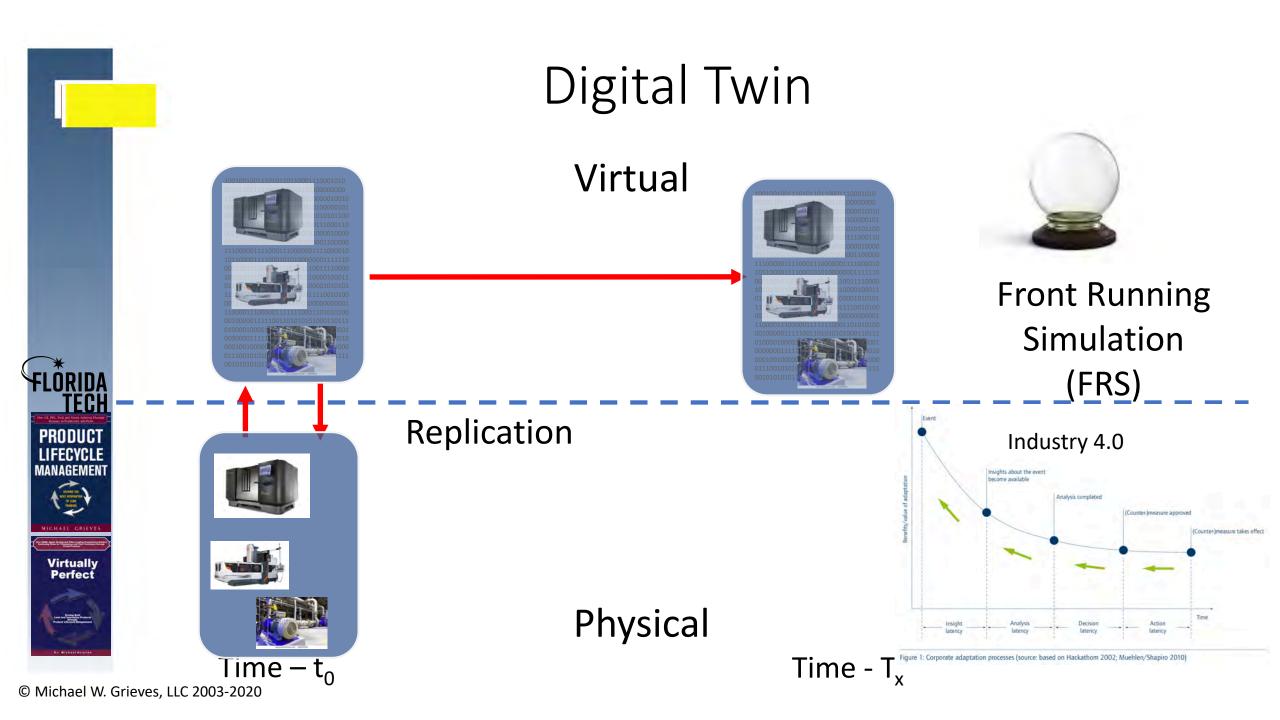
Digital Twin (DT)

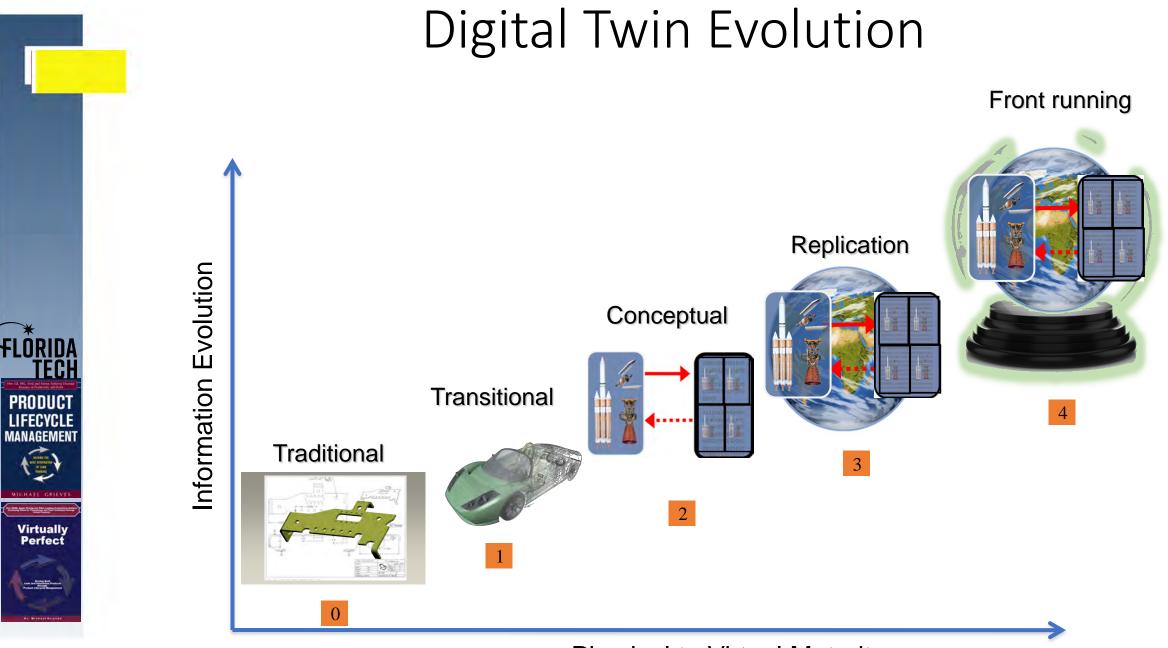
- Passive
- Offline
- Goal given
- Predictive

Intelligent Digital Twin (IDT)

- Active
- Online
- Goal Seeking/Minimizing Resources
- Anticipatory

Grieves, M., Intelligent Digital Twins and the Development and Management of Complex Systems, Forthcoming, 2021.

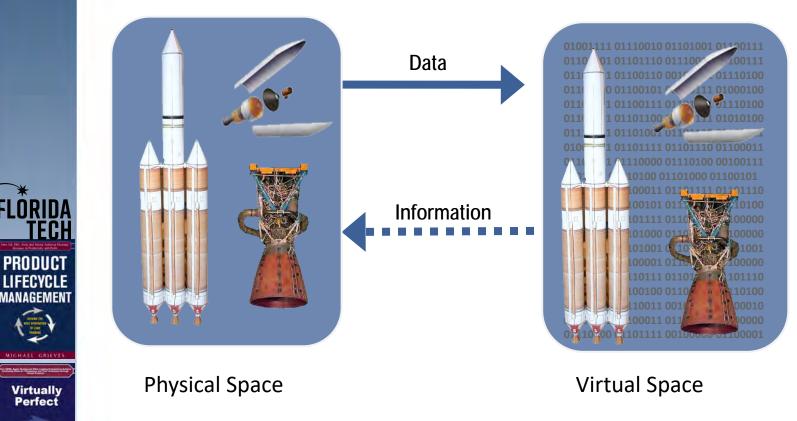




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Physical to Virtual Maturity

Intelligent Digital Twin (IDT) ML/AI



- Capture lessons learned
- Cued Availability
- Sensemaking
- FRS Multi-Scenario
- Decision Assistance / Making

Digital Twin Impediments

Technology

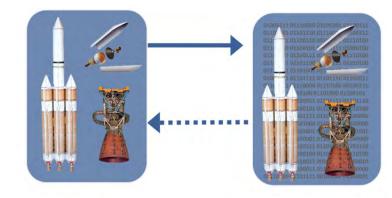
- Hardware
- Software
- Physics
- Risk vs. Uncertainty
- Cultural inertia

TECH

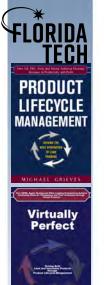
PRODUCT

LIFECYCLE

Virtually Perfect



Dr. Michael Grieves <u>mgrieves@mwgvp.com</u> <u>mgrieves@fit.edu</u>



IBM Digital Twin

December 1, 2020

Joe Berti VP of Offerings Management, AI Applications

Breaking Through To True Operational Success

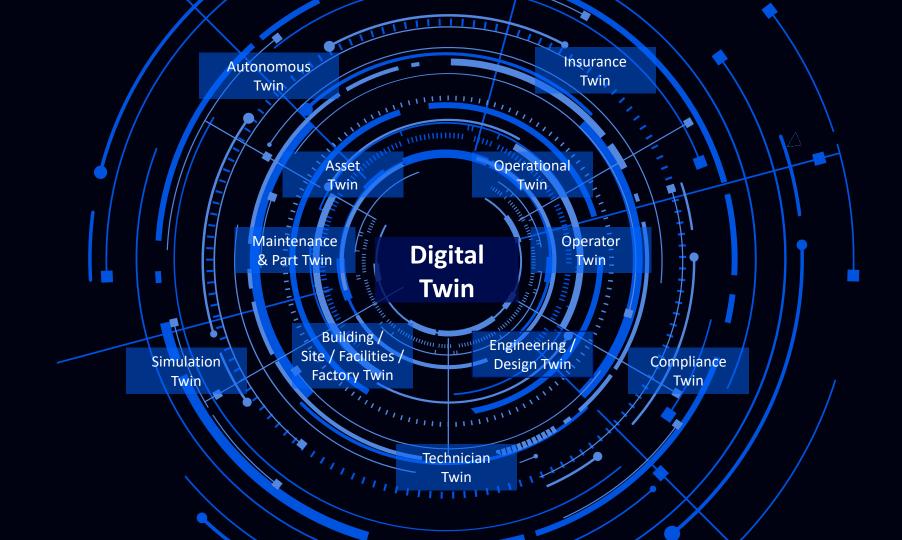
The time and need for *going digital* is more pressing as the move to AI occurs. Going digital has become a *top strategic goal* but getting there is *a daunting challenge*, especially for asset-intensive industries struggling with fundamental data and business process issues that are not designed for the digital age.

IBM digital twin immediately provides a complete digital solution for operational performance: *lack of data* and *business practices* that are *counter-productive* to going digital. IBM provides a process for *managing* digital asset content and *shortening the timeframe* to be *digitally enabled* quickly yielding *significant operational returns*.

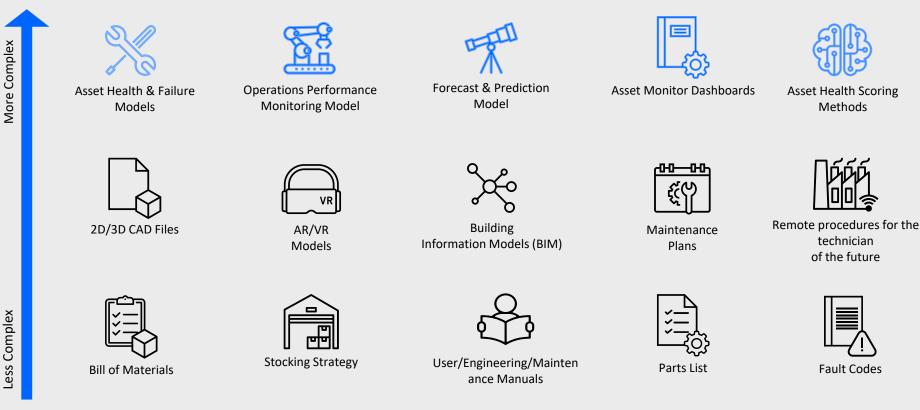
IBM digital twin strives to be a "no brainer" investment that is an end to end consulting, software and digital content solution. IBM provides a compelling alternative to the more common approaches: numerous tactical solutions and projects scattered across the organization. In todays business environment, boil the ocean strategies are a non starter. IBM is especially attractive for eliminating the alternative: "throwing money and people at the problem."

dig·i·tal twin

A digital representation of a physical thing. Combined with IoT, digital twins come alive, evolving into a living virtual model that mimics the experiences of it's physical twin.

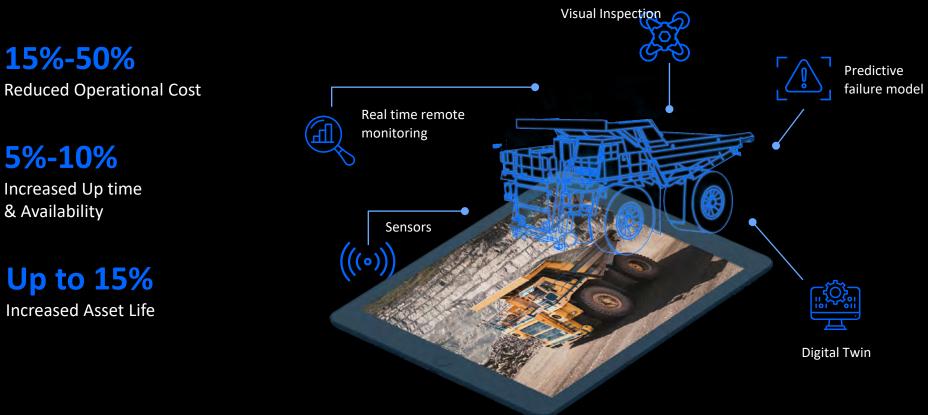


Most Common Digital Twin Content

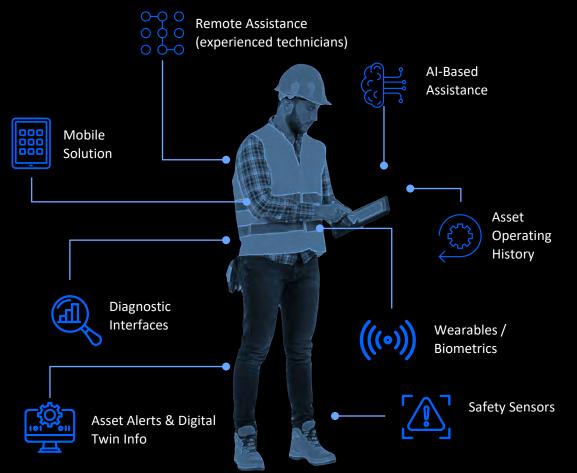


The operating model in asset intensive industries is changing

IoT is no longer a novelty



The operating model change also applies to technicians



Port of Rotterdam



Information

12 hours

earlier means **134.000** tons of CO² reduction (4%) Reducing waiting times can result in <u>additional</u>

188.000 tons of CO² reduction

Average household produces 4 tons of CO² per year

42 km long 12,686 ha of which 64% is land

\$60M

Annual maintenance budget (USD)

#1

Netherlands has the best quality of port infrastructure in the world Trimming **one hour** of berthing time for a single vessel results in

\$80,000/ hr

70,000 sea-going ship movements (in- or out of Port) per year. One-way trip from Singapore to Rotterdam costs

\$5-6M (USD) in fuel cost. Assume average fuel cost per ship movement to be \$2M (USD). Based on this assumption a 4% fuel cost savings for ships using Port of Rotterdam results in a total

\$5,6 Bn (USD)

savings per year

That is why their clients would want to choose Port of Rotterdam over their competitors.

Day in the life of ... A containervessel's captain

Departure Singapore, planned for 35 days

Full speed ahead! (but **burning** fuel)





Surprise 2: **Tides** and **traffic** so **not allowed** to berth

Angry customer







Surprise 3: **Broken bollard**, needs to be fixed first

Surprise 1: Bad weather!

Surprise 4: **Broken traffic sign** was not replaced, **detour** or **accident** also causing delay







Everything fixed, but tired captain returns **late, tired and having burned too much fuel** along the way.

Day in the life in... the Digital Twin 4.0

How is this different?

Environment integrated with external data sources



Intelligent workflows and process optimization

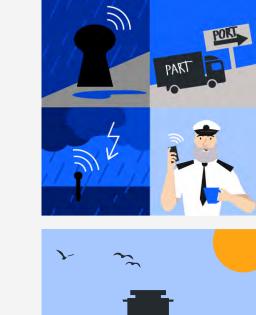
Autonomous behavior of Smart connected objects (or Assets)

0 1 0 1 1 0 1 0 0 1 0 1 1 0 1 0

Integrating legacy IT and unlocking true value of (more) data



The operating model in asset intensive industries is changing





Digital Twin 4.0 From Prototype to MVP

Based on this Digital Twin vision we just described the Port of Rotterdam resulted in a phase I prototype: Building the foundation SmartInfra IoT platform and first use case HydroMeteo.

HydroMeteo: Realtime data processing of sensor and predicted weather and water data, with anomaly detection to support harbour traffic management. With 70.000 sea-going ship movements a year and a cost of \$80,000 per hour per ship, optimizing the traffic has a huge impact on Rotterdam's competitive position.

http://port-of-rotterdam.eu-gb.mybluemix.net/



How the Port of Rotterdam is using IBM digital twin technology to transform itself from the biggest to the smartest

Business problem

- The Port of Rotterdam covers an area of over 41 square miles, employs close to 400,000 people, and serves nearly 30,000 sea-going vessels each year
- Needed to operate optimally within the Global Hub and within Europe's Industrial Cluster and become the world's first digital port

Solution

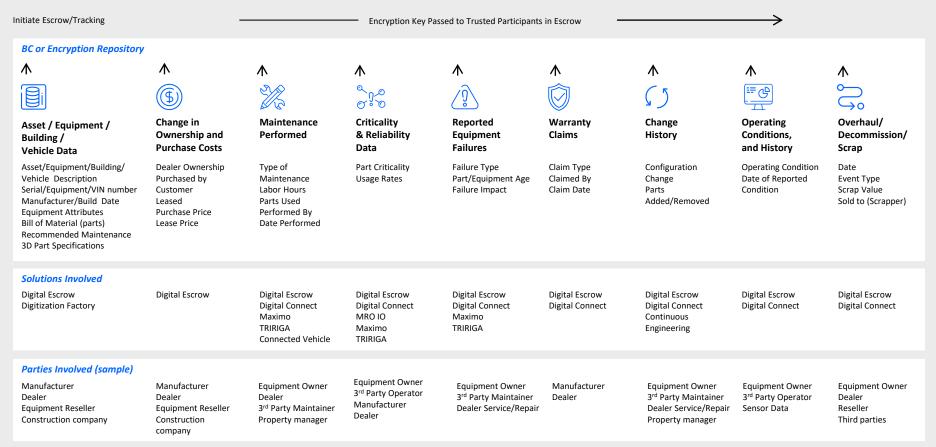
 Sensors throughout the dock facility to continuously gather real-time data about air temperature, wind speed, (relative) humidity, turbidity and salinity of the water, water flow and levels, tides and currents, smart quay walls, and sensorequipped buoys

Business Value Delivered

- Reduce waiting times and costs by predicting more accurately what the best time is to moor and depart
- Real-time access to information enables the Rotterdam Port Authority to better predict visibility and water conditions, lower fuel consumption rates, facilitate cost-effective per-ship payloads and ensure the safe arrival of cargo
- Shipping companies and the port may save up to one hour in berthing time, and enable more ships to pass through the port each day by using a digital dashboard for operations



The Solution: Strategic Digital Management







ORNL Resources to Support Digital Twin Applications for Nuclear Systems

Nuclear Energy and Fuel Cycle Division

Jeremy Busby and Prashant Jain December 1, 2020

ORNL is managed by UT-Battelle, LLC for the US Department of Energy



The historic beginnings of ORNL



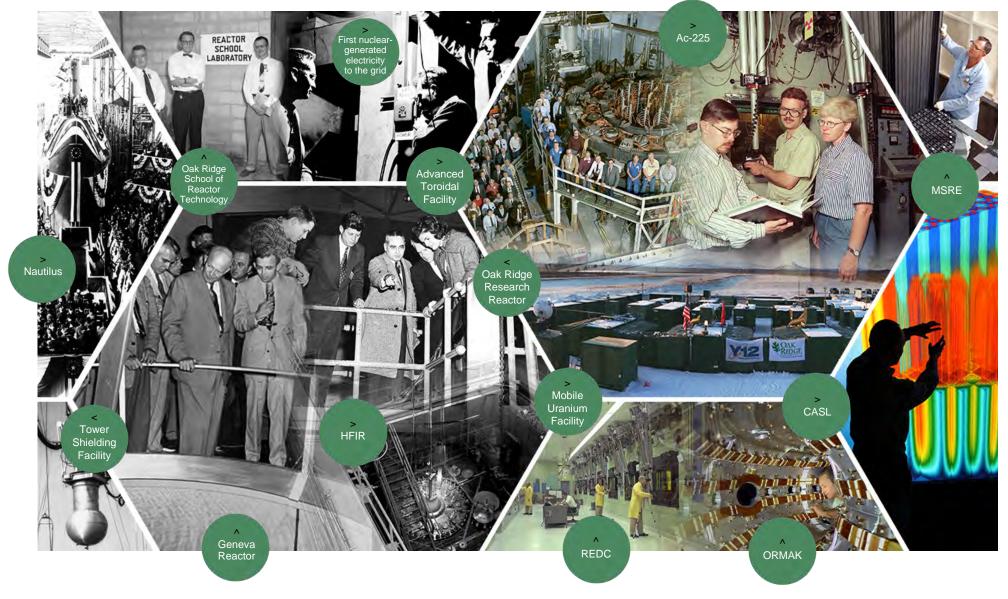


The Graphite Reactor: A solid foundation and true legacy



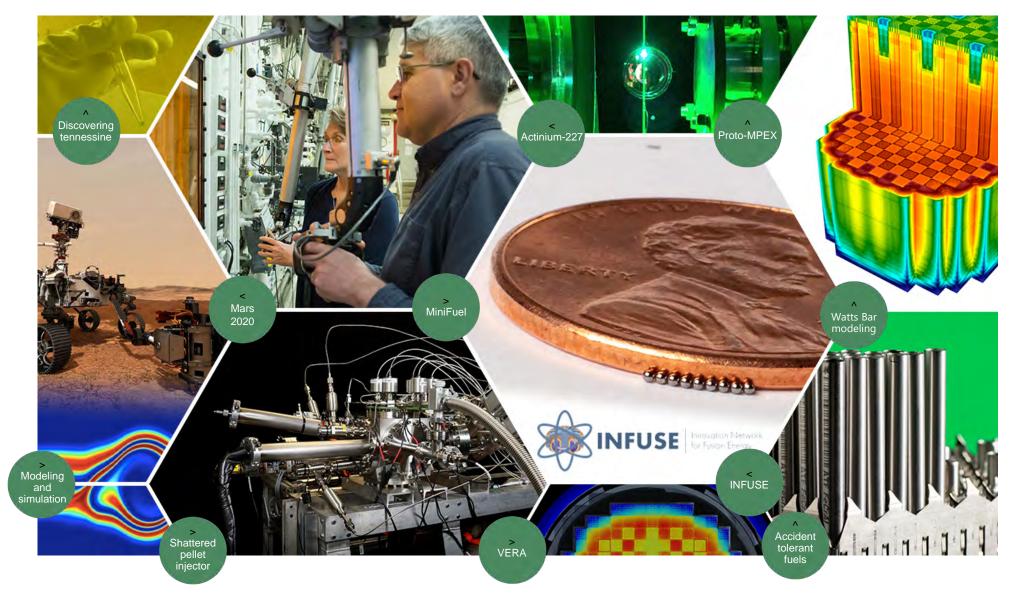


Expanding nuclear impact through the decades





Today's innovation in nuclear science and technology

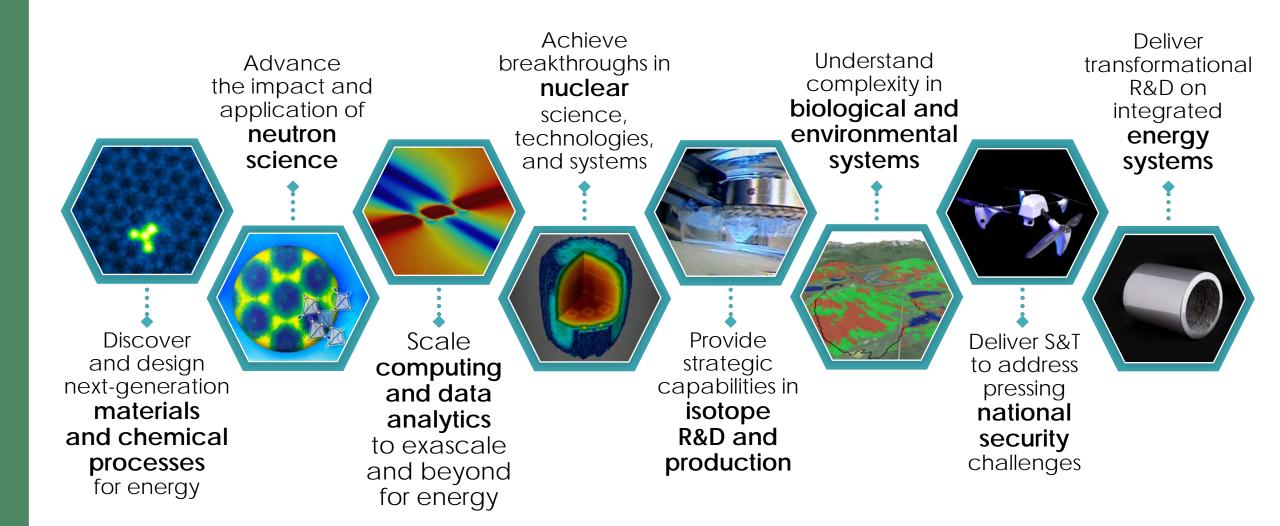








ORNL's major science and technology initiatives





The Transformational Challenge Reactor (TCR) Program Demonstrating a new approach to design and deployment

- Apply additive manufacturing technologies that enable rapid prototyping and geometric control to nuclear core design
 - Core design is driven by manufacturing
 - Demonstration with reactor deployment and successful operation is centric to showing the value of this approach

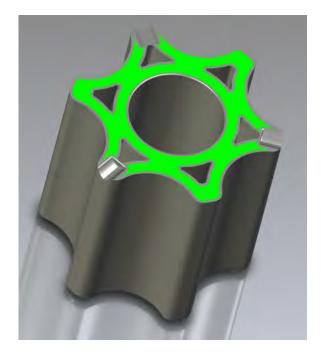
https://tcr.ornl.gov/



TCR is harnessing advances in manufacturing, materials, and computational sciences to enable advanced reactors

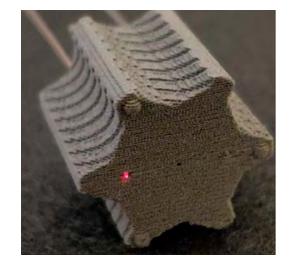
Design for advanced manufacturing Advanced highperformance Materials Distributed sensing towards autonomy

On the fly certification of critical components



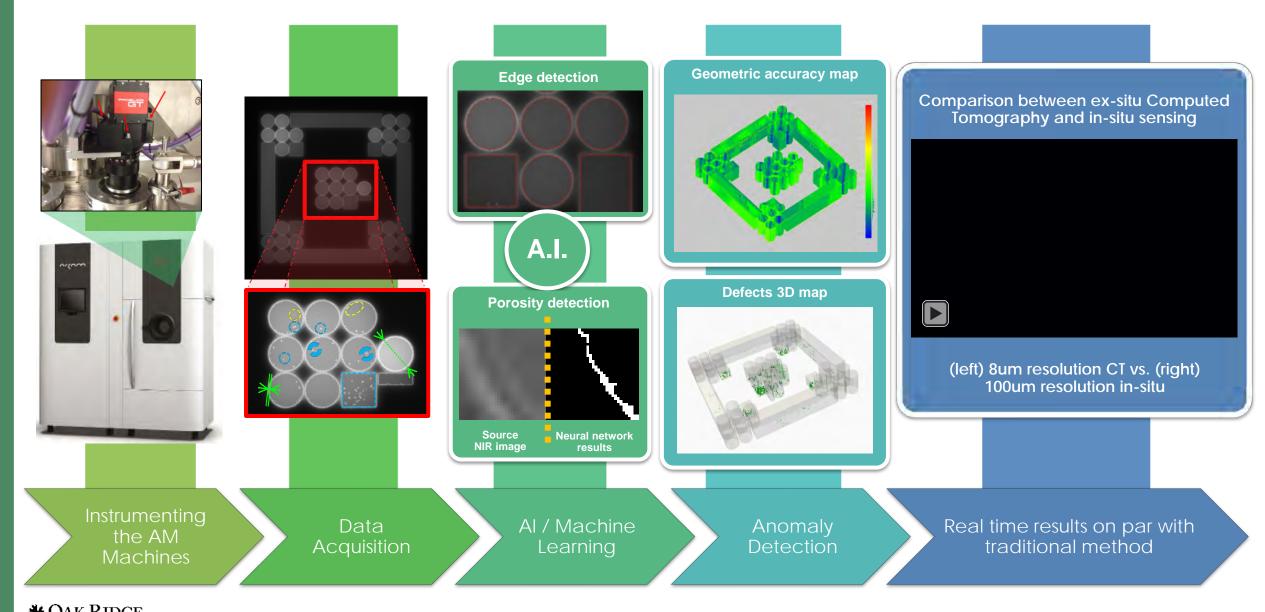
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TCR Developed "In-Situ" Quality Control of AM Processes using AI



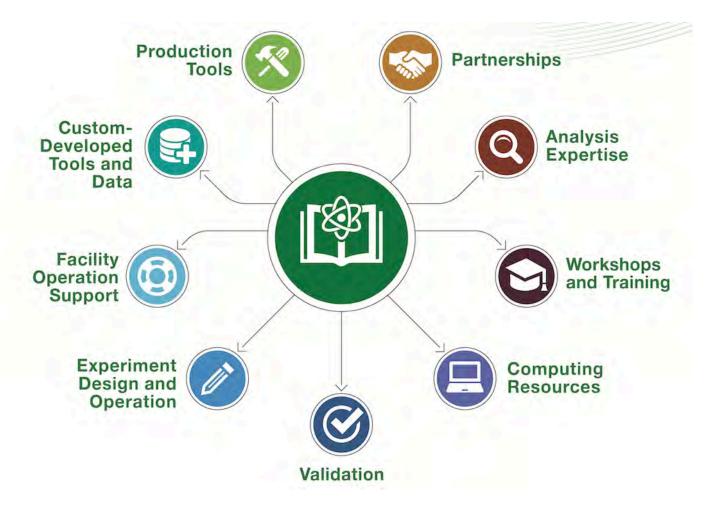
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11

Developing Digital Twins for Nuclear Can Build On Expertise in many areas

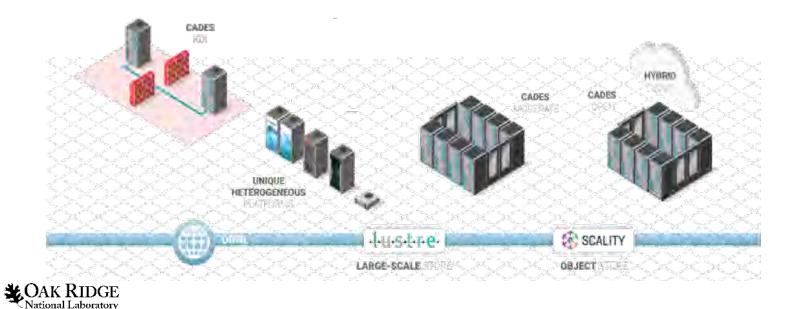
- ORNL has spent decades working in a number of key areas for nuclear applications
 - Fission energy systems to sustain the existing fleet and accelerate deployment of advanced reactors;
 - Fusion energy systems to improve understanding of plasma physics and accelerate facility design and safety assessment;
 - Isotopes and neutron sciences to accelerate design and improve efficiencies; and
 - Nuclear security solutions to support detection of proliferation support application of safeguards, and determine provenance of nuclear threats.

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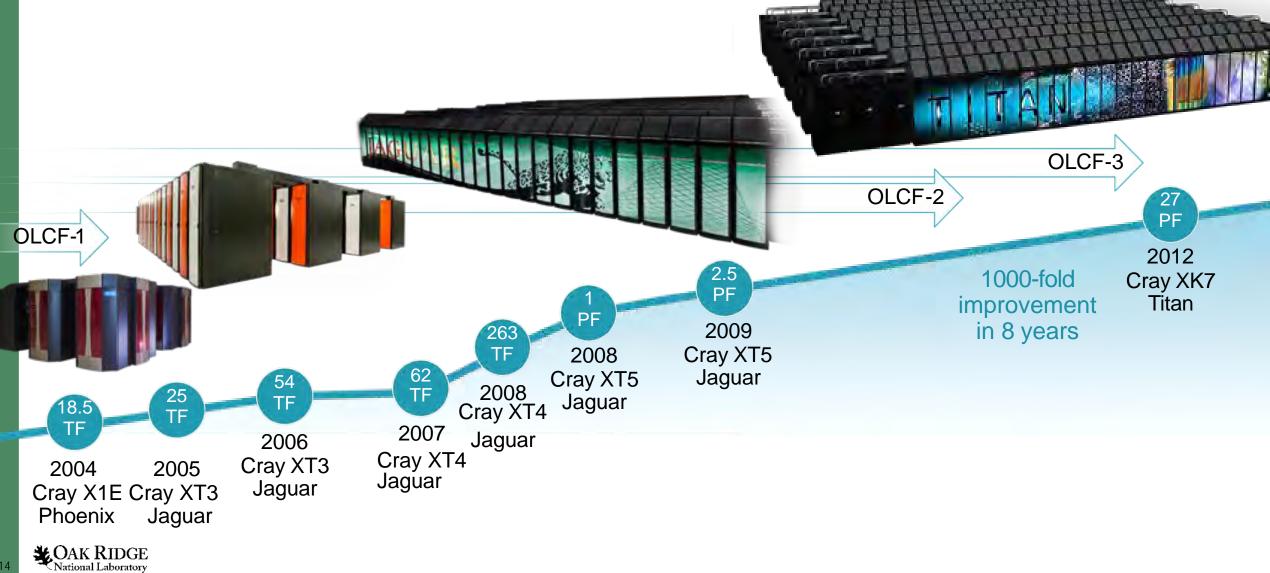
- Commodity clusters for production calculations
- Secure cloud computing
- Compute and Data Environment for Science (CADES)







ORNL has been pushing the limits for leadership computing



We are building on this record of success to enable exascale in 2021



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15



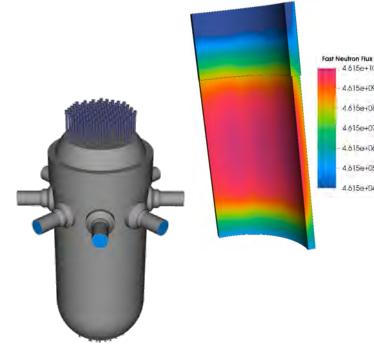
Production Tools

Tools distributed under license from RSICC

- SCALE Code System
- CASL VERA
- CTF
- ADVANTG
- AMPX
- SAMMY
- RPS-DET

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16





4.615e+10

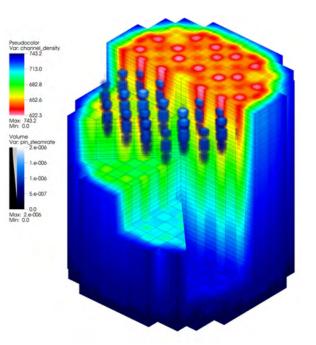
4.615e+09

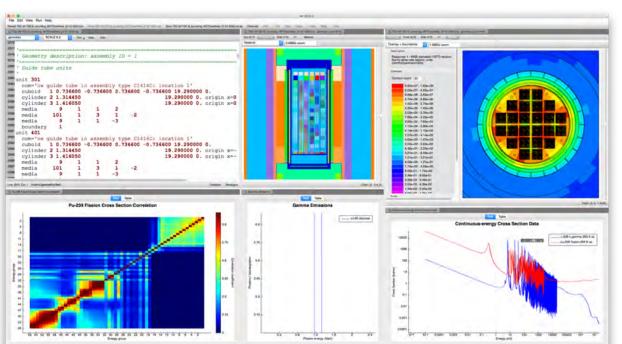
4.6150+08

4.615e+07

4.615e+06 4.615e+05

4.6150+04





CASL VERA – End User Product

Virtual Environment for Reactor Applications

• High Resolution:

- Fully coupled and pin-resolved neutronic, T/H, and crud growth physics
- Detailed rod-wise fuel performance analyses

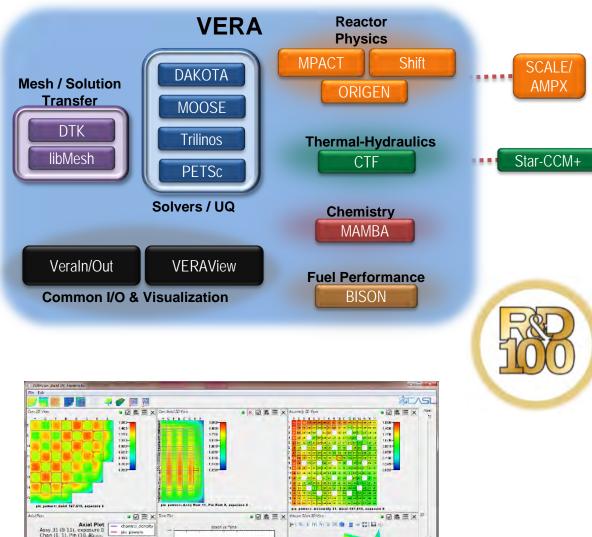
Integrated Applications:

- Modeling in-core and ex-core detector prediction of axial offset anomaly(AOA) due to CRUD deposition
- Identification of Pellet Clad Interaction (PCI) failure risk during load follow operation with Accident Tolerant Fuels
- Accumulation of vessel fluence damage over the reactor operating history
- Prediction of cladding integrity during reactivityinitiated transients
- Performance & Usability:

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- User-friendly I/O (e.g. automated mesh generation and data transfers)
- Integrated visualization tools

https://www.casl.gov/



0.6 0.8 1.0 1.2 1.4 1.6

660 680 700

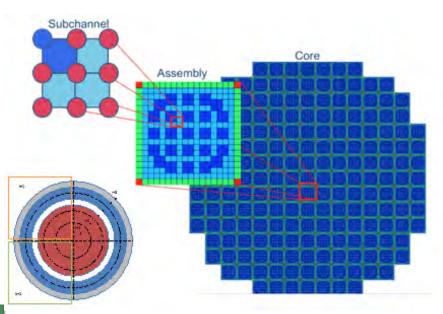
VeraView

CTF: Whole-Core Thermal Hydraulics Subchannel Code

- Two-fluid, three-field representation of the two-phase flow
 - Continuous vapor (mass, momentum and energy)
 - Continuous liquid (mass, momentum and energy)
 - Entrained liquid drops (mass and momentum)
 - Non-condensable gas mixture (mass)
- Cross flow between channels, Spacer grid models
- Internal pin conduction with dynamic gap model
- Parallel Solution = ~5 secs per solve

CTF enables in-line T/H analyses and direct local feedback to neutronics

https://www.ornl.gov/onramp/ctf

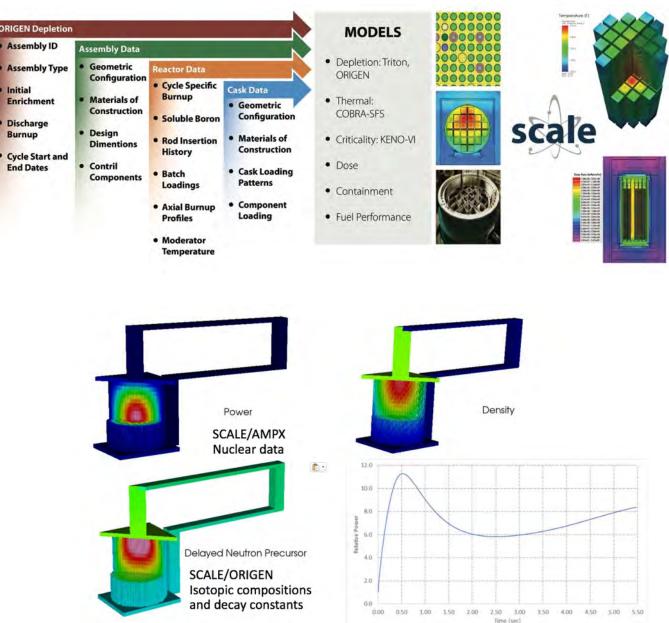


Custom-Developed Tools and Data

Initial

Burnup

- UNF-ST&DARDS
- NEAMS Workbench
- TRANSFORM
- SCALE and VERA Enhancements for Advanced Nuclear Energy Systems
- Fusion Plasma Physics
- Nuclear Security
- Exascale Computing Project
- Software Development and **Testing Environment**





19

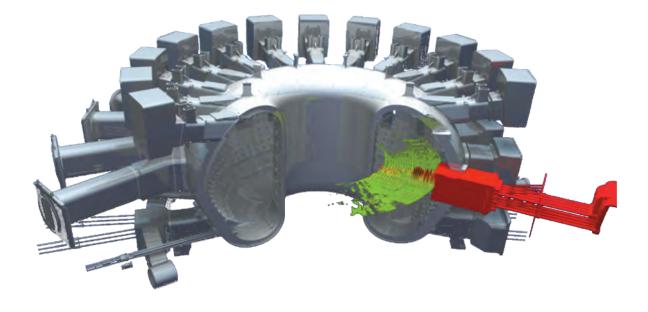


Analysis Expertise

- Nuclear data
- Radiation shielding and criticality safety
- Reactor neutronics
- Thermal hydraulics
- Nuclear materials
- Thermomechanics and fuels
- Reactor core analysis
- Accident analysis
- Fusion plasma physics
- Nuclear safeguards and security
- Fuel cycle analysis
- High-performance computing



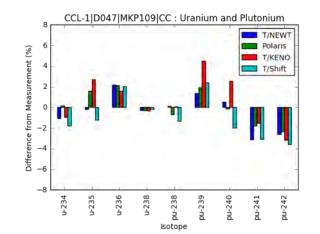


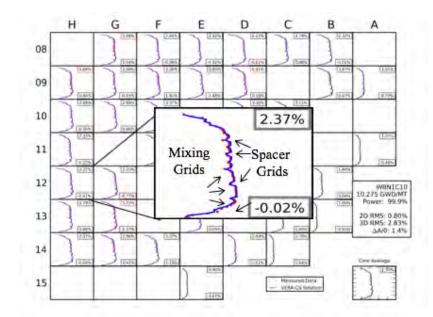






- Validation is the characterization of the suitability of a selected mathematical model and data to correctly predict and describe real-world physical phenomena. This places validation at the intersection of modeling, simulation, and experimental methods.
- ORNL has broad validation experience, including the collection and qualification of experimental validation data, leadership in international benchmark handbooks, development of problem-specific validation criteria, as well as comparison and visualization of large data sets.





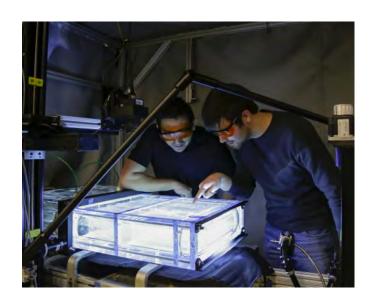


21



Experiment Design and Operation

- Irradiation experiments
- Thermal hydraulics experiments
- Nuclear data measurements
- Nuclear criticality experiments
- Fusion experimental capabilities

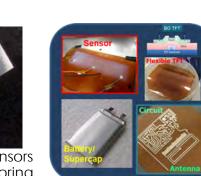






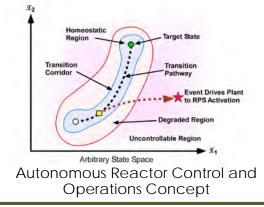
Instrumentation and Control: Capabilities

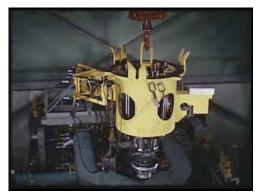
Measurements and Sensors



High Temperature Compatible and Embedded Sensors for Nuclear Process and Component Health Monitoring

Advanced Controls



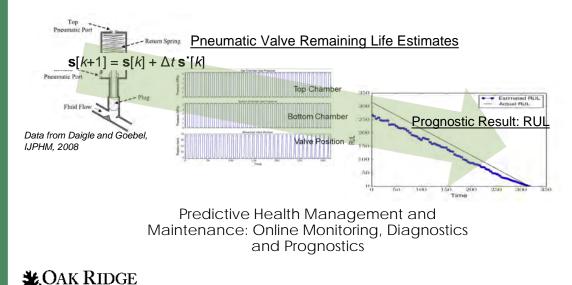


Remote Maintenance

Data Analytics and Computing

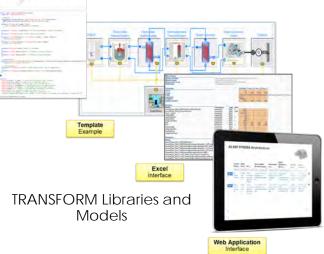
23

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Models and Testbeds

MODELICA

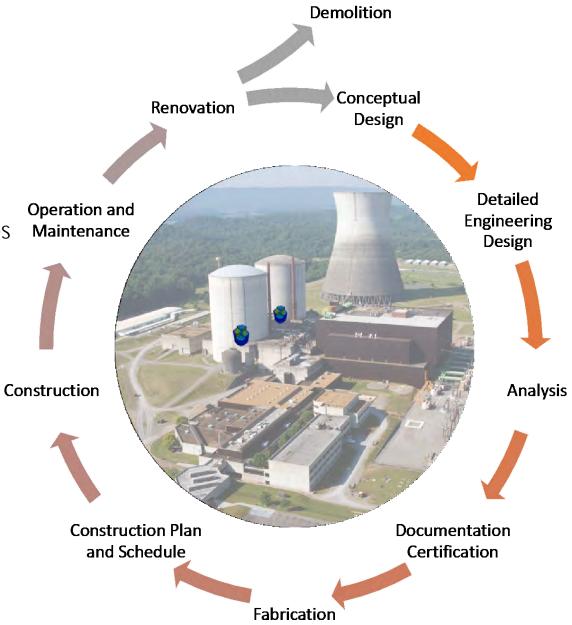




Molten Salt Flow Calibration Facility (Under Development)

Diverse ORNL Technologies Can Enable Advanced Reactor Deployment

- High fidelity core simulations enabling powerful digital twins
- Model based system engineering approach to plant design
- Consideration of integrated energy system
- Siting database integration
- Optimization of operation and maintenance cycles
- Wholistic consideration of safety and security
- Building information model approach to construction cycle optimization
- Testing to grow confidence in advanced component performance
- New understanding of fundamental materials
- Deployment of new sensors and measurement methods
- Al enabled autonomous control
- Licensing

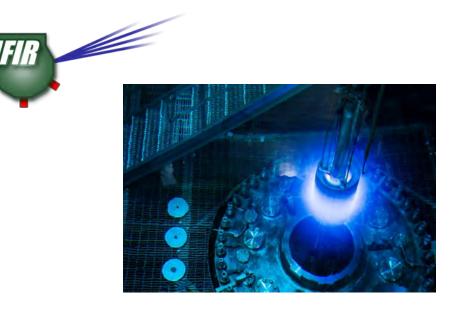






Facility Operation Support

- Safe, reliable operation of a nuclear facility requires facilityspecific modeling and analysis tools and expertise.
- Nuclear modeling and analysis tools are used for many facility needs, including but not limited to establishing, maintaining, and implementing the safety basis; redesigning components; designing, optimizing, and qualifying experiments; and upgrading instruments.









ORNL's High Flux Isotope Reactor can Provide a Test Bed to Mature Digital Twin Technology

- Located at Oak Ridge National Laboratory
- Achieved criticality / full power in 1965 / 1966
 - Has decades of remaining life
- Supports several high-impact scientific missions
 - World class capabilities serving a variety of national missions
 - Neutron scattering, isotope production, materials irradiation, ...
- Beryllium-reflected flux trap-type research reactor
 - 85 MW nominal full power (1.7 MW/L average power density)
 - 2.5x10¹⁵ n/cm²/s peak thermal neutron flux
 - 23-26-day-long fuel cycles

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- Downflowing light-water cooled and moderated
 - Primary coolant flow of 16,000 GPM
 - Inlet temperature / pressure of 120 °F / 468 psig
 - Outlet temperature / pressure of 156 °F / 358 psig



https://neutrons.ornl.gov/hfir



Workshops and Training

- ORNL workshops attract the global community to a venue that encourages innovation and excitement in a place where so much nuclear history has evolved. Recent meeting series that are especially applicable to ONRAMP include:
- Recurring workshops:
 - CASL/VERA Users Group (annual)
 - MSR Workshop (annual)
 - SCALE Users Group (annual)
- Possible future topics may include, but are not limited to:
 - CFD community of practice (broader than nuclear)
 - Experimental facilities, validation, and experiment design
 - HPC in nuclear applications
 - Hybrid variance reduction techniques
 - Modeling and simulation for advanced reactors
 - Modeling and simulation for fusion plasma physics and neutronics
 - Modeling and simulation for neutron science facilities
 - Modeling and simulation for research reactors
 - Nuclear data needs and opportunities
 - Nuclear security modeling
 - Sensitivity analysis and uncertainty quantification







Workshops and Training

- Training is provided by developers and expert users from our teams to address a variety
 of technical areas including but not limited to the following:
 - Nuclear criticality safety
 - LWR and advanced reactor modeling
 - Reactor safety analysis;
 - Radiation shielding
 - Spent nuclear fuel characterization for transportation/storage package designs, decommissioning and disposal
 - Verification, validation, and uncertainty quantification
 - Nuclear safeguards and security applications
 - Nuclear data processing and libraries generation
- New courses in FY19/20
 - Nuclear Data Fundamentals and AMPX Libraries Generation
 - VERA

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- ADVANTG
- Training presented at ORNL, NEA (Paris), NRC headquarters, utilities, nuclear suppliers, or R&D organizations.
- Additionally, many related workshops are presented at conferences and universities.



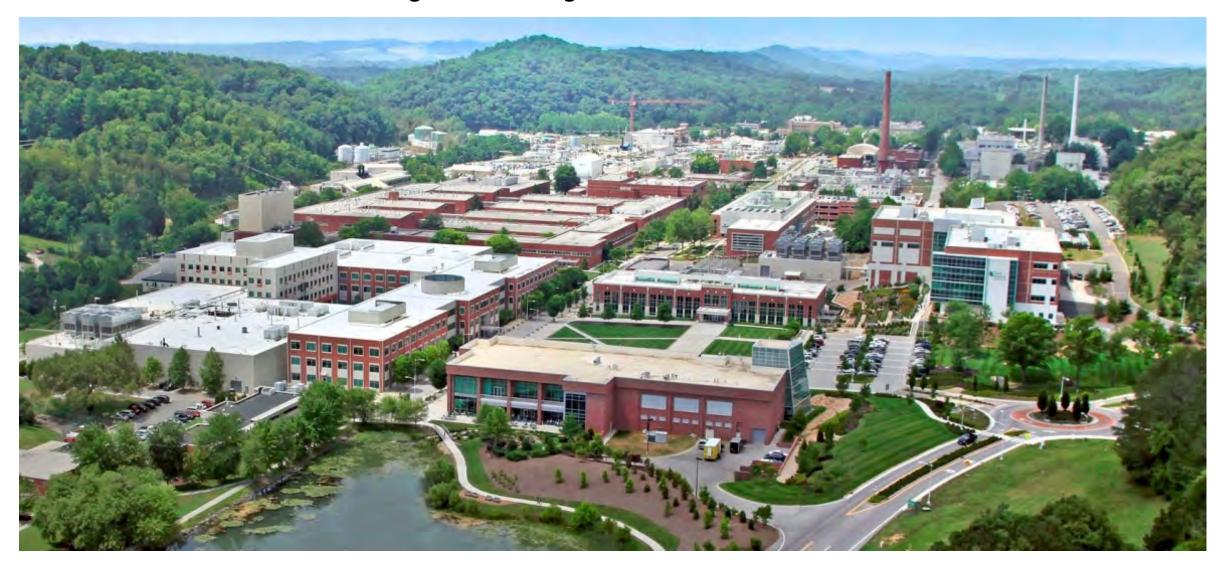


Partnerships

- Inform:
 - Participate in workshops and symposia to present current approaches and find out what is possible using ORNL's advanced techniques and computing. Tour ORNL facilities to observe state-of-art practices in facility operation and experimentation.
- Educate:
 - Choose from a wide array of training courses to gain hands-on experience using advanced tools.
- Analyze:
 - Partner with ORNL for specialized analysis of your systems, working in our collaboration space, and accessing world class computing resources.
- Enhance:
 - Establish partnerships to develop enhanced computational methods and data to better meet specialized needs and to realize the possibilities of HPC.
- Validate:
 - Quantify your validation basis by applying advanced approaches to assess available experiments and then enhance understanding through the design and operation of new benchmark quality experiments for licensing.
- Deploy:
 - Reap the benefits of quality assurance and archival analysis for licensing and deployment.
- Commercialize:
 - In certain cases, commercial licensing opportunities are available. These agreements may be on an exclusive or non-exclusive basis
 provided they are limited to a specific field of use. Said licenses are technologies that are bound by export control obligations
 which are available via RISCC.



Thank you for your attention!







NRIC Digital Engineering

December 1, 2020 Ashley E. Finan, Ph.D., NRIC director ashley.finan@inl.gov

inspire

empower

C

deliver

5-Year Program Objectives

Enable demonstration of at least 2 advanced reactors

- Make available infrastructure, sites, materials, expertise
- Provide regulatory support and coordination
- Best practices in public engagement

Prepare DOE/labs for continuing innovation and demonstration

- Develop best practices for planning/construction/demonstration of nuclear projects
- Develop enduring infrastructure and expertise
- Establish methods for efficient coordination among laboratories



WE'VE DONE THIS BEFORE

WE'RE GOING TO DO IT AGA

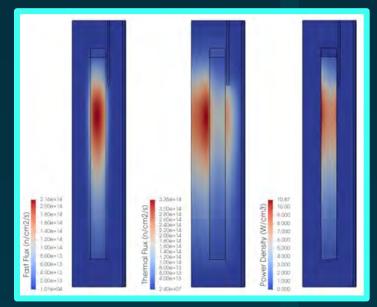
WITH SOME refinement

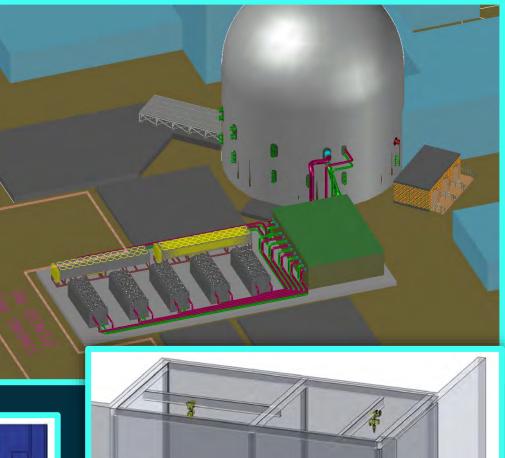
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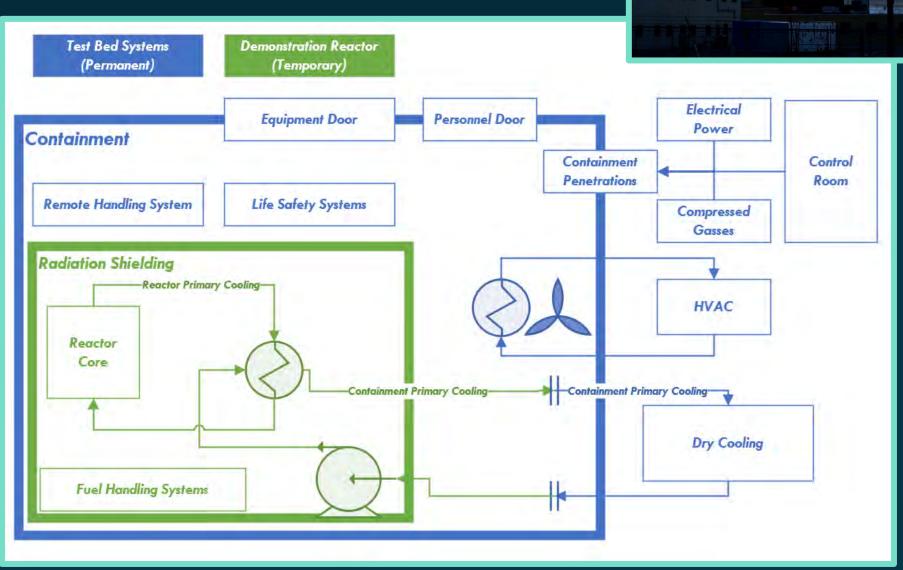
Mod. # 1 Empowering Innovators

- Private Sector Driven Effort
- NRIC Resource Team
- Virtual Test Bed
- Demonstration Resource Network
 - Experimental facilities
 - Fuel facilities
 - Test beds
 - Demonstration sites





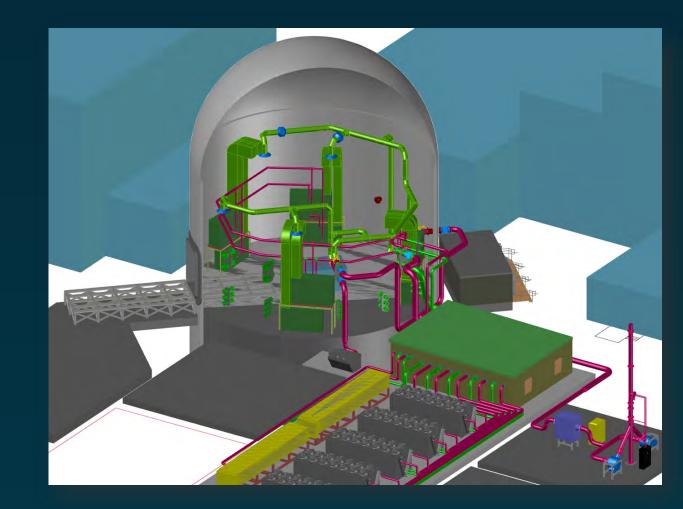
Demonstration Test Beds In Development



- User input received
- Functional and Operational Requirements Defined
- Concept of Operations Defined
- Digital engineering implemented
- Preconceptual design complete
- Request for Expressions of Interest released July 21 for A-E firm to complete design work in FY21

Pre-Conceptual Design NRIC-DOME Demonstration Reactor Test Bed

- Reactors producing less than 10MWt power
- Use of Safeguards Category IV fuels
- Modifications to equipment door to enable loading of Conex containers
- Cooling, electrical, ventilation, process fluid penetrations
- Ventilation system upgrades
- Electrical power system including safety class battery backup
- Control Room for ETB operations





Digital Engineering in Design -NRIC

- National Reactor Innovation Center: The National Reactor Innovation Center (NRIC) at Idaho National Laboratory provides resources for testing, demonstration, and performance assessment to accelerate deployment of new advanced nuclear technology concepts
- State of the Art: Document centric exchange of reactor design documents and information
- **Scope**: Transform the traditional engineering design ecosystem from a document-centric paradigm to a digital engineering framework to increase collaboration and efficiency.
- Opportunity: Powerful new software allows for the development of new products, services, and capabilities by using digital tools to improve real world outcomes. Industries ranging from construction to aerospace have implemented these techniques to bring down costs and increase productivity. NRIC is leading the way to begin applying these digital tools to advanced nuclear concepts.

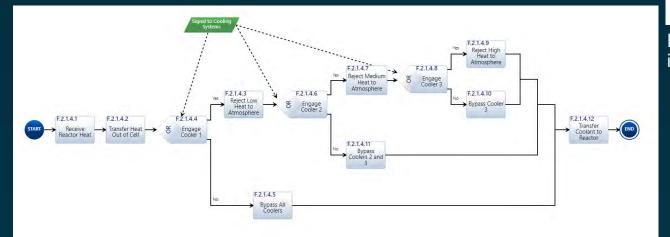
Complete Systems Level MBSE Architecture of Demonstration Test Bed

	Functional	System/Structure	Product
Conceptual	Approved	Preliminary	Preliminary for long- lead and key components
Preliminary	Updates as required	Approved	Preliminary
Final	Updates as required	Updates as required	Approved

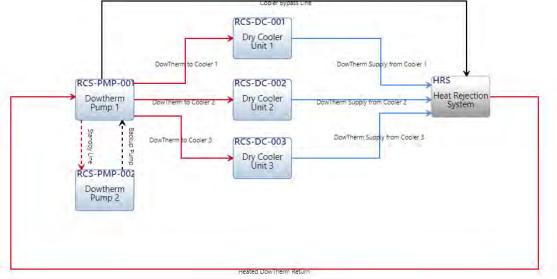


Model-Based Systems Engineering (MBSE) Environment for NRIC Test Beds and Industry Teams

- Emphasis on development of functional analysis (activity diagrams) and physical analysis (asset, internal block diagrams) over document creation
- System models (SysML/LML) are linked to the requirements document in the same tool environment to provide systemlevel traceability
- Models are integrated across teams (from NRIC test beds through contractor design teams)



Functional: Remove 50 - 500 kW Reactor Thermal Energy



Physical: Reactor Cooling System (RCS) Asset Diagram (gray box indicates live contractor input)

4.1.1 Maximum Thermal Load

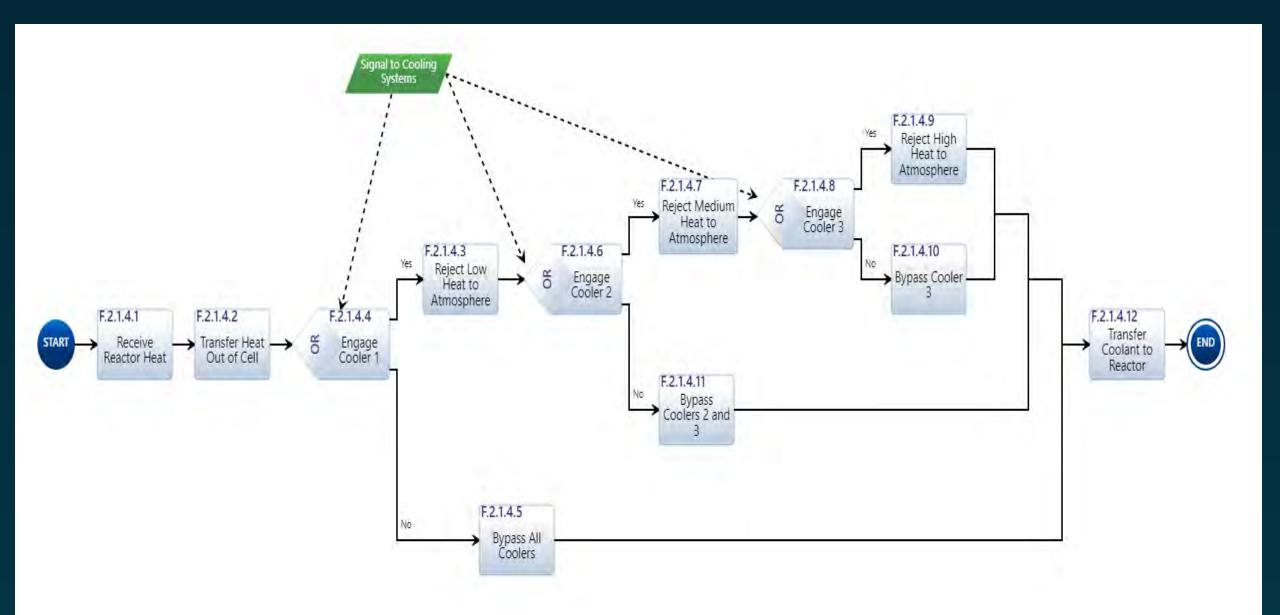
The thermal energy removal system shall remove a maximum of 500kW thermal energy from the ZPPR Cell

4.1.2 Minimum Thermal Load

The thermal energy removal system shall remove a minimum of 50kW thermal energy from the ZPPR Cell.

4.1.3 Variable Heat Remova

ZTB shall provide a feature for the thermal energy removal system which can vary the amount of thermal energy removed from the reactor.

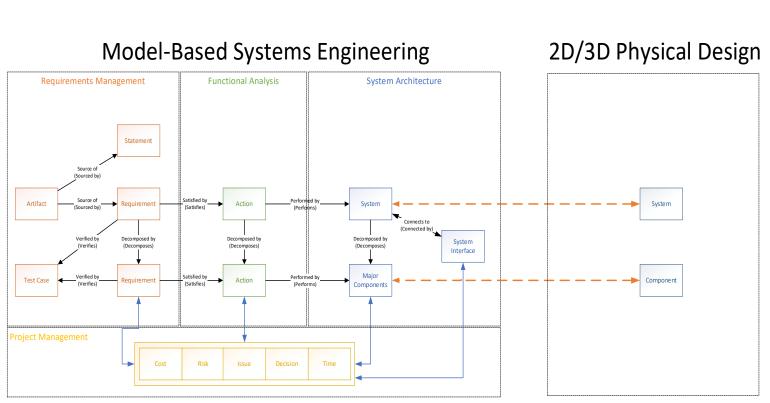




Functional Requirement: Remove 50 - 500 kW Reactor Thermal Energy

Building the Digital Thread with Model-Based Systems Engineering (MBSE) and digital engineering

- Computer Aided Design (CAD) bidirectional integration with MBSE models to reduce error transferring from systems through detailed design (leveraging existing laboratory university research on Deep Lynx)
- Generating reports in INL- and NRICcompliant formats to automate documentation needs at the system level
- Integration with the overall digital engineering ecosystem which will provide analysis integrations at the system, civil design physics, and nuclear physics codes
- Overall plan to integrate this system (used in design) with operating facilities to enable a full digital twin



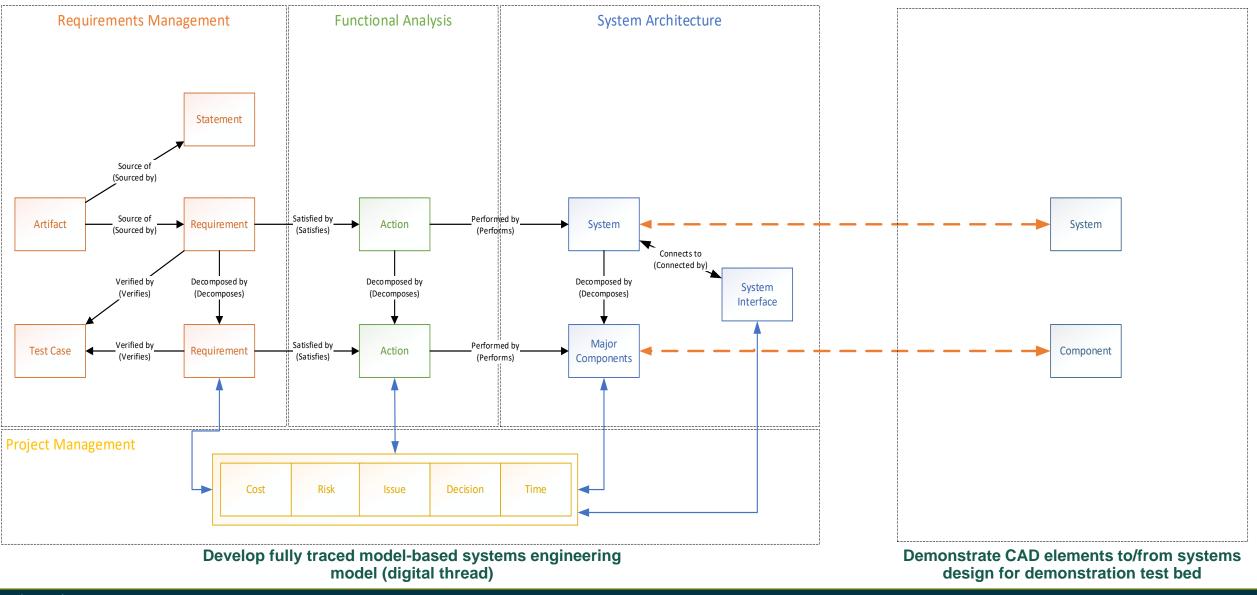
Develop fully traced model-based systems engineering model (digital thread)

Demonstrate CAD elements to/from systems design for demonstration test bed



Model-Based Systems Engineering

2D/3D Physical Design





Key Goals of NRIC Digital Engineering Approach

- Efficient and accurate interface between NRIC and industry partners
- Improved project outcomes through digital thread & MBSE approach
- Foundation design framework to enable digital twin
- Ultimate cost and schedule reductions in deployment through use of tools/methods demonstrated through NRIC



Thank you!

Questions?







The GEMINA Program:

What ARPA-E is Doing and Broader Opportunities

Jennifer Shafer Dec 1, 2020

First, Thanks to a Great Team!



Joel Fetter, T2M



Curt Nehrkorn, Tech SETA



Ray Duthu, Tech SETA



Lakshana Huddar, (former) Fellow



Caitlin Zoetis, Proj. Manag. SETA



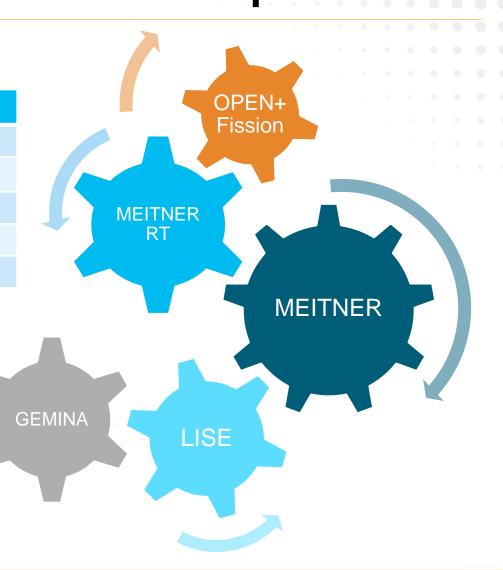
Geoffrey Short, Tech SETA



ARPA-E Nuclear Fission Landscape

Program/Cohort	Budget	Teams
MEITNER	~\$30M	9
MEITNER Resource Team	~\$10M	1
OPEN + Fission	~\$12M	5
LISE	~\$8M	4
GEMINA	~\$35M	9

Multiple groups of fission teams, all managed together to achieve economically viable nuclear power





U.S. Reactors are Shutting Down from O&M

Category	Fuel	Capital	Operating	Total
All U.S.	6.44	6.64	20.43	33.50
Single-Unit	6.42	8.92	27.32	42.67
Multi-Unit	6.44	5.99	18.46	30.89

- Table in 2017 \$/MWh
- Minimal staffing across best performing plants: ~750 FTEs
- Operations and Maintenance are the largest addressable categories

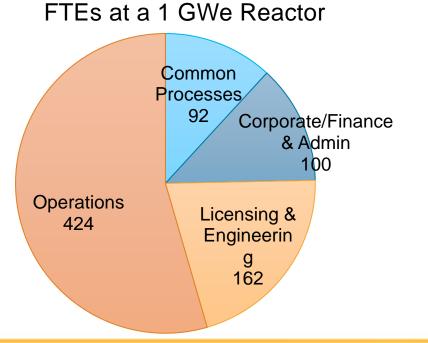


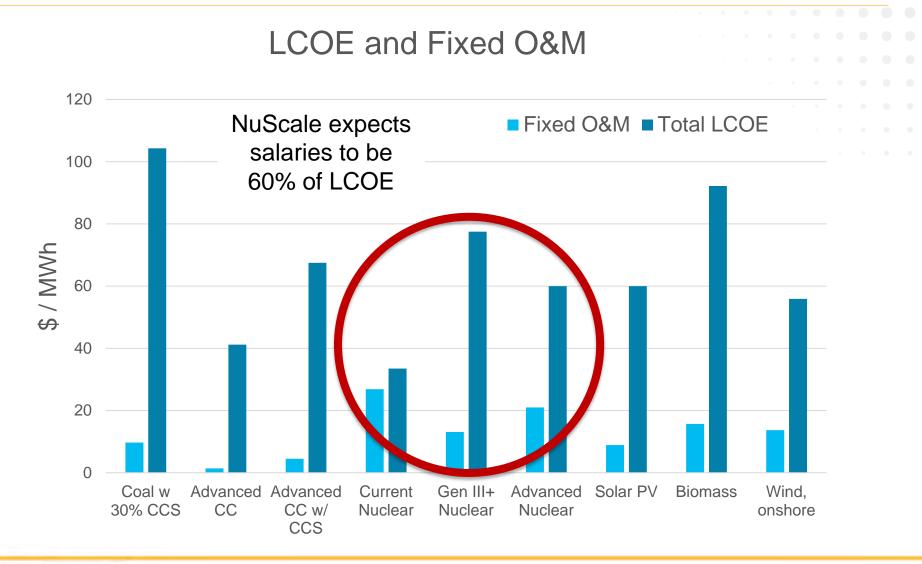


 Table: https://www.nei.org/CorporateSite/media/filefolder/resources/reports-and-briefs/nuclear-costs-context-201810.pdf

 3

 Pie chart: https://www-pub.iaea.org/MTCD/Publications/PDF/te 1052 prn.pdf

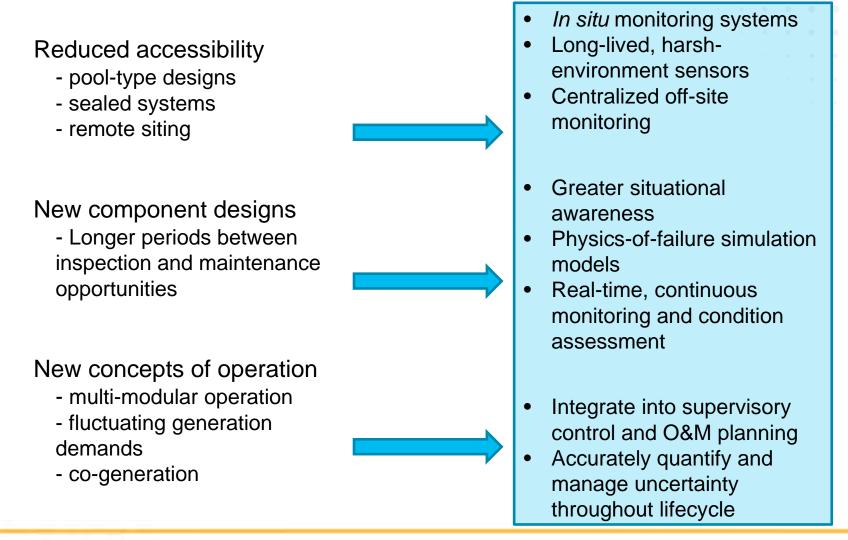
What Are the Costs? What About Next Gen?





https://www.eia.gov/outlooks/aeo/pdf/electricity_generation.pdf, https://www.innovationreform.org/wpcontent/uploads/2018/01/Advanced-Nuclear-Reactors-Cost-Study.pdf

Features of Advanced Reactor Designs Lead to New Requirements for On Line Monitoring





Challenge:

Can we operate advanced reactors with a cost profile comparable to a natural gas combined cycle plant? (a 15x reduction)

What this really means:

Can we get ARs up to the leading edge of development to make them nearly autonomous – which is what the rest of the industrial world is doing?



Leverage New Ideas and Sort it out NOW

- Lots of industries are developing better controls, better models, better data, better algorithms
- Focus on autonomy and machine learning (ML) is getting many questions answered
- Answer those questions specific for nuclear and prove out ideas in our systems and with our soft



- systems and with our software; aid in code validation
- Have tools the industry and the regulator can use



This Is an Essential Need

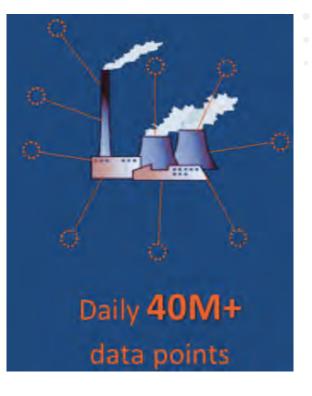
- Many developers have identified they need to develop these technologies
- In a test plan from an example company, Human-Machine Interface is an essential need that "would take full advantage of the technology advances since early LWRs were deployed." It includes things like:
 - Remote maintenance
 - All ops activities, from cold shutdown to full power operation of all integrated systems
 - Recovery from off-normal conditions



Lots of People Are Working to Improve Industrial Performance

- How do we know when plant conditions are becoming riskier?
- How can we avoid a plant trip or component failure?
- How can we reduce maintenance and insurance costs?
- How can we improve reliability and utilization without increasing risk?

Alarms / Operating Limits Asset Monitoring / Predictive Analytics Self Service Analytics Autonomous Risk Detection Systems



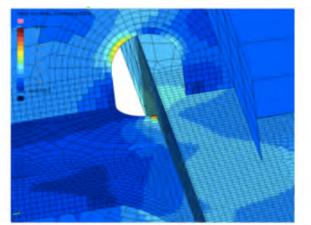


Source: Near-Miss Management, "Maximizing Uptime, Efficiency, and Safety of Industrial Operations through Early Risk Detection." AiChE

Lots of great results

Uptake reduced preventative maintenance work hours by 37% at Palo Verde nuclear plant





Halliburton builds models of wells to help construct them (using physics models and data gleaned from other wells). Can cut well design time by 80%.

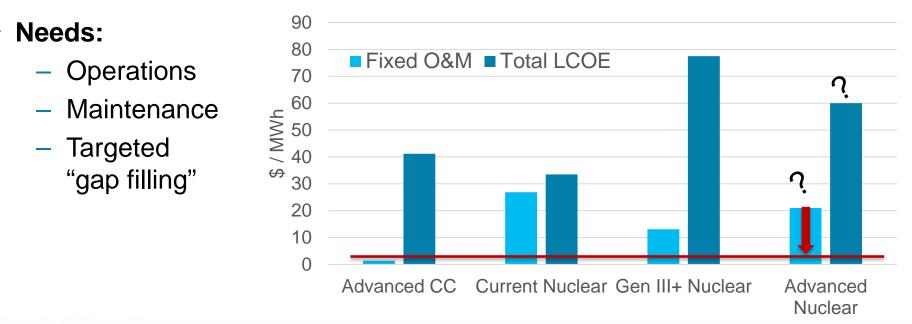
4x increase in predicted fatigue life without changing safety factors



Sources: Near-Miss Management, Akselos, BNEF Digital Twin Edition, Uptake Palo Verde case₁₀ study

Program Framing and Goals

- Goal: Advanced Reactor (AR) O&M cost of no more than 10% more than an advanced combined cycle plant
- Pathway: personnel reduction through automation, autonomy, and enhanced safety case







https://www.eia.gov/outlooks/aeo/pdf/electricity_generation.pdf,

https://www.innovationreform.org/wp-content/uploads/2018/01/Advanced-Nuclear-Reactors-Cost- 11

SSIBLE Study.pdf

Potential Impact

- Detailed plan and reasonably vetted tech for low O&M costs for advanced reactors: strong basis for cost estimates
- Substantially autonomous reactor operation:
 - Software and algorithms developed and ready for demo
 - Tools for design and regulation to accelerate deployment
- Significant reduction in staff for maintenance:
 - Map of task allocation and minimum staff required
 - Designs updated to integrate with autonomy needs
- Opportunities for demo: Existing test facilities, Southern Co's Large Component Test Facility, Micro reactors, NuScale, Versatile Test Reactor



Operating Advanced Reactors

Interdisciplinary teams will

- Build a digital twin for an advanced reactor system
- Build a cyber-physical / hardware in the loop¹ system at an existing non-nuclear test loop
- Assess what signals are needed with what accuracy; develop new inference methods as needed
- Gain validation data for nuclear software
- Practice control operations with injected signals: startup, shutdown, and transient scenarios
- Define a standard interface / approach for how to deal with uncertainty, simulation fidelity needs, inputs / outputs, signal measurement, etc.
- Examine impact of changes and feedback learning into design needs
- Wash, rinse, repeat



[1] HIL: real signals from a controller are connected to a test system that simulates reality, tricking the controller into thinking it is in the assembled product

New tech seeing some use in the field

- Accenture: Applying advanced analytics to the predictive maintenance of assets could save...up to 12% on scheduled repairs and lower overall maintenance costs up to 30%
- Exelon is using GE Predix to provide predictive assessments of key power plant components such as turbines Collect and analyze sensing information from equipment to determine whether the probability threshold of that equipment failing has been breached
- Framatome + IBM Watson IoT to use N-Vision data analytics service: proactive approach in maintenance and operations strategy

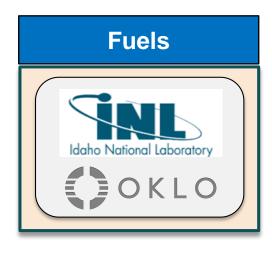


LISE Teams





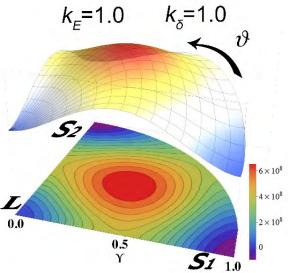






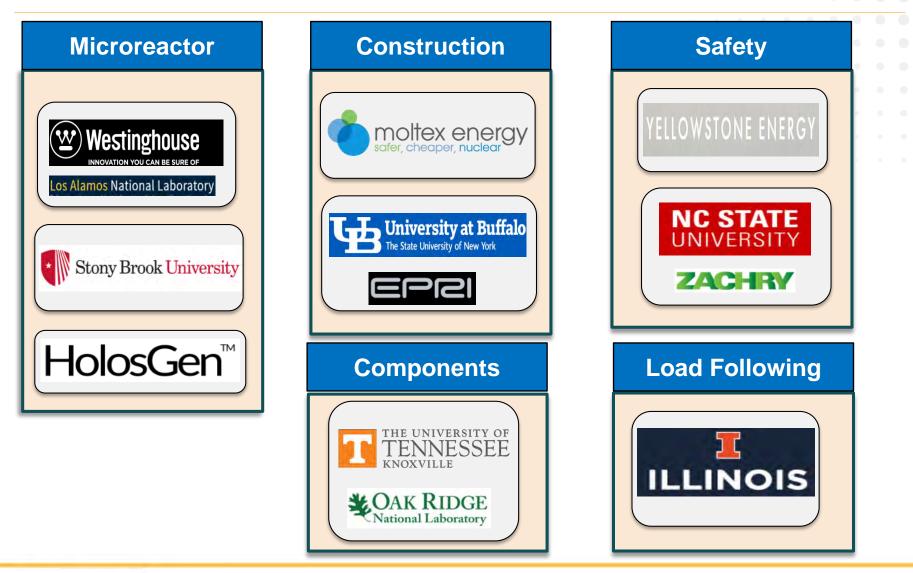
OPEN+ 2018: Five Teams, \$11.7M Fed

- CMU: Additive Manufacturing of Spacer Grids for Nuclear Reactors
- LBNL: MEMS RF Accelerators for Nuclear Energy and Advanced Manufacturing
- LANL: Advanced Manufacturing of Embedded Heat Pipe Nuclear Hybrid Reactor
- MIT: Multimetallic Layered Composites for Rapid, Economical Advanced Reactor Deployment
- UW-Madison: Accelerated Materials Design for Molten Salt Tech. Using Innovative High-Throughput Methods





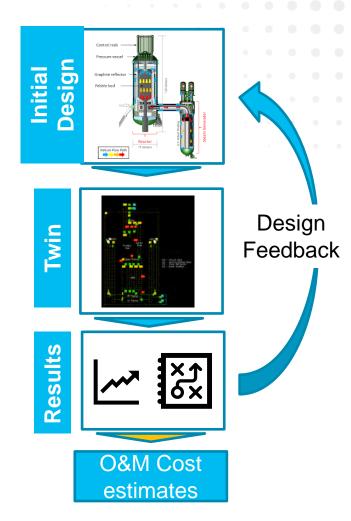
MEITNER Teams





Objectives and Measures

- Measures of success:
 - System provides correct recommendations for startup and shutdown; system avoids or mitigates accidents in off-normal scenarios
 - Ability to implement a change in the models and investigate the tradeoffs
- Provide authoritative data to regulators that leads to credible, expedited design and operator approvals
- Create knowledge of advanced nuclear system O&M that provide the basis for advanced reactor operating standards





Tools and Data Are Needed As Well

- We already know some of the knowledge and capability gaps that will prevent deployment of the first two efforts
- If we solve them concurrently, we'll be that much closer to deployment into test systems coming online in the early 2020s where they can be fully derisked
- Operations needs:
 - Data, e.g., thermophysical properties of molten salts; high temperature materials behavior
 - Key sensors, e.g., flow meter for molten salt
- Maintenance needs:
 - Some things must be done by robots



AR O&M: Opportunity to Do Things Differently

- Now is the time: no O&M plans, unfinished designs, no regs
- Implement "design for maintenance"
- ARs have a much stronger safety case, this opens the door to more change and more flexibility
- Most ARs have or will have different refueling schedules
 - Currently, maintenance is done during outage
 - Without outage, removal of time pressure
- A fleet of small, distributed reactors could have remote operators and one maintenance crew between them
- We need to generate tools and data to support licensing and fast learning



Key Points

LWR operations are dialed in after decades of experience

New reactors need long uptime right away -> use tech to change the learning curves

LWRs have refueling outages every 18 months when lots of maintenance is done

New reactors may refuel on much longer time scales (5-30 years), so new paradigm for maintenance

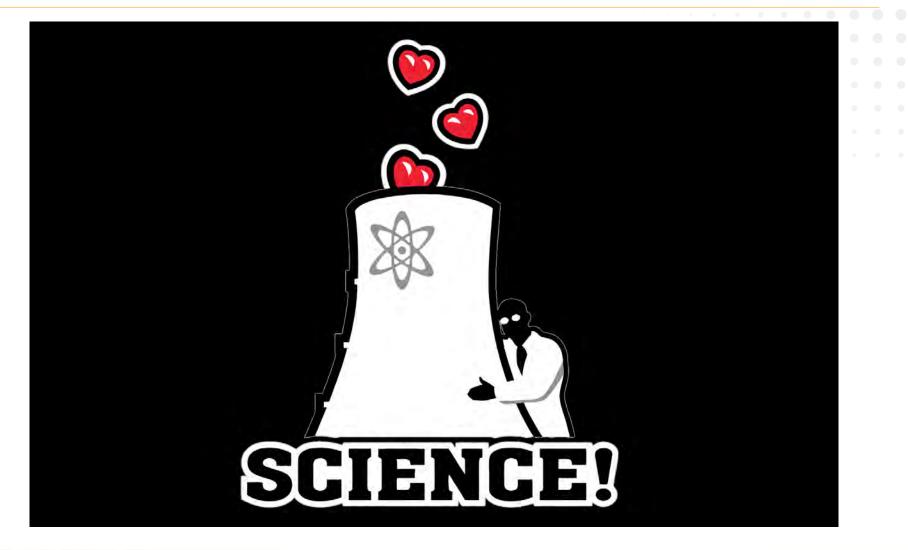
Cost profiles for LWRs are locked in via nuclear quality assurance procedures

These haven't yet been developed / don't exist for advanced reactors yet

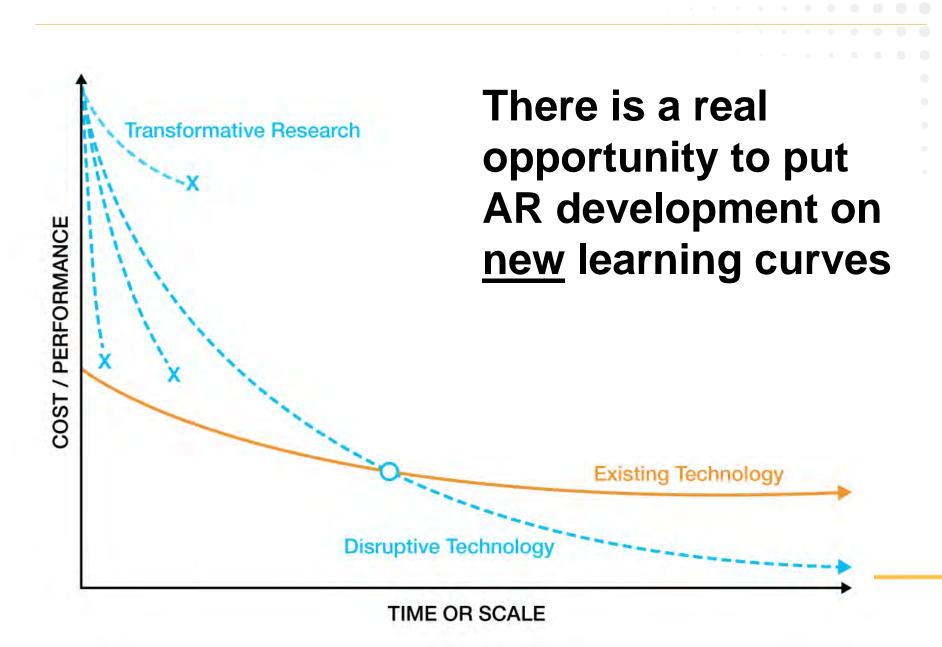
There are lots of technologies being developed to take advantage of and become just another industrial customer



NE Questions?



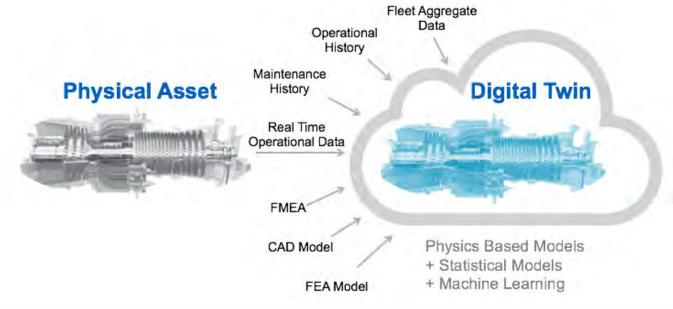




Digital Twins

Digital Twin:

 "A 'digital twin' is a physics-based, or data sciencebased, model of an asset that exists in real life. It should mirror digitally the exact characteristics and operating performance of the real device, so that operators can understand the...asset"

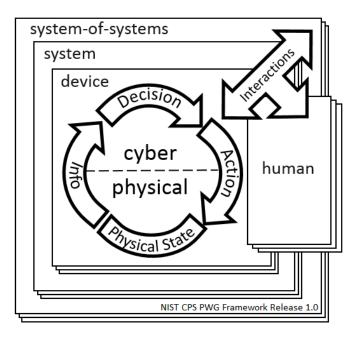




Cyber-Physical Systems

Cyber-physical system:

- "integrations of computation, networking, and physical processes...with feedback loops where physical processes affect computations and vice versa...
- CPS integrates the dynamics of the physical processes with those of the software and networking..."

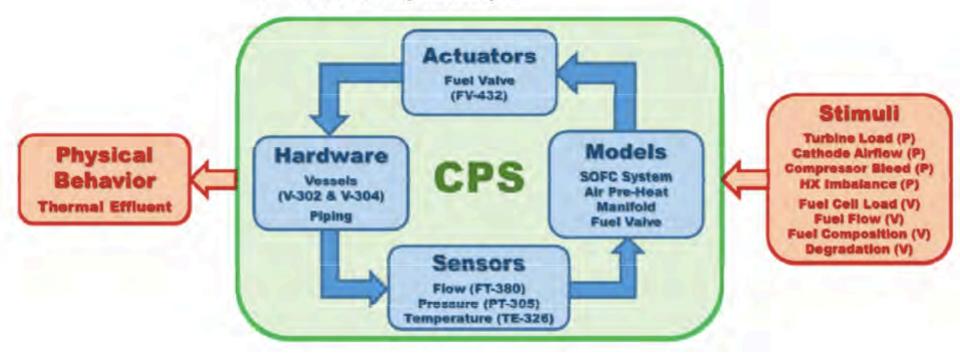




Cyber Physical System Example: Hyper

Cyber Physical Systems are used to replace physical systems that:

- 1. are irreplaceable,
- 2. are expensive,
- 3. not technically viable...yet.





What Might "Optimal Operations" Mean?

- Lower direct personnel costs
- Eliminate radiation to workers
- Reduce cost / amount of maintenance
- Reduce risk of human error
- Increase operational excellence
- Increase margin / safety envelope



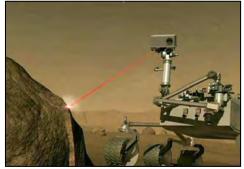
However,

- Increase cost of sensors / equipment / software?
- Shift problems and cost from one place to another?
- Increase need for certain highly specialized staff?
- How do we get the relevant innovators to think about nuclear?



Some Places to Draw Inspiration

- JPL currently deploys autonomous systems that:
 - Protect systems from detected faults and hazardous conditions (fault protection)
 - Perform critical events despite the presence of failures (orbit insertion; entry, descent and landing)



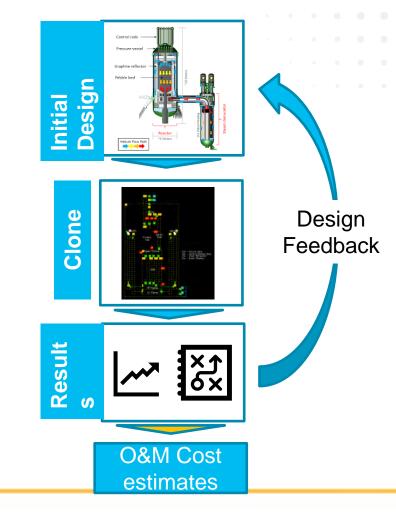
Curiosity Rover

- Increase mission effectiveness and return (auto-navigation, feature detection and science observation re-targeting)
- ExxonMobil has signed collaboration agreements with six other companies to accelerate the development of Open Process Automation (OPA) systems.
- Autonomous vehicles, Oil & Gas, DoD, jet engines, etc. (most industries are developing or adopting this stuff)

Credit: John Day https://www.marketwatch.com/press-release/exxonmobil-signs-collaboration-agreement-toaccelerate-development-of-open-process-automation-systems-2019-07-18

Learning feeds back into the design

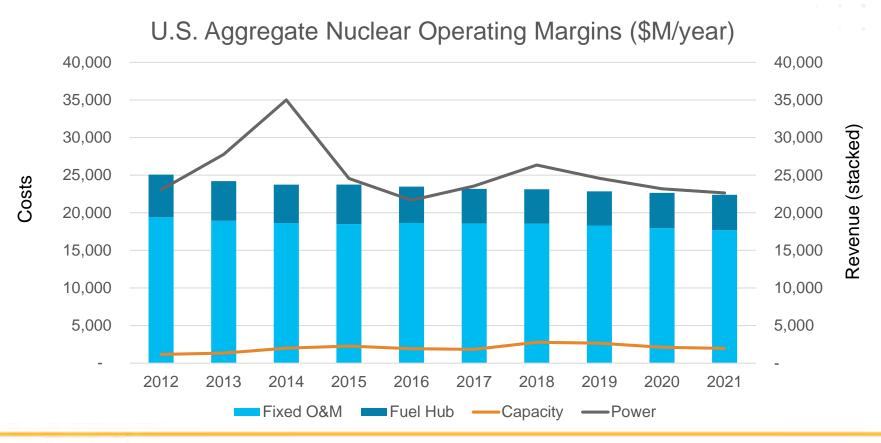
- **Purpose:** Use the model to inform O&M costs and practices.
 - Determine the necessary O&M staffing profile
 - Understand reactor dynamics in order to inform design enhancements
 - Document pro forma O&M costs
- Lead: Ideally, a reactor vendor and/or build contractor
- Key issues:
 - Further development will almost certainly be needed –
 "month 37" is especially important
 - → Ensure a home for the generic devices, perhaps at a laboratory.
 - Regulator input should be solicited to inform next stage development requirements
 - → Look to aviation sector for approaches to certifying simulators





U.S. Reactors are Shutting Down

5 reactors closed in the last 5 years
 14 more scheduled to close by 2025





Source: Bloomberg New Energy Finance, Reactors in the Red: Margins for U.S. Nukes. May 15, 2018

Southern nuclear challenge

- Second half of 2016, approached by a number of vendors for new data analysis techniques
 - 6 months of computer data (1 TB) available to each vendor
 - Develop and train over 1 month where issues are preidentified
 - New dataset (6 months) with no issues pre-identified, 2 months to analyze
 - Vendors reported findings
 - With lack of training information, similar results to deterministic approach



energy

Digital Twin Applications for Advanced Nuclear Technologies

Xe-100 Digital Technologies Overview

Ian Davis, Senior Digital Twin Systems Engineer

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December 1, 2020

X-energy was Created to Change the World

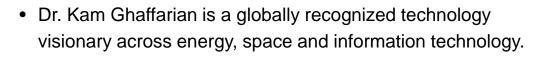




Dr. Kam Ghaffarian, Founder and Executive Chairman

"President Kennedy once said that we are in a space race and my work with NASA reflects the progress he had hoped for.

Today, I believe we are in an energy race. Providing clean energy across the world is my vision for X-energy and I believe that clean, safe, reliable nuclear energy is necessary to making this possible."

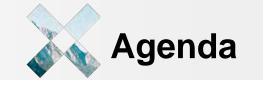


 Created and grew Stinger Ghaffarian Technologies (SGT), Inc. to \$650 million in annual revenue and 2,400 employees. SGT was ranked as the U.S. National Aeronautics and Space
 Administration's second largest engineering services company prior to being acquired by KBRwyle, subsidiary of KBR, Inc.



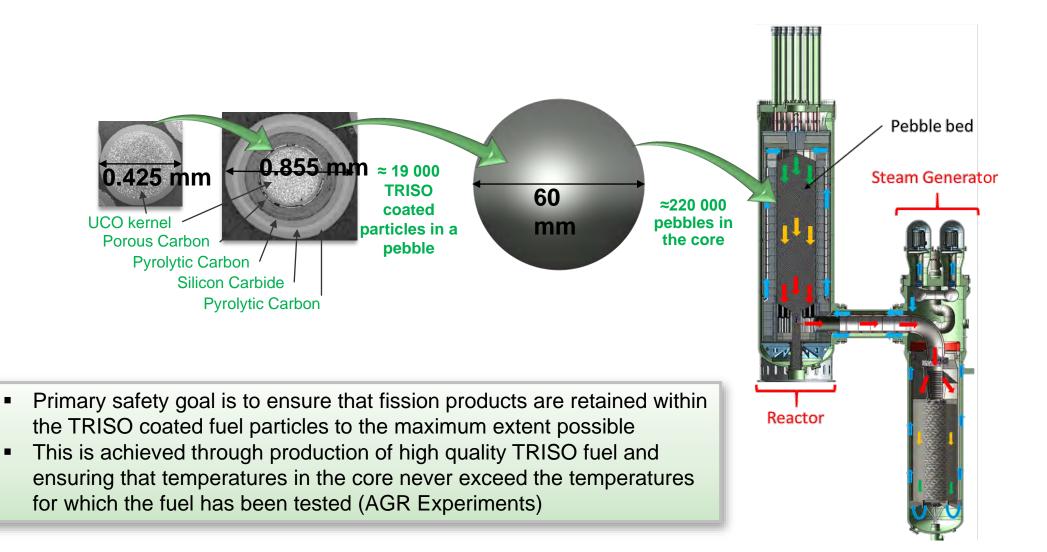


- Founded X-energy in 2009 to address innovation in critical energy solutions. X-energy was awarded ~\$60M from DOE to focus on an advanced nuclear reactor and TRISO fuel.
- Began Intuitive Machines in 2016 to leverage NASA technologies for commercial space and terrestrial applications. Intuitive Machines won its first Commercial Lunar Lander Contract from NASA in 2018.
- Began Axiom Space in 2017 to develop the first commercial space station, to be launched by 2021.



- 1. Company Introduction
- 2. Xe-100 Overview
- 3. Digital Twin Overview
- 4. Anomaly Detection in Systems & Components

UCO TRISO Particle – Primary Fission Product Barrier





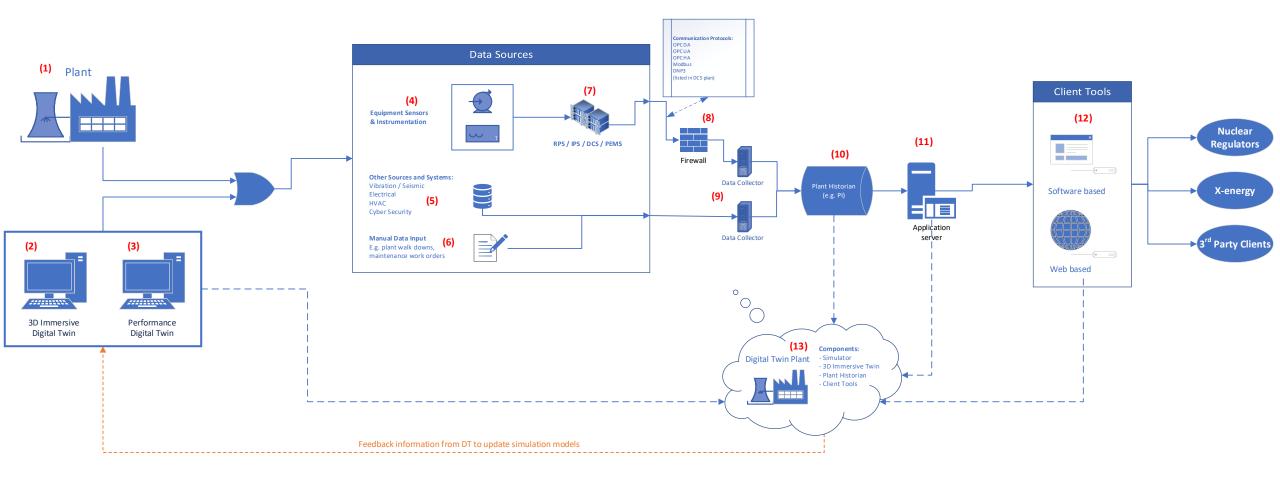
Standard X-energy plant have 4 Reactors - 4 Turbines producing 320 MWe, attributes include:

- 200MWth/80MWe Per Module
- Process heat applications Proven intrinsically safe
- Meltdown proof •
- Walk-away safe
- Modular construction
- Requires less time to construct (2.5-4 years)
- Road transportable for diverse geographic areas Uses factory-produced components Load-following to 40% power within
- •
- 15 minutes
- Continuous fueling; resilient on-site • fuel storage

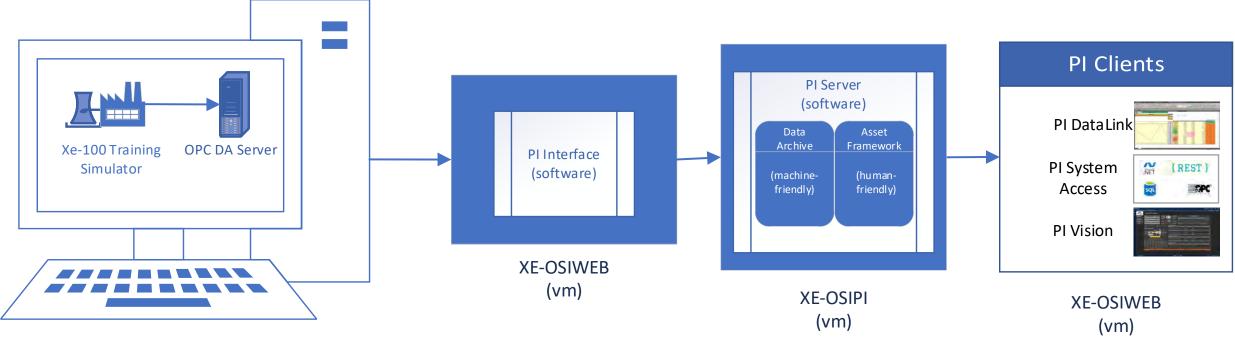






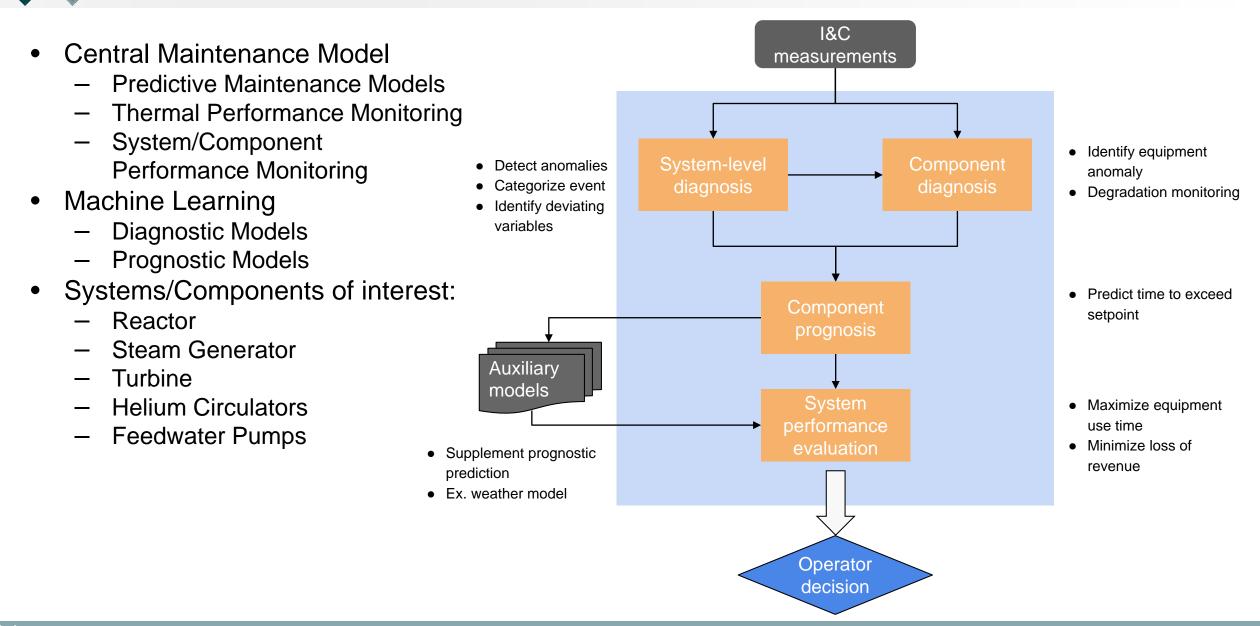


Training Simulator Integration with the PI System



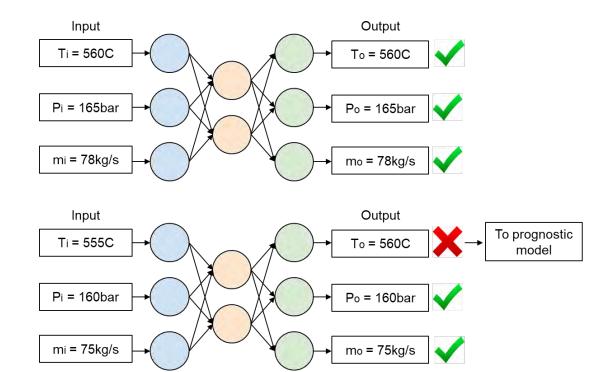
Model Server

Anomaly Detection with Machine Learning



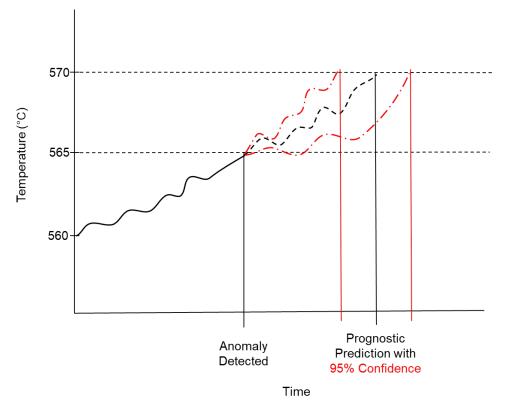


- The diagnostic model aims to
 - Detect system component anomalies
 - Identify deviating variables
 - Initiate the correct prognostic model
 - Be continuously trained online
- Machine learning algorithms include
 - Auto-Encoder (AE) for feature extraction
 - Long-Short Term Memory (LSTM) for temporal data





- The prognostic model aims to
 - \circ $\$ Predict time to abnormal condition
 - Provide time window to auxiliary models
- Machine learning algorithms include
 - Bayesian Neural Network (BNN) for uncertainty
 - \circ $\,$ AE-LSTM for input space reduction and temporal data $\,$
 - Convolutional Neural Network (CNN) for efficient spatiotemporal data processing





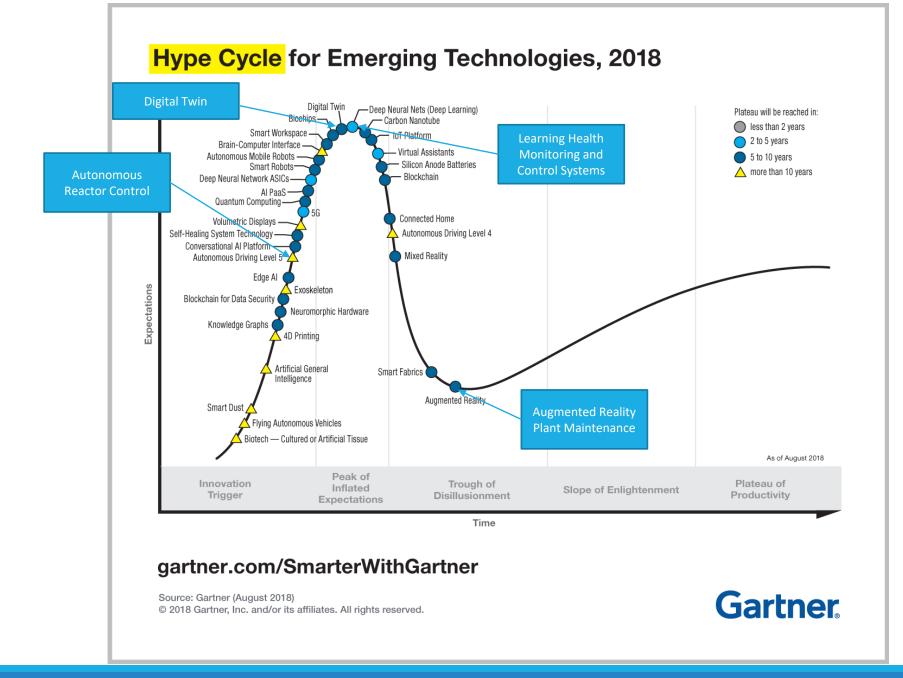
Digital Twin development for Advanced Reactors: Accelerating time to market, increasing safety margins, maximizing value.

DR. ANTHONIE CILLIERS

DECEMBER 1st 2020

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In order to achieve this mission, we must prioritize our efforts to focus on a clean energy technology that is *affordable* and *safe*.



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Digital Twin and simulator focus

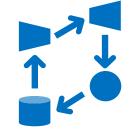
A Digital Twin consists of sophisticated models or system of models based on deep domain knowledge of specific industrial assets. The Digital Twin is informed by design, manufacturing, inspection, repair, online sensor and operational data. It employs a collection of high-fidelity computational physics-based models and advanced analytics to forecast the health and performance of operating assets over their lifetime.



Lifing Capital equipment predictive reliability **models** for personalized intervals, dispatch tradeoffs & long-term outage planning.



Anomaly Physics & data driven **models** for prognostics, early fault detection & asset specific failure mode management to reduce unplanned downtime.



Thermal Plant thermal cycle **models** to make informed operational tradeoffs, manage degradation and improve efficiency over the load profile.



Transient Physics & predictive **models** for achieving best plant operational flexibility while managing equipment & site constraints.

Fundamental Digital Twin uses

Outcomes

Online performance

Operational flexibility

Economic dispatch

related equipment

Life optimizing control

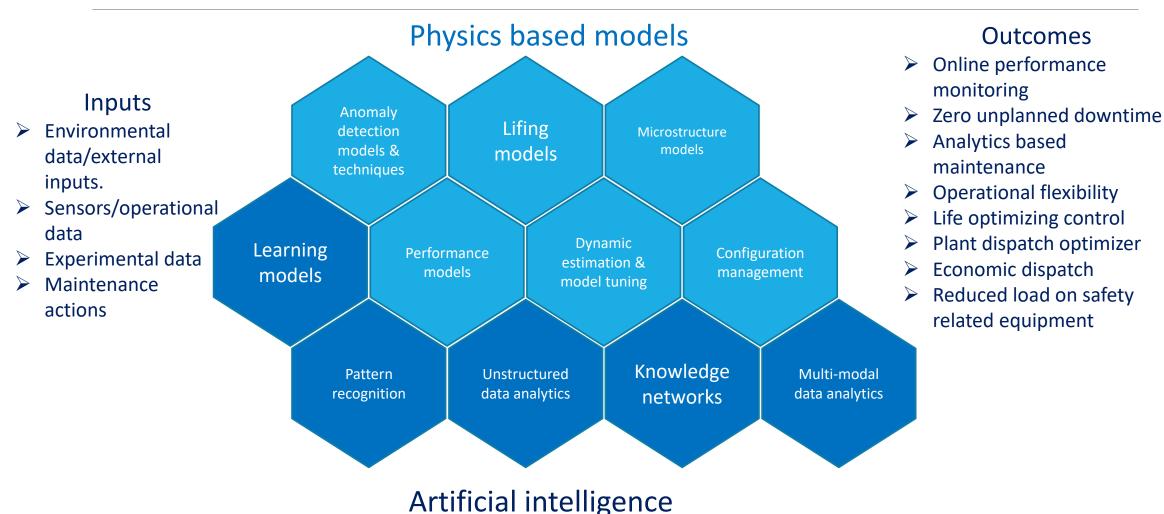
Plant dispatch optimizer

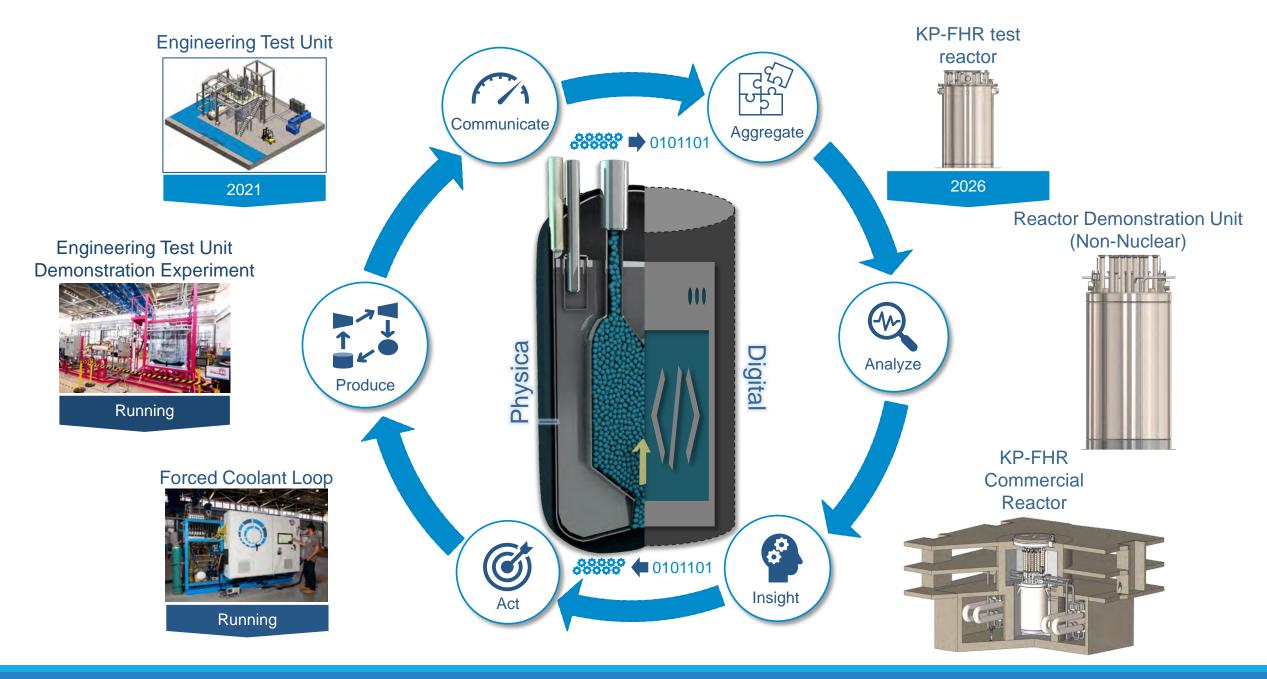
Reduced load on safety

monitoring

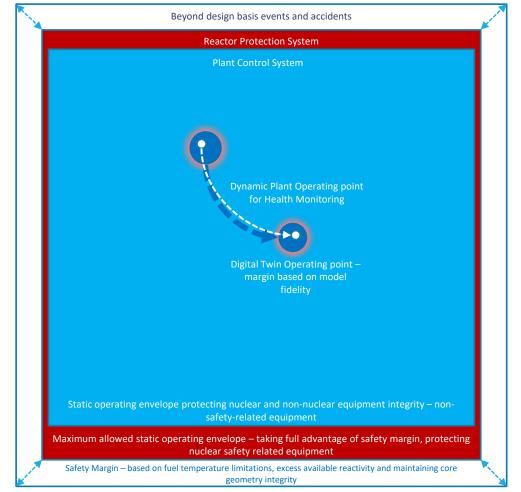
Analytics based

maintenance





Digital Twins: Increasing Safety Margins

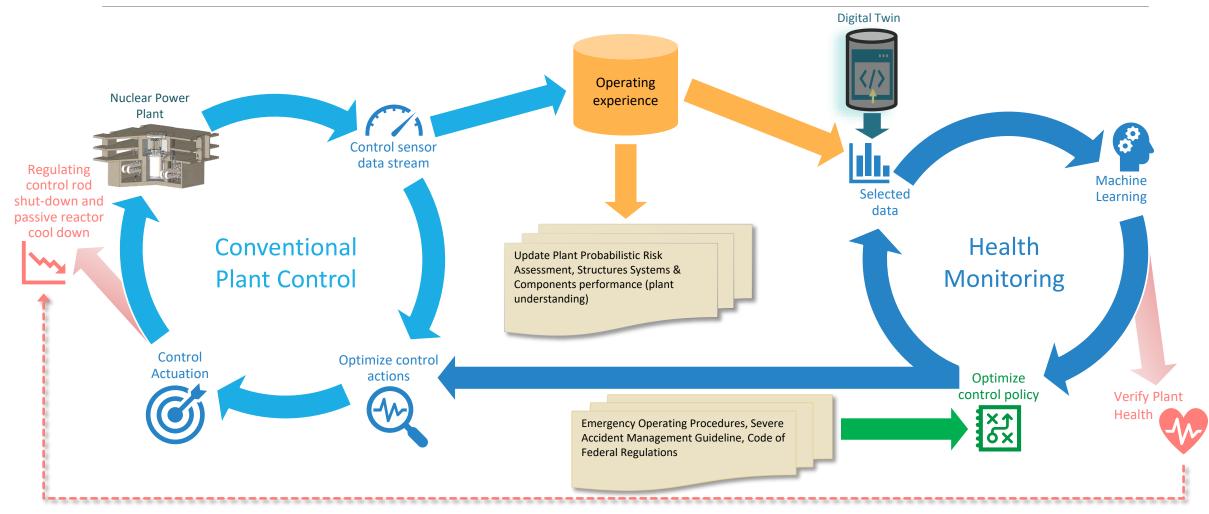


Outcomes

- Digital Twin Operating Point creates a virtual reference for plant operations.
- Identifies abnormal operations early before challenging System, Structure, Component integrity.
- Increases reliability of Non-Safety Related control systems.
- Reduces frequency of Safety Related system operation.
- Increases safety margin.

Source: Cilliers A.C., A deterministic approach for establishing a narrow band dynamic operating envelope to detect and locate hardware deterioration in nuclear power plants, North West University, South Africa, 2013

Digital Twins, maximizing operational value



Adapted from source: Singh. G., Gas Turbine Auto Tuner, Siemens Digital Innovation, MIT: The Digital Nuclear Power Plant: Design, License and Operate at Minimal Cost, April 30 - May 1, 2019

Thank You

Advanced Reactor Design Meets Silicon Valley

Digital twins as early design tools

Clyde Huibregtse, Oklo Inc.

How do we scope the design problem?

What we have:

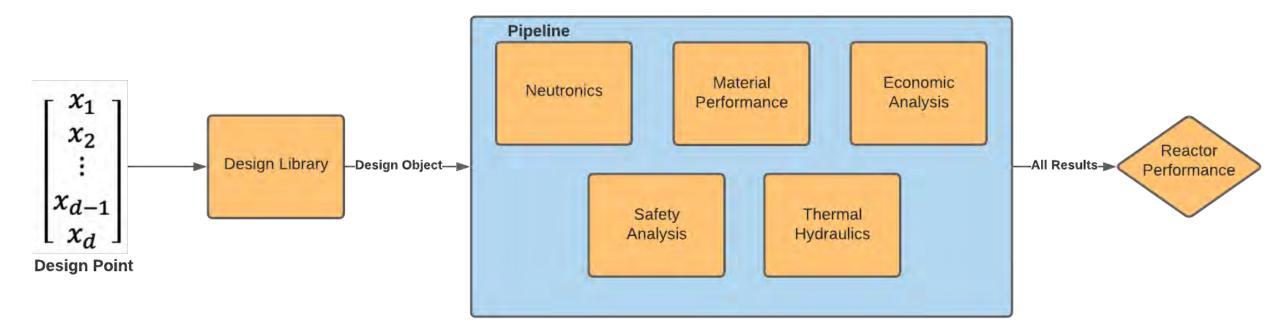
- High-level reactor concept and use-case (power, coolant, fuel type, etc.)
- Rough design and operating limits (material limits, peak temperatures, size)
- Ability to evaluate a design's performance based on economics, safety and operating state

What we want:

- Low-level choices for design parameters (design dimensions, material selections, operating conditions, etc.)
- Performance sensitivity to design parameter changes
- Ability to limit design search space by highly non-linear customizable acceptance criteria

Surrogate Model

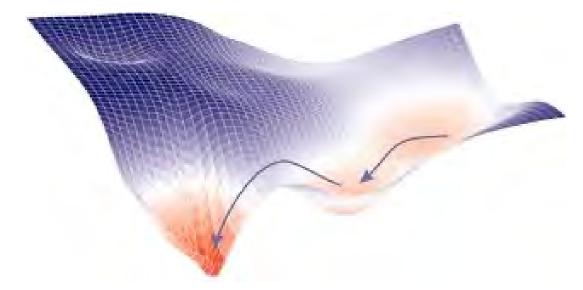
The Pipeline





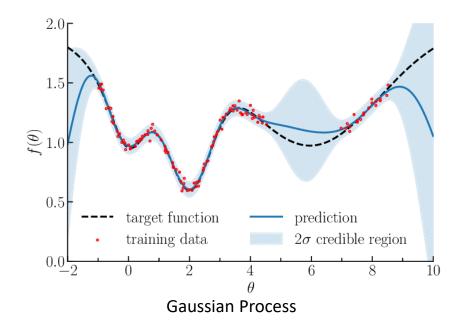
Our Holy Grail: The partial derivative

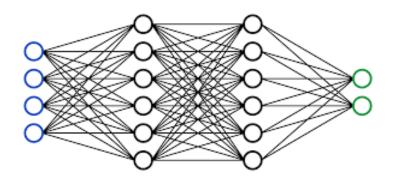
- "By how much does my reactor performance change as I tweak one of my dimensions?"
- Critical in modern convex optimization methods; Industry is dominated by gradient-based approaches
- Computing a derivative through a large, heterogenous computational stack is difficult, if not impossible
- (Industry is moving towards full-stack, algorithmic differentiation. Zygote.jl)



 $\frac{\partial \text{ perf.}}{\partial x_i}$

Surrogates make computing derivatives easy





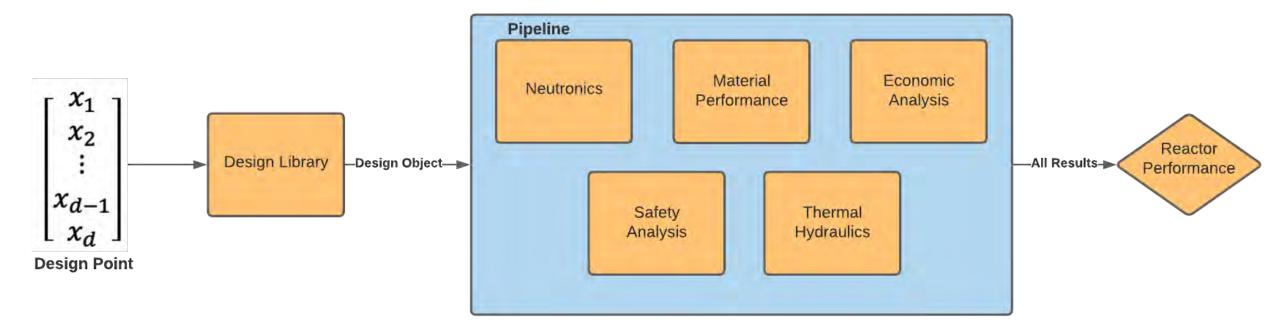
Artificial Neural Network

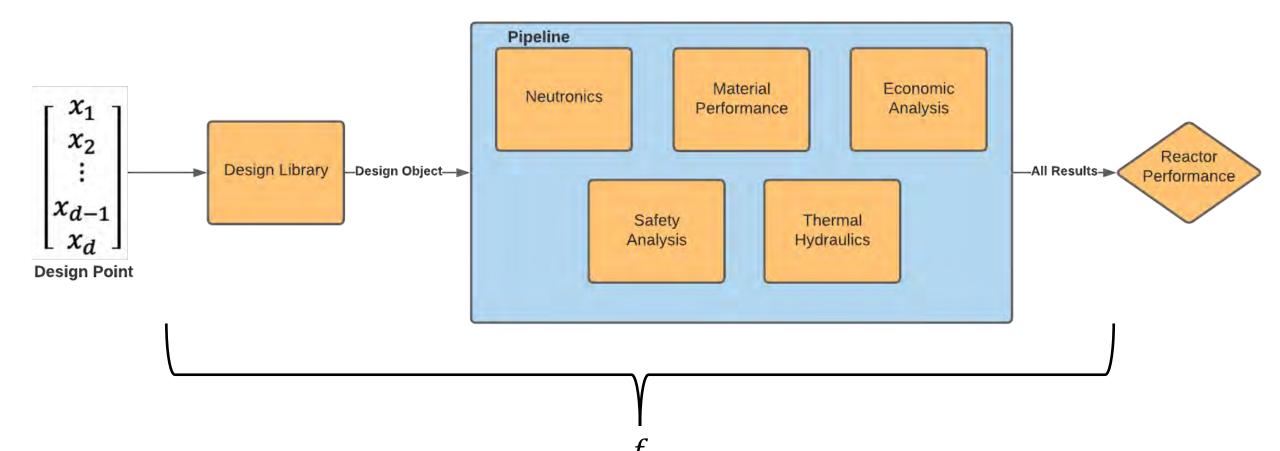
- Mimic the diverse, expensive computational stack with a **surrogate model**
- Surrogate should be:
 - Accurate surrogate should closely match the objective on unseen data points
 - Fast calls to the surrogate should be computationally cheap
 - Statistics informed surrogate should report the quality of its predictions
- Surrogate can be Neural Network, Gaussian Process, regression, etc.

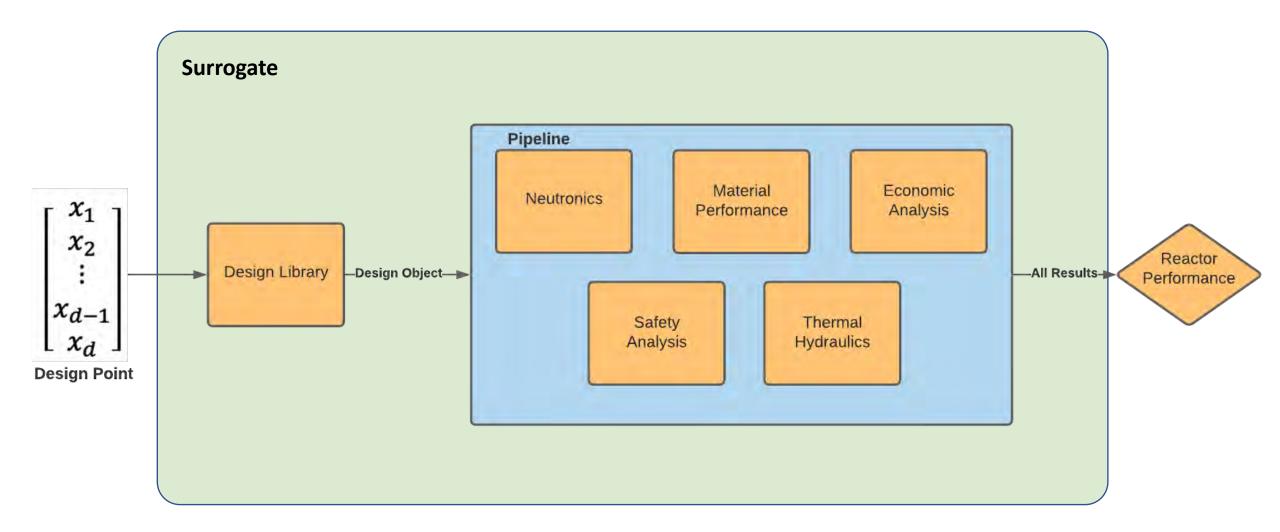
Surrogate formalism

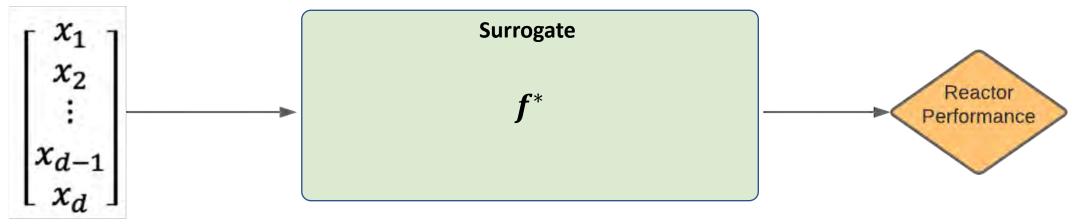
- Let \vec{x} be a vector of d dimensions **design point**
- Let *f* represent the stack of codes that computes the **performance** of a given design point
- Therefore, $y = f(\vec{x})$ is the scalar output of our performance function

- **Goal**: create **surrogate**, f^* such that $f^*(\vec{x}) \approx y$, and find minimum of f^*
 - f^* trained by collecting many (\vec{x}, y) pairs
 - Ideally, f^* responds to queries in the form: $f^*(\vec{x}) = y \pm \sigma$









Design Point

Training a surrogate

- \mathbb{R}^d is too large to search in earnest
- But we do have some intuition:
 - Most dimensions positive
 - Sense for relative scale given the use-case of the design
- Intuition gives a bounding **hyperspace** for the location of the optimal design point
- How do we efficiently span this hyperspace? Ensure surrogate is accurate where we need?
 - Grid in *d* dimensions?
 - Monte Carlo?
 - Low Discrepancy Sequences (LDSs)!

Generating efficient training data

• What is "efficient"? Low error in numerical integration

$$\int_{C^d} f(u) du \approx \frac{1}{N} \sum_{i}^{N} f(\vec{x}_i)$$
$$I(f) \approx \hat{I}(f; \vec{x}_1^N)$$

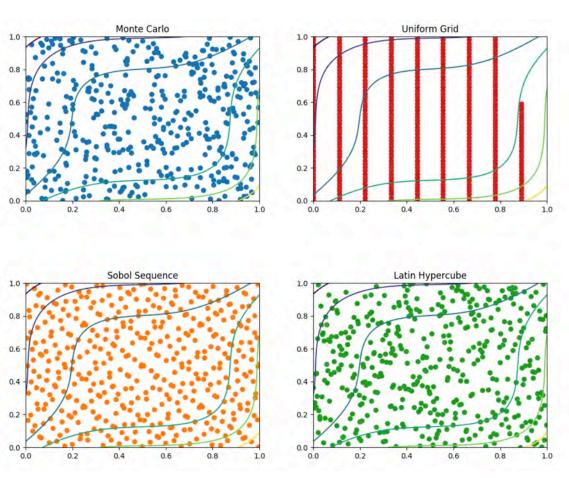
where,

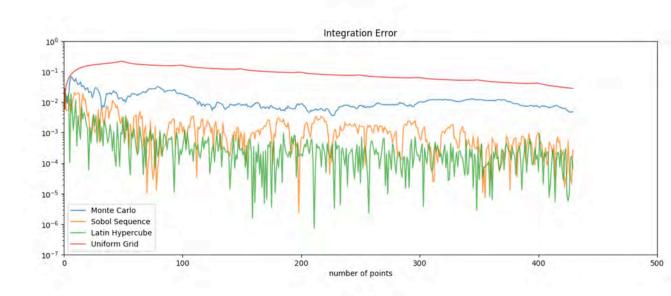
$$C^{d} = \prod_{j=1}^{d} [0, 1]$$

$$\overrightarrow{x_{1}}, \overrightarrow{x_{2}}, \dots, \overrightarrow{x_{N}} = P$$

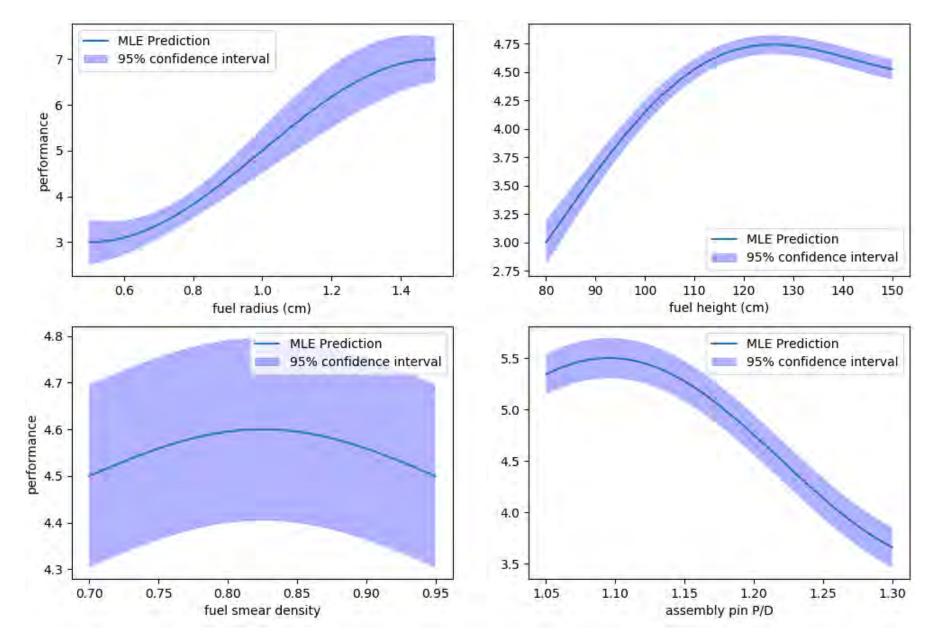
- What is the smallest P such that $|I(f) \hat{I}(f; P)|$ is minimized?
- "How well does our training data set P represent the important regions of f's domain?"
- Remember that each evaluation of $f(\vec{x_i})$ is **expensive**!

Choice of *P* matters!

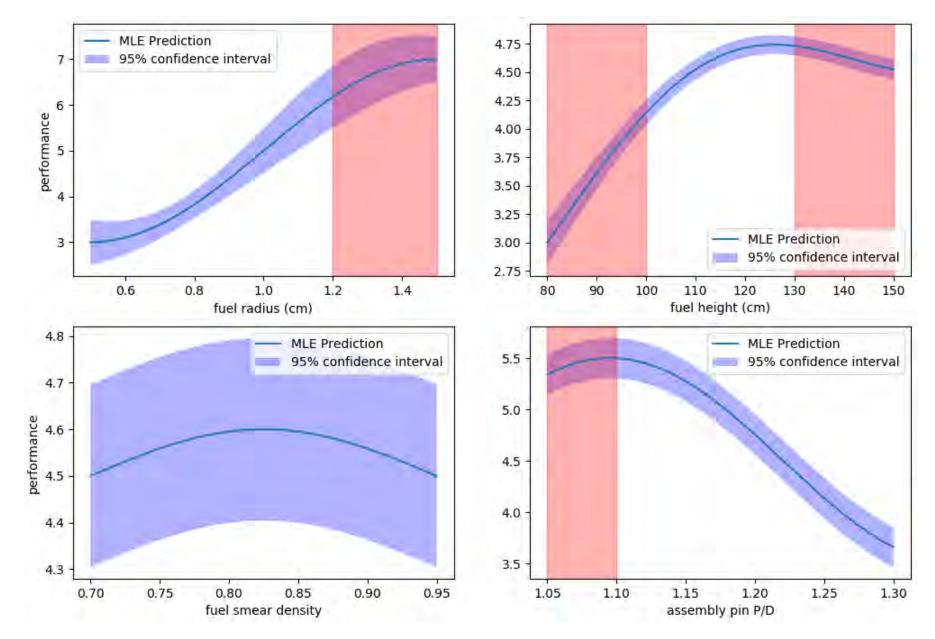




Sample partial derivatives



Sample partial derivatives



Thank you E



SENSOR T-044

GE Research

Industrial Digital Twins: GE Experience and Perspectives

Achalesh Pandey

Research Director- Al

Abhinav Saxena, PhD

Senior Scientist – Machine Learning

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Digital Twin

GE Experience

(ge)



CHANGES IN CUSTOMER DEMAND e.g. seasons, economy, ...



BLURRING COMPETITIVE MARKETS AND REGULATORY/ GOVERNMENT INFLUENCES

e.g. Policy, regulations, ...



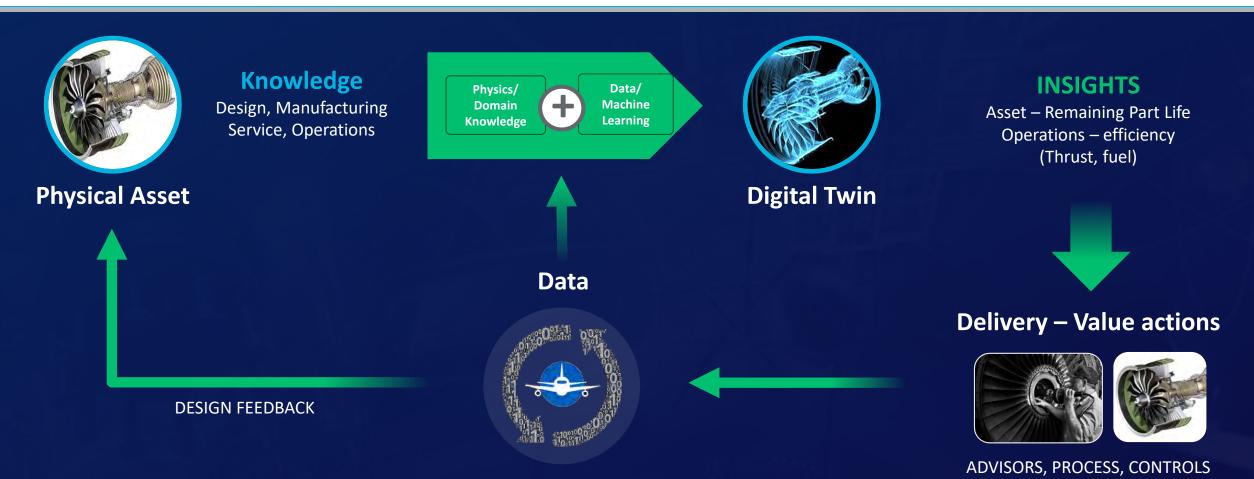
↑ DIGITAL CAPABILITIES @ LOWER COST

e.g. autonomous, tele, online, ...



Digital Twin

A personalized, living, learning, model



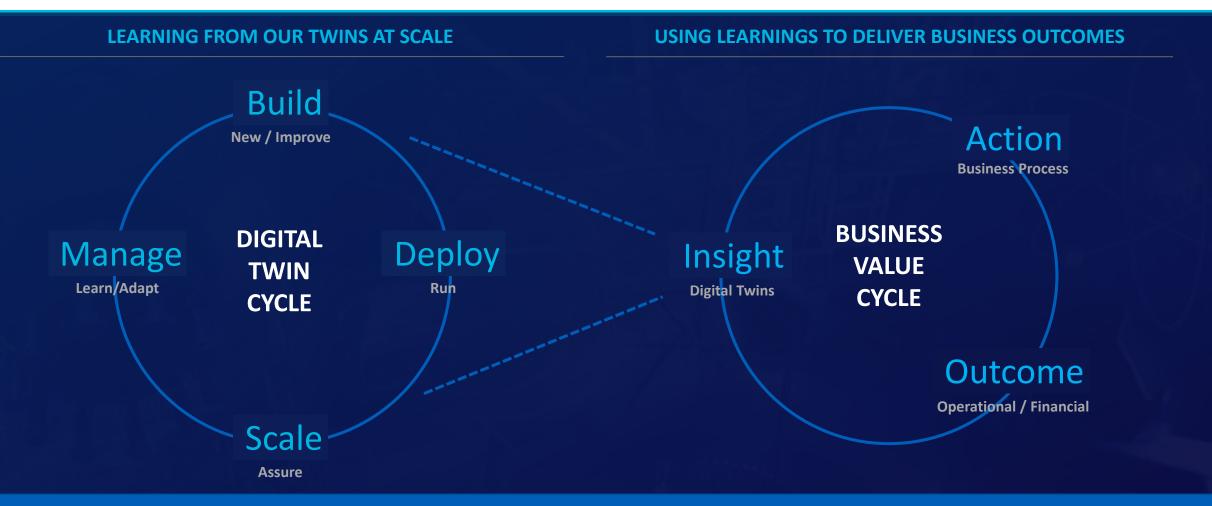
AI-BASED CONTINUAL LEARNING



CONTINUOUSLY IMPROVING BUSINESS OUTCOMES

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A framework for Adopting Digital Twins to Deliver Business Value



Continuous feedback loop to further advance Digital Twin capabilities



Aviation Customer Outcomes with Digital Twin

Sufficient Early Warning





DOMAIN

KNOWLEDGE

Compressor pressure, temperature, Exhaust gas temperature (EGT)

INDUSTRIAL DATA



BUSINESS OUTCOME



Predict Probability of Failure



Predict Compressor Issues

>30 days advanced warning

Continuous Prediction



Critical Engine Parts; TB Coating, EGT Environmental Parameters



Critical Engine Parts; Cumulative Damage



Optimized Inspection Schedule

>50% planned outage reduction \$10Ms saved per year

Dynamic Optimization



Fleet Engines, Routes, Environmental Data, CDM





Scenario Analysis and Optimization



Fleet Optimization

\$10Ms saved per year in lease costs



Energy Customer Outcomes with Digital Twin

Sufficient Early Warning



Condenser vacuum, Cooling water

temperature, ST MW



DOMAIN

KNOWLEDGE

INDUSTRIAL DATA



Fouling prediction, anomaly score



Increased lead detection time
11 days

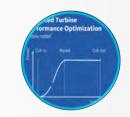
Continuous Prediction



Wind speed, temperature, turbine KW, speed, etc.



Pitch, TSR & Yaw optimal setpoints



ETPO - Performance optimization >1% AEP improvement \$ Millions saved per year

Dynamic Optimization



Generator load, frequency, transmission line flow (MW/MVAR), etc.



Load forecast, transmission constraints, economic dispatch



Higher grid utilization, lower cost >150GW footprint enabled \$ Millions saved per year



BUSINESS

OUTCOME

Healthcare Customer Outcomes with Digital Twin

Sufficient Early Warning





DOMAIN

KNOWLEDGE

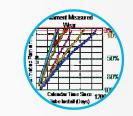
INDUSTRIAL DATA



CT Tube voltage, current, etc.



Predict probability of filament failure



BUSINESS OUTCOME

>5% decrease in overtime cost **\$100Ks** of savings per hospital system per year

Continuous Prediction



Brain MR Imaging, MR physics



Anatomy detection, optimal scan plane



Improved consistency, better image quality and reduced scan time (40-50%) **2X** throughput

Dynamic Optimization



Hospitals & Health Systems



Scenario Analysis and Optimization



45% workflow improvement, 2X growth \$ Millions saved per year



Digital Twin Example

Aviation Continuous Prediction Use Case

GE 90 HPT Stage-1 Shroud (Oxidation Failure) Harsh Environment Nominal Environment



S1S Oxidation Digital Twin (Physics + Data Driven)



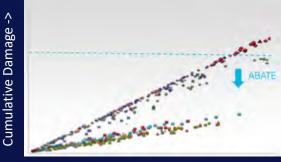


Operational & Environmental Variation



Environmental Data, Engine Operation Data, Configuration Data, City Pair Data, Design Data, Inspection Data

Analytics Based Removal



Cycles ->

On/Off- Wing Inspection

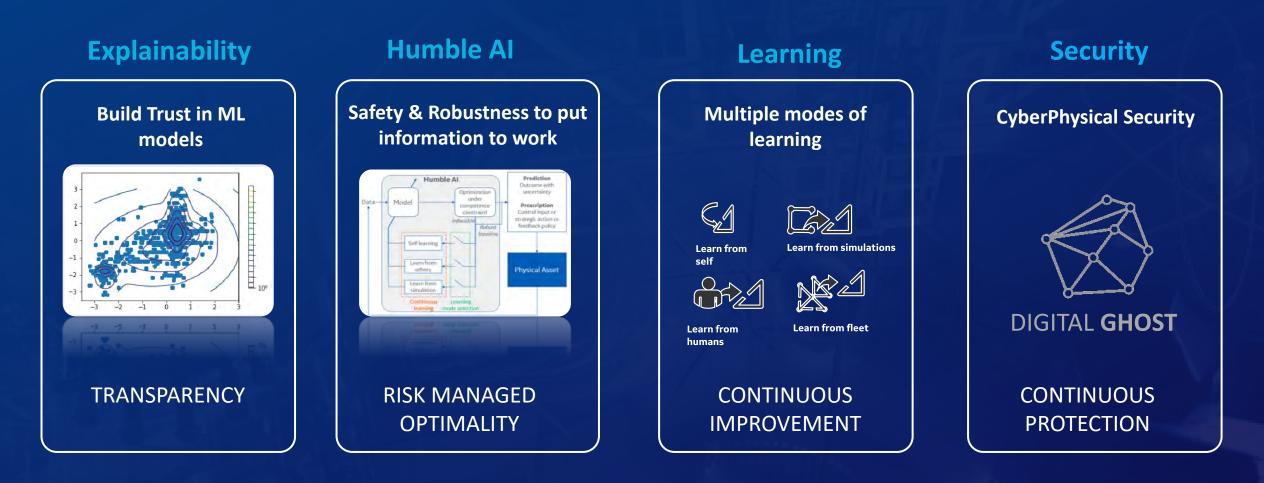


Feedback is used to continuously update Digital Twin Model



Research Focus: Advancing Digital Twin and Al

for Adoption and Scale



Realizing full value of data-driven analytics by putting information to action



General Electric Proprietary



Systems Level Digital Twin for Optimization

Understand and create fleet conditions for business flexibility

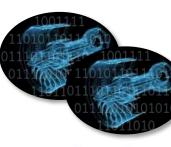
	ircrafts:	Left Engine Mounted:	Right Engine Mounted:
engine view aircraft view	Products (1) una 1923/07 (2) 1023 (2) 023 (2) 023 (2) 023 (2) 023 (2) 023 (2) 024 (2)		
nteraction		In Spares:	in Shop:





Engine life and performance, flight schedules, operator details - thrust, etc.

DIGITAL TWIN



MRO Service data - Simulated 'What-if' Futures: Operations, Spares/MRO TAT, Financials

VALUE OUTCOMES



Optimal engine assignment to routes, on-wing operations and MX



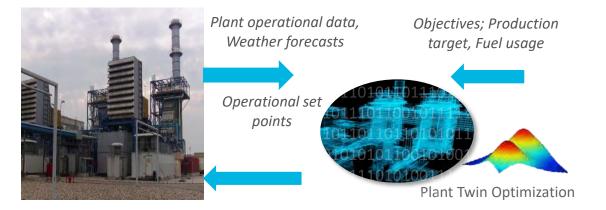
Realtime Optimization@ Edge

CCGT Plant Optimization





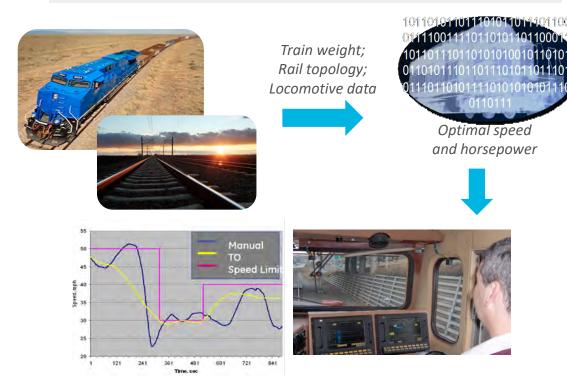
6FA Turbine Combined Cycle Plant



Continuously tuned combined cycle plant model

Automated Optimizer + Twin targeting > 1% fuel efficiency boost scalable across all turbine types

Evo Locomotive Trip Optimization



Minimize fuel consumption and emissions – generated per trip.

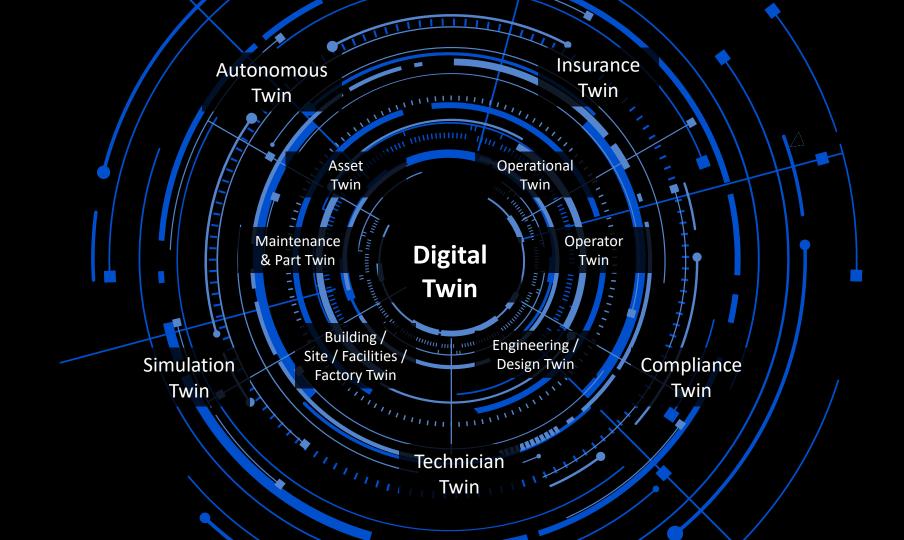
32K gallons / locomotive and 174K tons emissions decreased per year



IBM Digital Twin

Overcoming Digital Twin Data Scarcity and Accelerating the Journey to Predict

Presented By: Chandler Maskal Offering Manager, Digital Twin Exchange



Digital Twin pain points

Digital asset data availability Complex integration and lack of infrastructure and standardization Missing data and models to drive actionable insights leveraging AI

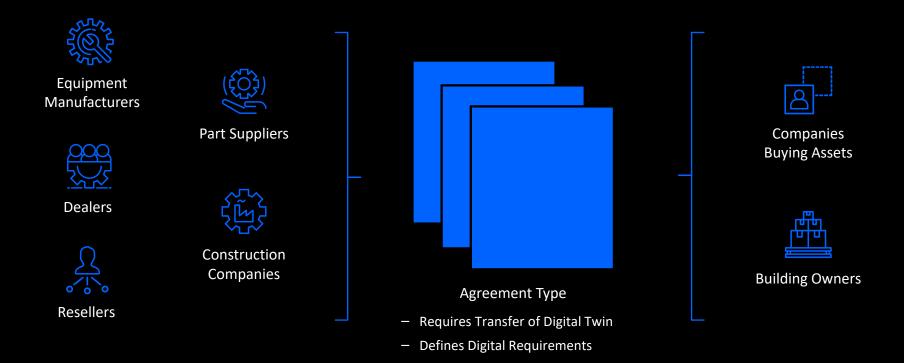
Energy & Utilities

Oil & Gas

Transportation

Manufacturing

The Solution: Strategic Digital Management



The Operational Performance Breakthrough: Strategic Digital Management

Short time-to-implementation

days and weeks

Rapid ROI

- A system that immediately begins to pay-for-itself

What's needed to drive "breakthrough" success:

Proven solution, team and team that:

- Has global deployment experience
- Is used to demanding customers demanding hard-dollar savings

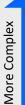
A strategic platform that will grow as technology proliferates and data grows

A focus on biggest data improvement paybacks first

A platform that drives your key strategies/ initiatives to effective action

Backed by consulting services and a digital factory to transform your business Solution Overview

Digital Twin Content





Bill of Materials

2D/3D CAD Files



Operations

Performance

Monitoring Model

AR/VR Models



Forecast & **Prediction Model**



Asset Monitor Dashboards



Asset Health **Scoring Methods**



Maintenance Plans



Remote procedures for the technician of the future



Fault Codes



Stocking Strategy

(BIM)

User/Engineering/ Maintenance Manuals



Parts List







Establishing digital twin standardization for easy deployment

Phase 1



Creation

_

Promote existing digital twins, formats and manual integration

Drive the creation of complex digital twins

Phase 2



_

Integration

Create automated integration capability to ingest digital twin data

Phase 3



Established Standards

Create format standard for use by OEMs and industry stakeholders aligned with industry specific use cases

Digital Twin Integrations – Spare Parts List into EAM Solution

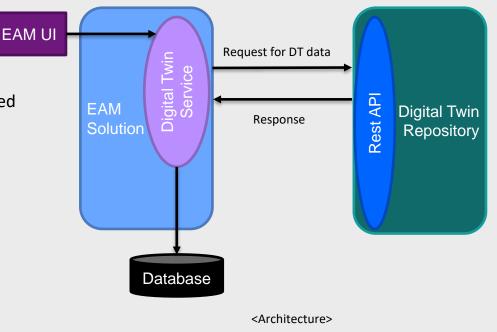
A repository of asset related digital data that are provided by various sources including the manufacturers is needed

Data objects created with the integration

- Item records with images
- The asset with Spare part records for the selected digital twin product with images and attachments
- Job plan and Job tasks with asset link record
- Json format

Custom Integration Mapping

- Additional fields can be mapped between EAM platform and digital twin repository
- An automation script can be used to customize mapping for an integration point in the integration.

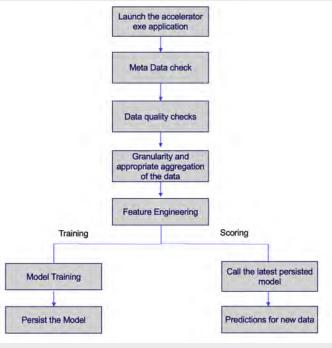


AI/ML Accelerators – Enabled Accelerator for Air Compressors

Accelerators analyze parametric data from assets like air compressors and predicts imminent failures thereby helping plant maintenance engineers/technicians plan the maintenance activity without disrupting plants operations

Air compressor downtime impeller failure/breakdown is critical, the event could impact the operations of a plant for days

- Generalized to work with air compressors that have an impeller Rotary and Centrifugal air compressors
- Can predict impeller failure/breakdown
- The accelerator is based on the experiences with several customers across industries from air compressor manufacturers to industries that heavily rely on availability of air compressors for their day to day operations
- Validated with multiple datasets from Compressors deployed in different geographies. The analytic model predicts the time to failure of the compressor (in days)



Architecture

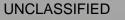


Pacing Optimization Enabled by Human Thermoregulatory System Digital Twin

Mark Buller, PhD U.S. Army Research Institute of Environmental Medicine

December 2020

The opinions or assertions contained herein are the private views of the author(s) and are not to be construed as official or as reflecting the views of the Army or the Department of Defense.





Motivation: Why is Pacing Important



BBC Search Weather More Sport News Gold Coast SPORT 2018 Commonwealth Games 📃 All Sport Home Football Formula 1 Cricket Rugby U Commonwealth Games > Results | Schedule | Medals | More • **Commonwealth Games: Brave Callum** Hawkins needed taking out of firing line By Tom English **BBC Scotland on the Gold Coast** O 15 April 2018 Commonwealth Games

© British Broadcasting Corporation 2018

"Hawkins made his move in the race around Mermaid Beach, 20km into the 42km Commonwealth Games marathon. Despite the stifling heat that would hit <u>29</u> <u>degrees</u> and that would cause such distress later on, the Scot began to turn it on and ease clear."

60% Humidity: Full Sun

Remember Callum Hawkins is from Scotland (not too warm or humid)

"His <u>lead was approaching two minutes</u> with only seven kilometres to run. ... *[His coach]* was excited but concerned. There was something about Hawkins' running style that perturbed him. There was a <u>little wobble</u> that he'd spotted that few others had spotted."

"I just wanted to keep pushing. I probably wasn't in the right mind to make the decision."

– Callum Hawkins







- Timed 8 mile road march
 - US Army Ranger Students
 - Part of a week-long series of pass/fail events
 - Must complete in 2 hrs. 10 min. or <u>fail class</u>
 - Carried ~32 kg (70 lb)
 - Warm (25 °C) and Humid (85% RH)
- Thermal illness risk elevated
- Students selected their own pacing strategy





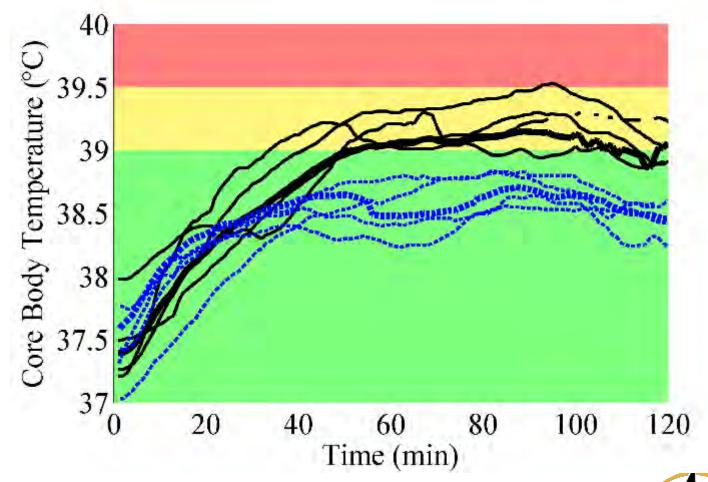






Ranger 8 mile timed Ruck March

Hot group, Warm group No difference in: Height, Weight, Fitness, Completion Time, Load Only know difference: Pacing Strategy



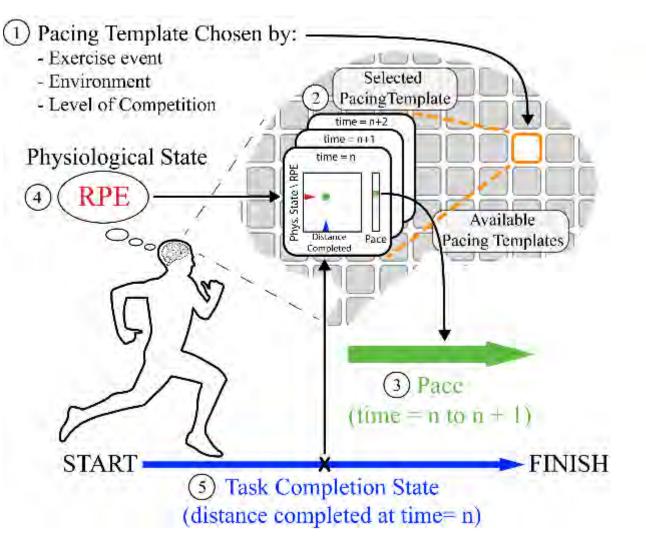






Black Box, Expert Learned Pacing Templates: Tucker (2009)

Athletes, use distance to goal, time remaining, and current RPE to optimally pace a race.

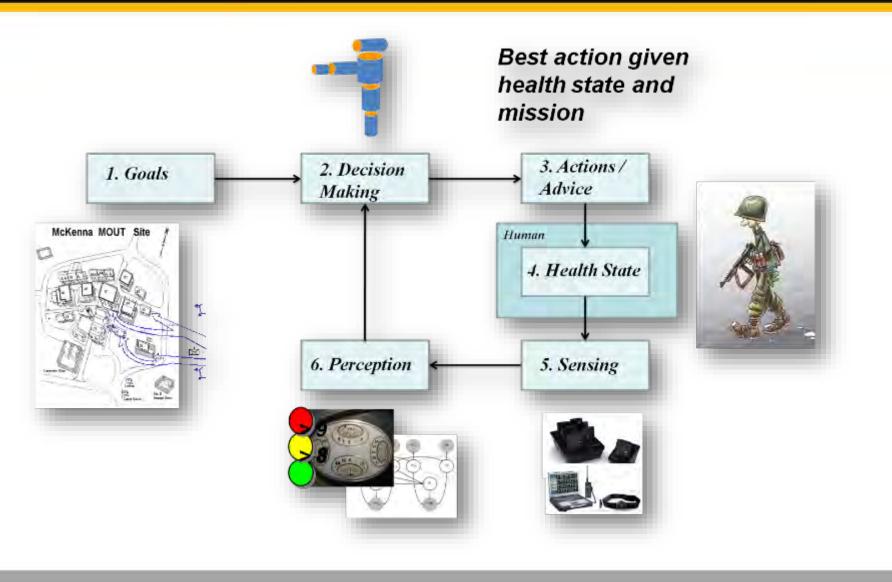






Pacing Control System

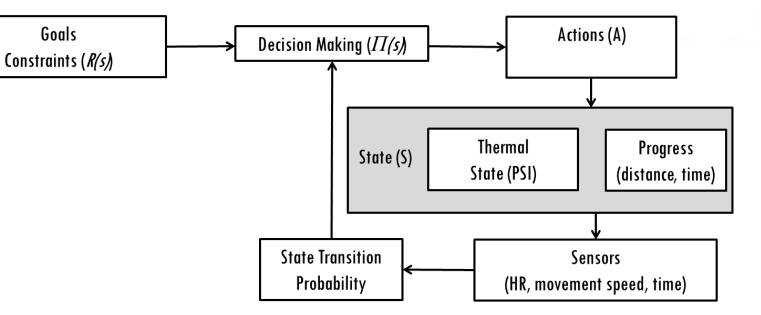








- State (S) :
 - Physiological Strain Index (PSI), distance remaining, time
 - PSI calculated from HR and estimated core temperature
- Actions (A) :
 - movement speed
- State Transition probability :
 - *P*(S'| *A*, *S*)
 - Likelihood of transitioning to a state (i.e., PSI value, distance, time) given current state and an action.
- Reward function *R*(*s*) :
 - Describes goals and constraints using "points"



$$PSI = 5\left(\frac{CT_t - CT_{rest}}{39.5 - CT_{rest}}\right) + 5\left(\frac{HR_t - HR_{rest}}{180 - HR_{rest}}\right)$$

PSI	
<5	Low
5-6	Moderate
7-8	High
9-10	Very High
>10	Extreme





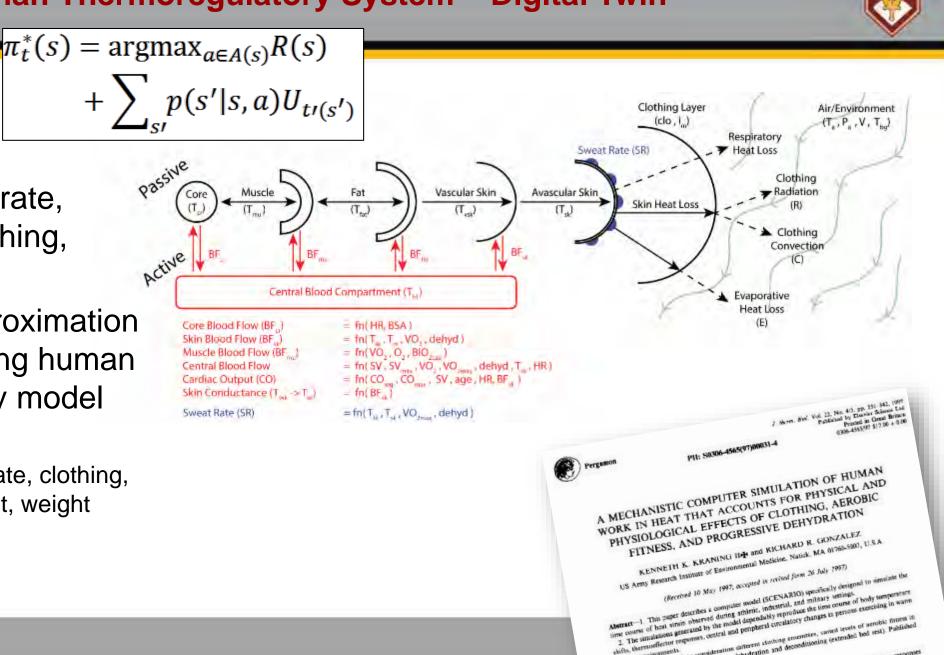
Human Thermoregulatory System – Digital Twin

Passive

Active

 $\geq P(PSI' | PSI, A)$

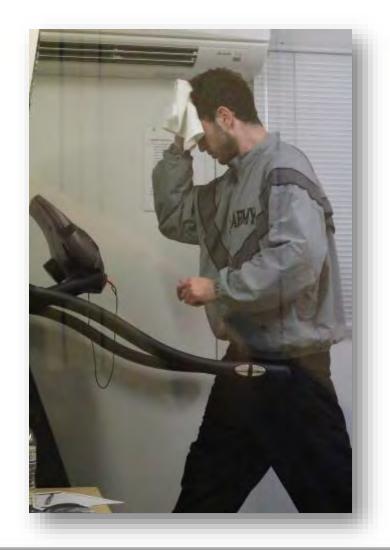
- > Non-trivial
- \succ Affected by work rate, environment, clothing, individual
- > Monte Carlo approximation (10,000 iters) using human thermo-regulatory model (SCENARIO)
 - \succ Inputs: metabolic rate, clothing, environment, height, weight





Real-Time Human Laboratory Experiment

- 16 Volunteers (11 M, 5 F)
 - » Fit: Able to run 3.2 km (2 miles) in 16 minutes
 - » Age 23 \pm 3 yrs., Wt. 67 \pm 9 kg
 - » Collected: HR, T_{SK} , T_{CR} , VO₂, movement speed
- Two trials:
 - SELF-PACED/UNGUIDED (1st trial) Instructions:
 - Complete 8.04 km (5 miles) in 60 minutes
 - Don't get too hot
 - Complete the 60 minutes as cool as possible
 - GUIDED (2nd Trial) Instructions:
 - Follow MDP application guidance as best you can
 - App designed to stop participants if their projected PSI was predicted to create to large a penalty
 - Environment:
 - 22 °C, 50% RH, minimal air movement
 - Hot Clothing:
 - U.S. Army Winter tracksuit (nylon)



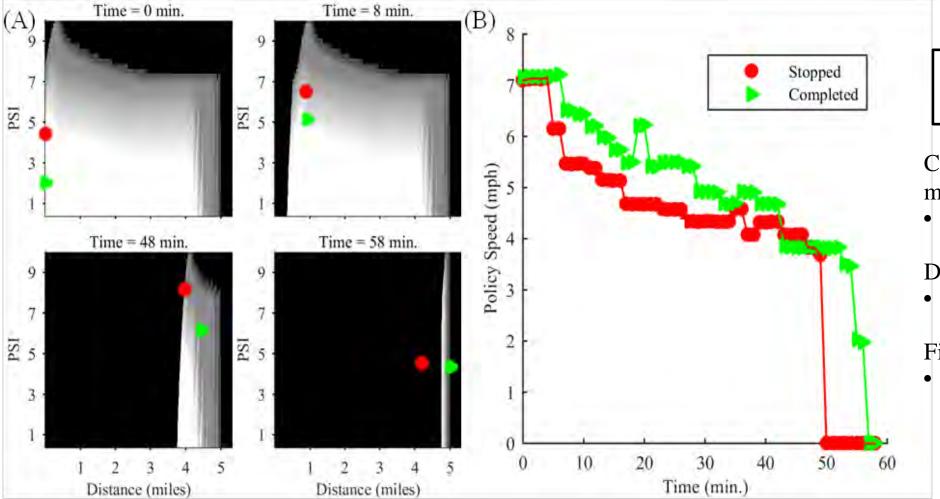




UNCLASSIFIED

Pacing Templates - No Longer a Black Box





Goals Constraints (*R(s)*)

Complete 8.04 km (5 miles) in 60 mins

• heavy penalty/negative points for failure

Don't get too hot "Safety"

• Heavy penalty/negative points for exceeding PSI of 7.5

Finish with a low PSI "End PSI"

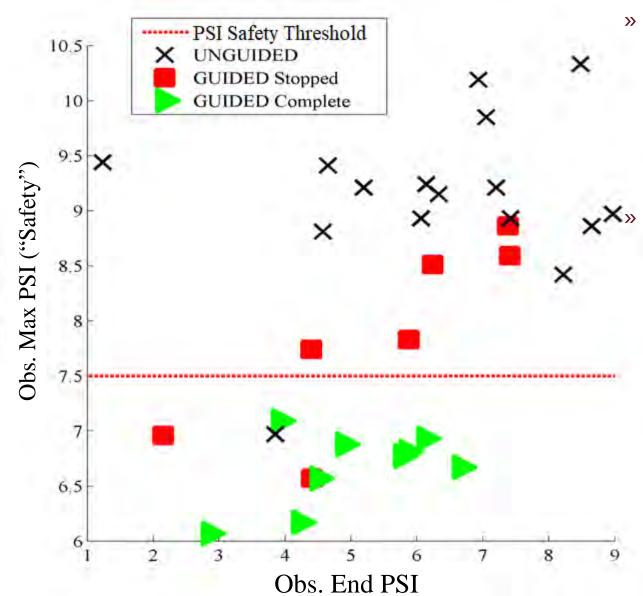
• The lower the ending PSI the more "bonus" points.





Results





» UNGUIDED (Self Paced)

- » 15 completed in 60 min
- » 1 stopped as they got too hot
- » GUIDED (MDP Policy)
 - » 9 guided to completion
 - » 7 guided to stop

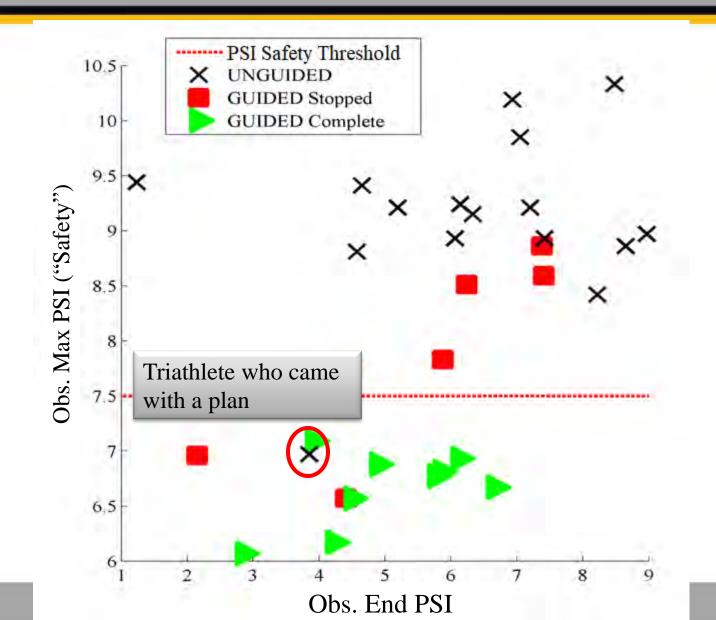
PSI	
<5	Low
5-6	Moderate
7-8	High
9-10	Very High





Results





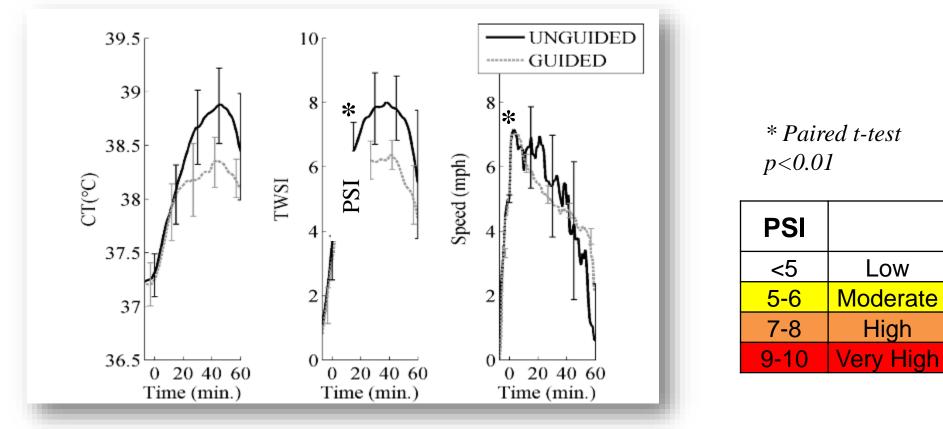
PSI	
<5	Low
5-6	Moderate
7-8	High
9-10	Very High







- » T_{CR} and PSI (max and mean) significantly lower when GUIDED
- » Total Energy Expenditure: GUIDED = 599 ± 84 W, UNGUIDED = 617±104 W





†1 subject dropped as they completed an old policy computed from incorrect transition probabilities





- Guidance effective at reducing PSI on novel task
- Only triathlete with a plan matched guidance policy performance
- Digital Twin shed light on a black box control surface
- Physiological feedback allowed machine pacing to the same level as an expert on a novel task



Distributed Fiber Sensor Enabled Digital Twin Modeling

Kevin P. Chen

pec9@pitt.edu

Department of Electrical and Computer Engineering University of Pittsburgh December 2, 2020

Digital Twin Applications for Advanced Nuclear Technologies Workshop



Computer Modeling can generate high resolution Digital Twin Models but...

• Are they real?

Spatial/Temporal Resolutions of Digital Twin Modeling Scale with Moore's Law...

• Is measurement devices scalable in similar fashion?





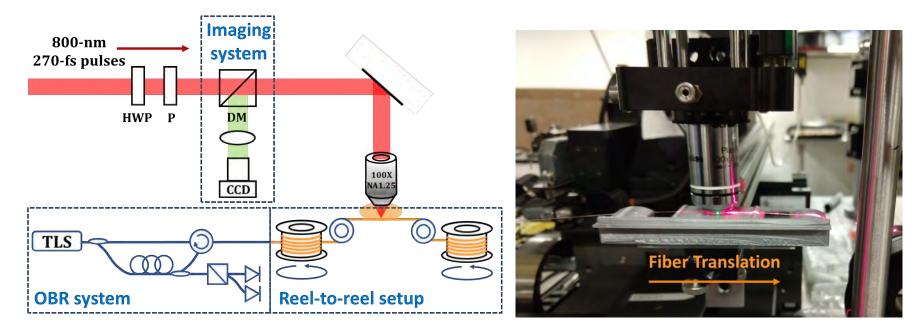


- Developing distributed fiber sensor technology to harness high spatial resolution data for NE applications
- Provide direct measurements to support the development of highly accurate Digital Twin (DT) models.
- Using sensor data to shed light on design optimization
 - High spatial resolution temperature measurements in reactor core.
 - High spatial resolution monitoring of solid oxide fuel cell
 - High resolution sensor data enabled DT modeling of additive manufacturing





Femtosecond Laser Fabrication for Sensor for NE Applications



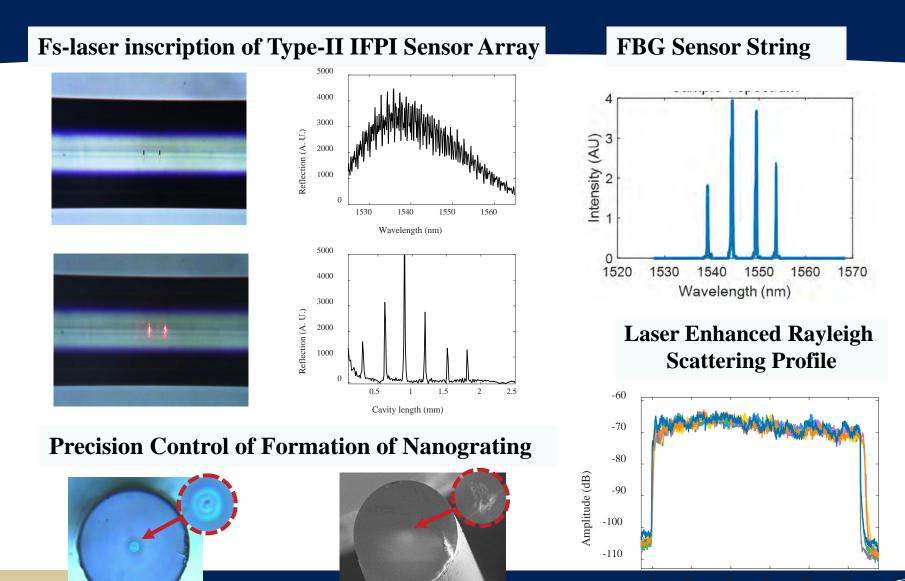
Reel-to-reel oil-immersion fiber writing setup

- Fast and simple fabrication of meters of fiber
- Can inscribe sensors on meters of optical fibers
- Real-time monitoring using an Optical Backscattering Reflectometer (OBR)





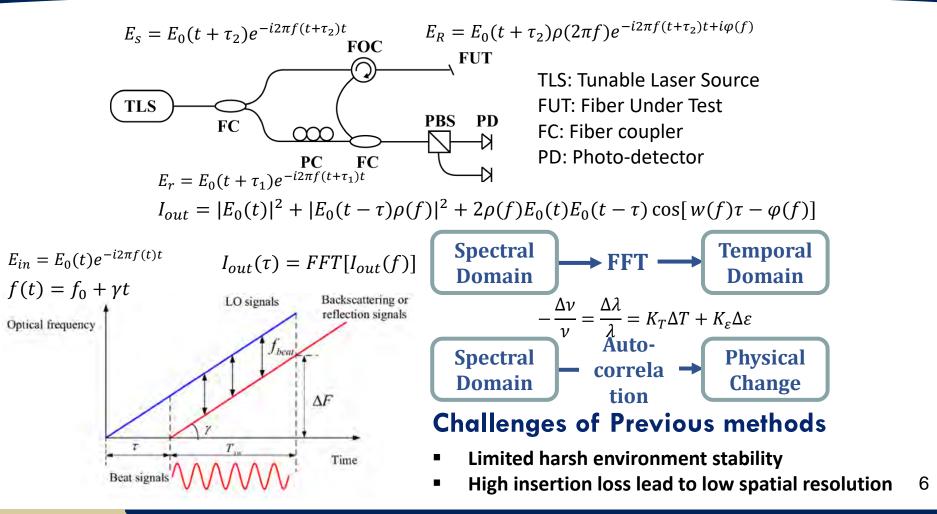
Reel-to-Reel Sensor Fabrications



Length (cm)

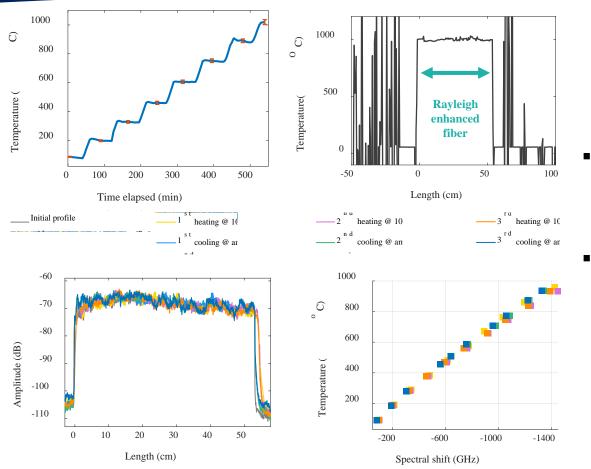


Optical Frequency Domain Reflectometry





Temperature Sensing Performances



- After 16 hours under 1000°C, the processed fiber section still functional compared to the unmodified fiber
- Robust and consistent operation after repeated heating and cooling cycles

M. Wang, et al. "Reel-to-reel Fabrication of In-fiber Low-loss and High-temperature Stable Rayleigh Scattering Centers for Distributed Sensing," IEEE Sensors Journal (2020).

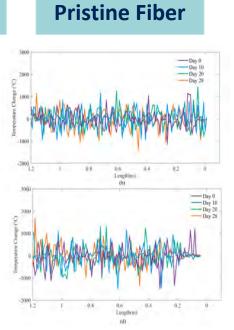


First High Spatial Resolution Temperature Profile Measurements of a Reactor Core

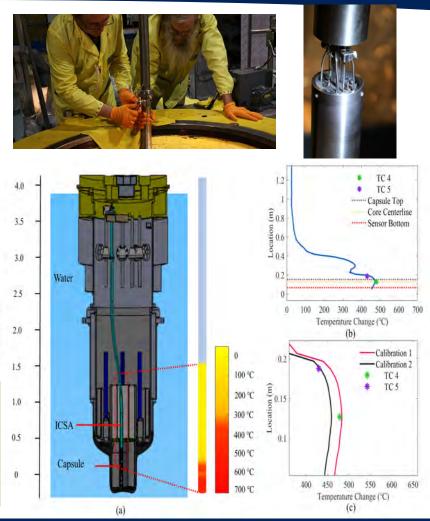
Location (m)

3500 Day 10 Day 20 Day 28 3000 2500 2000 1500 1000 14 12 1 08 0.6 0.4 0.2 Length (m) 14 0.8 0.6 0.4 0.2 Length(m) Day 10 Day 20 3000 2 2500 2000 1500 0.5 Lenith ini 1.4 1.2 1 0.8 0.6 0.4 0.2 Length(m) (c)

Laser Enhanced Fibers

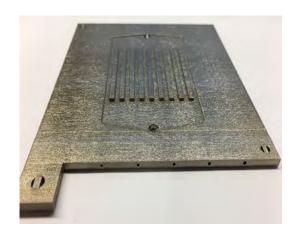


- In-pile measurements at (560°C, 1.4×10¹⁴ f n/s/cm²)
- 1.6-m core profile, 3-cm resolution, 1-s update rate.
- Laser enhancement critical

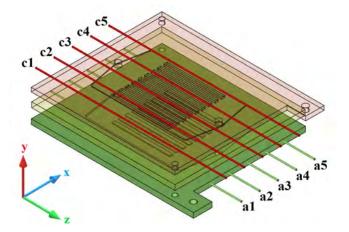


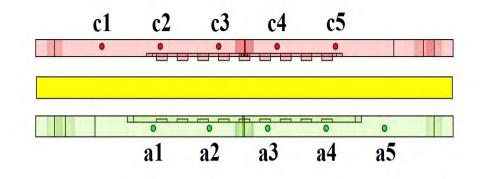


In-Vivo Monitor of SOFC and DT Modeling





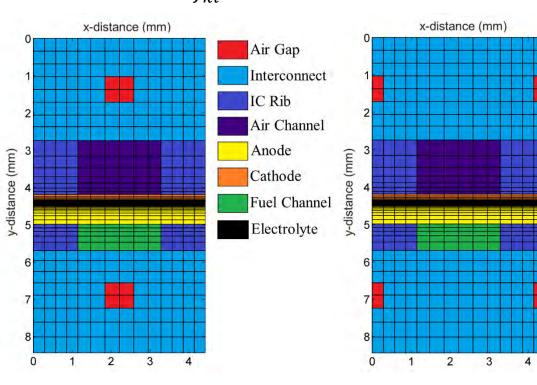


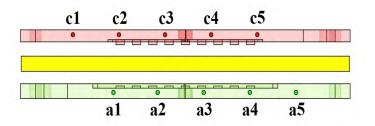




Digital Twin Modeling

• $\frac{\partial}{\partial t}(\varepsilon_k\rho_k\phi_k) + \nabla \cdot (\varepsilon_k\rho_k\vec{u}^{eff}\phi_k) = \nabla \cdot (\varepsilon_k\Gamma_{\phi}^{eff}\nabla\phi_k) + \varepsilon_k\rho_kS_{\phi} + \varepsilon_k\rho_kS_{\phi}$ f_{kl}





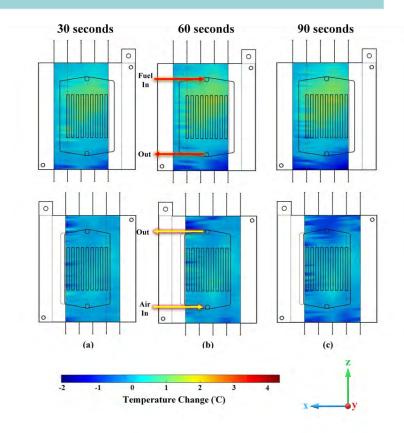
- 2D modeling using transport model
- Temperature

- Charge transport
- Fuel consumption rate
- Direct comparison with measurements

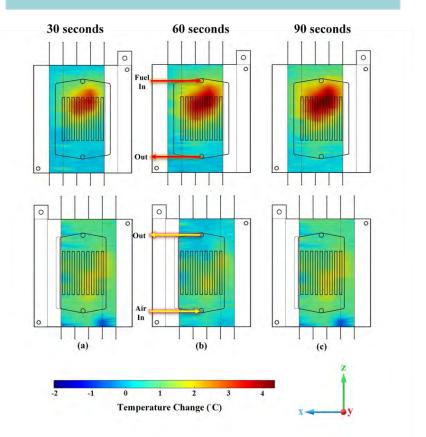


Sensor Measurements

750C, 100 sccm, 10% H2, 1A



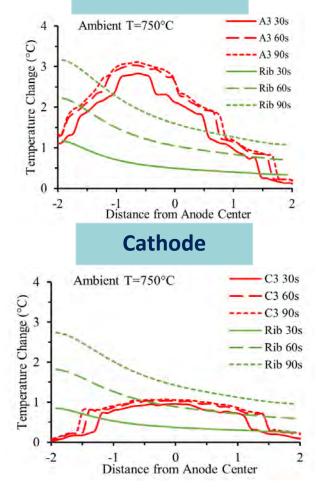
750C, 100 sccm, 50% H2, 2A

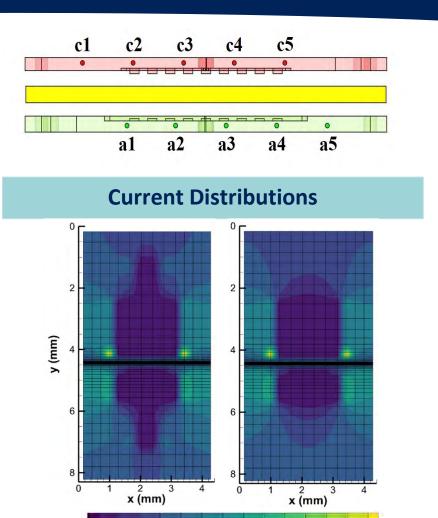




DT-Experiment Comparison

Anode





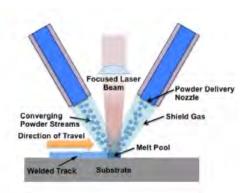
0 0.05 0.1 0.15 0.2 0.25 0.3 0.35 0.4 0.45 0.5 0.55 0.6 0.65

Current Density (A/cm²)



DT for Additive Manufacturing

Power Bed Based Additive Manufacturing



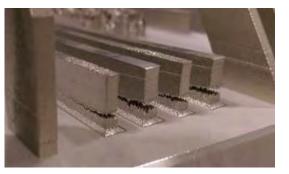


Can we develop an accurate DT models to predict, design, and optimize AM process for engineering

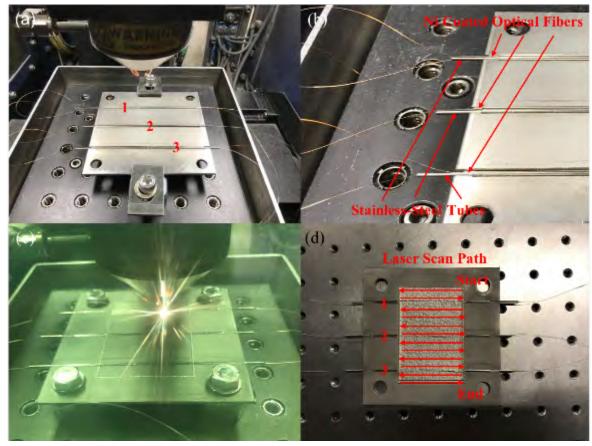
- High-T AM process are for NE
- Residual stress is a key road block
 - Corrosion performance
 - Manufacturing accuracy
 - High-term stability
- DT can be critical to design optimize AM process to minimize residual stress
- How to validate our DT model???







Sensor-Fused AM Process

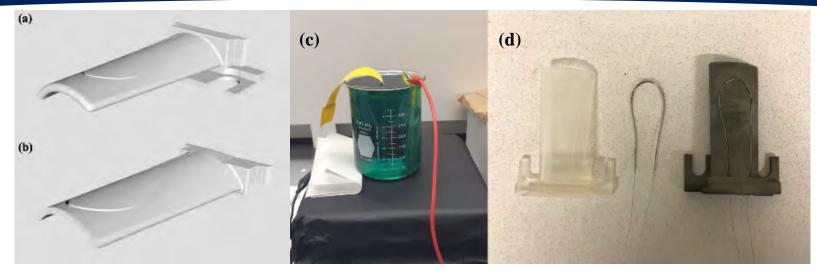


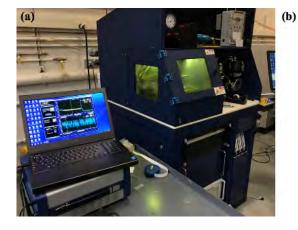
Sensor Fused AM Process

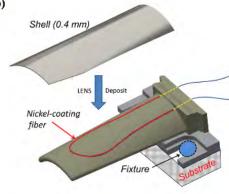
- High resolution real-time T & με measurements
- Design proper structures to embed sensors without disturbing AM process and part itself
- Real-time measurements to study AM process itself
- Post-process monitoring to study residual strain formation and relaxation.
- Compare, correct, and validate DT

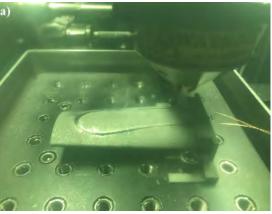


Embedding Fiber Sensor on Turbine Blade



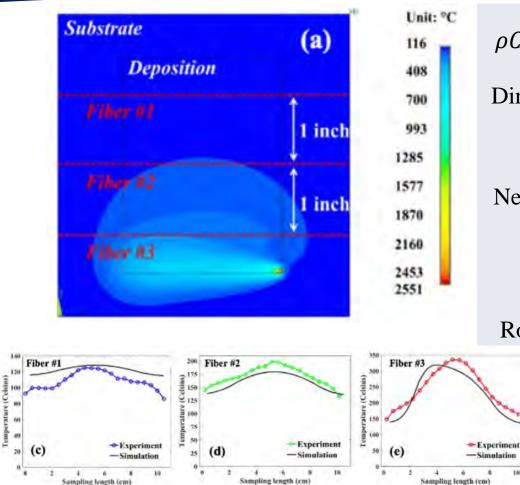








Temperature Modeling vs. Measurement



$$\rho C_p \frac{dT(\boldsymbol{r},t)}{dt} = -\nabla \cdot \boldsymbol{q}(\boldsymbol{r},t) + Q(\boldsymbol{r},t), \boldsymbol{r} \in \boldsymbol{V}$$

Dirichlet boundary conditions

$$T = \overline{T}, \qquad r \in S_D^T$$

Neumann boundary conditions:

$$-k\nabla T \cdot \mathbf{n} = \overline{q}, \ \mathbf{r} \in S_N^T$$

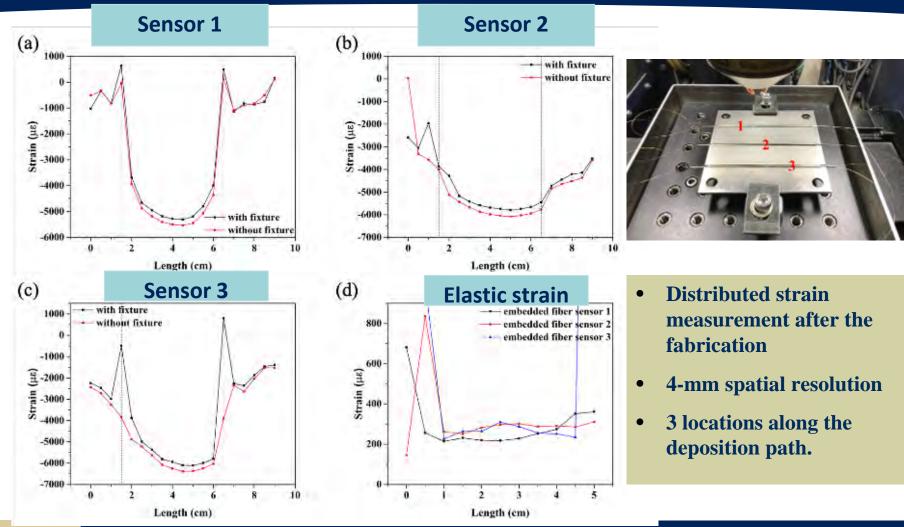
Robin boundary conditions:

18

$$-k\nabla T \cdot \boldsymbol{n} = h(T - T_a), \boldsymbol{r} \in S_R^T$$

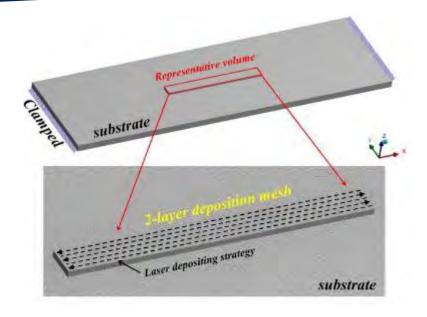


Elastic Strain vs Plastic Strain





Strain Modeling vs. Measurements



- 1000 +-Simulation 0 ⊖-Fiber #1 Fiber #2 -1000▲ Fiber #3 Plastic Strain (µE) -2000-3000 -4000 -5000 -6000 -7000 -80002 3 Sampling length (cm)
- Two-layer deposition (red box) as the representative volume of the large deposition area (upper).
- Laser scanning strategy in the laser fusion process (lower) Modelled.
- Measurements performed at 4-mm spatial resolution.
- Simulation consistent with modeling, provide confidence for model-based optimization.



Summary: Sensor-Fused DT Modeling

- Radiation-harden and high-T stable distributed fiber sensors provide mean to harness high spatial/temporal resolution data
- Providing direct measurements to support the development of highly accurate Digital Twin (DT) model.
- Sensor-fused DT modeling useful for several DT scenarios
 - Validate and provide accurate boundary conditions
 - > Test DT model developed for extreme environments
 - Provide most relevant evidence for DT-based design optimization
 - > ... many more



Thank you!

Email: pec9@pitt.edu



ARPA-e perspective: Digital twins as an enabler of low O&M costs

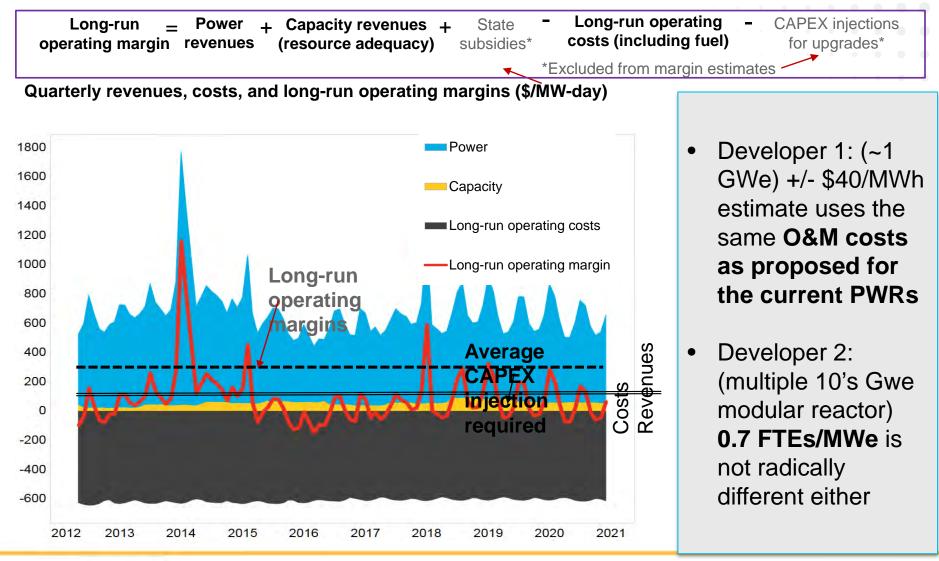
Or: If You Build It, Can You Run It?

Joel Fetter, Adviser to ARPA-e

Workshop on Digital Twin Applications for Advanced Nuclear Technologies

December 2020

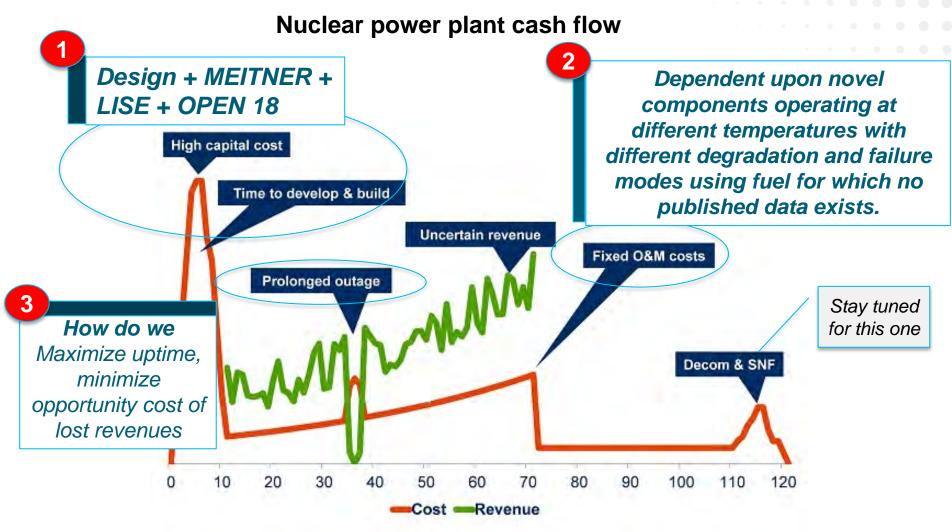
Provocation #1: LWR O&M costs are high: What are AR developers doing to reduce O&M costs?



Source: Bloomberg New Energy Finance, EIA, FERC Form 1 Note: Capital expenditures for uprates, enhancements, or regulatory compliance are not included. These CAPEX injections averaged almost \$6.74/MWh in 2016, or approximately \$150/MW-day, according to the Nuclear Energy Institute material and the second seco

CHANGING WHAT'S POSSIBLE

Provocation #2: Pro-forma financials need better data inputs – what really is the cost?

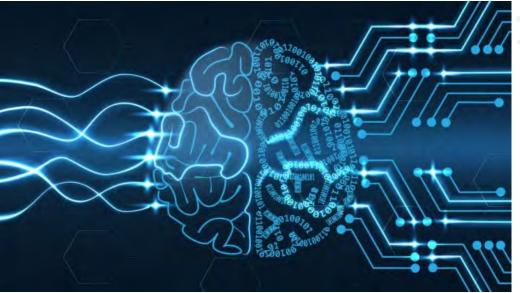




Graphic: https://nuclear-economics.com/5-revenue-certainty/

Provocation #3: The designs are maturing, but still optimizable for O&M

- Lots of industries are developing better controls, better models, better data, better algorithms
- Focus on autonomy and machine learning (ML) is getting many questions answered
- Answer those questions specific for nuclear and prove out ideas in our systems and with our softw



systems and with our software; aid in code validation

Have tools the industry and the regulator can use



Portfolio-at-a-glance

Funded programs

- BWRX 300
 - GE research
 - MIT / GE, focus on Thermalhydraulics
- Kairos
 - Argonne National Lab (sensors)
 - University of Michigan
- X-energy: O&M for XE-100
- Moltex USA: O&M for SSR
- Framatome (Metroscope)
- EPRI: Optimal component design life
- MIT: Irradiation data for MSRs
- (other portfolios) Southern Research (Robotics), NC State (construction)

Outcomes we're striving for

Digital twins of emerging advanced nuclear reactors (AR) that:

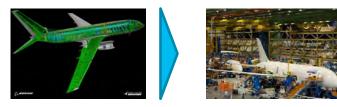
- Enable <u>flexible and economic operation</u> for the meeting needs of the future grid
- <u>Provide foundational insight</u> into the implications of new operating paradigms on proposed AR designs – early enough to modify them inexpensively
- Improve the <u>safety</u>, <u>reliability</u>, and <u>cost</u> <u>effectiveness</u> of emerging designs
- Provide <u>data-driven, design-specific foundations</u> for AR staffing and cost profiles
- <u>Support private markets</u> to identify and finance lowest cost, highest value designs
- <u>Accelerate</u> what are now extended and costly regulatory approval processes.



Parting thought: program takes inspiration from aerospace

Design

Digital design accelerated the 777 development process



Boeing 777: First new Boeing aircraft in 10 years:

- First jetliner 100% digitally designed
- Pre-assembly done digitally, eliminating need for costly preproduction mock-ups
- Five years from project launch to production, and 8 years from launch to commercial flight

Operational Simulation

Simulators support design of new aircraft, pilot training,



Based on flight equations and matches with physical data, simulators:

- Provide developers with insights into aircraft design trade-offs.
- Teach pilots especially in advanced aircraft – how to assimilate data from new/unique systems (e.g. F22 Raptor)
- Offer "Extended envelope" training teaches pilots, regulators how systems perform in extreme conditions.



What is Possible Now That Wasn't Before?

From 1985: "Future nuclear power plants are projected to be highly modularized, with the possibility of several plants being operated from one control room. AI-based diagnostics and control systems can make this possible by taking over the mundane day to day oversight tasks that the operators must perform to keep the plant running."





U.S. Department of Energy Assistant Secretary for Nuclear Energy Technology Transfer Artificial Intelligence Task Team. "Artificial Intelligence and Nuclear Power". June, 1985, p. 10

GE Research



Abhinav Saxena, PhD Senior Scientist – Machine Learning PI - GEMINA Award 2174-1511

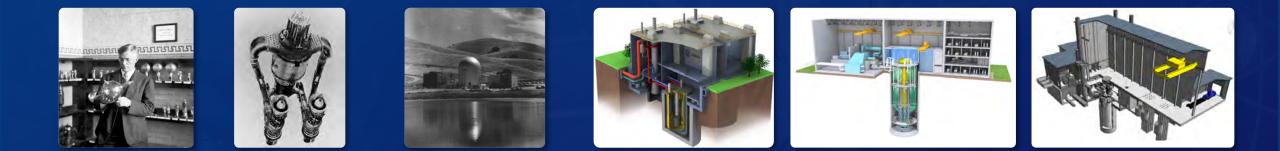
BWRX300 GE Hitachi Nuclear



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Decades of New Plant Innovation

Rich history of nuclear innovation ready to support advanced reactor market



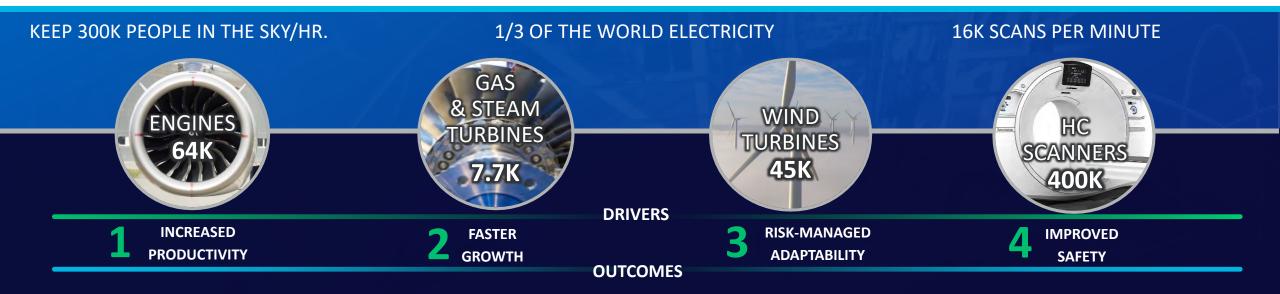
OVER 80 YEARS OF NUCLEAR EXPERIENCE AND INNOVATION

1939	1951	1955	1957	1981	1996	2014	2017	2018	
First GE involvement in nuclear physics	Aircraft nuclear propulsion	GE Atomic Division established	Vallecitos BWR AEC License #1	PRISM development commences	1 st ABWR built on time on budget	ESBWR NRC License	BWRX-300 launched	VTR Contract PRISM	



Digital Twins

GE Experience





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Digital Twins

GE Experience

KEEP 300K PEOPLE IN THE SKY/HR.

1/3 OF THE WORLD ELECTRICITY

16K SCANS PER MINUTE

Parts Twins Rotor failure prediction

> **Product Twins** Steam turbine performance

> > **Process Twins** Field engineer scheduling

Industrial Digital Twins



System Twins Performance optimizer



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Predictive Maintenance Digital Twins

Summarv

Team	Program Impact			
GE Research & GE Hitachi			maintenance to ↓ O&M labor costs /MWh in an Advanced Nuclear Reactor	BWRX300 GE Hitachi Nuclear
E SE	Program Targets			
UT Knoxville	Metric	From	То	MATION
	Automation ↓ labor costs	None	Automated workorders \downarrow Planning staff Online calibration \downarrow Tech and admin staff	AUTOMATION Design
Oak Ridge Natl. Lab	Predictive Maintenance ↓ labor & mat'l	Alarms	\downarrow Forced outages & trips AI-driven predictive algorithms $\rightarrow \downarrow$ Labor headcount	Deploy Analysis
	Trust	Human	Humble & explainable AI quantify uncertainty to establish trust in the models & encourage automation	Digital Twins Robustness of AI Methods
Exelon	Technology Summ	ary		
Energy Corp.	 Reactor Healt 	h – Causal,	vsics-informed machine learning, sensor optimization humble & explainable AI for predictive maintenance nomous risk-informed decisions for reconfiguration & maintenence	

Al-based predictive maintenance for lower O&M costs



- 10th generation BWR
- 300 MWe SMR
- World class safety
- Targeting LCOE competitive with gas
- Significant capital cost reduction per MW
- Scaled from licensed ESBWR
- Design-to-cost approach: targeting <\$1B total and <\$2,250/kW for NOAK
- Capable of load following
- Ideal for industrial applications ... district heating, desalinization and process heat
- Constructability integrated into design
- Initiated licensing in the U.S. and Canada

Operational by 2027

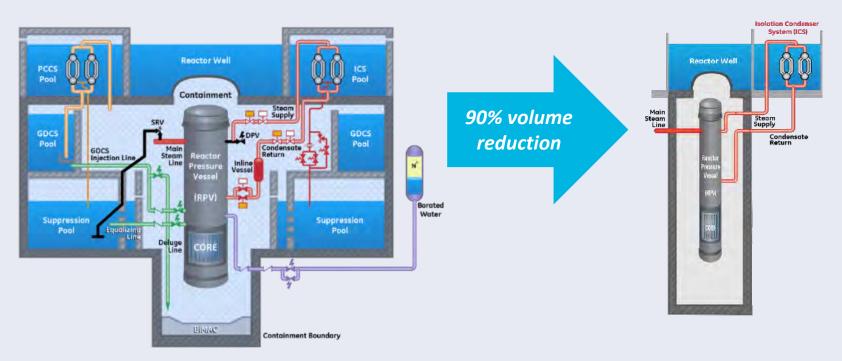






BWRX-300 Design Simplification

ESBWR



BWR<mark>X</mark>300

LOCA mitigation enables dramatic simplification

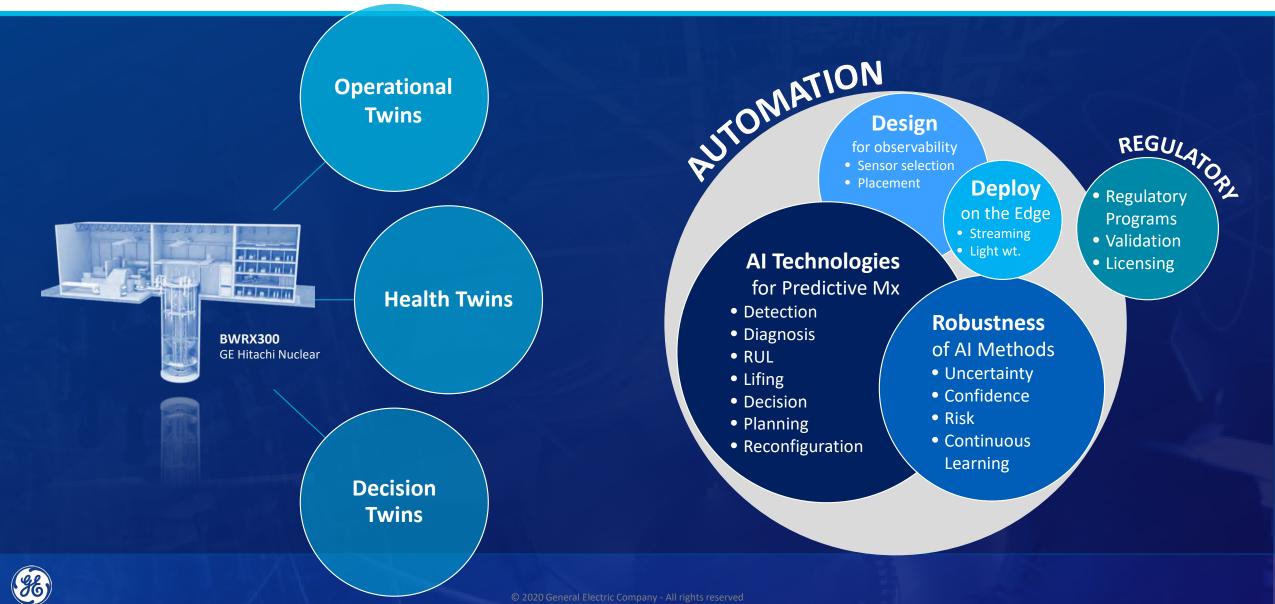
- LTR submitted to NRC and accepted for review
- Engineering design work confirmed 90% reduction in concrete vs. ESBWR

>50% building volume reduction/MW
>50% less concrete/MW



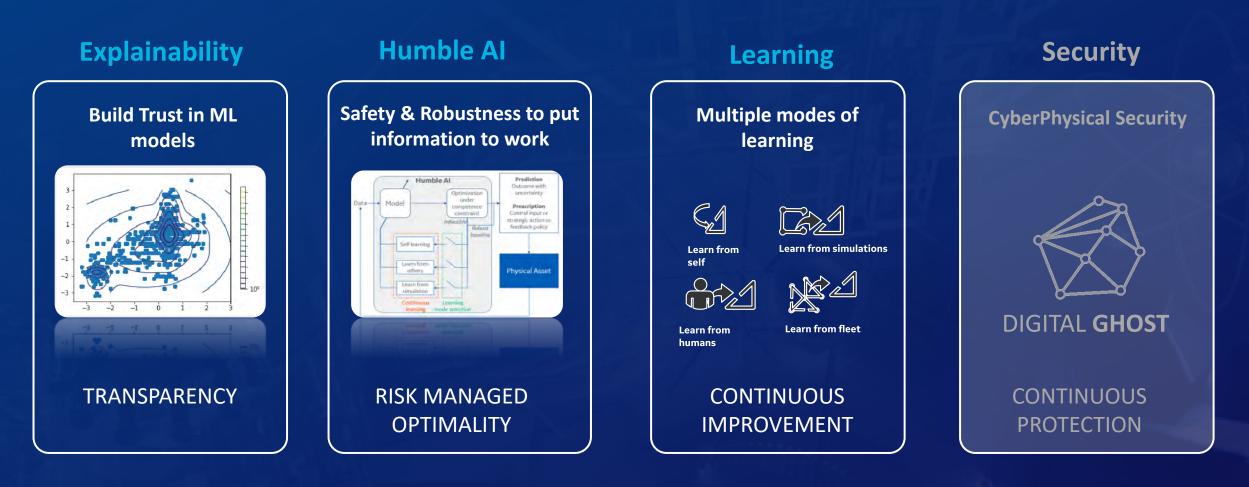
Predictive Maintenance Digital Twins (PMDT)

Technical Approach



Research Focus: Advancing Digital Twin and Al

for Adoption and Scale



Realizing full value of data-driven analytics by putting information to action



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Project Outcomes

GEMINA PMDT Project

86

DTs	 Development of operational and health digital twins for High value plant systems/components (low hanging) Reactor core critical components (challenging)
Demo	 Demonstration of automation Online sensor calibration – use flow-loop HIL testbed Work order automation – APM cloud deployments
R&D	 3. Research to address gaps and challenges Humble AI (challenging – technical development) Risk and Uncertainty - (challenging – integration)
Analysis	 4. Analyses Cost reduction opportunities and entitlements in SMR systems Regulatory needs and constraints



energy

Digital Twin Applications for Advanced Nuclear Technologies

Xe-100 ARPA-E GEMINA Program Overview

Yvotte Brits, Senior Nuclear Systems Engineer

December 2, 2020



- 1. X-Energy Introduction
- 2. Xe-100 Plant Overview
- 3. Xe-100 ARPA-E GEMINA Program Overview



X-energy was Created to Change the World





- Dr. Kam Ghaffarian is a globally recognized technology visionary across energy, space and information technology.
- Created and grew Stinger Ghaffarian Technologies (SGT), Inc. to \$650 million in annual revenue and 2,400 employees. SGT was ranked as the U.S. National Aeronautics and Space Administration's second largest engineering services company prior to being acquired by KBRwyle, subsidiary of KBR, Inc.



- Founded X-energy in 2009 to address innovation in critical energy solutions. X-energy was awarded ~\$60M from DOE to focus on an advanced nuclear reactor and TRISO fuel.



- Began Intuitive Machines in 2016 to leverage NASA technologies for commercial space and terrestrial applications. Intuitive Machines won its first Commercial Lunar Lander Contract from NASA in 2018.
- Began Axiom Space in 2017 to develop the first commercial space station, to be launched by 2021.

"President Kennedy once said that we are in a space race and my work with NASA reflects the progress he had hoped for.

Chairman

Dr. Kam Ghaffarian,

Founder and Executive

Today, I believe we are in an energy race. Providing clean energy across the world is my vision for X-energy and I believe that clean, safe, reliable nuclear energy is necessary to making this possible."

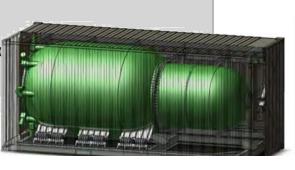
Reactor Solutions for Multiple Markets

Xe-100 For commercial power and process heat applications 80 MWe

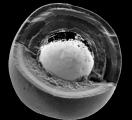
- •UCO TRISO fuel in pebble fuel elements
- •Multi-pass online refueling
- Passive safety
- Produces steam for electricity and/or process heat applications
- •Modules can be combined to generate GW-level power for large electricity demand

Xe-Mobile For remote locations and micro-grids 1-3 MWe

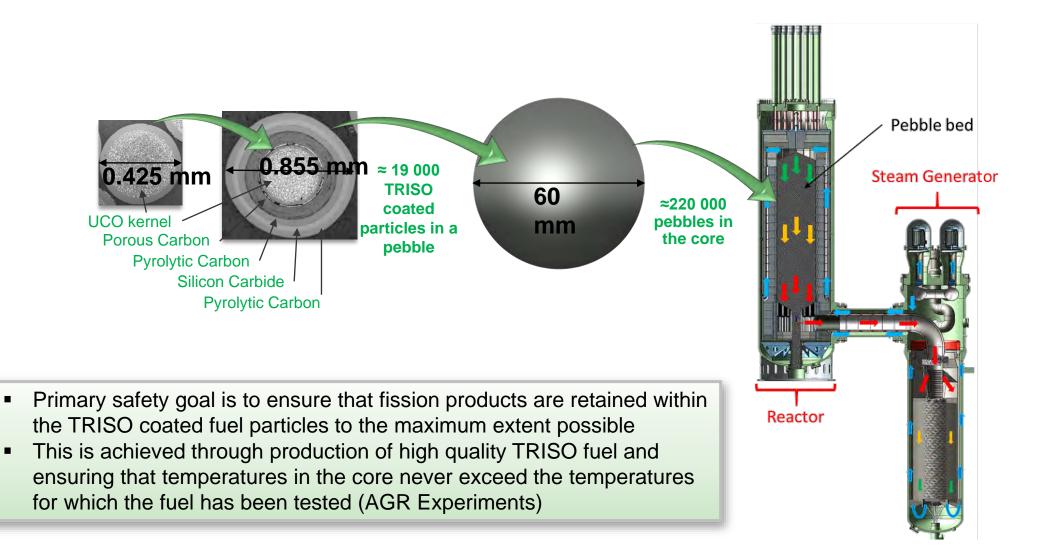
- •UCO TRISO fuel in pebble fuel or prismatic elements
- •Power conversion via gas or steam turbine generator
- •2-10+ year fuel cycles
- Passive safety
- Applicable for forward bases and disaster relief needs







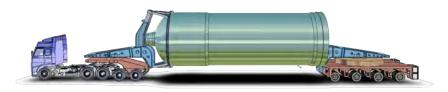
UCO TRISO Particle – Primary Fission Product Barrier

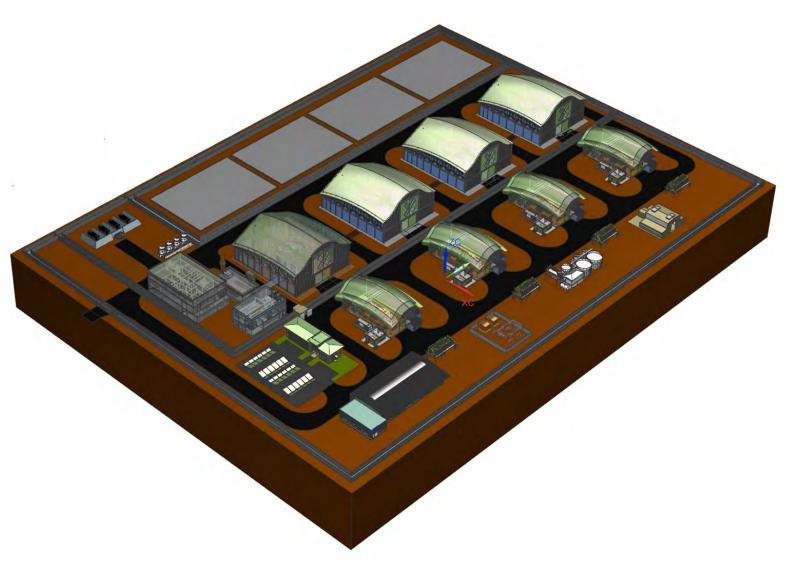




Standard X-energy plant have 4 Reactors - 4 Turbines producing 320 MWe, attributes include:

- 200MWth/80MWe Per Module
- Process heat applications Proven intrinsically safe
- Meltdown proof •
- Walk-away safe
- Modular construction
- Requires less time to construct (2.5-4 years)
- Road transportable for diverse geographic areas Uses factory-produced components Load-following to 40% power within
- •
- 15 minutes
- Continuous fueling; resilient on-site • fuel storage





Xe-100 ARPA-E GEMINA Project Objective

ARPA-E Xe-100 Plant Staff Target

Current Xe-100 Plant Staff

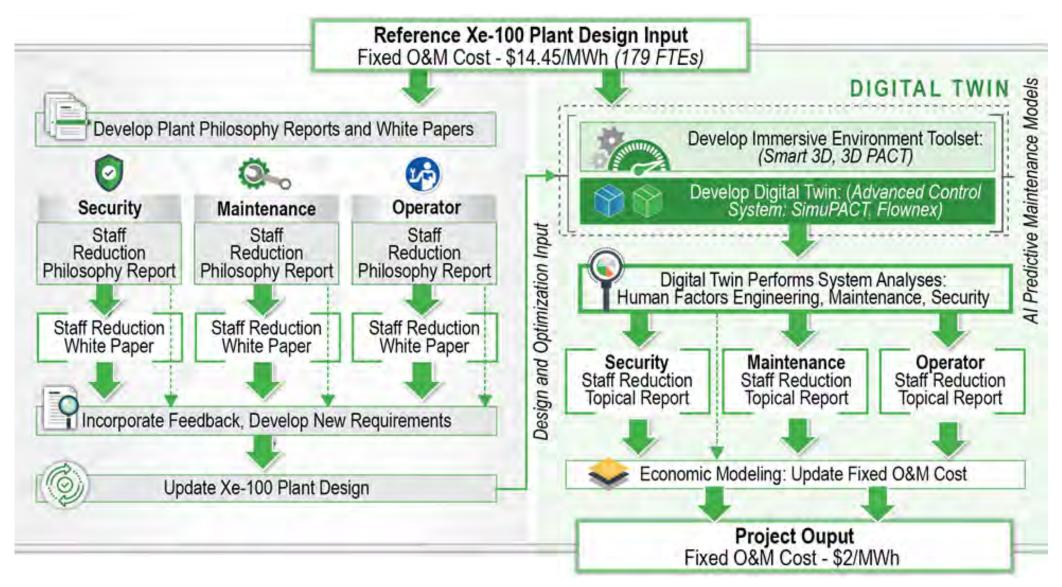
Division Nr	Number of Modules	Plant Staff
1	Plant Management	4
2	Operations Division	40
3	Security Division	50
4	Maintenance Division	52
5	Engineering Division	21
6	Administration Division	12
	Total	179

Conservative			Aggressive			
	Plant Staff		Plant Staff			
	4		4			
	30		18			
	20		20			
	20		20			
	21		10			
	12		12			
	107		84			

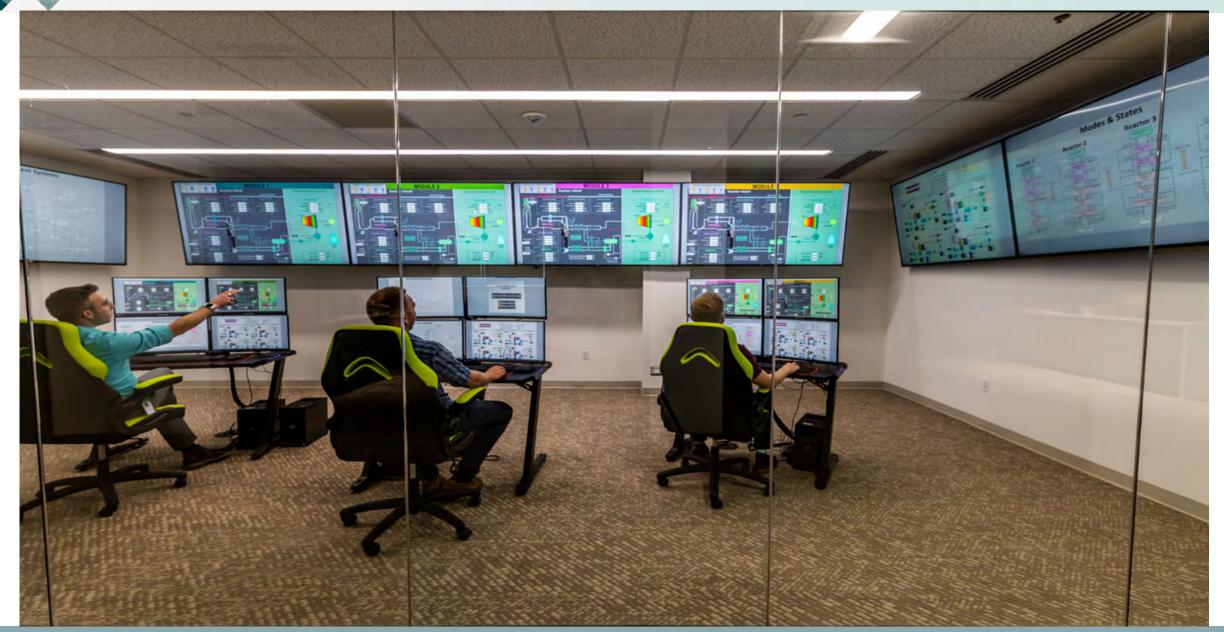
\$5	\$15	\$25	\$35	\$45	\$55	\$65	\$75	\$85
4-Reac	4-Reactor 300 MWe					\$14.45		
taff Redu	ction 4-R	Reactor 3	300 MWe		\$12	.18		
del Servi	ces 16 Xe-1	100 Reactor	s 1200 M	We	\$3.05			
del Servi	ces 24 Xe-1	00 Reactor	s 1800 MV	Ve	\$2.03			
	4-Reac	4-Reactor 300 M taff Reduction 4-F	4-Reactor 300 MWe taff Reduction 4-Reactor 3 del Services 16 Xe-100 Reactor	4-Reactor 300 MWe taff Reduction 4-Reactor 300 MWe del Services 16 Xe-100 Reactors 1200 M	4-Reactor 300 MWe taff Reduction 4-Reactor 300 MWe del Services 16 Xe-100 Reactors 1200 MWe	4-Reactor 300 MWe \$14 taff Reduction 4-Reactor 300 MWe \$12 del Services 16 Xe-100 Reactors 1200 MWe \$3.05	4-Reactor 300 MWe \$14.45 taff Reduction 4-Reactor 300 MWe \$12.18 del Services 16 Xe-100 Reactors 1200 MWe \$3.05	4-Reactor 300 MWe \$14.45 taff Reduction 4-Reactor 300 MWe \$12.18 del Services 16 Xe-100 Reactors 1200 MWe \$3.05



Overall Project Lifecycle (24 months)

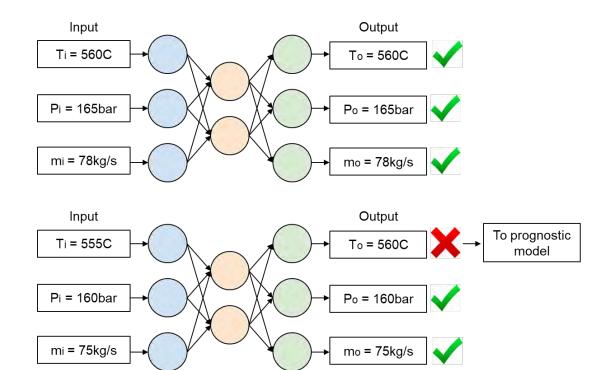


Xe-100 Digital Twin for Operator Staff Reduction in progress...



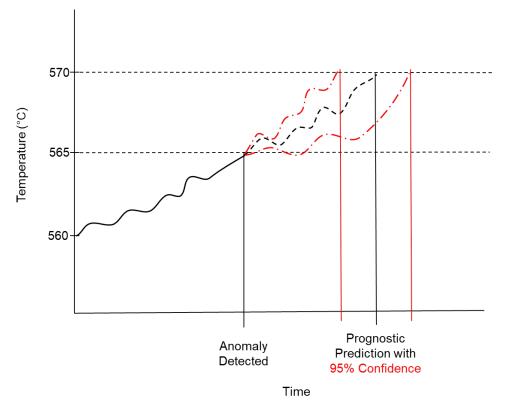


- The diagnostic model aims to
 - Detect system component anomalies
 - \circ Identify deviating variables
 - Initiate the correct prognostic model
 - Be continuously trained online
- Machine learning algorithms include
 - Auto-Encoder (AE) for feature extraction
 - Long-Short Term Memory (LSTM) for temporal data





- The prognostic model aims to
 - \circ Predict time to abnormal condition
 - Provide time window to auxiliary models
- Machine learning algorithms include
 - Bayesian Neural Network (BNN) for uncertainty
 - \circ $\,$ AE-LSTM for input space reduction and temporal data $\,$
 - Convolutional Neural Network (CNN) for efficient spatiotemporal data processing



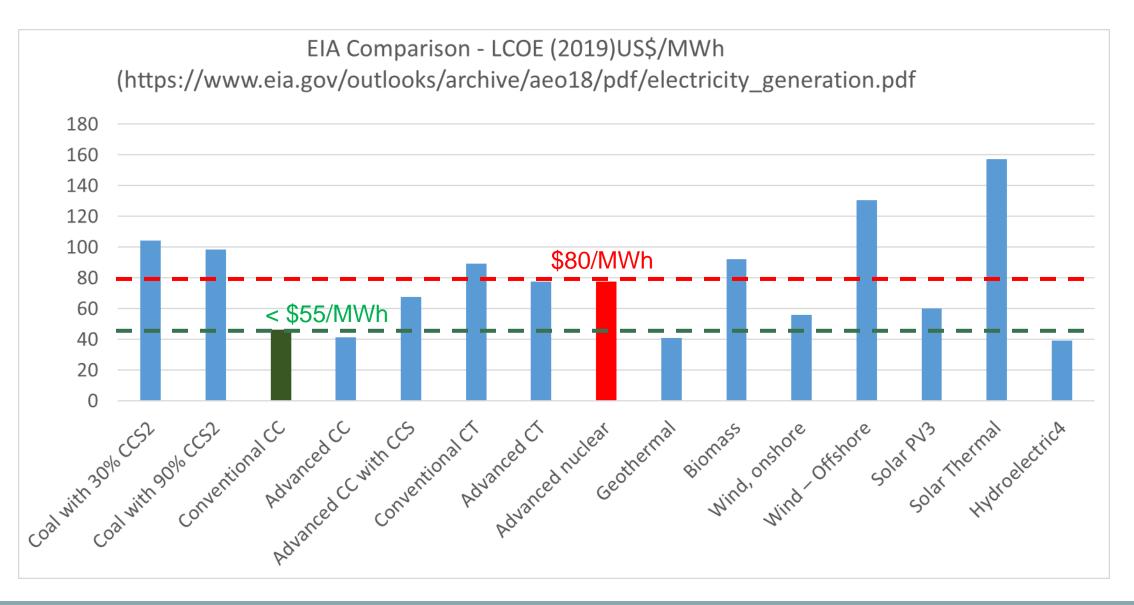
Xe-100 Immersive Environment Toolset for Security and Maintenance Staff in progress...







Successful deployment of Advanced Reactors in the US requires lower plant cost





Questions





Kairos Power ARPA-E Gemina Projects: Project "SAFARI" – Secure Automation for Advanced Reactor Innovation Project "MARS" – Maintenance of Advanced Reactor Sensors and Components.

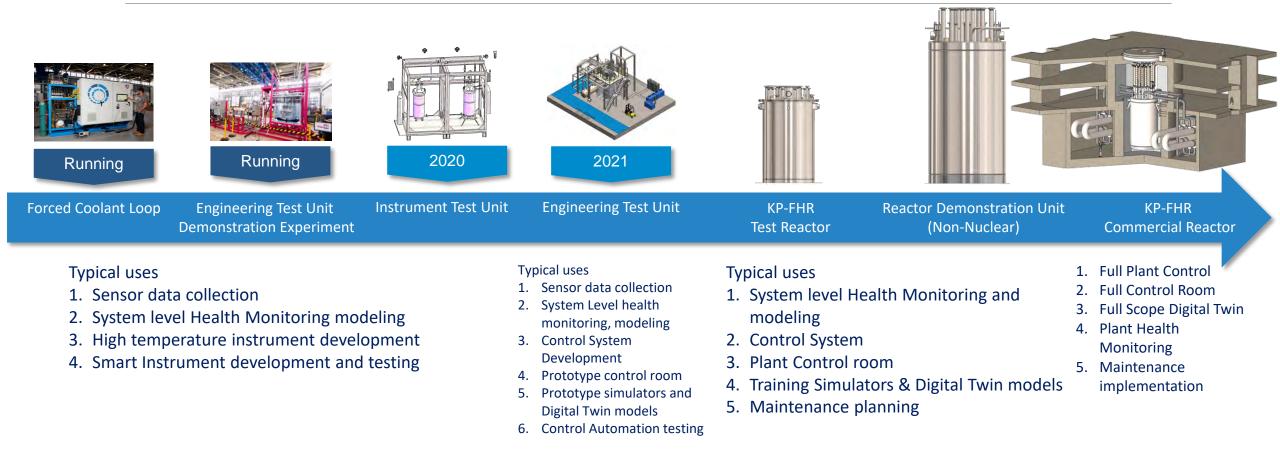
DR. ANTHONIE CILLIERS

DECEMBER 2ND 2020

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In order to achieve this mission, we must prioritize our efforts to focus on a clean energy technology that is *affordable* and *safe*.

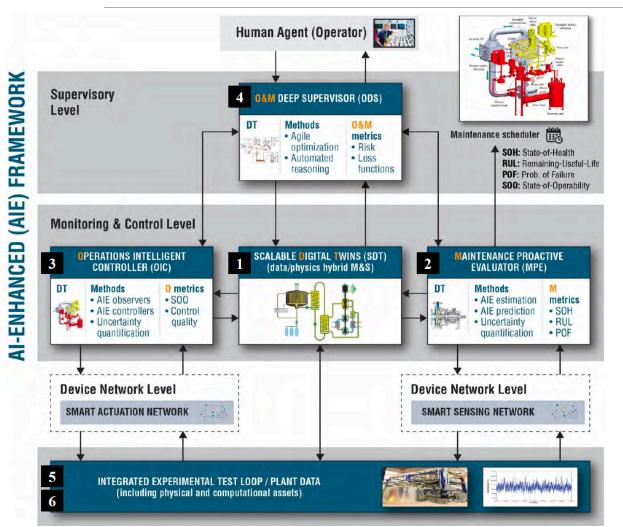
Iterative systems supporting Digital Twin development projects



Project "SAFARI" – Secure Automation for Advanced Reactor Innovation & Project "MARS" – Maintenance of Advanced Reactor Sensors and Components.

TWO COMPLEMENTARY PROJECTS SUPPORTED BY THE KAIROS TEST PROGRAM

Project "SAFARI" – Secure Automation for Advanced Reactor Innovation (University of Michigan)



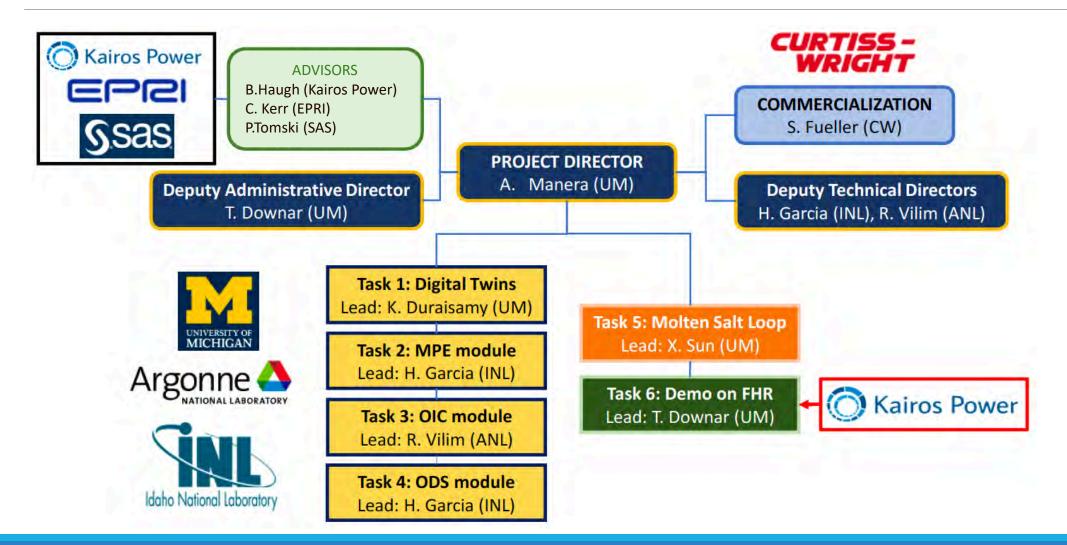
Deliver a capability enabling smart functionalities in advanced reactor systems (ARS) such as

- autonomous operations (AO),
- flexible operations (FO), and
- predictive maintenance (PM).

This has the potential to dramatically lower operations and maintenance (O&M) costs compared to currently operating LWRs.

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Project "SAFARI": Partners

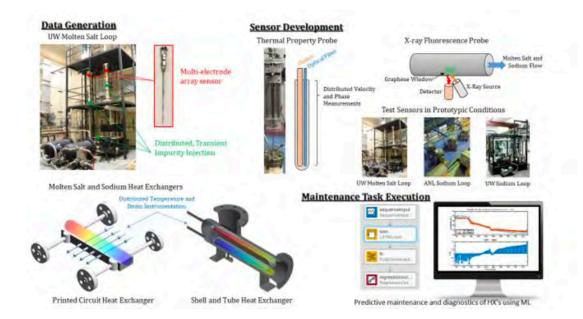


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Project "SAFARI": Activities

- The work is organized into 6 main tasks:
- Task 1 Development of Scalable Digital Twin (DT) module and SAFARI interface
- Task 2. Development of the Maintenance Proactive Evaluator (MPE)
- Task 3 Development of Operations Intelligent Controller (OIC) module
- Task 4 Development of O&M Deep Supervisor (ODS) module
- Task 5 Demonstration of Developed Capability using a Molten Salt Loop with Feedbacks
- Task 6 Application of the developed capability to Kairos-FHR

Project "MARS" – Maintenance of Advanced Reactor Sensors and Components (Argonne National Laboratory)



Develop advanced distributed sensing and data generation techniques to

- characterize critical components and systems,
- increase sensor diversity,

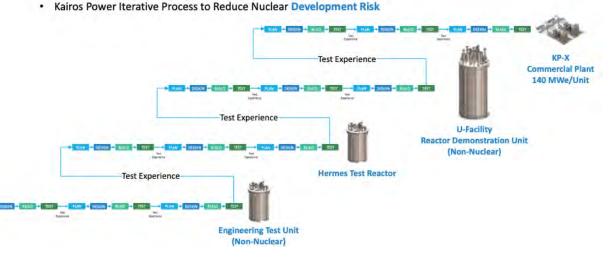
Develop multifunctional sensors which

- measure several process variables simultaneously,
- automate maintenance tasks through machine learning-enabled fault detection and diagnostics, and
- informs intelligent sensor placement to achieve autonomous operation.

Project "MARS": Kairos Power Development Strategy

Develop O&M cost-saving solutions for the Kairos Power fluoride saltcooled high temperature reactor.

- Kairos Power is a mission-driven engineering company focused on the delivery of a clean, affordable and safe energy solution through the integrated design, licensing and demonstration of advanced reactor technology.
- Focused on reducing technical risk through a novel approach to test iteration often lacking in the nuclear space.
- Constructing a non-nuclear research and development laboratory in Albuquerque, designed to test the high-temperature salt technology.
 Technologies demonstrated to be successful in reducing O&M costs, will be incorporated into the KP-FHR design.
- The licensing strategy reduces licensing risk and facilitates licensing certainty for customers via active pre-application engagement with the Nuclear Regulatory Commission (NRC) during the design process prior to plant construction.
- Licensing process improvements are anticipated by implementation of the Licensing Modernization Project (LMP), which provides a methodology for the identification and focus on safety significant portions of the design and safety analysis during the licensing review.



Risk Reduction



Project "MARS": Outcome

- The proposed technology will decrease the O&M costs through automation of maintenance and increasing durability of components through better sensing.
- Characterization of critical components, such as the heat exchangers, combined with automation of maintenance task execution through machine learning and early detection of faults is expected to reduce staffing requirements and reduce the O&M cost.
- Early and distributed detection of oxide impurities will reduce the risk of corrosion of components.
- Multifunctional sensors will reduce the number of sensing units while providing requisite information about coolant process variables.
- Reactor maintenance will be enhanced through early detection of adverse thermal hydraulic effects, such as thermal stratification in the reactor vessel.

Thank You

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Synopsis of Westinghouse Machine Learning, Artificial Intelligence, and Digital Twin Developments for Nuclear Power Applications

for the Workshop on Digital Twin Applications for Advanced Nuclear Technologies, December 2020

Brian M. Golchert, Ph.D. Gregory A. Banyay, Ph.D., P.E.



WESTINGHOUSE VISION & VALUES

Leading Infrastructure Services Provider to the Power Generation Industry

estinghouse





Outline

- Definition / History
- Selected areas where digital twins (in conjunction with Machine Learning (ML) / Artificial Intelligence (AI)) have been or will be applied at Westinghouse:
 - 1. Reduction of physical testing & maintenance
 - Destructive test elimination at our manufacturing facility
 - Baffle-former bolt predictions
 - 2. Automated analysis of inspection or monitoring data
 - Concrete crack detection using drones
 - Neutron noise monitoring of reactor structures
 - Reactor failure prediction
 - 3. Process optimization
 - Component condition monitoring
 - Fan operation
- Looking ahead

Items are briefly discussed at a high level in subsequent slides, emphasizing relevance to advancing digital twins



Background on Digital Twins

"A rose by any other name would smell as sweet"

What are digital twins?

- Digital twins (DT) are software that combine:
 - Plant data
 - Numerical models (CFD, FEA)
 - Statistical analysis (predictions)
- DT provides a virtual simulation of what is being modeled:
 - Components
 - Systems
 - Processes
- Current commercial analysis software provides more detail faster (close to real time) than ever before
 - This is key to the implementation and use of DTs

Digital twins have been around for a long time. They are just getting a lot better!

WEC has been doing 'DT' for years

- The following examples of high value services offered by Westinghouse are essentially digital twins:
 - WESTEMS
 - MAAP
 - BEACON
 - RESM
 - POMS
 - PFM
 - Simulators



A Brief History of Westinghouse Innovation

- Westinghouse has a long history of applying new technology and ideas to the nuclear industry such as:
 - First nuclear company to use a commercial finite element analysis (FEA) code
 - Pressurized thermal shock
 - Accident tolerant fuel (ATF)
 - CVAP for AP1000®
 - AP1000 passive safety plant technology
- This presentation will focus on recent activities involving digital twins with emphasis on those applications that employ advanced analytics and simulation methods

""AP1000 is a trademark or registered trademark of Westinghouse Electric Company LLC, its affiliates and/or its subsidiaries in the United States of America and may be registered in other countries throughout the world. All rights reserved. Unauthorized use is strictly prohibited. Other names may be trademarks of their respective owners."



Current emphasis in Westinghouse is on digital twin applications that build upon Westinghouse experience and leverages operational data

Area 1: Destructive test elimination

"By indirections find directions out"

Status	Complete	Bree Bree Bree
Issue	Part of the fuel fabrication facility at Westinghouse was experiencing large amounts of scrap due to poor prediction of fuel pellet composition and pellet testing. In part, the pellet composition was determined by subject matter expert input.	Addback Advisor (Version 1.0_TI) - C X
Value Proposition	ML model used to replace costly fuel pellet pre- production process Project paid for itself in less than a year!!	Select Lot Number Iddition Enter Desired Density XX Select Part Number 10014E01H01 Select Fart Number 10014E01H01 Select Green Chart ID SPA01 Select Sinter Furnace ID PL1A Select Sinter Furnace ID PL1A
Plan	Exposed WEC to potential value of ML. Built trust in ML. Inspired and enabled new applications. Developed generic tool accessible to all WEC engineers.	Select Push Cycle Time 18
Application	Almost any type of production process.	Machine learning methodology eliminated costly part of manufacturing process saving \$1M+ /yr



induced vibration

Area 1: Baffle-Former Bolt Predictions

"Better three hours too soon than a minute too late"

Status	Complete	Proportion of Failed Bolts vs. Time (EFPY)
Issue	Irradiation assisted stress corrosion cracking (IASCC) imparts damage to baffle-former bolts that hold together core support structures necessary for the structural integrity of the reactor assembly.	Percentile Predictions Compared with OE
Value Proposition	Semi-empirical predictive methodology used analysis of operating experience data in conjunction with mechanics- based modeling (for stress re- distribution).	
Plan	Deployed predictive methodology for >5 plants to improve model validation and assist in decision making for inspection timing & replacement part purchases.	5% 0% 0.00 5.00 10.00 15.00 20.00 25.00 30.00 35.00 40.00 45.00 Image: Plant 1 Image: Plant 2 Image: Plant 3 0.50 0.05 0.05 Image: Plant 1 Image: Plant 2 Image: Plant 3 0.50 0.05 Image: Plant 3 Image: Plant 4 Image: Plant 4 Image: Plant 4
Application	Probabilistic reliability analysis served to prognosticate remaining time to significant degradation, to assist in maintenance planning.	IASCC susceptibility can be exacerbated by dynamic stress, such as that due to flow-

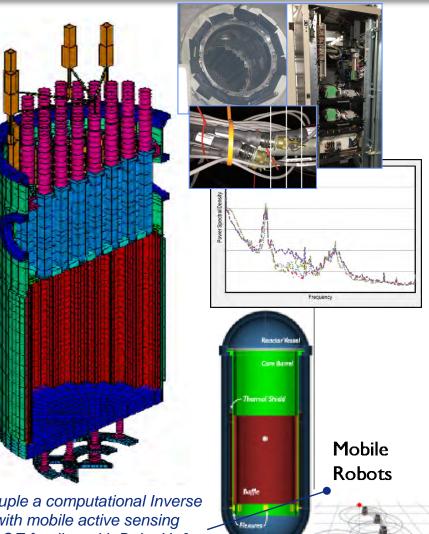


Area 2: Structural aging monitored by neutron noise

"...with a team of little atomies..."

Status	On going
Issue	Reactor internals degrade over time and it is necessary to know the current condition of these components.
Value Proposition	Use ML/Digital Twins to evaluate diagnostic certainty to assess reactor internals degradation due to aging. This will help reduce engineering cost to perform analysis.
Plan	Create a <u>condition</u> -based monitoring digital twin with machine learning that can be used to evaluate primary equipment, to enable movement away from <u>time</u> -based paradigm by EPRI MRP [®] requirements.
Application	Any reactor type internals

Reactor Internals structural dynamics are observable from neutron fluence signal due to turbulent and acoustic excitation





Visionary extension of this work to couple a computational Inverse problem (i.e., damage identification) with mobile active sensing (i.e., robotics) being researched via DOE funding with Duke U. & Sandia Nat'l Lab!

** Electric Power Research Institute Materials Reliability Program

Area 2: Concrete crack detection using drones

"Once more unto the breach..."

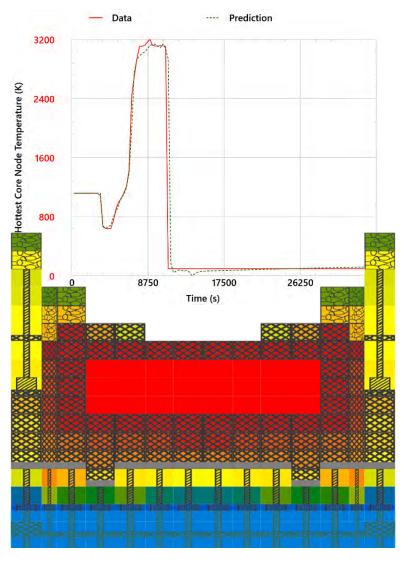
Status	On going	Initial image Identify segments	
Issue	Human inspection of large nuclear buildings is very slow, time consuming, and requires subject matter expert evaluation.		
Value Proposition	Machine learning of drone-captured images to significantly reduce cost of concrete crack identification.		
Plan	Develop machine vision system and skillset that can help automate visual inspections inside and outside the nuclear industry. Use subject matter expert elicitations to help train the ML algorithm.	Final crack Combine segments identification	
Application	Wide range of nuclear and non-nuclear	cracks	



Area 2: Manage a Severe Accident in Real Time

"...melt, thaw, and resolve itself into a dew!"

Status	Completed
Issue	Using conventional means, it was difficult to initialize the state of a digital twin corresponding to a plant in the midst of a severe accident.
Solution	A deep, recurrent neural network was developed to identify the state of an ongoing accident when provided with either plant data or simulator data. The items calculated by the neural network include the core mass distribution, fission product releases, and the size of a vessel failure that may be present.
Application	Initialize a digital twin in the midst of a PWR severe accident to perform faster than real-time evaluation of the impact of various accident management strategies. Principles involved include multiple physics, including nuclear phenomena and fluid dynamics
Value Proposition	Reduce cost associated with an accident by improving the outcome due to increased knowledge of the accident state.





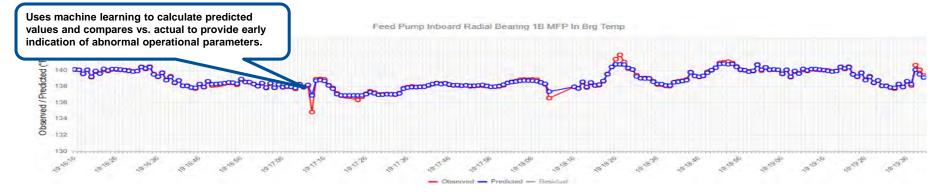
Product of Fauske and Associates, a Westinghouse subsidiary 10

Area 3: Optimization through Component Condition Monitoring

"Though this be madness, yet there is method in it"

- Mission: improve plant economics by reducing O&M costs
- Developing predictive maintenance software to monitor component health and optimize maintenance activities:
 - Rapid to deploy and easy to use
 - Component agnostic
 - Anomaly detection, diagnostics, and remaining useful life
 - Utilizes data from diverse sources and sampling rates
- Partnered with experienced Advanced Pattern Recognition Software provider.





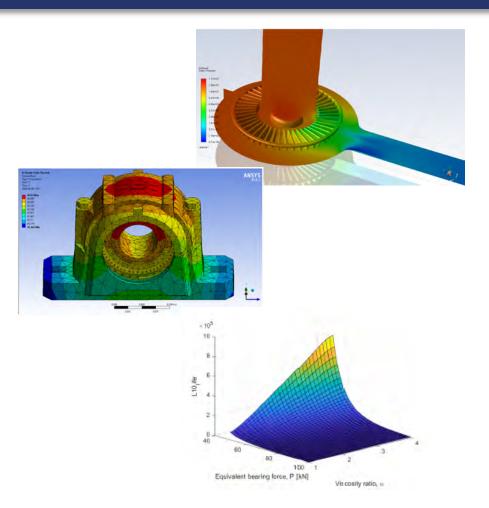


Emphasis on not only anomaly detection, but also **diagnostics** and **prognostics**

Area 3: Swedish Fan Optimization

"No, lago, I'll see before I doubt"

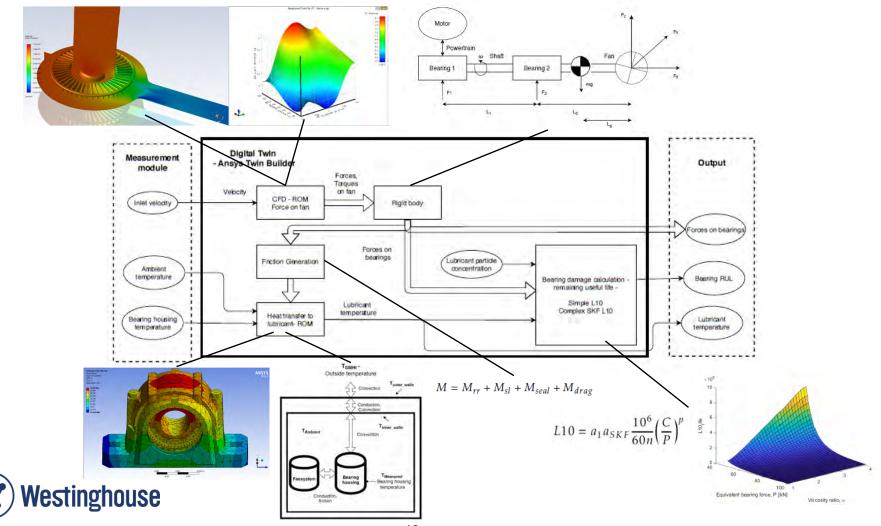
Status	On going
Issue	Fans in a Swedish factory were experiencing significant down time which led to lost production.
Value Proposition	A Digital Twin/ML condition monitoring model created to minimize fan downtime.
Plan	Development of methodology for integrating physics-based reduced order models in Westinghouse condition monitoring platform.
Application	Almost any type of manufacturing facility.



A Digital Twin Enhances Condition Monitoring capabilities by using **simulation as a virtual sensor** in real-time



Area 3: Swedish Fan Optimization



Looking Ahead

"Was this ambition?"

- Future works in progress or being considered
 - Extension of Digital Twins to:
 - Advanced Reactors (e.g., eVinci, Lead Fast Reactor)
 - Entire reactor systems such as steam generators and pump seals
 - Advanced manufacturing (e.g., fuels, additive manufacturing)
 - Fatigue and probabilistic fracture mechanics applied to systems
 - Source tracing for environmental substrate contamination
- Working together to help nuclear thrive
 - Westinghouse continues its history of working with its customers, regulators, academia, and national laboratories to apply new technology in order to improve our understanding of nuclear power operations
 - We welcome any dialogue regarding mutual research or development activities.

Nearly all nuclear analyses can be enhanced through their incorporation into digital twins



Questions?

• For further information, please contact:

Brian M. Golchert, <u>golchebm@Westinghouse.com</u> Digital Twin Project Technical Lead

Gregory A. Banyay, <u>banyayga@Westinghouse.com</u> Modeling & Simulation Hub Technical Lead



BWXT®

BWX Technologies, Inc.

Digital Twin for the Manufacture and Qualification of Additively Manufactured Nuclear Components

Matt LeVasseur, Director of Research Ryan Kitchen, Data Scientist

Quality-grade Digital Twin to inspect-during-build

How it Works

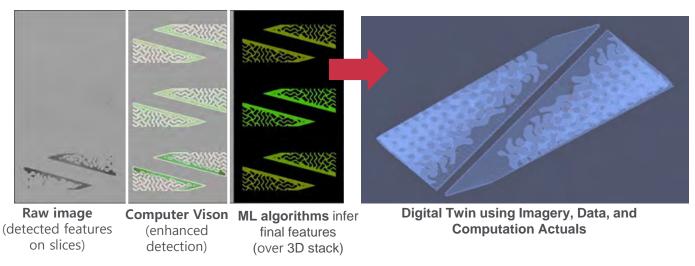
- Many additive manufacturing (AM) technologies build layer-by-layer
- A camera may be used to take images insitu (during fabrication).
- Images between layers may be restacked to form a 3D representation
 - Becoming common to visualize AM parts, and many machines include a camera for calibration or general process quality
 - A true "digital twin" if it actually represent the as-built for inspection purposes
- Imagery reveals some features directly, and some are inferred computationally
 - Features may change, form, or heal later in processing (cracks, pores, stress features)
 - Computer vision and ML/deep networks are used to discover indicative patterns

The Benefit

- Additive manufacturing is most economic for low-volume, customized and complex geometries
 - Hard-to-Source replacement components
 - Manufacture of advanced reactor and microreactor designs, that take advantage of optimized geometries
- o Qualification of AM nuclear components
 - Accurate digital twins could replace or augment DE and NDE inspection, streamlining cost of qualifying nuclear components
 - Digital twins may be used as simulations of asbuilt components, in place of as-designed
- Algorithms can relate as-built digital twin to mechanical properties



Objective: reduce cost of development for novel components

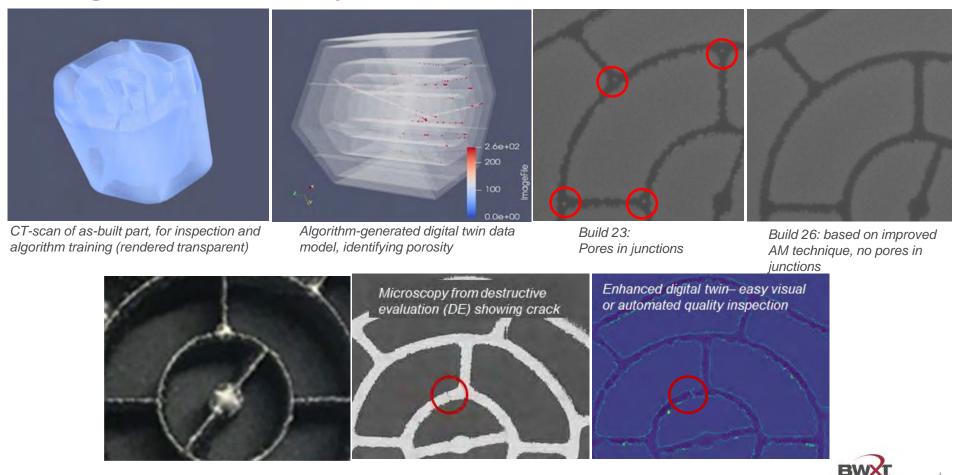


The Need

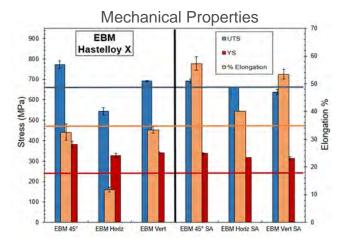
- Advanced nuclear concepts suffer from cost to qualify and deploy a technology
 - Reducing costs of testing combined with increased fidelity of data on the as-built can change the economics of nuclear and additively built development
 - Suited to creative microreactor design and deployment



Digital Twin for Quality Assessment



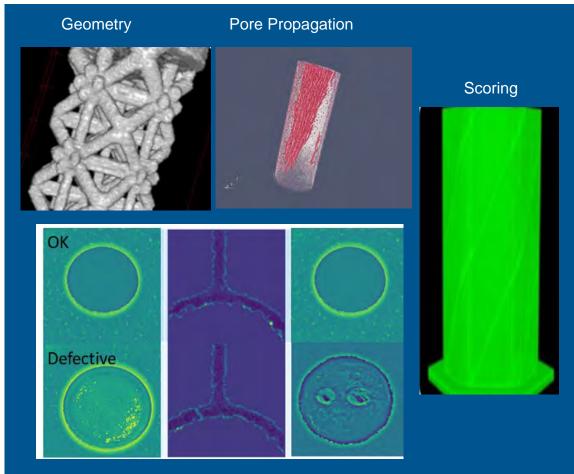
Features of Interest



Digital Twin algorithms help infer/predict mechanical properties.

This above chart shows the results of an AM component after tensile strength testing.

Color-coded horizontal thresholds represent ASTM B572 standards for rod N06002 (Hastelloy X).

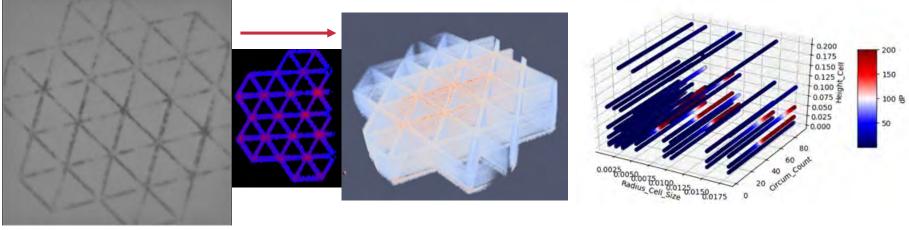




Digital Twin (as-built) as input to multiphysics simulation

Merging data and physics: feed digital twin as-built data model back into design simulation to guide design-for-manufacturing and predictive component test (e.g. additive manufacturing):

- o See effects of true geometry, defects, fuel and moderator placement
- Neural networks + computer vision: supporting build technique development, physics simulation, design, predictive test and performance validation



Example: digital twin based on in-situ melt pool to predict thermally induced stresses and other properties. This as-built model may then be fed back into simulation tools.

Example: Reactivity comparisons of as-built



End of Slides





EPRI's Digital Twin Related Activities for Nuclear Applications

Hasan Charkas Principal Technical Leader

Digital Twin Applications in the Nuclear Industry Dec 1-4, 2020



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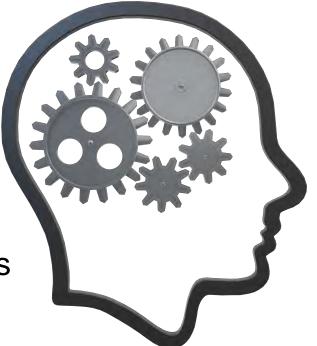
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EPRI Digital Twin Engineering Overview

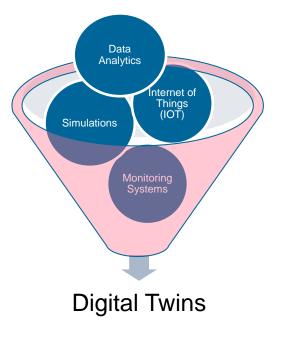
- Rapid advancement in 3D visualization and modeling technologies can lead to opportunities for using digital twins (DT) in managing the lifecycle of nuclear assets
- Formed an internal cross-cutting team for collaboration
- Near term the team is working on the following:
 - What impact do DT applications have on nuclear power plants construction, operation, maintenance and decommissioning?
 - What DT applications can be deployable in the near future?
- Launched 2 projects: advanced reactors and chemistry areas
- Developed a technical insight document (<u>3002020014</u>) and working on technical videos
 - targeted first video release in December 2020
- On going discussions in several technical virtual meetings with industry stakeholders to identify opportunities for coordination or collaboration



EPRI Digital Twin Engineering – Path forward

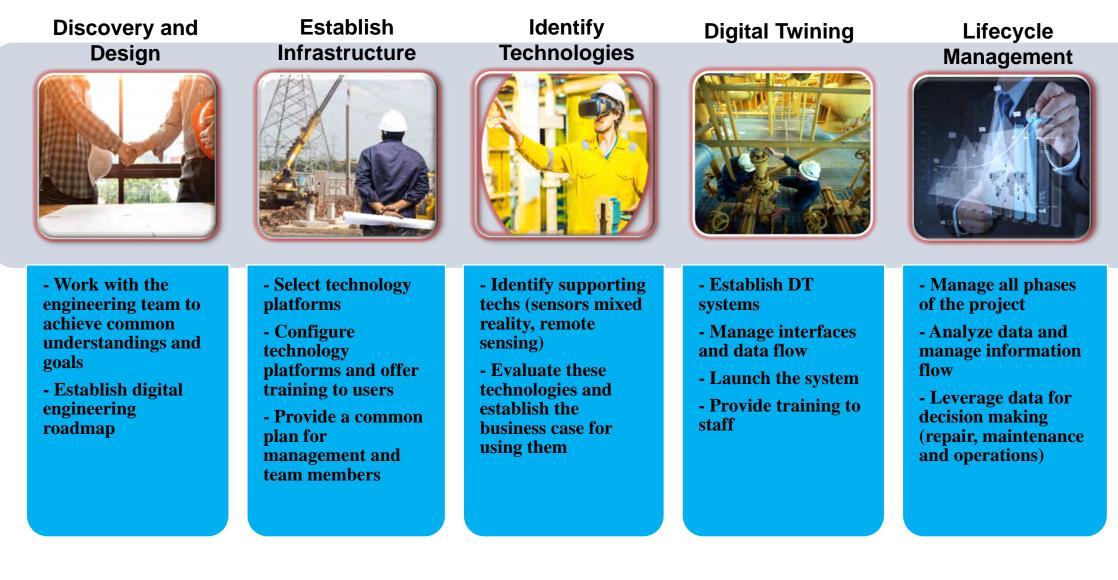
- Establish industry guidelines, best practices, and recommendations for DT implementation
- Informative videos and a series of coordination and collaboration webcasts with utilities, regulators, and vendors
- Project collaborate with Artificial Intelligence (AI) and Data Driven Decision Making (3DM) initiatives at EPRI
- Additional questions:
 - What research is needed for the industry to advance the use of DT?
 - How can DT optimize the life cycle of nuclear assets? Predictive maintenance, risk management, informed decisions?







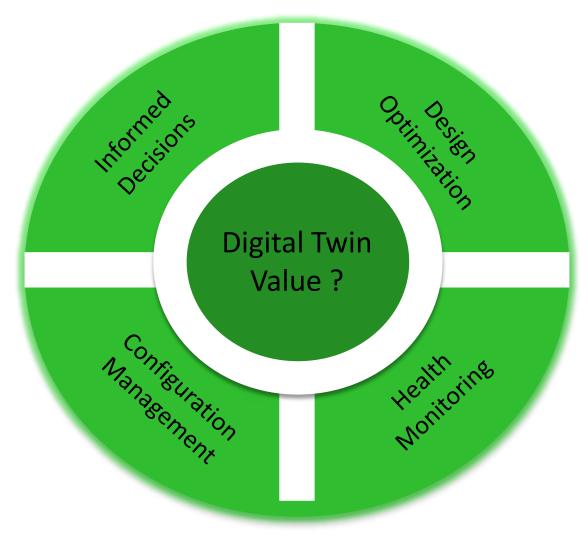
Digital Twin Project Phases





Digital Twin Applications for Advanced Reactors (ARs)

- Objectives:
 - Explore benefits, challenges and potential AR applications.
 - Summarize available tools, software, sensing technologies and monitoring strategies for equipment, structures and components (SSCs)
 - Establish industry guidelines, best practices and recommendations for implementing DTs in ARs life cycle management
 - Estimate costs and potential savings of implementing digital twin technology
- Value:
 - Help answer the question of where it makes sense to use DTs in ARs





Digital Twin Applications for Construction

- Holistic view of projects activities
- Effectiveness assessment of new construction techniques through progress monitoring
- Simulate what-if scenarios and run through steps of construction to identify challenges
- Automated progress reports, as-built configurations, including deviation from design, faster response to field changes
- Other reachable applications
 - Advanced manufacturing and fabrication
 - Performance of steam generators







Together...Shaping the Future of Electricity





INNOVATING NUCLEAR TECHNOLOGY

Digital Twins for Advanced Reactor Applications

Presented by: Jake Houser, Ph.D. Senior Research Engineer, AMS

Hash Hashemian, Ph.D. President, AMS

AMS Corporation 9119 Cross Park Drive, Knoxville, TN 37923 www.ams-corp.com

December 2, 2020



We Test the Instrumentation and Control Systems of Nuclear Facilities





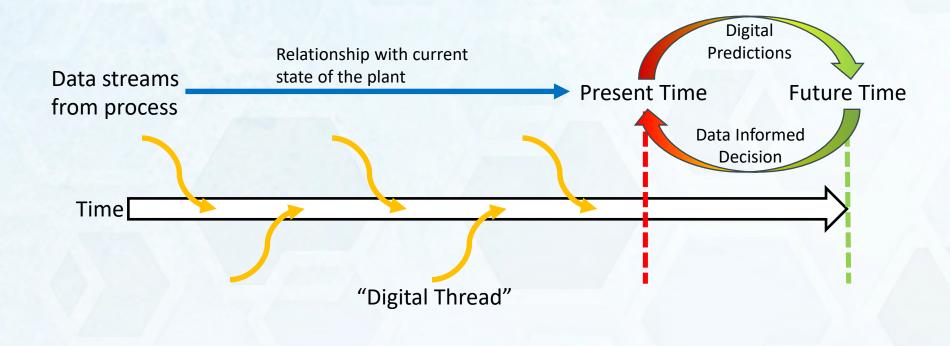


- I&C Testing
- Rod Control
- Cable Testing
- Software Reliability
- EMC/Wireless
- Reactor Diagnostics
- On-Line Monitoring
- Custom Data Acquisition



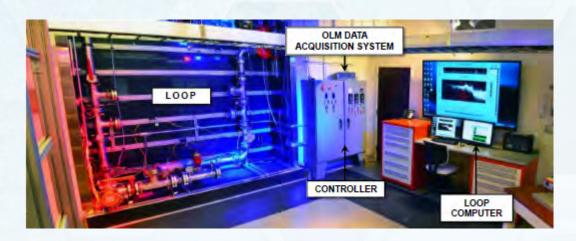


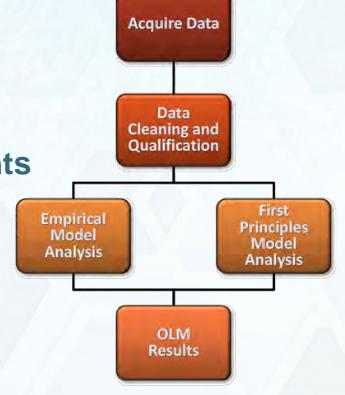
Digital Twins are living, data informed models of complex systems



On-line Monitoring (OLM) for New Generation of Reactors

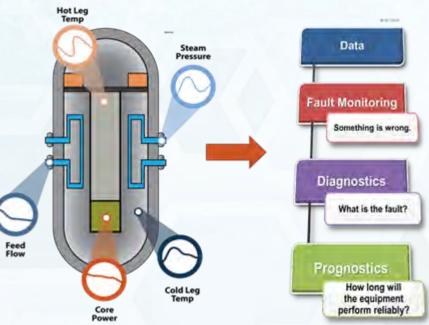
- Constructed flow loop to mimic SMR thermal-hydraulics
- Developed empirical and first principles models
- Compared data informed model
 results to experimental measurements





Takeaways from OLM for New Generation of Reactors

- Four critical components in developing OLM for new reactors:
 - For monitoring I&C performance, sampling frequency must be >1000 Hz
 - Redundancy helps verify calibration and separate process problems from I&C system issues
 - OLM can verify that response time of process sensors remains intact and sensors can register process changes
 - Algorithm/software package depends on I&C system architecture, sensor location, redundancy, and sampling frequency
 - Highly redundant systems → straight and weighted averaging of signals
 - No redundancy → process modeling needed to establish process state



Online Monitoring System to Support Autonomous Remote Microreactor Operations

- Establish I&C system sensors for process measurements and structural health monitoring (SHM)
- Determine ability of embedded sensors to provide quality measurement data
- Develop AI / ML based online monitoring (OLM) technologies for autonomous operation and predictive maintenance of microreactors







AMS has been working with the industry for over 30 years to extend safety related pressure transmitters calibration intervals. Attempts to do this using modeling techniques have been challenging.

- Working with NRC to approve simple averaging technique through DOE Pathway III grant
- NRC is currently more receptive to working to facilitate both efficiency and safety
- Safety related applications are always more difficult and will be challenging for Digital Twin implementation

→ Regulatory issues should be addressed in conjunction with development of Digital Twin architecture. Applications targeting efficiency, maintenance, and troubleshooting are promising.



INNOVATING NUCLEAR TECHNOLOGY

Thank You Questions?



On AI Research at ORNL and its application at SNS

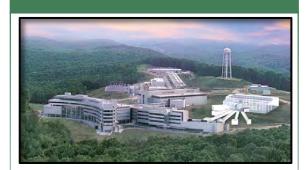
David Womble Al Program Director Oak Ridge National Laboratory



ORNL is managed by UT-Battelle, LLC for the US Department of Energy



We are at a "tipping point" in AI/ML



- Facilities distinguish DOE
- Sensors are
 ubiquitous

Data

Data is plentiful. We are "bit-rich"

Computing



- DOE has an HPC mission
- Computing is
 "exaflop scale"
- Specialized HW for data analytics and "edge" applications

Algorithms

$\mathbf{x}^{(i)} \xrightarrow{\mathbf{a}_{1}^{(1)}} \underbrace{\mathbf{a}_{1}^{(i)}}_{i,o} \underbrace{\mathbf{a}_{2}^{(2)}}_{i,o} \underbrace{\mathbf{a}_{2}^{(2)}}_{i,o} \underbrace{\mathbf{a}_{2}^{(3)}}_{i,o} \underbrace{\mathbf{a}_{2}^{(3)}}_{i,o}$

- Pre-defined models
- Computationally tractable training for ML

Accessibility



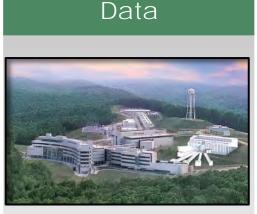
- Everyone has a PC and internet access
- A lot of data and SW are open-source
- Foundational research needed to bring AI/ ML to DOE mission

Al won't replace the scientist, but scientists who use Al will replace those who don't.*

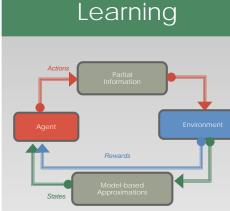
*Adapted from a Microsoft report, "The Future Computed"



ORNL Strategic Directions and in AI/ML



- Facilities operation and control
- Experimental design
- Data curation and validation
- Compressed
 sensing

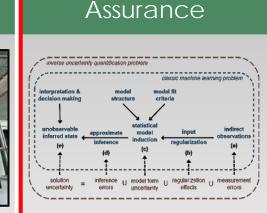


- Physics informed
- Accelerating learning
- Stability and robustness
- Foundations of ML formulations - RL, GANs, GNNs, BNNs
- Dimension reduction
 and encoding

 Algorithms, complexity and convergence

Scalability

- Levels of parallelization
- Mixed precision arithmetic
- Communication
- Implementations on acceleratednode hardware



- Uncertainty
 quantification
- Robustness
- Explainability and interpretability
- Validation and verification
- Causal inference and hypothesis generation

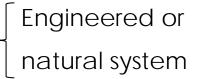
Workflow



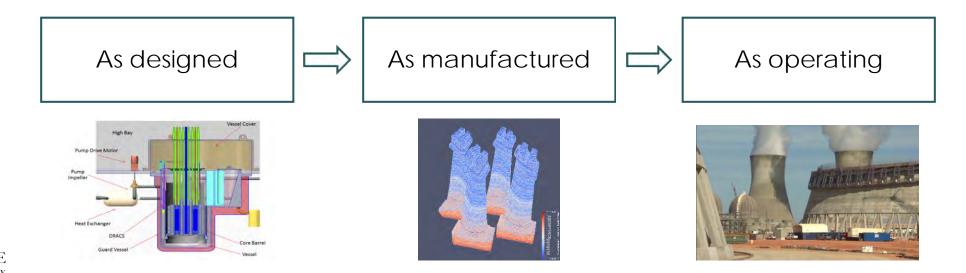
- Edge Al
- Compression
- Online learning
- Federated learning
- Infrastructure
- Augmented intelligence and HCI

What do I mean by "digital twin"

- A model that
 - Captures the current state of a system, -
 - Continuously updated
 - Is individualized to a specific system,
 - Can be used
 - to assess the health of the system,
 - In system control
 - And in making decisions, e.g., maintenance
 - Is "causal"









"As operating" is inherently a machine learning problem

- Most systems cannot be "completely characterized" by either equations or observations once they are put in operation.
- But we can collect data.
- Challenges
 - "Informed" learning
 - How do we incorporate physical "constraints" (physics-informed)
 - How do we blend traditional models with ML models/updates
 - Getting the right data
 - Widely varying temporal and spatial scales
 - Stochastic nature of the problem
 - Causal analysis
 - ASSURANCE

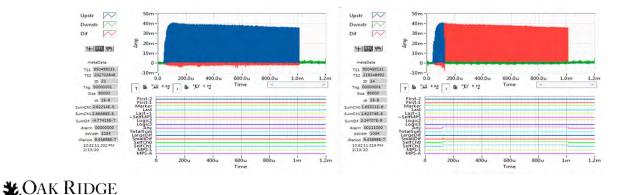


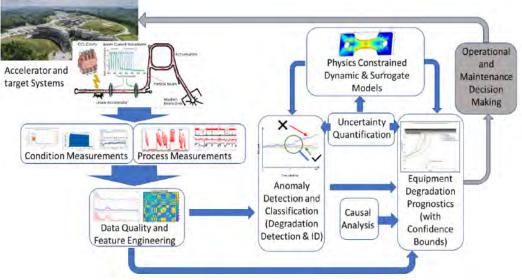
An "easy" first step is anomaly detection

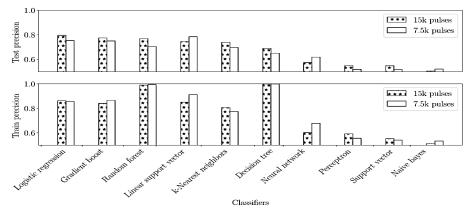
- Is there something wrong with the system?
- Need a baseline from "good" data
- If we have sufficiently well-labeled data, this is a classification problem, although you can only "learn" the current decision process.
- For the SNS accelerator an "anomaly" is an errant beam
 - Best results in the 90% accuracy but still not good enough
 - We have more (and new) data now

National Laboratory

Ideally, the results associate with equipment failure





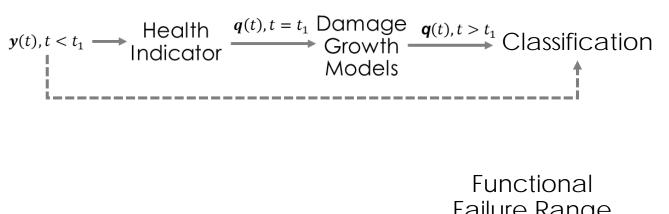




From: M. Rescic, R. Seviour, W. Blokland, "Predicting particle accelerator failures using binary classifiers," Nuclear Instruments and Methods in Physics Research Section A, Volume 955, 2020, 163240, ISSN 0168-9002, https://doi.org/10.1016/j.nima.2019.163240.

A second step is ETTF

 Anomaly detection: Classify predicted health indicator data into two or more classes



ETTF

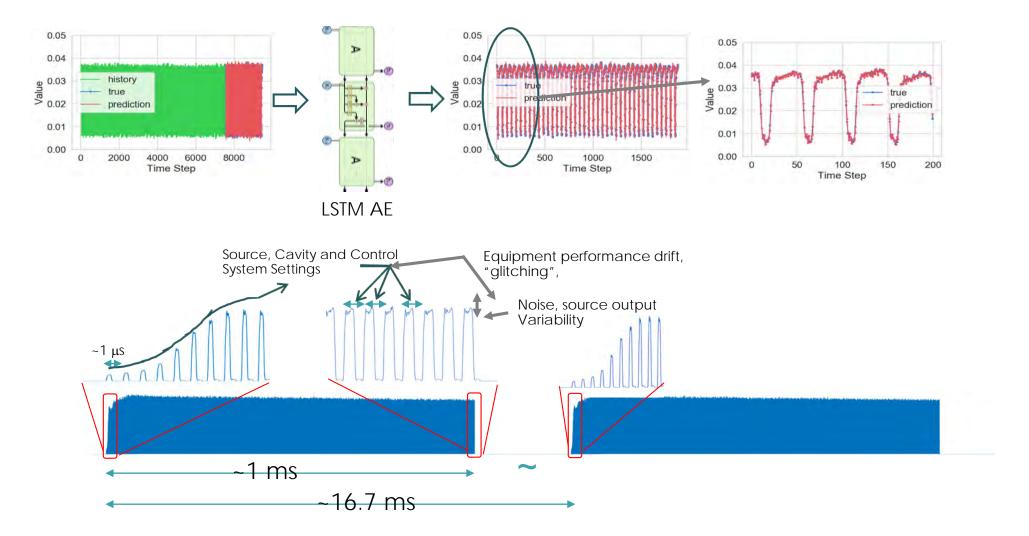
- Identify time-to-failure using damage growth models $y(t), t < t_1 \rightarrow Health$ Indicator $\frac{q(t), t = t_1}{Growth} = t_1 Damage$
- Detect deviations from nominal based on time-series predictions

$$y(t), t < t_1$$
 \longrightarrow Time-Series $\hat{y}(t), t > t_1$ $\epsilon(t), t > t_1$

Models



A predictive model is next, followed by a "causal" model





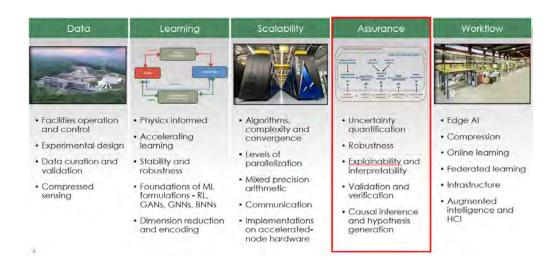
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Summary of challenges

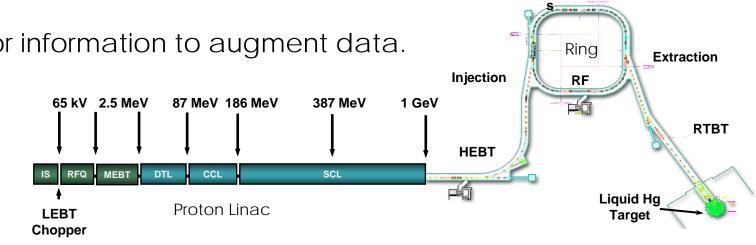
- The single biggest challenge may be assurance. Why should I trust an AI?
 - Includes
 - UQ
 - Validation
 - Reproducibility and Replicability
 - Causal Analysis
 - Includes getting the right data
 - Why should I trust a "digital twin" in a control system?
 - Bias is not just a social issue
- "Informed" learning. Using prior information to augment data.
- Dealing with scales

CAK RIDGE

Noise and uncertainty



Collimator





OVERVIEW OF DIGITAL TWIN WORK AT ANL



RICK VILIM Nuclear Science and Engineering Division Argonne National Laboratory



Workshop on Digital Twin Applications for Advanced Nuclear Technologies December 1-4, 2020

OVERVIEW OUTLINE



Digital Twin – What is it?

Why the Interest in Digital Twins

Autonomous Operation

AI/ML - Enabler of Autonomous Operation

Digital-Twin Projects at ANL

Applications

Challenges





DIGITAL TWIN– WHAT IS IT?

Two variants: Data-driven and physics-based

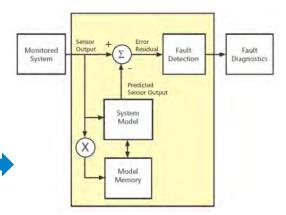
Digital Twin (DT)

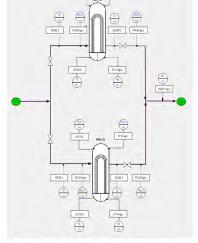
An analytic representation in combination with sensor data provides improved performance for tasks such as diagnosing operational anomalies, understanding system health, and improving system efficiency.



Data-driven (DD)

Constructed using sensor data taken from plant operating history





Physical + Virtual Sensor Set



Physics-based (PB)

Constructed from first principles, which may include conservation balances and constitutive relations

WHY THE INTEREST IN DIGITAL TWINS

Enabler of AI/ML methods for autonomous operation

Increased automation can reduce O&M costs. More efficient allocation of staff and increased plant availability

COMPARATIVE COST (mil	ls/kWh) + U.S. EIA 2019
50	_
5	
o	
5	
o	
5	
0	
Light Water Reactor	Gas Turbine

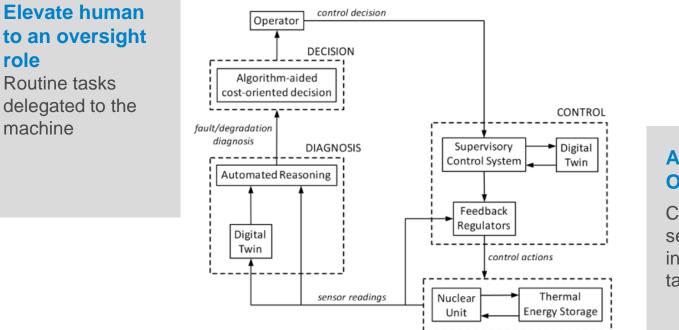
Digital twin models underlie many of the Al/MI methods that can support autonomous operation





AUTONOMOUS OPERATION

A long-term goal for O&M cost reduction



Autonomous Operation

PLANT

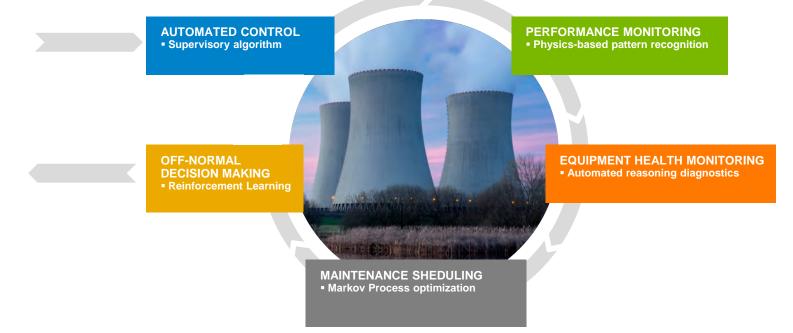
Conceptualize as a set of interconnected tasks and activities





AI/ML - AN ENABLER OF AUTONOMOUS OPERATION

A logical progression is envisioned with successive introduction of AI/ML technologies







A SAMPLING OF ANL DIGITAL-TWIN PROJECTS All currently underway

PROBLEM	APPLICATION	DIGITAL TWIN	CUSTOMER
1 - Health Monitoring	FW Pump-Motor Set ^a	Physics-based	U.S. Utility
2 - Health Monitoring	HP FW System	Physics-based	U.S. Utility
3 - Maintenance Scheduling	BOP Sensor Selection ^a	Physics-based	U.S. Utility
4 - Performance Optimization	Moisture Carryover ^b	Data-driven	U.S. Utility
5 - Lifetime Extension	Pressure Vessel Nozzle ^c	Combined	DOE NE
6 - O&M	Adv Reactor Automation	Combined	DOE ARPA-E
7 - Design	Adv Reactor Safety ^d	Combined	DOE NE
8 - Manufacturing	Adv Manufacturing	Data-driven	DOE NE
9 - Future Electric Grid	Integrated Energy System	Combined	DOE NE
a	h	d	_

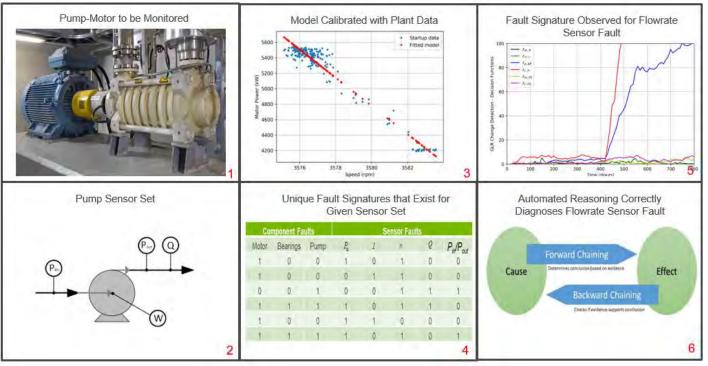
^a This presentation ^b R. Vilim presentation, ^c S. Mohanty presentation, ^d R. Hu presentation

ENERGY Argonne National Laboratory is a U.S. Department of Energy laboratory managed by UChicago Argonne, LLC.



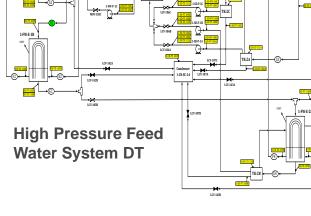
APPLICATIONS (1/2)

Health Monitoring – Equipment diagnostics using physics-based DT



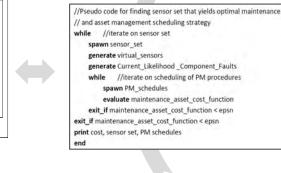


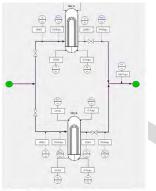




APPLICATIONS (2/2)

Maintenance Scheduling – Sensor selection using physics-based DT





Physical + Virtual Sensor Set

		Comp. Label	Comp. Type	Fault		Comp. Label	Comp. Type	Fault
	1	1-FW-E-1A	FWH	Fouling	21	PT-100	Press. sensor	Sensor fault
	2	1-FW-E-1A	FWH	Tube leak	22	1-FW-P-1A	Feed pump	Pump fault
	3	1-FW-E-1A	FWH	Shell leak	23	1-FW-P-1B	Feed pump	Pump fault
	- 4	1-FW-E-1A	FWH	Tube block	24	MOV-150A	Valve	Leakage
	5	1-FW-E-1B	FWH	Fouling	25	MOV-150A	Valve	Blockage
	6	1-FW-E-1B	FWH	Tube leak	26	MOV-150B	Valve	Leakage
	7	1-FW-E-1B	FWH	Shell leak	27	MOV-150B	Valve	Blockage
	8	1-FW-E-1B	FWH	Tube block	28	1-SD-P-1A	Drain pump	Pump fault
PRO-AID	9	FE-105	Flow sensor	Sensor fault	29	1-SD-PT-100A	Press. sensor	Sensor fault
	10	FW-TE-109A	Temp. sensor	Sensor fault	30	1-SD-FT-100A	Flow sensor	Sensor fault
	11	FW-TE-110A	Temp. sensor	Sensor fault	31	1-SD-PT-108A	Press. sensor	Sensor fault
	12	SD-FT-102A	Flow sensor	Sensor fault	32	1-SD-P-1B	Drain pump	Pump fault
	13	SD-TE-110A	Temp. sensor	Sensor fault	33	1-SD-PT-100B	Press. sensor	Sensor fault
	14	ES-PT-100A	Press. sensor	Sensor fault	34	1-SD-FT-100B	Flow sensor	Sensor fault
	15	FW-TE-109B	Temp. sensor	Sensor fault	35	1-SD-PT-108B	Press. sensor	Sensor fault
	16	FW-TE-110B	Temp. sensor	Sensor fault	36	1-SD-P-1C	Drain pump	Pump fault
	17	SD-FT-102B	Flow sensor	Sensor fault	37	1-SD-PT-100C	Press. sensor	Sensor fault
	18	SD-TE-110B	Temp. sensor	Sensor fault	38	1-SD-FT-100C	Flow sensor	Sensor fault
	19	ES-PT-100B	Press. sensor	Sensor fault	39	1-SD-PT-108C	Press. sensor	Sensor fault
	20	FW-PT-158	Press. sensor	Sensor fault				

Fault ID	Fault	r2, r6, r9	
15	Sensor E2.T	25.7%	
16	Sensor F2.T	25.7%	
17	Sensor G2.w	25.7%	
18	Sensor G2.T	25.7%	
5	FWH 1B, Fouling	10.1%	
7	FWH 1B, Shell leak	5.1%	
19	Sensor D2.P	5.0%	
ę	Other faults	< 0.1%	



OUTSTANDING CHALLENGES

Need to minimize contributors to risk posed by digital twins

- Sensor Assignment
 - What is the sensor set needed to ensure correct and complete inferencing of plant state and present and future condition?
- Explainable Results/Human Factors
 - Are the results presented in a way that one can understand how they were arrived at?
- Uncertainty Quantification
 - Are the results presented in a way that their reliability is easily understood?
- Validation and Verification
 - Do we have assurance that the results are correct?





Business Confidential

THANK YOU

MORE INFORMATION @ https://www.anl.gov/nse/artificial-intelligence-andmachine-learning



Argonne National Laboratory is a U.S. Department of Energy laboratory managed by UChicago Argonne, LLC



Christopher Ritter Director, Idaho National Laboratory Digital Innovation Center of Excellence

Extending a Digital Engineering Framework through Operations



65% of MegaProjects Fail

- Failure definition:
 - 25% over budget
 - 25% behind schedule
 - Not able to meet business objectives within one year of the facility start date
- Some factors in failure: Ineffective Interface Management (B5)
 Inadequate Document Management Plan (B6), Inadequate Integrated Schedule (C7), Ineffective Change Management (C11), Unfit Documents, Procedures, and Processes (C13)



South Carolina VC Summer (Westinghouse/ Post & Courier)

Real World Examples

- Kilopower Project KRUSTY Test
 - Design change from 316-L Stainless
 Steel to 304 Stainless Steel
 - Miss-communication between reactor and mechanical designers occurred for change in materials of shielding
 - Caused schedule delays
- Airbus A380 program
 - Use of CATIA v4 and CATIA v5 in different designer's home countries
 - Data integration issue caused a miscalculation of wiring length
 - Wires were ultimately cut too short, leading to massive schedule delays and a > \$1 billion cost overrun





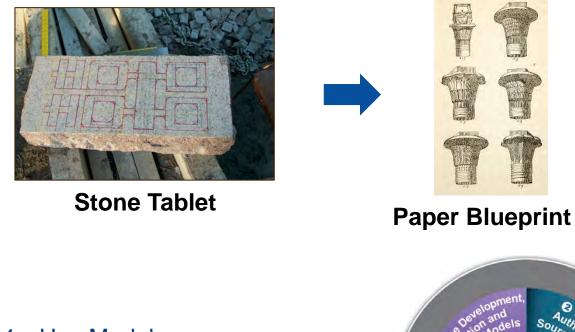
Requirement changes and downstream tooling notified, design marked invalid



Engineer is aware of change needed and resolves conflict

IDAHO NATIONAL LABORATORY

The Vision of Digital Innovation: Digital Engineering & Digital Transformation



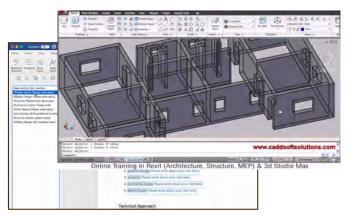
1. Use Models MBSE & BIM

2. Source Of Truth Central Datawarehouse

3. Technological Innovation Lab & University Research



Digital Innovation



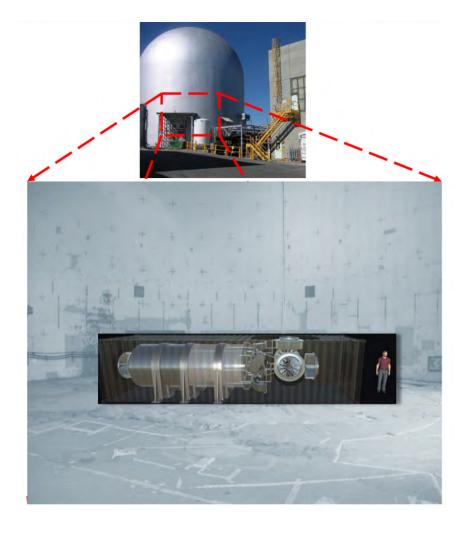
Information Management

4. Infrastructure and EnvironmentCloud Computing & HPC5. Transform CultureTraining & Cultural Integration

IDAHO NATIONAL LABORATORY

Digital Engineering in Design - NRIC

- National Reactor Innovation Center: The National Reactor Innovation Center (NRIC) at Idaho National Laboratory provides resources for testing, demonstration, and performance assessment to accelerate deployment of new advanced nuclear technology concepts
- State of the Art: Document centric exchange of reactor design documents and information
- **Scope**: Transform the traditional engineering design ecosystem from a document-centric paradigm to a digital engineering framework to increase collaboration and efficiency.
- **Opportunity**: Powerful new software allows for the development of new products, services, and capabilities by using digital tools to improve real world outcomes. Industries ranging from construction to aerospace have implemented these techniques to bring down costs and increase productivity. NRIC is leading the way to begin applying these digital tools to advanced nuclear concepts.



Digital Engineering in Design – Versatile Test Reactor

- The VTR will provide support for progress in multiple important science and technology areas including:
 - Testing and qualification of advanced reactor fuels.
 - Testing and qualification of innovative structural materials.
 - Testing of innovative components and instruments.
 - Validation of advanced modeling and simulation tools, and the versatility to support future technical missions.
- Advanced digital engineering ecosystem across design and construction: Connection of requirements management, BIM, pipe stress, seismic, traceability analysis, etc.



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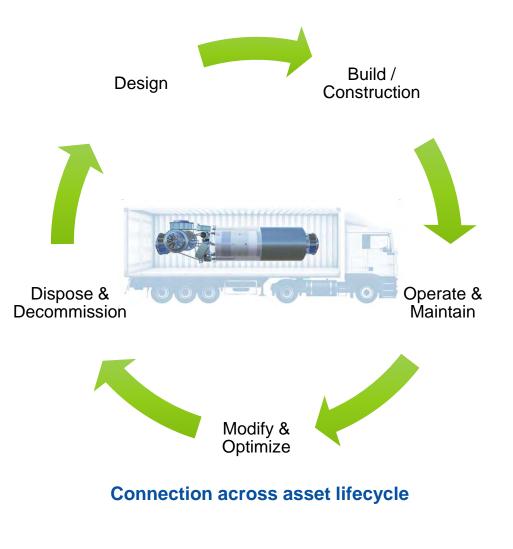
Digital Engineering in Operations – NNSA Digital Twin

- State of the Art: Safeguards analysis is typically SME based without models; When models exist, they are disconnected, have no AI/ML integration, and no digital twin capabilities
- Problem: Development of new advanced reactors (Gen IV) increases importance of new methods to understand diversion and misuse scenarios and determine mitigation pathways
- **Opportunity:** for comprehensive understanding of nuclear fuel cycle facility operations to significantly strengthen nuclear safeguards and nonproliferation regime
- Future opportunity: to support diversion/misuse detection for both item (LWR) and bulk (MSR) type advanced reactors. As well as indicators for clandestine reactors



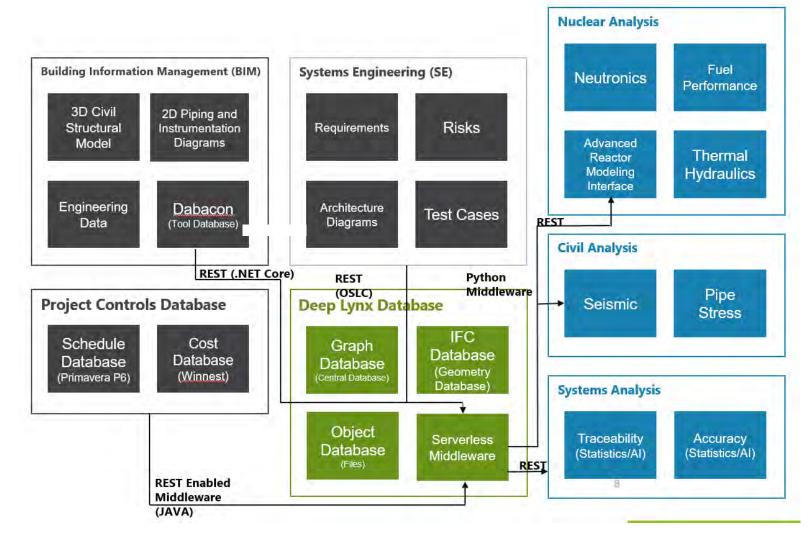
DIAMOND Ontology

- Project Objective(s): This ontology allows for a generic, common framework to enable digital engineering programs. Like previous successful Idaho National Labs initiatives (ex. MOOSE), this data ontology will allow for a common framework to be shared, allowing for more complex energy projects to be undertaken and utilize a plug and play model.
- **Technical Challenges**: (1) Ontological compatibility with other domain ontologies: Mitigated through BFO use (2) Right sized ontology development to ensure the ontology is deep enough to be useful but flexible enough to support multiple designs (3) Verification of the ontology to ensure that functional specifications are executable; this is mitigated by the use of the Monterey Phoenix event trace system
- **Approach**: (1) analysis and selection of top level meta models (BFO/LML) (2) development of lower ontological decompositions for nuclear design using subject matter input to create an easily extendable ontology framework (3) validation and verification of the DIAMOND ontology for nuclear reactor behavior models using Monterey Phoenix (MP)



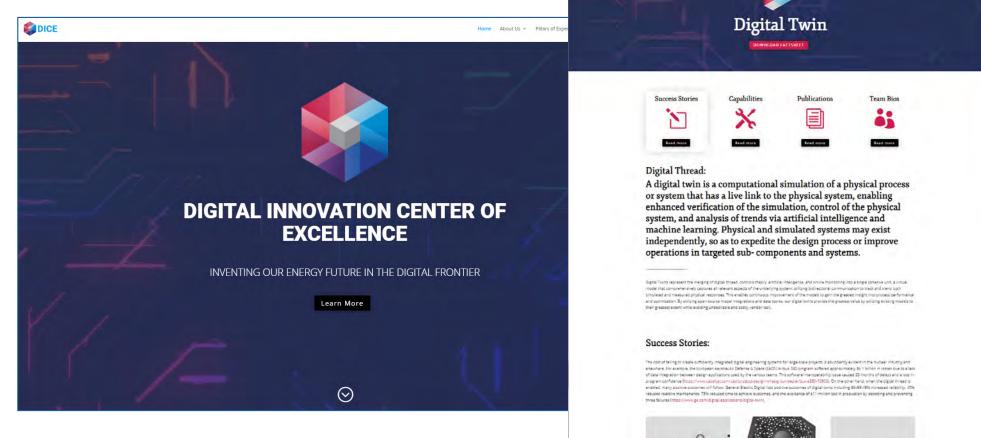
Deep Lynx Datawarehouse

- Ontology: Utilizes ontology for a standardized, common data model to enable a generic framework independent of tool/solution
- Central Software Framework: This allows for a common software framework to be shared, allowing for code reuse and minimal point-to-point integrations
- Central Datastore: This is utilizing the Microsoft Azure Postgres Hyperscale Database which allows a balance between scalability and historical stability



IDAHO NATIONAL LABORATORY

Digital Innovation Center of Excellence (DICE)



DICE

VTR

FRSATILE

Versatile Test Reactor (VTR)

dice.inl.gov

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HOME ADDITION - DOLLARS OF DESIGNED - CARE STO

W NRIC

National Reactor Innovation Center (NRIC)

Transformational Challenge Reactor (TCR) Program



Any Questions?

- Christopher Ritter
- Director, Digital Innovation Center of Excellence
- Email: <u>Christopher.Ritter@inl.gov</u>
- Phone: 208-526-2657 (office) / 301-910-1818 (cell)



Digital Platform for the Transformational Challenge Reactor

Presenters:

Benjamin Betzler - Design Thrust Lead

Vincent Paquit - Digital/Manufacturing/Testing Thrust Lead

Digital Twin Applications for Advanced Nuclear Technologies

ORNL is managed by UT-Battelle, LLC for the US Department of Energy

This work has been authored by UT-Battelle, LLC, under Contract No. DE-AC05-00OR22725 with the U.S. Department of Energy



TCR is bringing to bear additive manufacturing (AM) and artificial intelligence (AI) to deliver a new approach

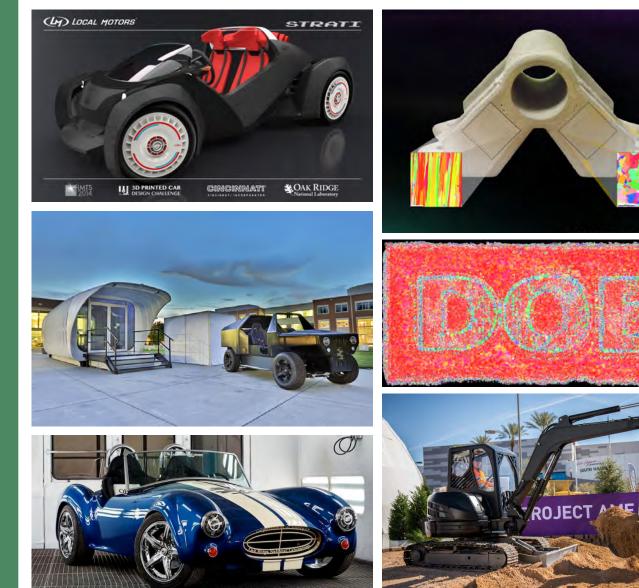
Using AI to navigate an unconstrained design space and realize superior performance Leveraging AM to arrive at highperformance materials in complex geometries Exploiting AM to incorporate integrated and distributed sensing in critical locations Using AI to assess critical component quality using in situ manufacturing signatures



tcr.ornl.gov



Scientific drivers



- Certification of AM components by conventional methods eliminates the business case for AM components
- Limited understanding of local and global processing state for additive manufacturing

Develop new certification methodologies for manufacturing technologies

TCR – Digital Platform

Objective: Develop a digital platform and associated processes to couple data analytics with design and manufacturing data for use in rapid prototyping and quality evaluations of manufactured products.





CAK RIDGE Digital Platform for Manufacturing

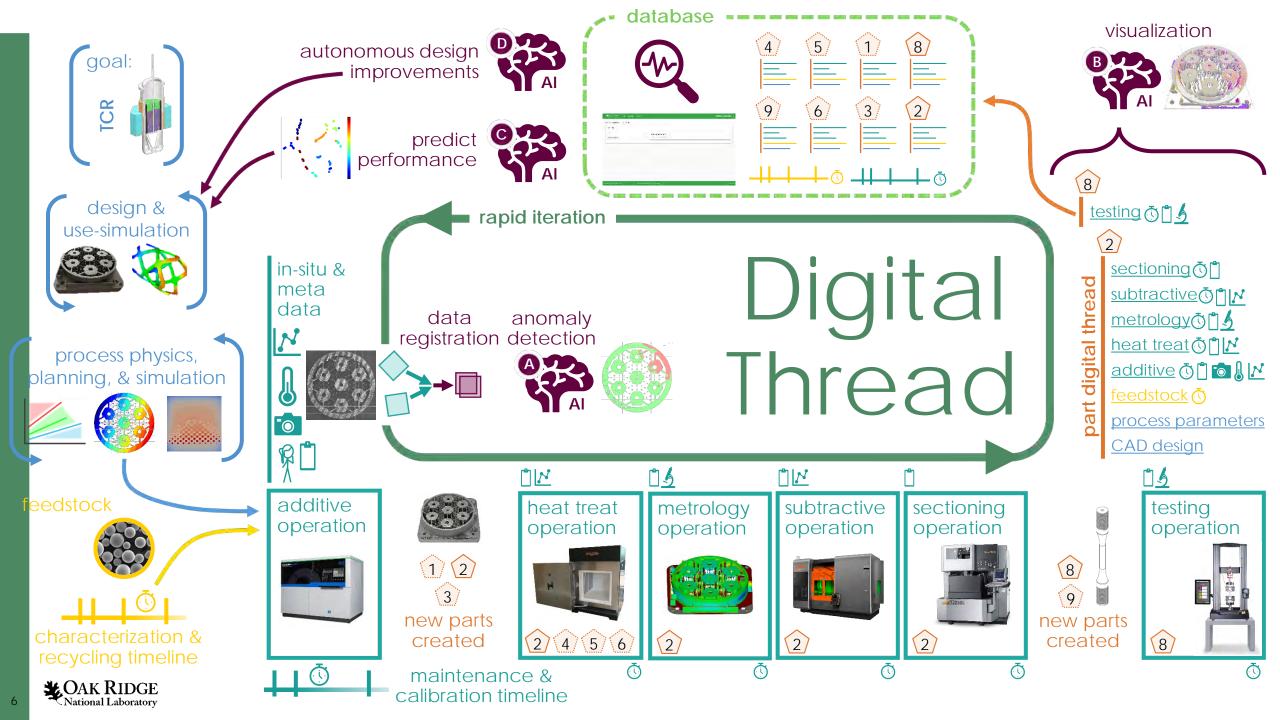


Advanced manufacturing technologies produce valuable datasets at every stage of the manufacturing workflow. Collecting, structuring, and analysis such data is paramount to understanding, optimizing and validating the manufacturing process.

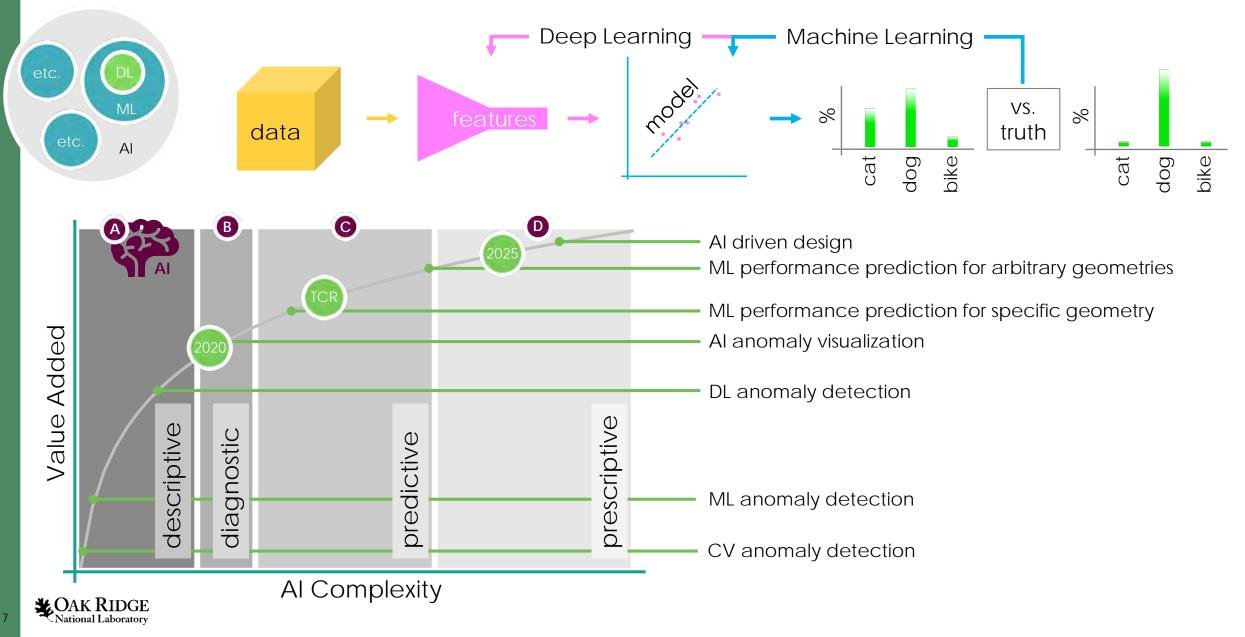
IT Infrastructure	Voxelized Parts	Data Producers	Digital Clone	Data Workflow
• Wired network		Sensor data	Unified Data	Data Management & Tracking
• WiFi network			Architecture	Signal Processing
• IoT		Modeling		Computer Vision & Image Processing
Storage systems		Intent		n-D Data Visualization
HPC systems		In-Situ		Modeling & Simulation
J		in-situ		Data Analytics & Machine Learning
 Embedded Systems 		Ex-situ & Properties	DREAM.3D	Process Optimization
			Open Source Edition	Certification, Verification & Validation

Cybersecurity

Digital Thread



Augmented Intelligence for Advanced Manufacturing



CAK RIDGE Research Activities

Data management

Metadata search

		Owners Lawrence Office 1	UPLOAD SEARCH	EXPLORE	WAITING TO ANALYZE	STATS
Build(s):	ConceptLaserM2-ORNL1				Sharun	
Action	Name	Start Date	End Date	Status	Material	Setup Tec
8	Framatom Arch	2020-02-04	2020-02-04	Suppressful	316L/Provail/22	Alka Sm
16	Airfoils & TCR Moderator Pieces	2020/02-07	2020-02-07	Successful	316L/Prissaid/27	Alka Sin
ĺθ.	Kairos Impeller	2020/02-12	2020-02-12	Successful	316L/Praxair/27	Alka Sin
25	MDF Framatome Factoriers 01	2020 02 26	2020-02-26	Suppreseful	316L/Praxair/27	Alka Sin
125	Fastener Assembly	2020/02-06	2020-02-06	Suppressful	316L/Prasair/27	Alka Sin
36	Framatome Fastener Components	2020-02-14	2020-02-14	Suppressful	316L/Frasair/27	Alka Sin
10	TCR Moderator Preces	2020-02-03	2020-02-03	Successful	316L/Presait/27	Alka Sin
. 8	Framatom Middle Section	2020-02-05	2020-02-05	Successful	316L/Presait/22	Alka Sin
8	Inner Mask Mold Bottom Section	2020-04-08	2020-04-08	Suppressful	-316L/Praxait/27	Alka Sin
8	Theta impeller and TCR Endcaps	2020-00-12	2020-03-12	Successful	316L/Priosait/27	Alka Sin
	LTER BY PARAMETER(S)					

Data viewer

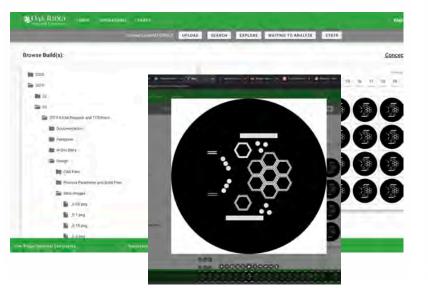
8

In-situ quality control



Sensor development





CAK RIDGE Process Correlation Campaign

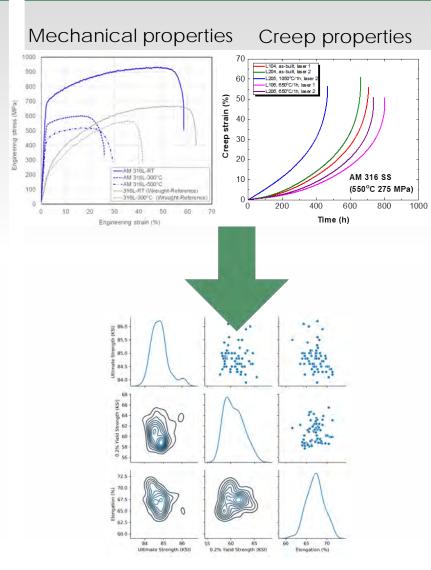
Standard Cluster

Build 0.1 Layout

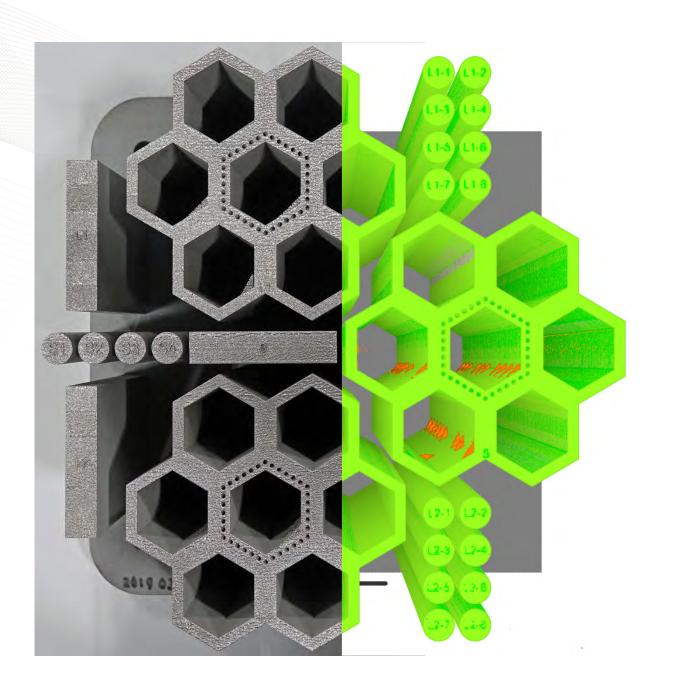
....

2,784 SS-J3 specimens

Data Correlation



Location Specific Sample Extraction



Questions?

Contact: paquitvc@ornl.gov betzlerbr@ornl.gov





Digital Twin-Based Asset Performance and Reliability Diagnosis for the HTGR Reactor Cavity Cooling System Using Metroscope

Eric Helm

12/02/2020

Restricted Framatome

Digital Twin-Based Asset Performance and Reliability Diagnosis for the HTGR Reactor Cavity Cooling System Using Metroscope

Technology Summary

Digital twin-based diagnosis with Metroscope with high reliability for passive and active cooling in SC-HTGR Reactor Cavity Cooling System

Technology Impact

Close an estimated fixed O&M cost gap of \$9.9/MWh to the goal of \$2/MWh by a 50% reduction in key plant staff categories for **overall project impact of \$3.7/MWh** contribution to the ARPA-E mission for the SC-HTGR design.

Proposed Targets

Metric	State of the Art	Proposed		
Fault detection with minimum sensors	Statistical methods need many sensors and intolerant to drop-outs	Reduce O&M for sensor calibration by 50%		
Reliable fault detection	Statistical methods in practice yield majority false positives	Automate diagnosis for aux.systems with >90% reliability		
Minimize effort to perform detection	Burden of asset health is on systems engineers	Commercialize software to reduces sys .eng. effort by 50%		

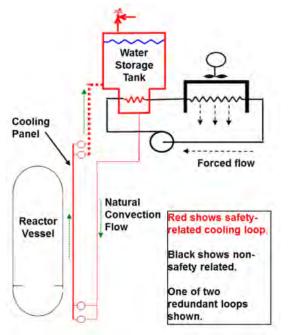
Digital twin-based diagnostics for \$3.7/MWh O&M impact with rapid tech-to-market approach

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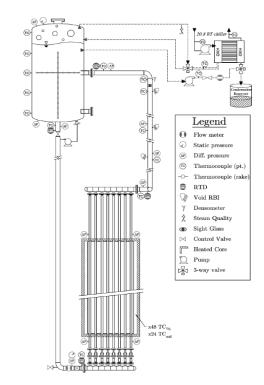
Digital Twin Applications for Advanced Nuclear Technologies Workshop - E. Helm - Dec 2, 2020

Physical System of Interest and Data Source

SC-HTGR Reactor Cavity Cooling System



Natural Convection Shutdown Heat Removal Test Facility (ANL)

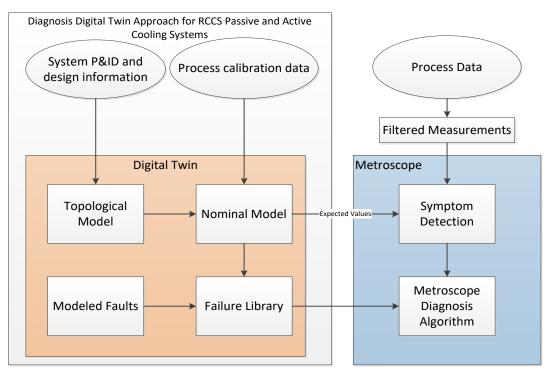


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Digital Twin Applications for Advanced Nuclear Technologies Workshop – E. Helm – Dec 2, 2020

Technical Project Scope

- Develop the modeling approach
- Model the NSTF (RCCS) digital twins
- Calibrate the digital twins with NTSF data
- Develop the failure library
- Test and validate failure detection performance



Develop the generic capability to determine needed sensors and find faults with no operating fault data

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What is Metroscope?

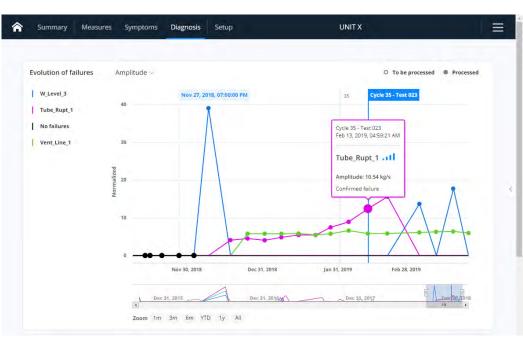
METROSCOPE is Knowledge-Based AI (as opposed to Statistical AI and Machine Learning).

It relies on 2 core features:

- a knowledge base (the Digital Twin) embedding expertise,
- an inference engine (the AI).

It is meant to address problems and decisions where engineers need both expertise and data

METROSCOPE software provides the software tools for automated diagnosis and visualization of fault characteristics.



Metroscope combines digital twin methodology with automation and easy to understand software

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Technical Challenges

- Address more than one single steady-state system mode
- Determine main failure modes of the RCCS and model in a failure library
- Find or generate meaningful validation data for failure modes
- Determine the best sensor + soft sensor mix

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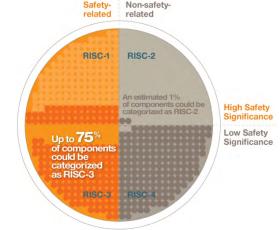
Value and Impact Challenges

- Prove meaningful failure detection impact related to consequential functional failures
- Link to inspection actions that would have formed the regulatory basis for the place
- Demonstrate the cost savings

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Technology-to-Market Challenges

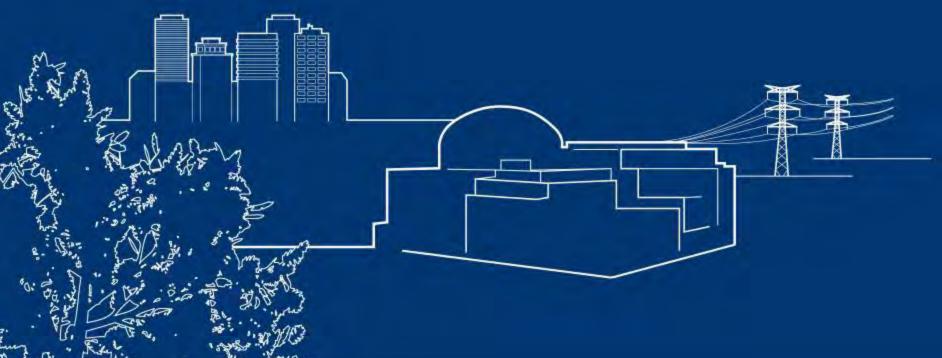
- Adapt the Metroscope method quickly
- Join with utility partner to estimate the value-added for a sample PWR or BWR system
- Examine the viability of risk informed approaches (10 CFR 50.69) to leverage this type of monitoring for increased safety assurance and savings
- Follow the HTGR research with a LWR pilot



8

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framatome Thank You!



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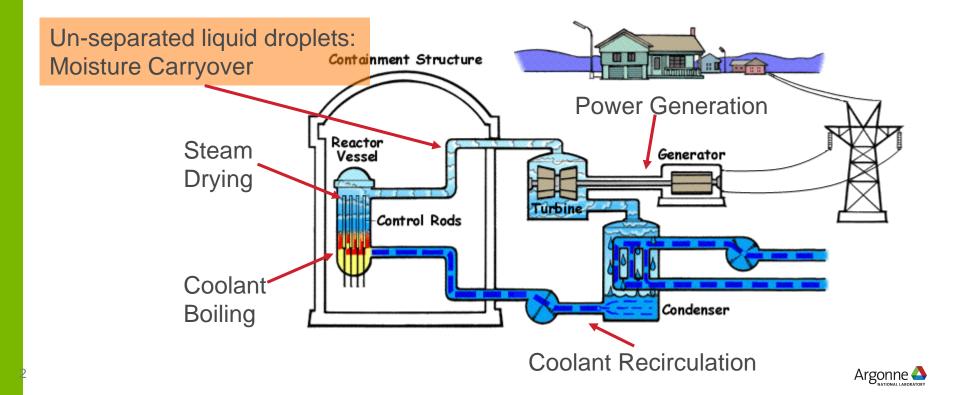
DATA-DRIVEN OPTIMIZATION OF MOISTURE CARRYOVER IN AN OPERATING BWR



RICK VILIM, HAOYU WANG Nuclear Science and Engineering Division Argonne National Laboratory

Workshop on Digital Twin Applications for Advanced Nuclear Technologies December 1-4, 2020

BWR STEAM SUPPLY SYSTEM



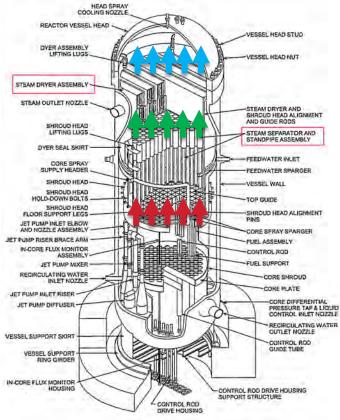
STEAM DRYING PROCESS

Steam drying in GE BWR/4 reactor :

(1)Steam Separator, upgrading the steam quality from ~30% to ~90%;

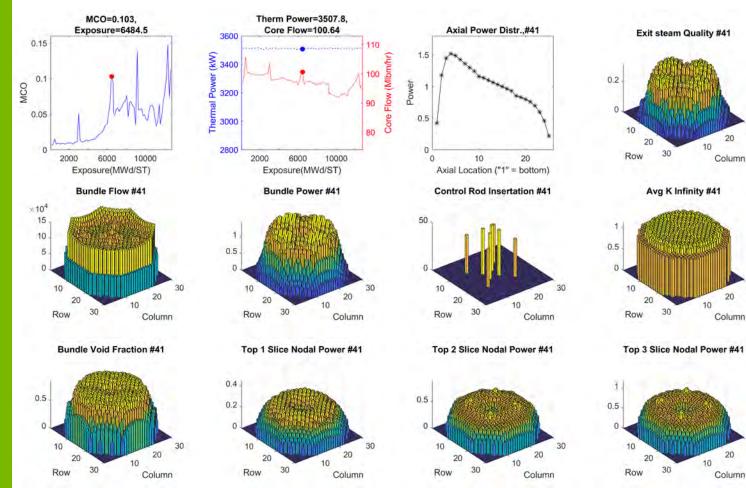
(2)Steam Dryer, upgrading the steam quality from ~90% to ~99.9%.

Saturated Steam Separators will elevate the Moisture Carryover





MCO DATA



Data:

30

20

20

- 6 completed Cycles; ۰
- 540 Measurements; ٠
- 7,000+ variables; ٠

Task: 30

Model and Predict MCO

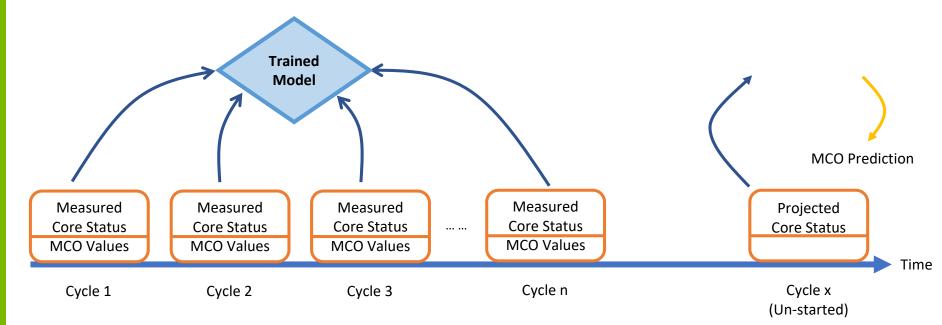
Keys:

- Feature Selection •
- Diversity of training . samples



OUT-OF-CYCLE PREDICTION: OBJECTIVE

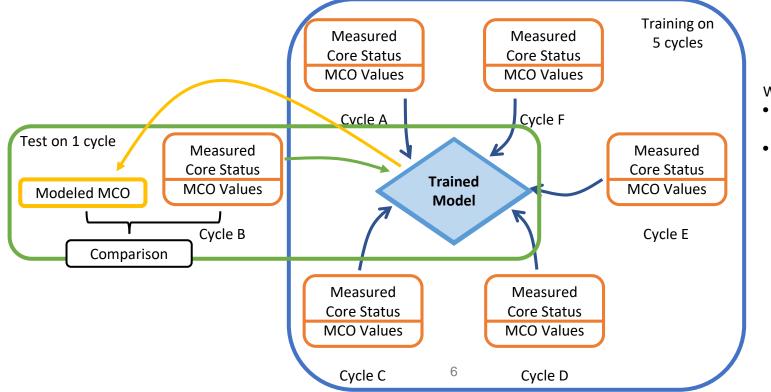
Accurate prediction of the MCO of an un-started cycle, using the projected core status (operating plan).





OUT-OF-CYCLE PREDICTION: METHODOLOGY

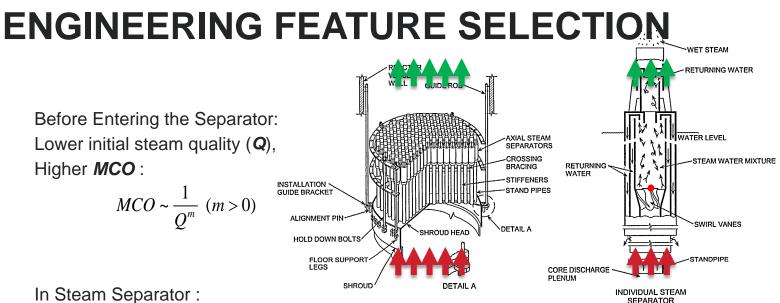
Using 6 cycle data from commercial reactors, train model on 5 cycles, then test on 1 cycle



We assume:

- Both reactors are identical in structure
- All cycles share the identical loading plan.

Argonne



In Steam Separator :

Mixture passes swirl vanes, "Centrifuge"; Liquid Drops hit the wall and get separated.

```
Lower Liquid Velocity (V_I),
Higher MCO :

MCO \sim \frac{1}{V_{I}^{n}} (n > 0)
```

Combining both Q and V_{I} , Define a Local Feature K:

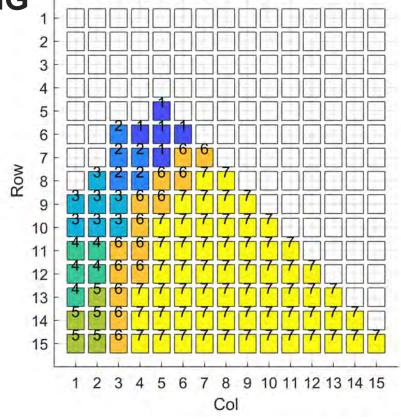
$$K = \frac{1}{Q^m \cdot V_L^n} \ (m > 0, n > 0)$$



FUEL LOADING AND GROUPING

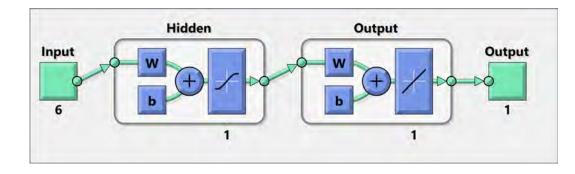
- In GE BWR/4 reactor core: 764 bundles;
- In Each 45 degree section: 101 bundles.
- Reducing the input feature space by a factor of 7, which reduces the computation demand while avoids repeated variables;
- Increasing the number of points by a factor of 8.

Bundle Partition Scheme, 7 Groups



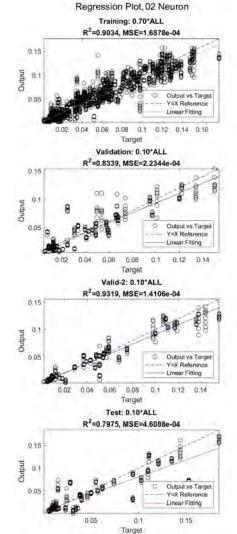


DATA DIVISION FOR VALIDATION



Divide Data into 4 Sets

- Training: Optimize neural network parameters, aiming at lowest MSE;
- Valid #1: Stop training when the MSE of Val.#1 stops decreasing, avoid overfit;
- Valid #2: Not used in model training process, for model selection only;
- Test: Independent test, providing a measure of model predictability.

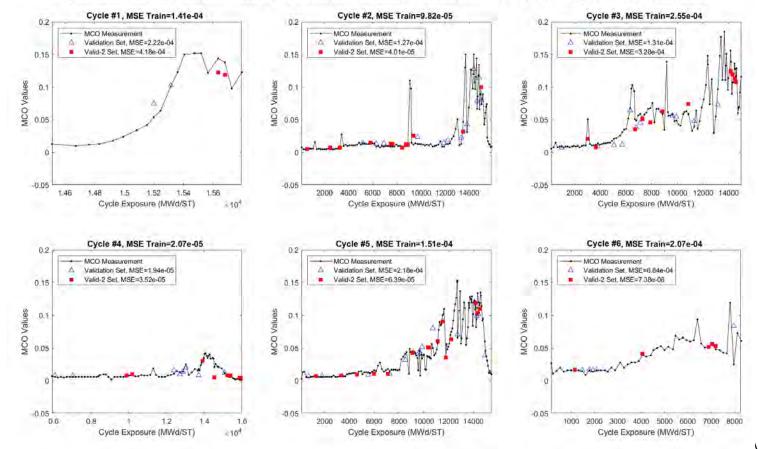


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IN-CYCLE MODELING: VALIDATION

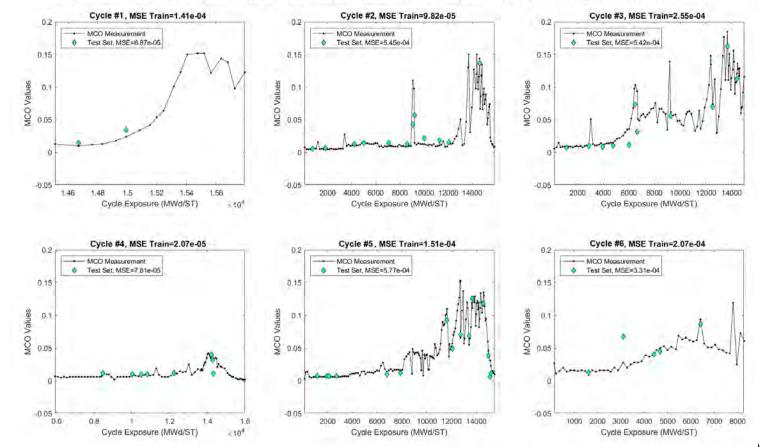
In Cycle Model #199, 70% Train(MSE=1.69e-04), 10% Valid(MSE=2.23e-04), 10% Val2(MSE=1.41e-04), 10% Test(MSE=4.61e-04), 02 Neuron





IN-CYCLE MODELING: TEST

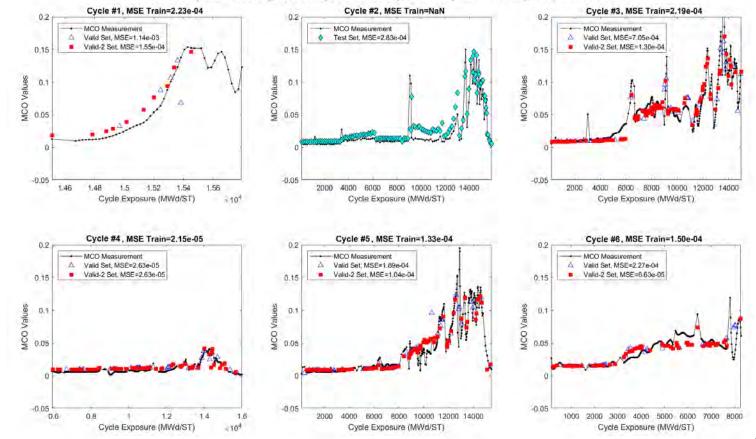
In Cycle Model #199, 70% Train(MSE=1.69e-04), 10% Valid(MSE=2.23e-04), 10% Val2(MSE=1.41e-04), 10% Test(MSE=4.61e-04), 02 Neuron





OUT-CYCLE MODELING: TEST

00% MovHrz Cycle #2, Model #604, Prog_AL_Qn2VLn2_Q00VLn1, 02 Neuron 70% Train(MSE=1.55e-04), 10% Valid(MSE=3.65e-04), 20% Val2(MSE=1.01e-04), Independent Test





Business Confidential

THANK YOU





Role and Status of VR/AR/MR in Digital Twins in the Nuclear Industry

Rizwan-uddin Nuclear, Plasma, and Radiological Engg. University of Illinois

December 1 – 4, 2020 (Virtual) **Workshop on Digital Twin Applications for Advanced Nuclear Technologies** (ANL/INL/ORNL/...)

ILLINOIS | NPRE

Digital Twins

- There are as many (slightly varying) definitions of digital twins as there are people developing and using them.
- "Digital twins," are emulations of real systems that contain simulated communication networks, devices, and other cyber and physical components.
- Live data streamed into the digital twin add additional possibilities



Digital Twins

- Goal is to optimize asset performance and utilization.
- Use it for monitoring, diagnostics and prognostics
- Combine sensory data, with past experience and human expertise to improve performance.
- Use digital twins to find root cause of issues and improve productivity



Digital Twins and VR/AR/MR

 While most other aspects of "digital twins" can be taken advantage of using just the "digital twin", any desired improvement in design and operational efficiency or in safety assessment that involve human factors and human machine interface will benefit from a marriage between digital twin and the VR/AR/MR technology.



- VR/AR/MR allow the traditional 2D representations of the digital twin (or of parts of it) to be presented in 3D.
- Add interactivity



Quick overview of VR, AR and MR in the context of digital twins



Nuclear, Plasma, and Radiological Engineering, Illinois

VR/AR/MR

- Just the way there are many differing definitions of "digital twins," so is the case with AR and MR.
- But, it is getting better; and some sort of consensus is developing.



Nuclear, Plasma, and Radiological Engineering, Illinois

Virtual Reality (VR)

- Physical world is hidden; and the human being (as opposed to an avatar) is *fully immersed* in a "digital" world; with a strong sense of being present (only) in that "digital world".
- Presence in the digital world can be very "strong", leading to dizziness (roller coaster)





Virtual Reality (VR)

- Immersive
- Interactive

- Typical headsets:
 - Oculus Rift
 - Oculus Go
 - HTC Vive

Images from: https://www.oculus.com/



HTC VIVE



www.pbtech.com

www.amazon.com





VR (functionality, and uses)

- User of VR can simply be a stationary "spectator" or can walk around the digital environment.
- In more advanced VR applications, the physical user can interact with her digital surroundings. For example, the user with the help of a hand-held device, can shoot at digital objects, press a "virtual/digital" button to raise or lower a "virtual" control rod, or press a "virtual/digital" button to start a "virtual" pump. It can be used to virtually assemble a set of virtual parts.



VR in Nuclear

- Real locations can be displayed without risking radiation exposure
- Data can be displayed in the context of its environment
- Training using VR before actual attempt

• Industry involvement:



Tan

VR (functionality, and uses)

- Training and Education, and its use in dose reduction for plant workers was considered to be two of the main applications of VR in the nuclear industry
- A VR model of the actual physical plant when complimented with a digital twin is likely to provide an even better environment for increased operational efficiency and safety of NPPs



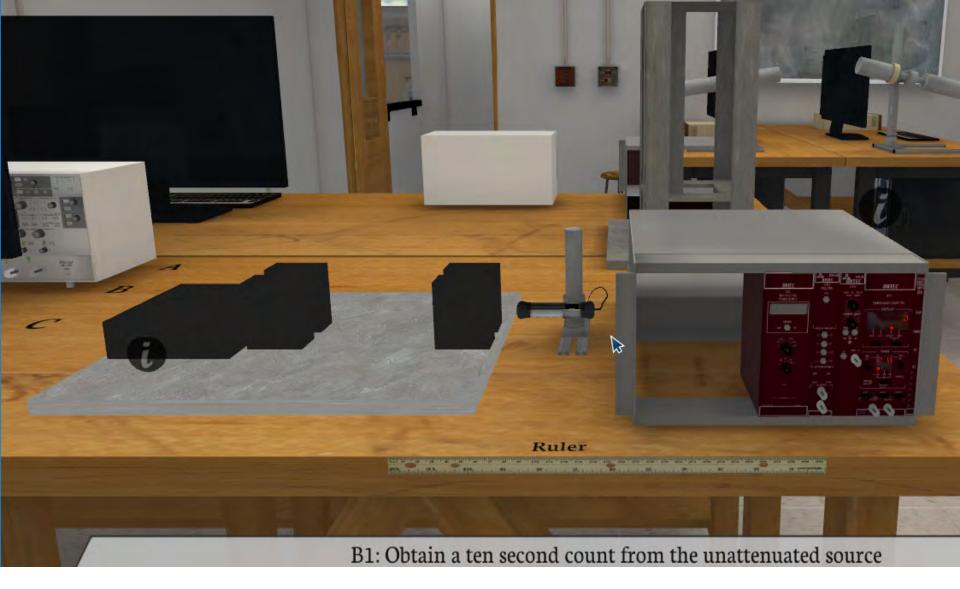




- The bottom line is that in a VR system, "everything" is digital/virtual.
- There is no interaction with the surrounding "physical" world.



Nuclear, Plasma, and Radiological Engineering, Illinois



VR Use Case: Lab Training

VR in a Digital Twin

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Augmented Reality (AR)

- One definition of AR:
- Augment the information you get when looking at a physica scene through the camera of your cell phone
- So, it is the physical world around you, but the information is augmented *digitally* on your camera screen.





Augmented Reality (AR)

- Example:
- You turn your cell phone camera on, and aim it at a building. A software then adds (overlays) a text that gives you information about the scene (Eifel Tower, Statue of Liberty, Taj Mahal)
- A camera on your car projects the scene in front of you on a screen in your car, and overlays direction or other information (GPS-based info)



Augmented Reality (AR)





Nuclear, Plasma, and Radiological Engineering, Illinois

Augmented Reality: functionality and uses

• Training (overlay information)

(Using *Unity* and the *Vuforia* Engine, we can scan and detect 3D objects, and place buttons, video players, buttons, and lines to describe components of those 3D objects.)



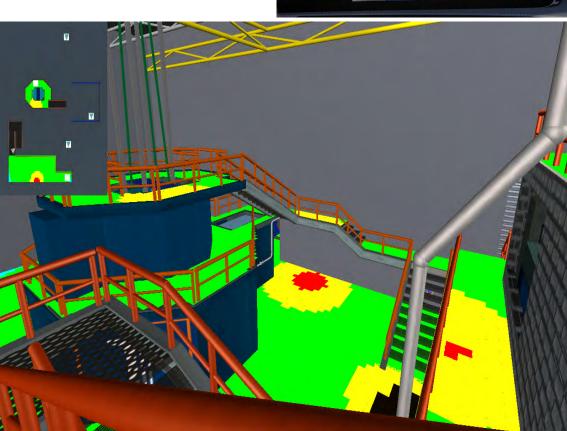


Nuclear, Plasma, and Radiologica

Augmented Reality: function and uses

- Dose reduction

 (Overlay a
 transparent
 radiation field on
 the scene; and
 show a path of leas
 dose")
- Sky is the limit



Al-Zalloum, MS Thesis, Illinois. APPLICATION OF SHORTEST PATH ALGORITHMS TO FIND PATHS OF MINIMUM RADIATION DOSE, 2009





Mixed Reality

- In mixed reality, one can see the physical world (without a camera), but "digital" objects can be superimposed in the physical world.
- Needs special gadgets such as Microsoft's Hololens









Nuclear, Plasma, and Radiological Engineering, Illinois

Mixed Reality (MR)

• MR headsets display holograms for the user





https://www.microsoft.com/en-us/hololens





Holograms are digital objects made of light and sound

They appear in the physical world around us.

Holograms can be controlled by gaze, gestures and voice commands



Nuclear, Plasma, and Radiological Engineering, Illinois

Mixed Reality

- As long as "digital twin" is that of an existing facility, MR may have limited application
- However, if digital twin is that of a facility that is under design, or under construction, then several applications of MR can be easily imagined





A 2018 paper from Japan

JOURNAL OF NUCLEAR SCIENCE AND TECHNOLOGY 2018, VOL. 55, NO. 9, 965–970 https://doi.org/10.1080/00223131.2018.1473171

RAPID COMMUNICATION





NPRE

OPEN ACCESS OPEN ACCESS

Radiation imaging using a compact Compton camera inside the Fukushima Daiichi Nuclear Power Station building

Yuki Sato^a, Yuta Tanifuji^a, Yuta Terasaka^a, Hiroshi Usami^a, Masaaki Kaburagi^a, Kuniaki Kawabata^a, Wataru Utsugi^b, Hiroyuki Kikuchi^b, Shiro Takahira^b and Tatsuo Torii^a

ABSTRACT

The Fukushima Daiichi Nuclear Power Station (FDNPS), operated by Tokyo Electric Power Company Holdings, Inc., went into meltdown in the aftermath of a large tsunami caused by the Great East Japan Earthquake of 11 March 2011. The measurement of radiation distribution inside the FDNPS buildings is indispensable to execute decommissioning tasks in the reactor buildings. We conducted a radiation imaging experiment inside the turbine building of Unit 3 of the FDNPS by using a compact Compton camera and succeeded in visualizing high-dose contamination (up to 3.5 mSv/h). In addition, we drew a three-dimensional radiation distribution map inside the turbine building by integrating the radiation image resulting from the Compton camera into the point cloud data of the experimental environment acquired using a scanning LRF. The radiation distribution map shows the positions of these contaminations on a real space image of the turbine building. The radiation distribution map helps workers to easily recognize radioactive contamination and to decrease their own exposure to radiation because the contamination cannot be observed with the naked eye.



Recent paper from Japan (2018)

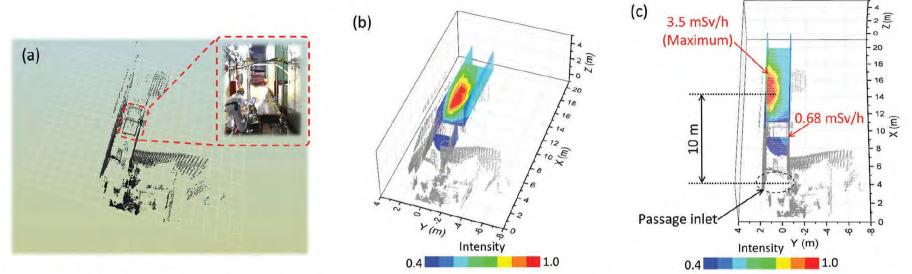
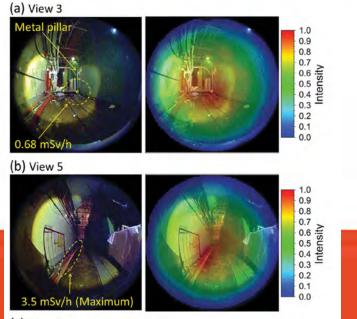


Figure 3. (a) Point cloud data of experimental environment acquired using scanning LRF. Combined metal pipes can be seen. (b,c) Radiation distribution map prepared by superimposing radiation image and point cloud data. In panels (b) and (c), the viewpoints are different.





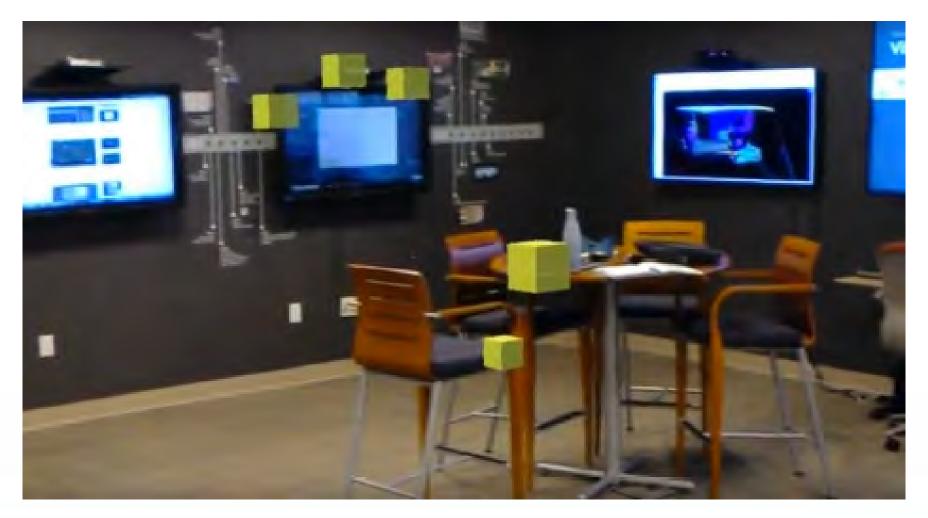
PNNL Student Internship Project

 Model a room and an interactive AR/MR application to collect data at specific locations, and enter data in an online database



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Mixed Reality -- Hololens

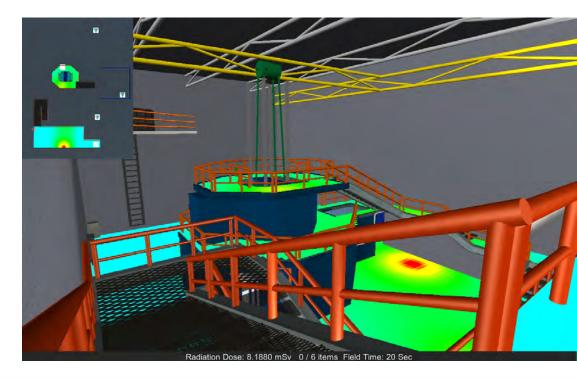




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Dose Minimization Game

 Goal: Collect six objects in the TRIGA reactor building while minimizing dose



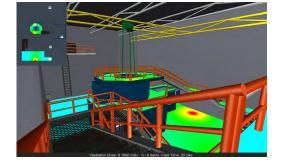
NPRF

- Display radiation map on demand
- Virtual dosimeter

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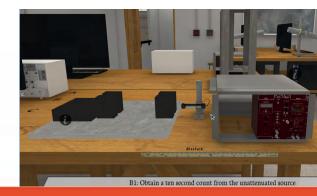








Where do you want to go?







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Thankyou!!



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Campus VR Model

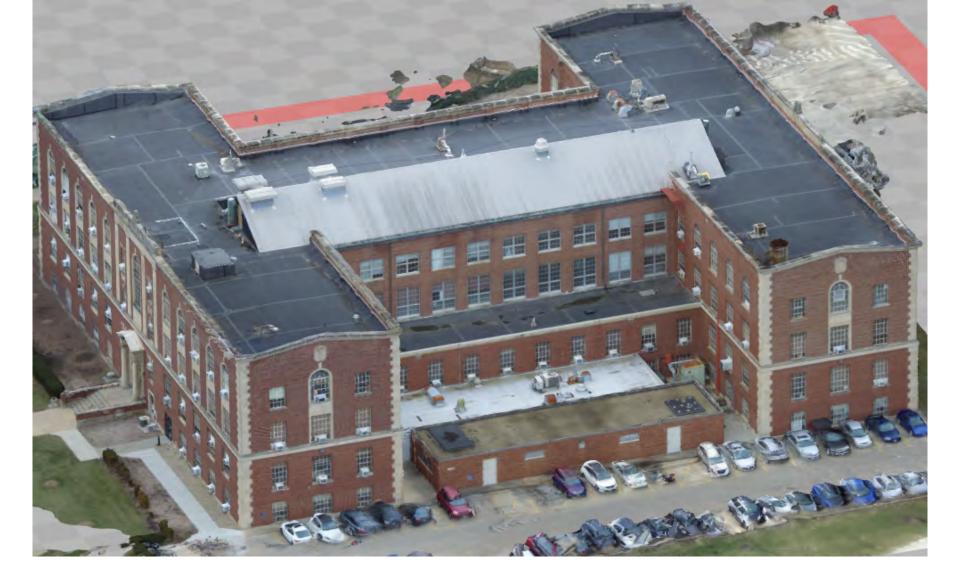




MR in Nuclear

- Headsets can guide a user through collecting radiation measurements
- Collected data can be displayed to the user at its location

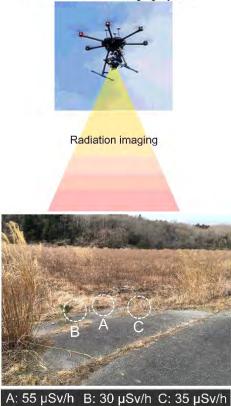




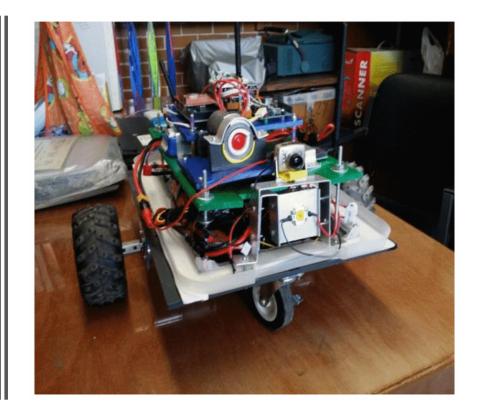
Drone Photogrammetry 3D Model Generated in Agisoft Photoscan

Radiation Measurement Via Drone/Robot

Remote Radiation Imaging System



Yuki Sato, Yuta Tanifuji, Yuta Terasaka, Hiroshi Usami, Masaaki Kaburagi, Kuniaki Kawabata, Wataru Utsugi, Hiroyuki Kikuchi, Shiro Takahira, Tatsuo Torii. (2018) Radiation imaging using a compact Compton camera inside the Fukushima Dailchi Nuclear Power Station building. Journal of Nuclear Science and Technology 55:9, pages 965-970.



Vázquez, R. M., & Gutiérrez, E. (2015, November). Mobile robot for gamma radiation detection with long range remote control. In Mechatronics, Electronics and Automotive Engineering (ICMEAE), 2015 International Conference on (pp. 175-180). IEEE.







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ON-LINE AI/ML & COMPUTATIONAL-MECHANICS BASED PREDICTIVE TOOLS FOR A DIGITAL-TWIN FRAMEWORK

SUBHASISH (SUBH) MOHANTY

Argonne National Laboratory Nuclear Science and Engineering Division

> Argonne National Laboratory is a U.S. Department of Energy laboratory managed by UChicago Argonne, LLC

The results presented are mostly based on research work sponsored through <u>DOE-Light Water Reactor</u> <u>Sustainability Program</u>, <u>US-NRC steam generator tube</u> <u>integrity program</u> and some of the results also based on my PhD thesis work at Arizona State University, sponsored by US Air Force Research Laboratory.

2nd December 2020



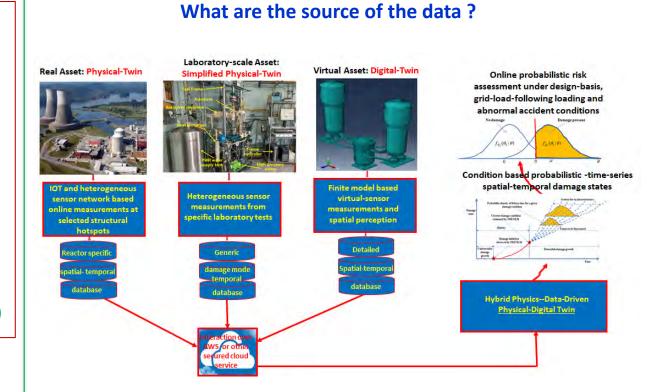
<u>Big Picture of a Digital Twin Framework</u>: In context of reactor component structural state & life prediction

Digital-Twin models:

- ➔ Need to operate in real-time:
 - # To predict the state of structure at a given time.
 - # To predict the remaining life of a structure at a given time.
- ➔ Primarily to be data driven:

Need historical data.

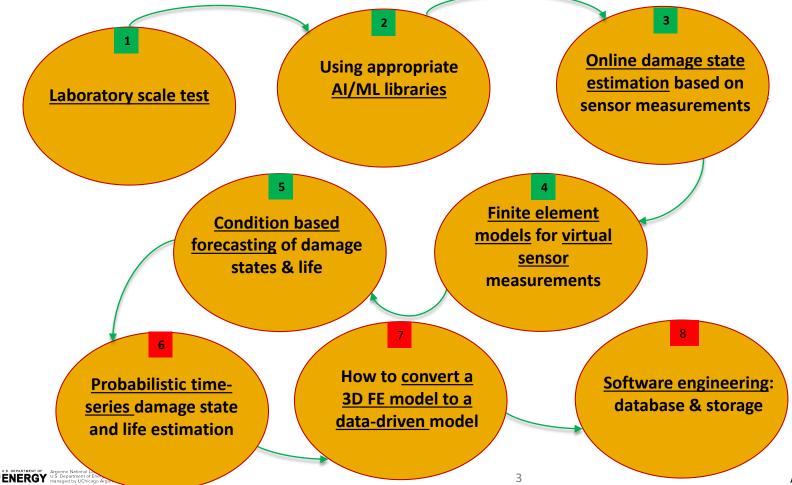
Either from <u>actual sensor</u> measurements or from <u>virtual</u> <u>sensor</u> measurements (e.g. from physics based finite element model)







Outline of my talk: Physical-Digital Twin- What are the building blocks?





Example results: <u>Laboratory scale test</u> for understanding and modeling the <u>material-damage time evolution</u> over entire fatigue life

Why laboratory scale test important:

Getting data from actual reactor component is not easy due to:

Expensive and inaccessibility# Component currently may not be instrumented# Regulatory hurdles for putting new sensor

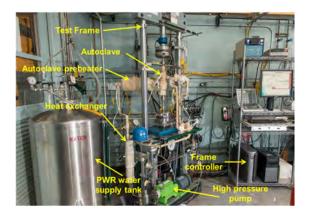
Often getting long duration data is nearly impossible because:

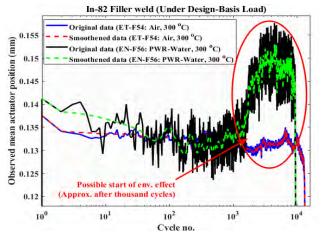
Actual reactor component can have many years of life before failure.

→ Laboratory scale e.g. fatigue test could generate the required historical data

Inexpensive

Possible to capture different failure modes over entire fatigue life from start to final failure.





← ANL's PWR water test loop

← Comparison of in-air versus PWR-water sensor measurements from fatigue tests (under hundreds of design-basis-loading cycles)





Example results: Using appropriate <u>data-driven model & AI/ML libraries</u>

→ Many AI/ML libraries available: such as TensorFlow, Kerras and Scikit-Learn, Apache Spark, Pytorch, Gaussian process, etc.

Need exhaustive qualification

→ Many type of model possible with many combinations of input-output mapping

Need appropriate selection of input features

Need exhaustive qualification of the model

➔ Data-driven models are good for modeling unmodellable nonlinear-complex physics, but

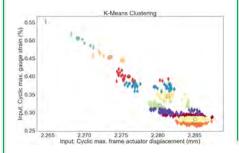
Can be sometime disastrous if simply use as a Blackbox

Results from model should have some physical sense

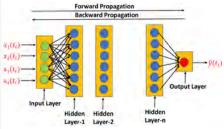
Need domain knowledge understanding

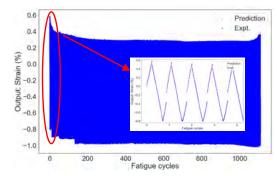
Example of a hybrid Scikit-Learn clustering and Kerras deep-learning based framework for <u>predicting strains</u> <u>from other sensor measurements</u>

Scikit-Learn based K-mean clustering of fatigue test data



Keras based deep learning regression framework





 Prediction of timeseries strains subjected to hundreds of fatigue loading cycles.



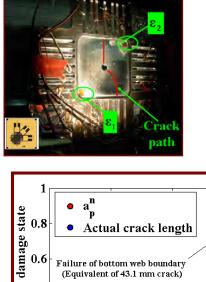
Example results: Online damage state estimation based on

heterogeneous sensor measurements

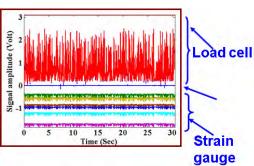
gauge & load-cell measurement based sensor network 000 ... 000 ... 000 ... 000 X 2 00

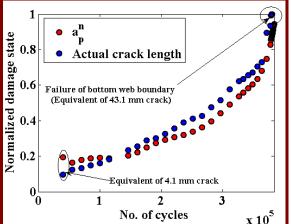
Biaxial test frame (at Arizona state university) with strain-

Instrumented cruciform specimen



Heterogeneous sensor measurements from a single loading cycle





 Online estimated damage states (and their comparison with normalized visually measured crack length) for thousands of fatigue cycle





3

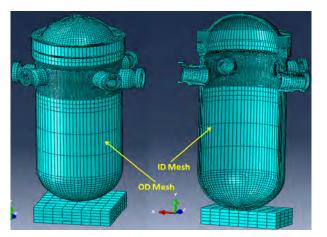
Note: This slide results are based on my PhD Thesis work at Arizona State University

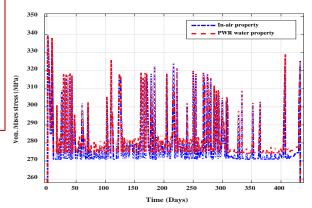
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Example results: Physics based <u>finite element</u> model for <u>spatial-temporal</u> <u>virtual sensor</u> measurements

Why physics based model (e.g. 3d FE model)?

- → Structural damage are in general geometry dependent
 # Need physical measurements at many locations
- Physical measurements at many locations are not possible
 # Expensive to put so many sensors
 # Sometime it is not feasible due to inaccessible locations
- → Developing system level finite element models
 - # not only for predicting the state of the structure at a given time (temporal)
 - # but also for predicting the state of the structure at any given location (spatial)





Pressure
 vessel and
 nozzle assembly
 finite element
 model for a
 two-loop PWR

Stress states (at a typical ID side location of the hot-leg nozzle) over an entire fuel cycle subjected to grid-load-following thermal-mechanical loading cycles.



<u>Condition based prognostics of online forecasting of time-series states & life</u>

Why we need condition based prediction ?

➔ Online monitoring and/or NDE

gives the damage state of the structure at a given time.

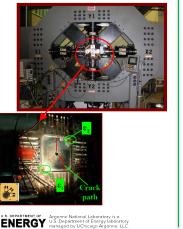
→ How do we know what is going to happen in future ?

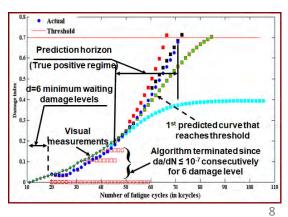
Future structural states at a given time.

How much life left at a given time.

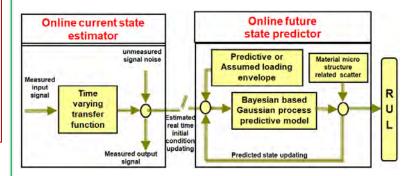
Strain-gauge instrumented cruciform specimen

Forecasted states: Correct prediction horizon spans approx. <u>40% of crack</u> propagation life

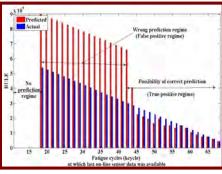




Example: Recursive prognostic (physics-guided) model to predict the fatigue damage state of a structure



<u>Forecasted remaining useful life (RUL)</u>: Good correlation between prediction and actual RUL in true positive regime





Note: This slide results are based on my PhD Thesis work at Arizona State University

THANK YOU





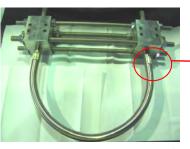
Extra slides

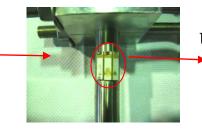




Example results: <u>Types of sensors</u> and sensor network for tracking a particular damage mode (<u>Use of ultrasound sensor for incipient damage tracking</u>)

- → Sensor type selection depends on what type of damage we want to track?
- ➔ How early or severity of the damage to be tracked?
- → Large area damage monitoring and/or few structural hotspots ?

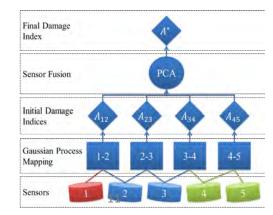




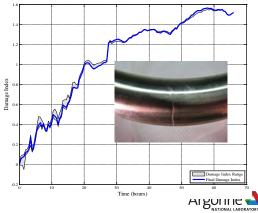
Example online stress-corrosion damage tracking in a steam generator tube using ultrasound sensor network

Ultrasound sensor network

Hierarchical sensor network & sensor fusion based damage state estimator



Online estimated damage states and cracked steam generator tube





Example results: Physics based component-scale model with time-dependent 4b damage initiation modeling subjected to hundreds of fatigue cycles

Damage evolution in a component is function of: →

Cyclic hardening/softening

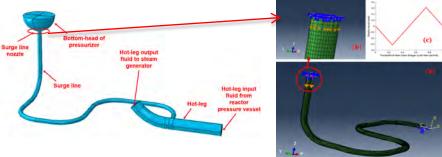
Cyclic interaction of loading and environment

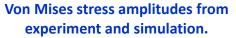
- Spatial-temporal time-dependent state of a component → can be estimated using cyclic plasticity based finite element model.
- Need HPC based parallel computing for modeling → thousands of fatigue cycles.

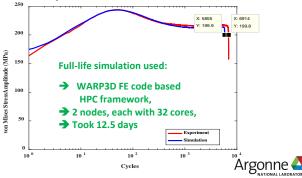
Computation time as a function of CPUs to simulate PWR SL for <u>10</u> fatigue cycles using ABAQUS

Number of CPUs	4	8	12	22
Computation time (hr)	25.1	10.3	8.9	3.5

PWR surge-line finite element model









Example results: Physics based component-scale model with <u>time-dependent</u> <u>damage propagation</u> modeling

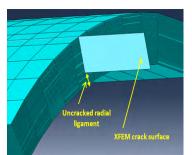
- Demonstration of time-dependent crack propagation modeling through extended finite element method (XFEM).
- → Steam generator tube (of 1.27mm thick) rupture simulation under severe accident condition.
- → With presence of initial crack:

can be through online measurements

can be from NDE measurements

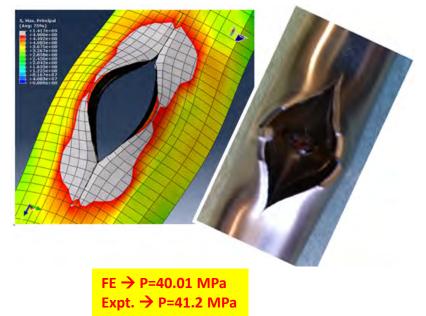
→ Internal applied pressure : P= 0 to burst.

Initial crack ->



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Extended finite element (XFEM) model simulation vs experiment results of a steam generator tube under severe accident conditions





Example results: <u>Probabilistic time-series</u> damage state and life

0.2

100

14

101

Why probabilistic model ?

- Structural damage are inherently dependent on time-dependent material-damage evolution
- Time-dependent material evaluation are stochastic due to

Surface finish of component.

Variation in material microstructure.

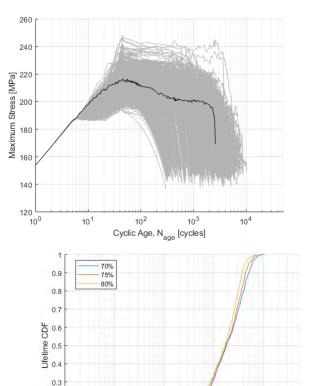
Variation in material interaction with loading and coolant environment.

➔ Probabilistic models such as Markov-Chain-Monte-Carlo (MCMC) can be used for predicting

time-series probabilistic damage states.

probabilistic life.

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 10^{2}

Fatigue Life, N., [cycles]

103

 10^{4}

Experimentally vs. Markov-Chain-Monte-Carlo (MCMC) simulated maximum stress profiles for a 316 SS test specimen, fatigue tested under PWR-water-coolant environment.





Physics-data-driven model: How to convert a 3D FE model to a data-driven model

Why do we need to convert the FE model to data-driven model ?

➔ For an effective predictive model for a complex 3D geometry, sensor measurements are required from many locations:

Geometry dependency !

Physics-based model e.g. 3D FE model can be a choice for having

Virtual measurements (temporal data) at

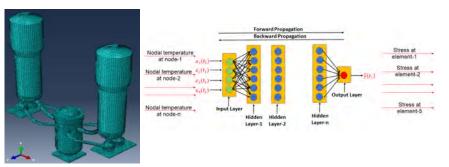
millions of spatial nodal points (spatial data).

➔ However, FE model are good for offline computation not for online.

Computationally intensive – Software license cost

→ Then what we do for online model to capture the spatial damage ?

Schematic of AI/ML based FE model data mapping



System-level AI/ML based data-driven model

→ Need of <u>parallel AI/ML code/libraries</u> e.g. APACHE Spark to operate with timeseries FE data from millions of nodal, elemental and/or integration points

e.g. data = [x1, x2, x3, x4]



distData = sc.parallelize(data)



Software engineering: Common database & Interacting over cloud

- How to store large amount of data from:
 - **#** Directly from plants: Temporal data
 - **# Experiment: Temporal data**
 - # FE and/or CFD models: Temporal & spatial data
- Heterogeneous data source:
 - # Image (Object type database!)
 - **#** Model parameters and time dependent
 - real/virtual sensor data (Relational database !)
 - # Manual entry data (Unstructured data !)
- → Need <u>commonly used database</u> to explore:
 - # e.g. SQL or NO-SQL
- Need common place to store the data for largescale multi-organization working on a common goal:
 - # Cloud: AWS, etc.!

Snapshot of the automatically updated SQL based material model database for reactor Digital-Twin model

Use temperature-dependent data umber of field variables:

Vield Stress

At Zero Plastic

Strain 420140000

421853851

SQL Based Material Properties DATABASE (DB) File

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62	3771	0.28	419.06063842	39638.879516	88315.159097	685.75510921	N
63	3792	0.28	419.90936279	40112.619018	88442.867794	683.20530914	1114
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65	2125	0.28	421.25338745	40130.917015	88932.852798	688.62278192	
56	3949	0.28	420.93289184	40236.851972	89771.396155	695.95528798	
67	3044	0.28	420.88967895	40346.690209	89311.681230	687.47406542	
68	2775	0.28	421.85385131	39935.793681	89365.925409.0	690.45903383	
69	3154	0.28	426.84109497	38147,346689	79673.259011	625.51228225	
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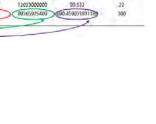
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Parameter C1

ABAQUS Properties Module



Gamma 1

Temp



Qualification of the Pickering A Test Facility

Workshop on Digital Twin Applications for Advanced Nuclear Technologies

Richard Henry (OPG), John Sladek (CNSC) | 1-4 December 2020



Where a brighter tomorrow begins.

Overview

- 1 | What is the Pickering A Test Facility (PATF) and how is it used?
- 2 | What are the regulatory requirements and process for qualification of Software Tools?
- 3 | How was the PATF qualified?

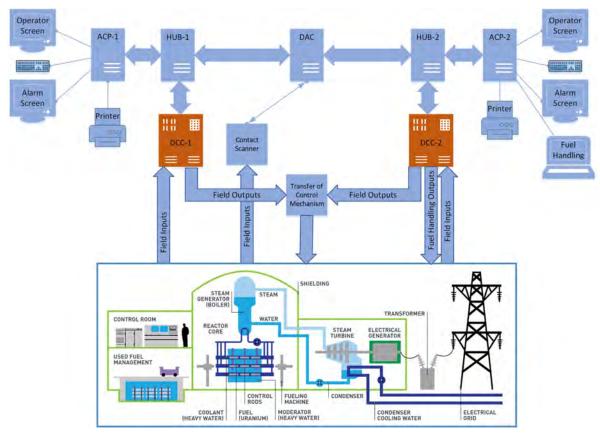


Context

- Pickering A 4x540 MW CANDU Pressurized Heavy Water Reactors
 - Placed in service 1971-73. Currently two units in service. Two units in safe store.
- Digital Control Computers (DCC) control major plant processes (Reactor power, Boiler pressure, Online fuel handling)
- Obsolescence issues with legacy test facilities used for software Verification and Validation testing.
- A software-based test facility (digital twin) was developed.
- Qualified September 2011
- Followed existing
 qualification processes
- No new processes
 required



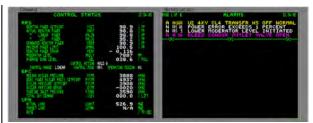
DCC System Context



Pickering A Test Facility Overview

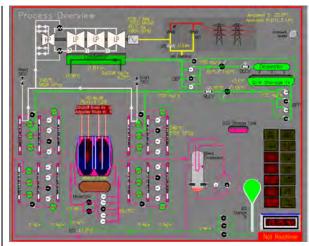
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- High-fidelity DCC <u>Emulation</u>
 - Function and timing
 - Instructions, peripherals, process input/output, interrupts





- <u>Simulated</u> Operator Human Machine Interface
 - Annunciation computer, keyboard and displays
 - Control room panels
- Contact alarm scanner



- Plant Simulation
 - Process models
 - Simulated relay logic
 - Simulated stand-alone devices (e.g., PLCs, digital meters)

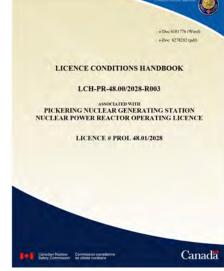
Test Execution for V&V Activities

- Software engineering tool used within the development lifecycle
- Test execution controlled by scripting language (Python)
 - Able to control DCC, simulation, etc.
 - Can simulate operator actions (keyboard, panel switches)
 - Can override simulation with substitute values
 - Observe/record process parameters or internal DCC states
- Save and restore "storepoints"
 - Available for all major plant modes of operation (0%-100%FP)
 - Process model state
 - Internal DCC state (memory, disk, registers)
- Allows for development of fully-automated repeatable tests
- Useful for training new staff in DCC fundamentals
- Scenario based analysis of control algorithm changes

```
# Restore 100% full power storepoint
restore('/home/patf/storept/FPSS ibm1800.stp')
# Set Digital Input 0x41, bit 10 on DCC1 off
# Set Digital Input 0x41, bit 10 on DCC2 off
dcc1.di[0x41].b10=0
dcc2.di[0x41].b10=0
# Print simulated reactor neutron power
print "Starting Power: ", getcdb ('RNTPOW')
# On ACP1, pull up the RRS display and
# decrease reactor power by 10% at rate 3
pace.acp1.keypress("<RRS>")
pace.acp1.keypress("RD3<ENTER><EXECUTE>")
\# Run for 30 seconds
run for (seconds=30.0)
# Print simulated reactor neutron power
print "Ending Power:", getcdb ('RNTPOW')
# Print DCC1 Analog Input @Address /1D01
# in ADC counts and eng. units
print dcc1.ai[0x1D01], dcc1.ai[0x1D01].eng
# Display core location 0x1800
print dcc1.core [0x01800]
# Print dcc registers A, Q and I
print dcc1.a, dcc1.q, dcc1.i
```

Regulatory Requirements for Software QA

- The Pickering Nuclear Generating Station License and Condition Handbook (Effective 17 April 2020):
 - "The licensee shall implement and maintain a management system"
 - Licensing basis publication: CSA N286, NPP **<u>OA Program Requirements.</u>**
 - Compliance Verification Criteria include:
 - OPG N-CHAR-AS-002, Nuclear Management System,
 - OPG N-PROG-MP-0006, Software
 - "The licensee shall implement and maintain a design program."
 - Guidance: N290.14, <u>*Qualification of Digital Hardware and Software</u></u> for Use in Instrumentation and Control Applications for Nuclear Power Plants</u>*
 - "The licensee shall implement and maintain a safety analysis program."
 - Guidance: N286.7, Quality assurance of analytical, scientific and design computer programs for nuclear power plants



Software Qualification

- N-PROG-MP-0006, Software, defines processes for all types of software including,
 - <u>Software Engineering Tools</u> follows OPG N-STI-69000-10002, *Qualification of Software Engineering Tools*
 - This is key to the qualification of PATF for the use case for V&V of software
 - Testing requirements based on categorization of **Target** software (RTPC or SESA)
 - Real Time Process Computing (RTPC). QA Requirements based upon software classification:

Organizations or Countries	Safety	ns in nuclear plants		
United States	S	(not specified)		
	Safety-Related	Safety-Related 5		
Canada	Category 1	Category 2	Category 3	Category 4

http://www.world-nuclear.org/uploadedFiles/org/WNA/Publications/Working Group Reports/safety-classificationfor-iandc-systems-in-npps.pdf

¹⁸ • Scientific, Engineering and Safety Analysis (SESA) software follow CSA N286.7 QA requirements

N-STI-69000-10002

• Qualification requirements for software engineering tools:

Software engineering tools are those used to <u>support any aspect of the software engineering lifecycle</u>, including: requirements gathering and specification; design and code production; review and static verification of requirements, design and code; <u>test case generation, execution, and results analysis</u>; configuration management and change control; and training.

- Method
 - 1. Identify target software classification and categorization
 - 2. Select tools
 - 3. Determine impact severity of tool failure
 - 4. Determine mitigating circumstances
 - 5. Select a qualification approach
 - 6. Perform qualification activities
 - 7. Configuration management of software engineering tools
 - 8. Software engineering tool qualification report

Determine Severity of Tool Failure

- Guidance: "Consider the failure modes for **each use** of the **software engineering tool**, and identify the relevant failure effects on the target software. Classify each failure effect based on its potential impact on the safety, functional, reliability, performance or security requirements of the target software."
- Classification Scheme:

Failure Type	Description
Direct	the tool is incorporated in the target application. Tool is to be considered pre-developed software and qualified to the same degree of rigor as the target software.
Indirect-Causal	Tool failure can introduce errors in the target software which if undetected could result in the target software failing to meet the above requirements.
Indirect-Preventive	A tool failure effect can result in the non-detection of errors in the target software which could result in the failure of the target software to meet the above requirements.
Minimal	A tool failure could have an impact on the target software but no mechanism has been identified that could result in the target software failing to meet the above requirements.
No Impact	A tool failure can have no impact on the target software in meeting the above requirements.

Identify and Classify Mitigations

- Identify any mitigations that eliminate or reduce the impact of the failure effect.
- Classify mitigations as one of the following:

Class	Description
None	
Single	 Single reliable mitigating activity or procedure which defends against impact. Must be independent of the failure effect (efficacy of mitigation not diminished or nullified by the failure effect). Examples: Testing, review, checksum comparison
Multiple	 Multiple reliable mitigating activities or procedures which defend against impact. Must be independent of the failure effect. Must be independent of each other (having no other common failure mechanism) Example: Review of outputs by two independent individuals using different methods.

Determine Qualification Approach

- Qualification grade is determined based upon:
 - Target software classification and categorization
 - Impact severity of tool failure
 - Mitigations
 - Result is: NSR2, NSR3, O, and A
- Qualification method is based on grade
 - NSR2 / NSR3: Follow RTPC Category II/III qualification method (e.g., CSA N290.14)
 - A: Follow SESA qualification method (e.g., CSA N286.7)
 - O: Select qualification method from. For example:
 - Acceptance testing
 - Widespread industry usage (for same purpose)
 - Operating history from third party

Results for PATF

- Target Software Classification and Categorization: System is used for testing of DCC software which is Categorized.
 → Category 2.
- <u>Impact</u>: failure of the software test tool could result in nondetection of errors in the target software.
 - → Indirect-Preventive
- <u>Mitigation</u>:
 - Several sets of tests are performed by independent individuals (e.g., unit testing, subsystem testing, integration testing and validation testing)
 - However, some of these tests could all potentially make use of the PATF, so there could be a common failure in the mitigating activities. → Single
 - Qualification approach is "O" (as per lookup table)
 - Qualification method selected to be "Acceptance"

	Table 3.5-1 Qualification G				
Software Categorization		Mitigating Circumstances			
and Classification	Impact of Failure Effect	None	Single	Multiple	
	Direct	See Note below			
Nuclear Safety Related Category I	Indirect-Causal	NSR2	NSR3	0	
	Indirect-Preventive	NSR3 O		0	
	Minimal O		0	0	
	Direct	See Note below			
Nuclear Safety Related	Indirect-Causal	NSR3	0	0	
Category II	Indirect-Preventive	0	0	0	
	Minimal	0	0	0	
	Direct	See Note below			
Nuclear Safety Related	Indirect-Causal	0	0	0	
Category III	Indirect-Preventive	0	0	0	
	Minimal	0	0	0	
	Direct	See Note below			
Scientific, Engineering	Indirect-Causal	A	0	0	
and Safety Analysis	Indirect-Preventive	А	0	0	
	Minimal	0 0		0	
1	Direct	See Note below			
Common Grade	Indirect-Causal	0	0	0	
	Indirect-Preventive	0	0	0	
	Minimal	0	0	0	

^{p13} Testing"

Acceptance Testing of PATF

- PATF development team had extensive experience with qualification of DCC hardware and with the software QA processes
 - DCC's previously replaced with hardware emulators (1995-2001)
- Qualification tests focused on:
 - Quality of emulation:
 - Included execution of all OEM diagnostics
 - Test cases for DCC features, based on OEM documentation
 - Test cases to test all features of scripting language
 - Testing of integration of Emulated DCC with plant simulation
 - Testing of transfer of control mechanism
- Qualification tests were implemented using scripting language
 - Simplified qualification testing of a new release of PATF in 2014

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Approved by:	Richard He Section Ma						

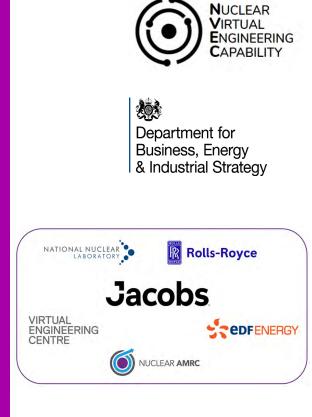


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Dr. Albrecht Kyrieleis





Challenging today. Reinventing tomorrow.

Agenda

- Context for NVEC
- NVEC elements
- Case studies
- Future developments



The Nuclear Innovation Programme			~£30M		~£150N	NUCLEAR VIRTUAL ENGINEERING CAPABILITY
		Research Theme	Apr 18	Apr 19	Apr 20	Apr 21
	Advanced Fuels	Accident Tolerant Fuels Coated Particle Fuels Pu containing fast reactor fuels Reactor physics Nuclear Data				
	Reactor Design	Thermal hydraulic model development Thermal hydraulic facility development Reactor safety and security Virtual engineering Modelling and simulation				
	Spent fuel recycle	Development of proliferation resistant spent fuel recycle technology				
		Materials testing and development Advanced component manufacturing Large scale manufacturing / assembly Prefabrication module development Codes and standards				
		Strategic assessments Fast reactor knowledge capture Regulatory engagement Access to irradiation facilities				
3	Advanced Modular Reactors	Feasibility Study Design Development				©Jacobs 2020

Challenge



- Innovative nuclear power plants needed to meet the UKGovernment commitment of net zero carbon emissions by 2050
- By 2030 deliver
 - Cost savings of 30% on new build, 20% on decommissioning
 - £2bn domestic and international contract wins
- 'Silo' practices
- Information sharing
- Innovation
- Cost management

The NVEC Partnership





- Jacobs Lead
- Wider partners include: Digital Catapult, University of Bangor, University of Bristol, EvoMetric
- Collaboration includes: UKAEA, Fraser-Nash, Sellafield Ltd, Menai Science Park

NVEC Elements



- Develop collaborative digital environment to support the nuclear life cycle
 - Use existing technology where possible
 - Open and highly flexible
- Develop operating model, standards and guidance
- Demonstrate benefits of digital environment in various case studies
- Involve stakeholders
- Early adoption
- Develop 'community' which can assume responsibility for
 - issuing guidance, maintaining standards,
 - discussing and resolving common technological issues
 - ensuring a **common approach** across the sector

NVEC Phase 2 Environment



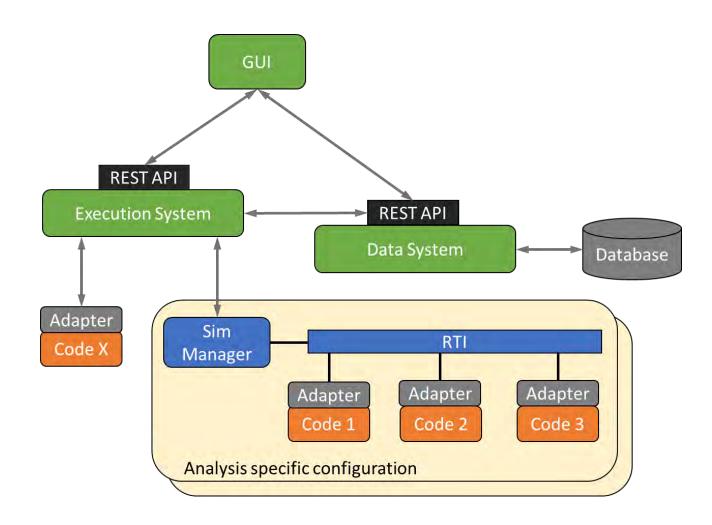
Design	Build	Operation	Decommissio	ning Waste storage			
NVEC Digital Environment							
Commo Enviror		Common Model Environment	ing	Standards for virtual engineering			
Enable 'si source of	•		Run distribu analyses o common d	on Enable			

Benefits



- Reduced costs
 - Single source of design data; collaborative environment
 - o Increased return on investment through efficient operation & maintenance
 - o Lower risk leading to reduction in financing costs
- Shortened development times
 - Efficient Design & licensing ; Integrated multi-physics approach
 - More reliable prediction of development times, allowing better synchronisation
- Enhanced credibility, operability, reliability & safety
 - o Real time understanding of plant, better planning and predictive maintenance
 - o Enhanced training & skills development
 - Reduced risk and perception of risk
- Cross-discipline transfer of expertise; joined-up industry
- Enables innovation and new technology adoption; diverse users

Architecture



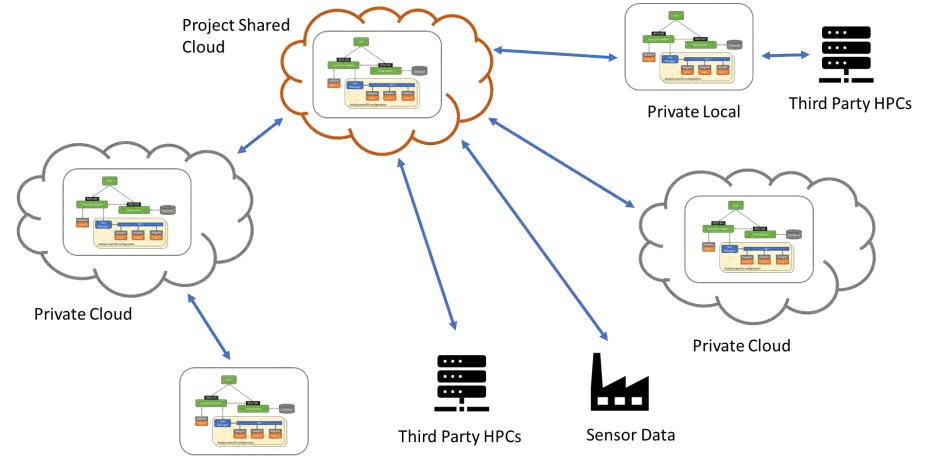


Status

- Initial implementation of all components complete
- Continuous improvement
- Deployed on different systems
- Application to various cases on-going

Networked Architecture

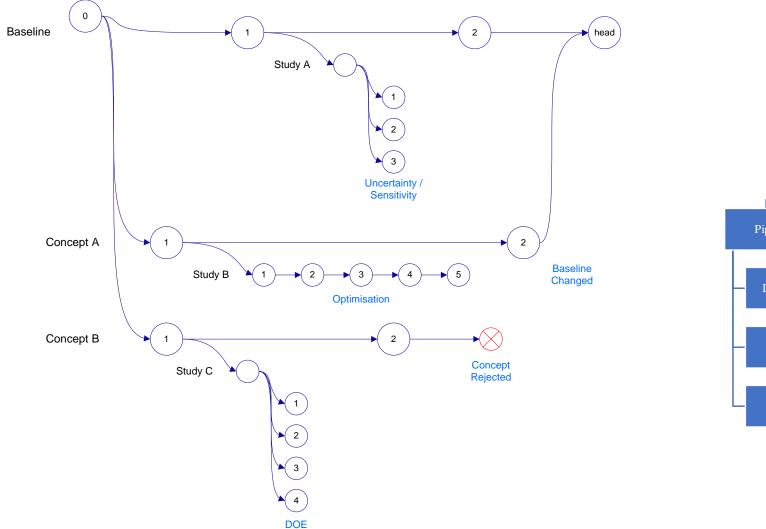


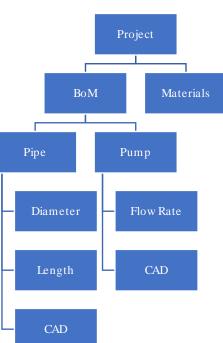


Private Local

Change Control using the Data System

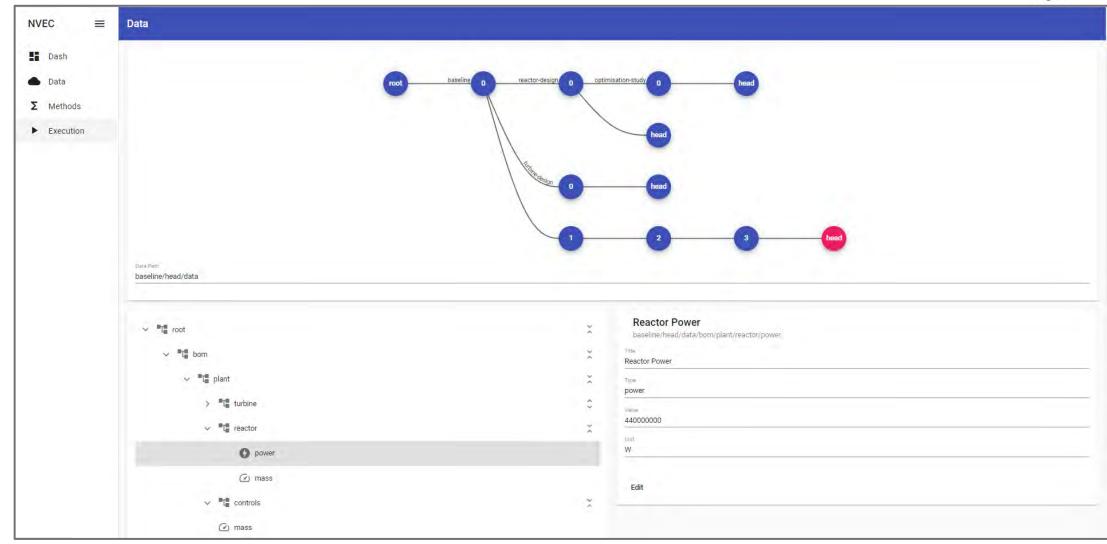






Graphical User Interface





NVEC Multi-Scale Simulation

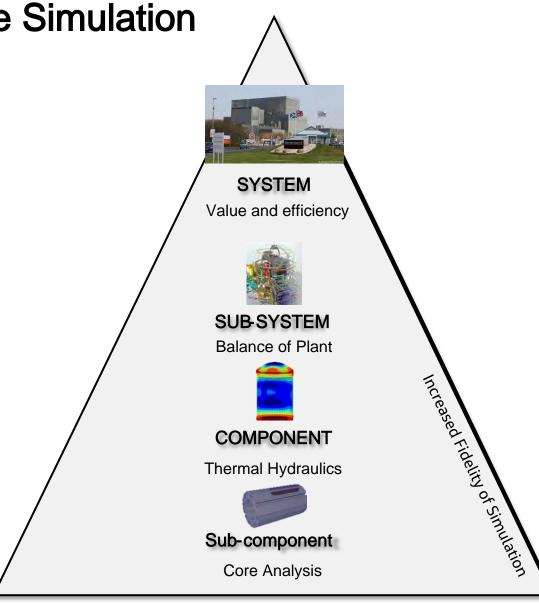
Features

Break down 'model into hierarchy of components'

'Equation Orientated' modelling approach or dedicated code

Code coupling via 'plug and play' modular design

Scalable to allow deployment in a range of applications





Benefits

Single tool can analyse many different designs with few changes

Rapid turn-around from concept to outcome from an analysis

Detailed component analysis via dedicated code where required

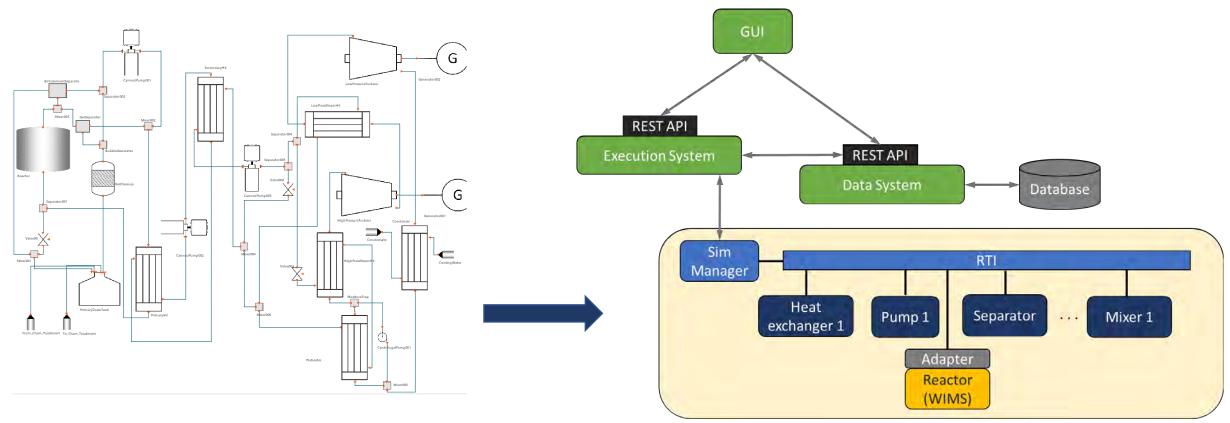
Analyse faults faster as plant simulator and control system can have common features

Case Study: System Level Modelling

NUCLEAR VIRTUAL ENGINEERING CAPABILITY

- Component libraries can be re-used
- Simulation of operational sequences

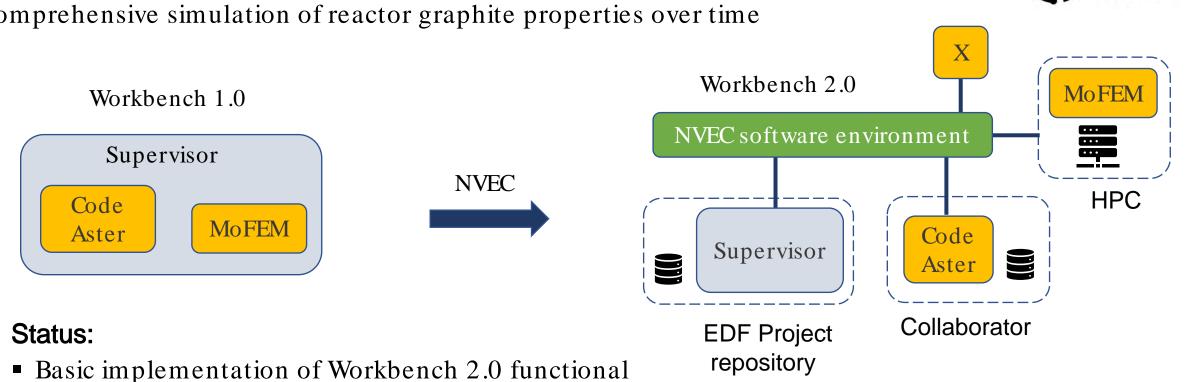
- Complex system model of Molten Salt AMR developed and implemented
- Reactor analysis optionally using WIMS code



Multi - Scale Simulation GUI

• [Screen shot Sys Lev Sim GUI]





Case Study: AGR Graphite Workbench

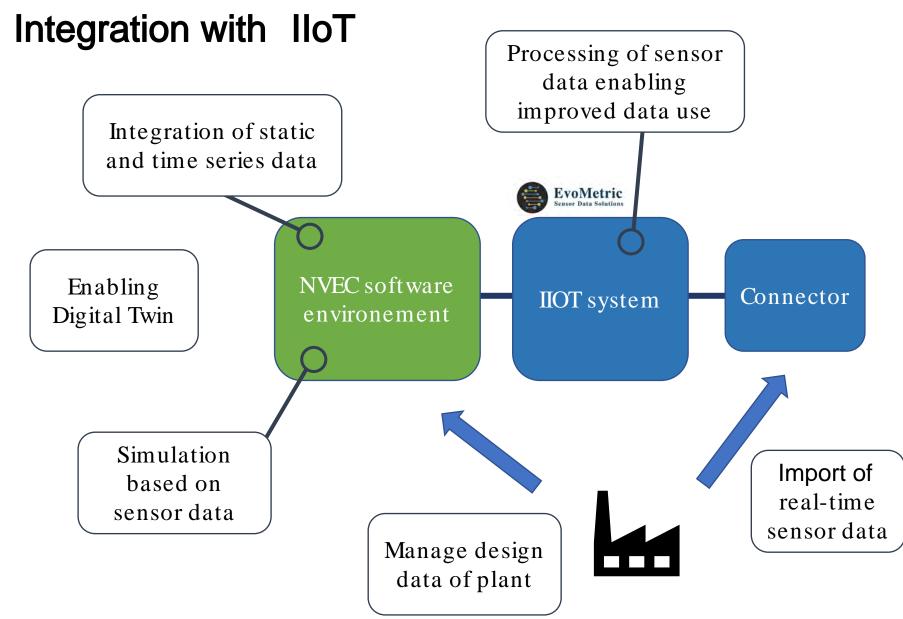
Comprehensive simulation of reactor graphite properties over time

Benefits of integration with NVEC

- Enables sharing of computational infrastructure / increased fidelity / inclusion of future modules
- Improved collaboration between sub-contractors
- Standardisation of analyses: less QA effort and training

UCLEAR

IRTUAL ENGINEERING





Status:

- Design in progress
- Plant at NAMRC setup to export sensor data
- Model of plant sub-system developed (Frazer-Nash)
- Collaboration with Digital Catapult / SMEs on related methods/technologies

Further Applications



Decommissioning

- Integration of simulations with point cloud data from innovative decommissioning project (IIND)
- Reactor Physics
 - Completed design of coupling of codes for key workflow (WIMS-ENIGMA) in NVEC
- THOR(Thermal Hydraulic Open-Access Research Facility), University of Bangor
 - Collaboration started aiming at involving NVEC from design stage onwards
 - Data Model for THOR developed

FAITH

- Application of NVEC approach and tools on-going

SME Discovery Workshop (held in Sept '20)

- Exploring opportunities for collaboration
- NVEC enabling innovation through SMEs



DIEMinnovations







CI RADAR







Future Developments



- SMR
 - Use NVEC for key requirements: e.g. engineering data management, design, change control
 - Initial NVEC evaluation version in development for Rolls-Royce SMR
- Fusion (STEP, CHIMERA)
 - Develop requirements / information model
- System level digital representations enabling optimisation strategies
 e.g. AMR, process heat, Hydrogen-Nuclear combinations
- Implementation of operational Digital Twins for existing facilities
- Further development of application for FAITH and THOR/ THUNDER

Future Developments



- Advanced Materials & Manufacturing: more effective structural integrity management
- Increased effectiveness of safety case support
- Social factors study of benefits/obstacles of digitalisation in nuclear
- Develop working practices to support an information management strategy between disciplines
- Develop standards/guidance aiming at a 'NVEC Community'
- Link to the Construction Sector Deal

Summary

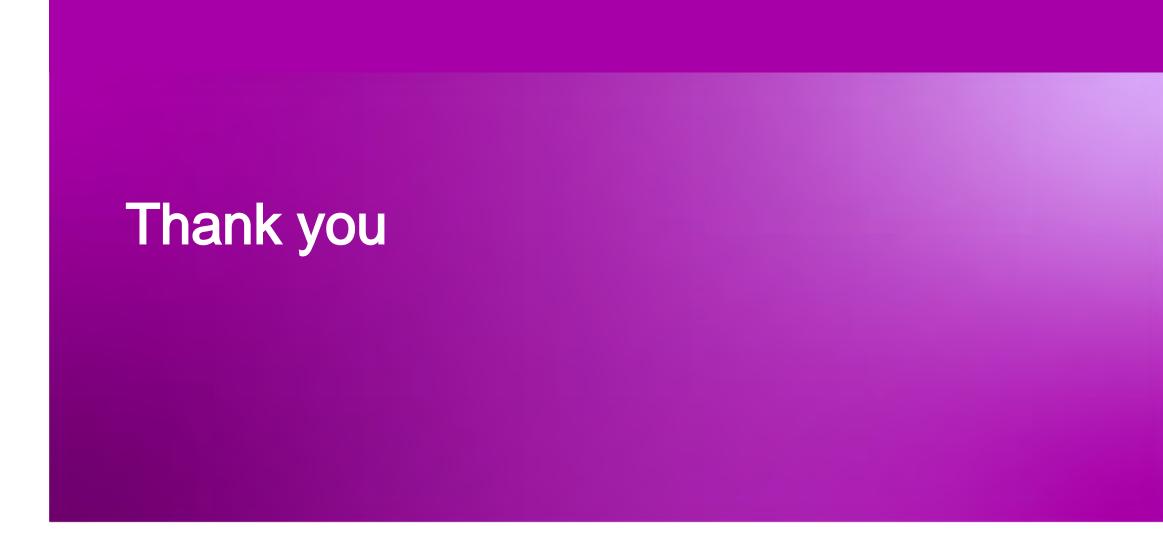


- NVEC to help deliver UK Government net zero carbon emissions target by 2050
- Key challenges: 'Silo' practices, information sharing, innovation, cost management
- Development of collaborative digital environment along with standards/guidance
- 'NVEC community': responsible for issuing guidance, maintaining standards, ensuring a common approach across the sector
- Various case studies on-going demonstrating benefits of NVEC
- Broad range of future opportunities

Acknowledgments

- C. Phelps, A. Aslam (Jacobs)
- D. Bowman, K. Vikhorev (University of Liverpool VEC)
- M. Bankhead (NNL)
- C. Jackson (Rolls-Royce)
- J. Draup, P. Martinuzzi (EDF-Energy)
- S. Marr (NAMRC)









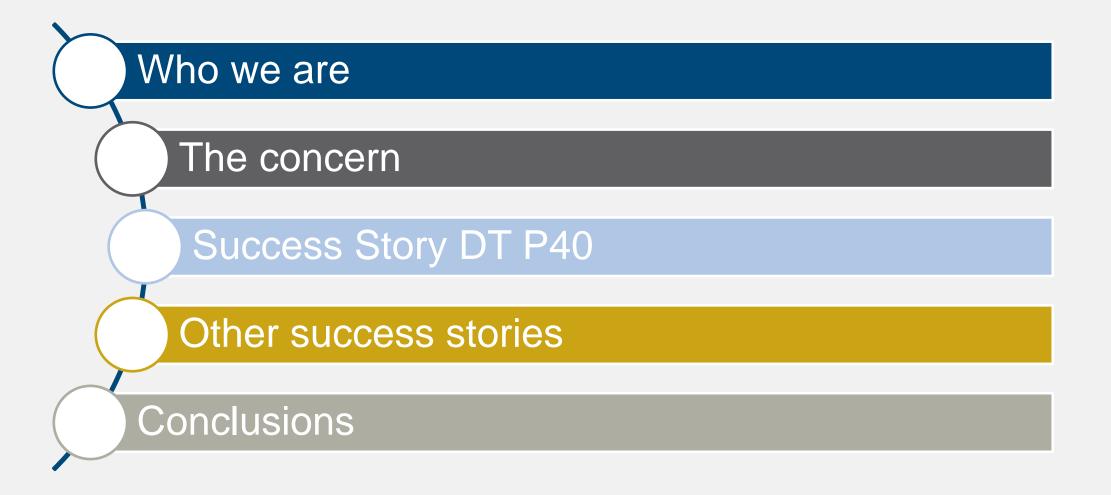


> NRC&INL&ORNL 2020 – December 3rd Susana López (<u>slopez@tecnatom.es</u>) Pablo Rey (<u>prey@tecnatom.es</u>)





CONTENTS

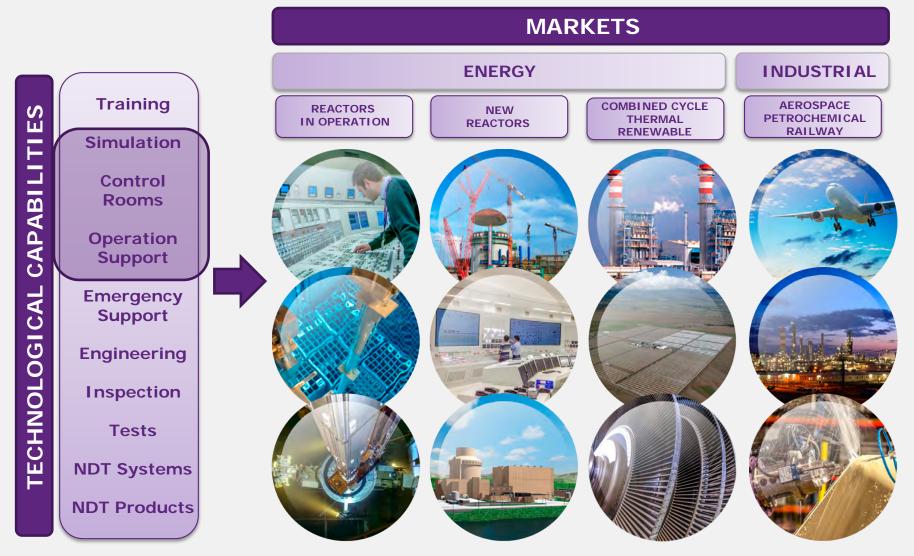




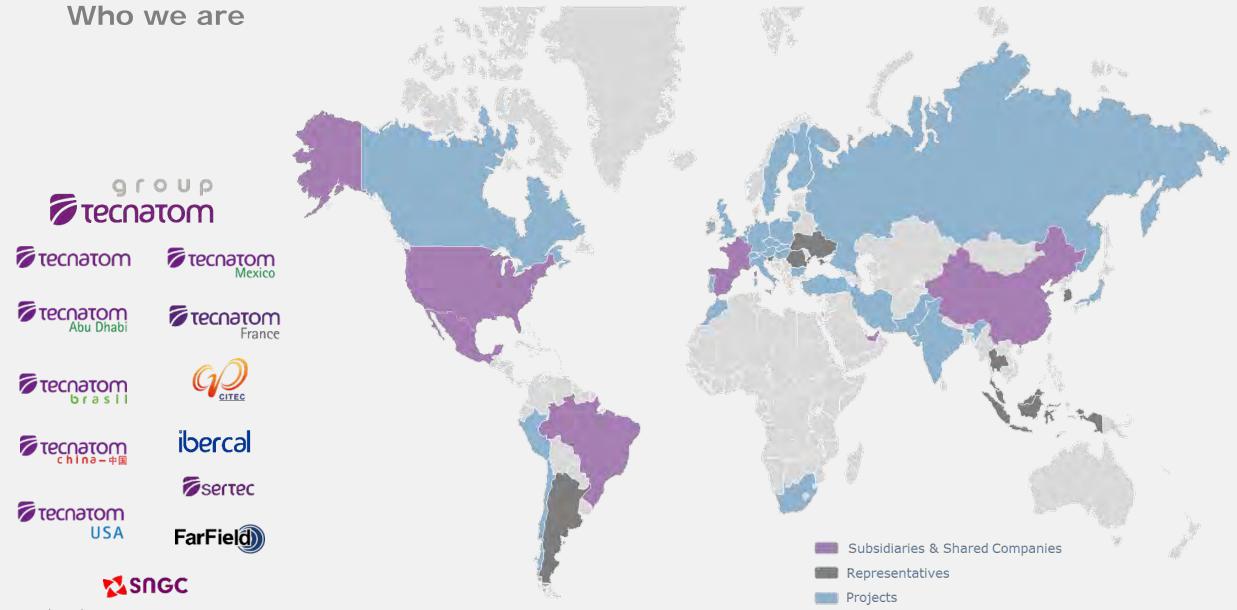




Who we are













The concern



BWR/6 manufactured by General Electric Electrical power 1,092.02 MW Located in Spain

Key facts

- 1st coupling: 14 October 1984
- Commercial operation: 11 March 1985

BOP DCS

- Honeywell TDC 3000 since 1988
- Installed in Full-scope simulator in 2002
- Migrated to Experion in 2005
- Design modifications: control room digitalization



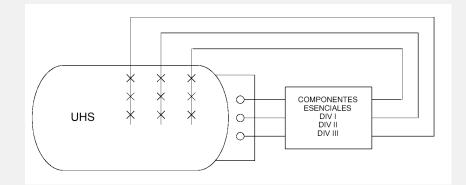
The concern : BWR NPP heat sinks

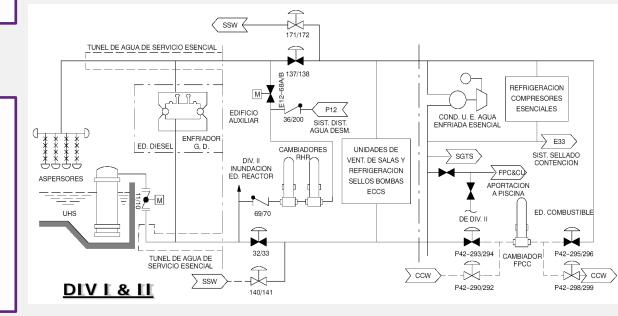


Forced draught towers (Auxiliary systems)

Alternative: Ultimate Heat Sink (UHS)

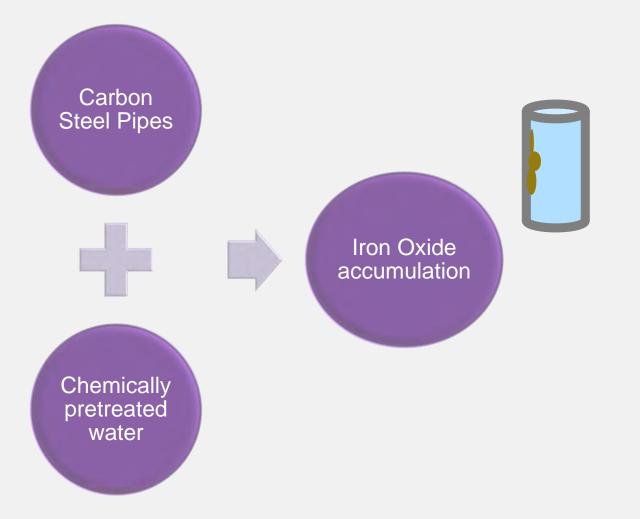
- Pond: 30 days autonomy
- ESW: 3 cooling water pumping and distribution sub-systems
- LOCA or LOOP





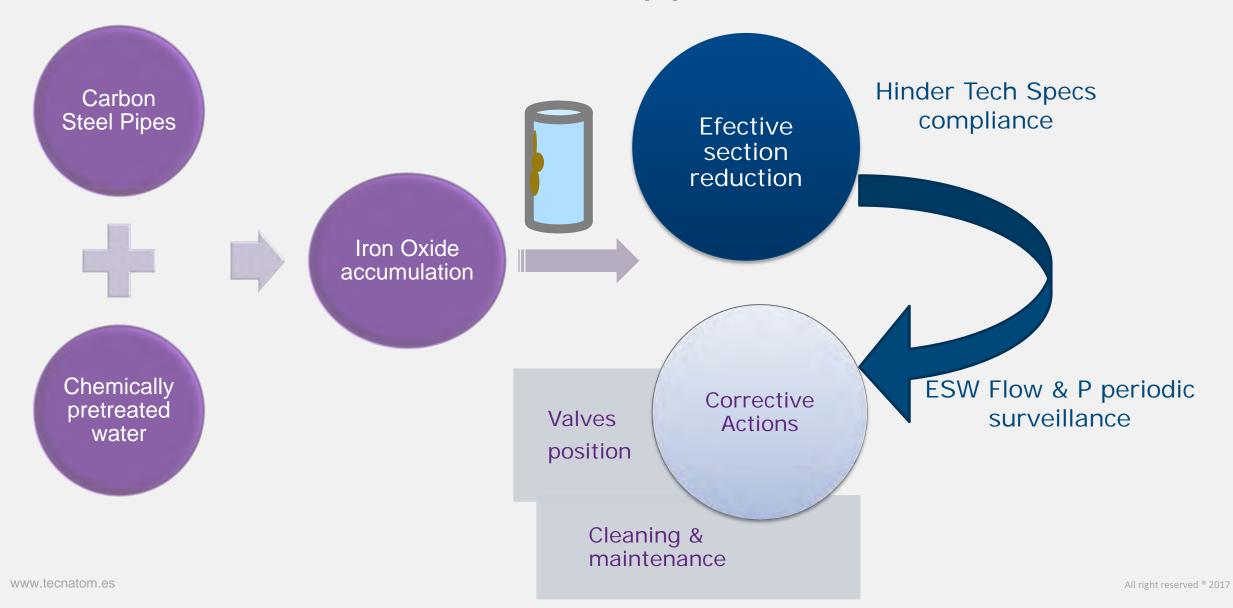


The concern: Essential Services Water pipes effective section





The concern: Essential Services Water pipes effective section



10



The concern : the solution

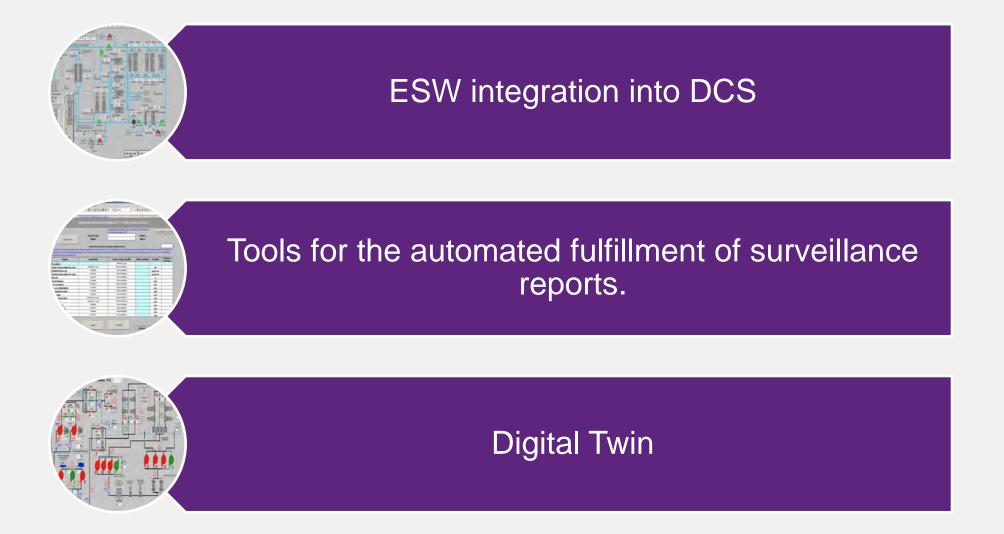
Digital Twin to ensure compliance with Technical Specifications



Success story DT P40

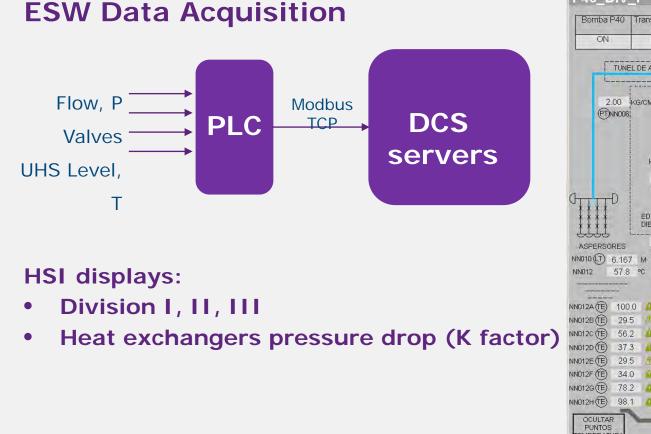


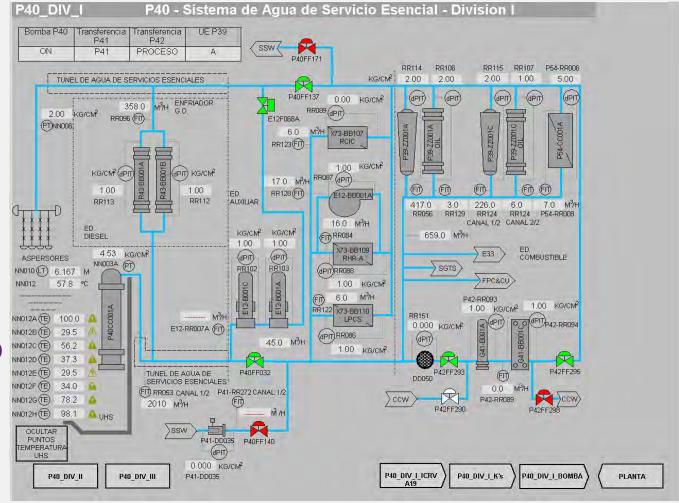
Project DT P40





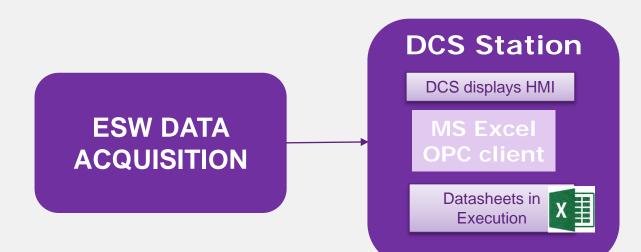
Project DT P40 : ESW integration into DCS







Project DT P40 : Tools for automatization of surveillance reports



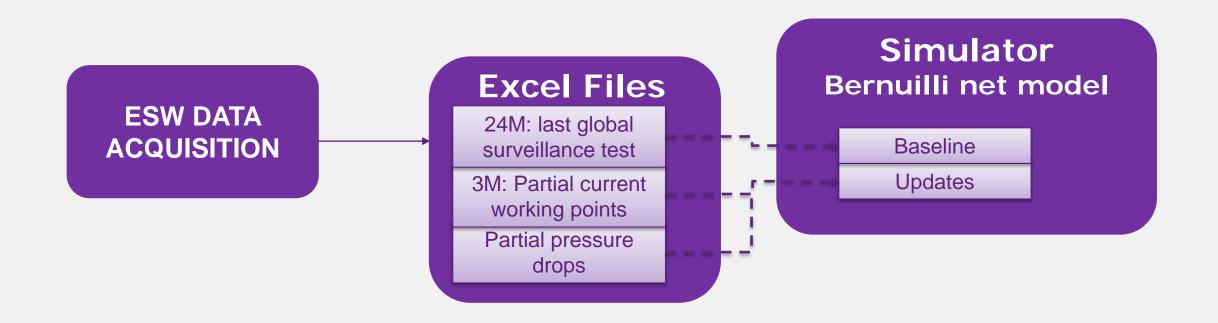
Each division surveillance books:

- 24M: Minimum flow surveillance (all components)
- 03M: Pumps functional capacity (current and historical working point)
- Pressure drop factor (K = dP/Q2)
 - Heat exchangers & filters
 - Graph over time

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6 Nota; Par en funció 7 tener todo 9 0 1 P39 ALINE 2 CAUDAL E 3 PRESIÓN F 4 PRESIÓN E	n de la unidad de enfriamiento que esté is los equipos alineados al P40. Equipo Equipo Exado: DESCARGA BOMBA (P40-CC001A) EETORIO AL UHS DESCARGA BOMBA (P40-CC001A)	abierta, mediante la incomunicación de ain en servicio, La toma de datos se realizará Instrumento P40-RR053 (1° canal) P40-NN006	e de P52, la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux, punto P40 R19 AUXR053 A P40 R19 P40NR055 A	sistema estable durante al m	Unidades Unidades m3/h kg/cm2 man	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota: Par en funció 7 tener todo 9 0 11 P39 ALINE 12 CAUDAL I 13 PRESIÓN F 4 PRESIÓN I 5 NIVEL UHS	n de la unidad de enfriamiento que esté os los equipos alineados al P40. Equipo ADO: ECCARGA BOMBA (P40-CC001A) ECTORIO AL UHS S	abierta, mediante la incomunicación de ain en servicio. La torna de datos se realizará Instrumento P40-RR053 (1º cana) P40-NR005 P40-NR005 P40-NR005 P40-NR005 P40-RR096	e de P52, la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el v P40 R19 aux_punto P40 R19 aux_punto P40 R19 P40RR053 A P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056	sistema estable durante al m	Unidades Unidades m3/h kg/cm2 man kg/cm2 man	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota; Par en funció 7 tener todo 9 10 11 Pas ALINE 12 CAUDAL 1 13 PRESIÓN I 14 PRESIÓN I 15 NIVEL UNS 16 GD-I (R43-	n de la unidad de enfriamiento que esté os los equipos alineados al P40. Equipo ADO: DESCARGA BOMBA (P40-CC001A) EETORIO AL UHS ESCARGA BOMBA (P40-CC001A) S BB001AB)	ableta, mediante la incomunicación de ain en servicio. La toma de datos se realizará Instrumento P40-RR053 (1º canal) P40-NN006 P40-NN005A P40-NN010 P40-RR096 E12-RR007A	e de P52, la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s P40 R19 aux, punto P40 R19 Aux, punto P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056	sistema estable durante al m	Unidades Unidades m3/h kg/cm2 man kg/cm2 man m	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota: Par en función 7 tener todo 9 0 11 P39 ALINE 12 CAUDAL I 13 PRESIÓN I 14 PRESIÓN I 15 NIVEL UHS 16 GD-J (R43- 17) 17 PHR-A (E1) 18 G41 A-C (C)	n de la unidad de enfriamiento que esté is los equipos alineados al P40. Equipo EADO: EADO: ESCARGA BOMBA (P40-CC001A) RETORIO AL UHS S BB001A/B) 2-B001C/A) G41-B001A/BB001C)	abierta, mediante la incomunicación de ain en servicio, La toma de datos se realizará Instrumento P40-RR053 (1° canal) P40-NN006 P40-NN006 P40-NN006 P40-NN006 P40-RR066 E 12-RR007A P42-RR089	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux_punto P40 R19 sucromotos A P40 R19 P40R055 A P40 R19 P40R056 P40 R19 P40R056 P40 R19 P40R056 P40 R19 P40R056 P40 R19 P40R057A	sistema estable durante al m	Unidades Unidades m3/h kg/cm2 man kg/cm2 man m m3/h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota; Par en función 7 fener todo 9 9 0 11 Pas ALINE 12 CAUDAL I 13 PRESIÓN I 14 PRESIÓN I 15 NIVEL UNS 16 GD-J (R43- 17) 17 RHR-A (E1) 18 G41 A-C (Ú) 19 CONDENS:	n de la unidad de enfriamiento que esté isolos equipos alineados al P40. Equipo alineados al P40. EQUIPO AL UNS DESCARGA BOMBA (P40-CC001A) ETORIO AL UNS DESCARGA BOMBA (P40-CC001A) S BB001A/B,B 2-B001C/A) G41-B001A/BB001C) ADOR P33-2Z001A	abierta, mediante la incomunicación de ain en servicio, La toma de dalos se realizará Instrumento P40-RR053 (1° cana) P40-NN006 P40-NN003A P40-NN003 P40-R036 E12-RR007A P42-RR089 P40-RR056	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR053 A P40 R19 P40RR058 P40 R19 P40RR058 P40 R19 P40RR058 P40 R19 P40RR058 P40 R19 P40RR058	sistema estable durante al m	Unidades Unidades m3/h kg/cm2 man kg/cm2 man m m3/h m3/h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota; Par en funciói 7 tener todo 9 0 10 P39 ALINE 12 CAUDAL I 13 PRESIÓN I 14 PRESIÓN I 15 NIVEL UNS 16 GD-J (R43- 7) 17 RHR-A (E1 18 G41 A-C (0) 19 CONDENS; 20 Oil P39-ZZ	n de la unidad de enframiento que esté as los equipos alineados al P40. Equipo alineados al P40. Equipo alineados al P40. PECORIO AL UNS DESCARGA BOMBA (P40-CC001A) S BED01ALBIS 2.8001C(A) G41-B001ABB001C) ADOR P39-Z2001A 2014	abierta, mediante la incomunicación de ain en sarvicio. La torna de dalos se realizará Instrumento P40-RR053 (1º canal) P40-NN006 P40-NN006 P40-NN010 P40-RN096 E12-RR007A P42-RR099 P40-RR056 P40-RR129	a de P52, la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s P40 R19 aux, punto P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR055 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR129	sistema estable durante al m	Unidades 	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota; Par en funció rener todo 7 tener todo 90 0 11 P39 ALINE 12 CAUDAL I. 13 PRESIÓN I. 14 PRESIÓN I. 15 NIVEL UNS 16 GD-1 (R43- 17 RHR-A CL 18 G44 A-C (0) 19 CONDENS. 20 OI P39-ZZ 21 CONDENS.	n de la unidad de enframiento que esté os los equipos alineados al P40. Equipo (Constant) Equipo (Constant) Escaraça BomBa (P40-CC001A) Escaraça BomBa (P40-CC001A) S (Constant) BB001A,B) 2.B001A,B) 2.B001A,B1 2.B00A,B1 2.B00A,B1 2.B00A,B1 2.B00A,	abierta, mediante la incomunicación de ain en sarvicio. La torna de datos se realizará Instrumento P40-RR053 (1º cana) P40-NN006 P40-NN006 P40-NN006 P40-NR066 E12-RR007A P42-RR068 P40-RR056 P40-RR056 P40-RR124 (1er cana)	e de P52, la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el v P40 R19 aux, punto P40 R19 aux, punto P40 R19 AURROS3 A P40 R19 P40 RROS3 P40 R19 P40 RROS6 P40 R19 P40 RROS6	sistema estable durante al m	Unidades Unidades 	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota; Par en funció rener todo 7 tener todo 10 2 11 P39 ALINE 12 CAUDAL L 13 PRESIÓN F 14 PRESIÓN F 15 NIVELUNS 16 GDJ (R43- 7) 7 RHR-A (E1 8 G41 A-C (C) 19 CONDENS. 20 OI P39-ZZ 21 CONDENS.	n de la unidad de enframiento que esté os los equipos alineados al P40. Equipo (Constant) Equipo (Constant) Escaras BomBa (P40-CC001A) Escaras BomBa (P40-CC001A) S BB001AB (P40-CC001A) S BB001AB (P40-CC001A) Constant Escaras BomBa (P40-CC001A) S BB001AB (P40-CC001A) Constant S BB001AB (P40-CC001A) Constant S BB001AB (P40-CC001A) Constant S BB001AB (P40-CC001A) Constant C	abierta, mediante la incomunicación de ain en sarricio. La torna de datos se realizará Instrumento P40-RR053 (1º cana) P40-NN006 P40-NN006 P40-NN007 P40-RR056 E 12-RR007A P40-RR056 P40-RR056 P40-RR056 P40-RR124 (1er cana) P40-RR124 (1er cana)	e de P52, la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el v P40 R19 aux, punto P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR053 A P40 R19 P40RR058 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR124 P40 R19 P40RR124 A	sistema estable durante al m	Unidades Unidades Mah kgiem2 man m m3h m3h m3h m3h m3h m3h m3h m3h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota; Paren (unció) 7 tener todo 9 0 1 P39 ALINE 2 CAUDAL I: 3 PRESIÓN I: 4 PRESIÓN I: 5 NIVEL UNS 6 GD-1 (R43-7) 7 RHR-A (E1) 8 G41 A-C (0) 0 OI P39-ZZ 21 CONDENS; 22 OI P39-ZZ 23 P54-A (P5)	n de la unidad de enframiento que esté so los equipos alineados al P40. Equipo (Constant) Equipo (Constant) EADO: (Constant) EADO: (Constant) EECARGA BOMBA (P40-CC001A) EESCARGA BOMBA (P40-CC001A) S BB001AB) 2-B001C(A) G41-B001A/BB001C) ADOR P39-Z2001A 2001A ADOR P39-Z2001C 2001C E001C 4-CC001A)	abierta, mediante la incomunicación de ain en servicio, La toma de datos se realizará Instrumento P40-RR053 (1° canal) P40-NR005 P40-NR005 P40-NR005 E12-RR007A P42-RR089 P40-RR056 P40-RR056 P40-RR129 P40-RR129 P40-RR129 P40-RR124 (1er canal) P40-RR124 (2° canal) P40-RR124 (2° canal) P40-RR124 (2° canal)	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s P40 R19 aux, punto P40 R19 aux, punto P40 R19 A0RR053 A P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P42RR059 P40 R19 P42RR059 P40 R19 P40RR123 P40 R19 P40RR123 P40 R19 P40RR124 A P40 R19 P54RR008	sistema estable durante al m	unidades Unidades unidades kgiem2 man kgiem2 man m3h m3h m3h m3h m3h m3h m3h m3h m3h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota: Pare en función 9 en función 9 caracterization 10 Paresión I 11 Paresión I 12 CAUDAL L 13 PRESIÓN I 14 PRESIÓN I 15 NIVEL UNS 16 GDJ (R43- 17) 17 RHR-A (E) 18 G41 A-C (1) 19 ONDENS: 20 OIP 39-ZZ 21 CONDENS: 22 OIP 39-ZZ 20 OIP 39-ZZ 21 ONDENS: 22 OIP 39-ZZ 22 OIP 39-ZZ 21 OIP 39-ZZ 22 OIP 39-ZZ 22 OIP 3	n de la unidad de enfriamiento que esté se los equipos alineados al P40. Equipo EADO: EAD	abierta, mediante la incomunicación de ain en servicio, La toma de datos se realizará Instrumento P40-RR053 (1° cana) P40-NN005 P40-NN005 P40-NN005 P40-RR058 E E12-RR007A P42-RR089 P40-RR056 P40-RR124 (1° cana) P40-RR124 (1° cana) P54-RR006 P40-RR124 (2° cana)	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR129 P40 R19 P40RR124 P40 R19 P40RR124 B P40 R19 P40RR124 B P40 R19 P40RR124 B P40 R19 P40RR124 B	sistema estable durante al m	enos una hora, el Unidades m3/h kg/em2 man kg/em2 man m3/h m3/h m3/h m3/h m3/h m3/h m3/h m3/h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota: Pare en función 9 en función 9 itenertodo 11 Paresión fi 12 CAUDAL L 13 PRESIÓN fi 14 PRESIÓN fi 15 NIVEL UHS 16 GOL (R43-1) 17 RHR-A (E1) 18 GATA-C (DI) 19 CONDENS. 20 DIP3-ZZ 20 PS4-A (P5) 24 SALA LPC 26 SALA LPC	n de la unidad de enfinamiento que esté as los equipos alineados al P40. Equipo alineados al P40. Equipo alineados al P40. EscarGA BOMBA (P40-CC001A) DESCARGA BOMBA (P40-CC001A) S ESCARGA BOMBA (P40-CC001A) S BB001AB) 2.8001C(A) G41.8001A BB001C) ADOR P39-ZZ001A 2001A ADOR P39-ZZ001A 2001A ADOR P39-ZZ001C 2001C 4.CC001A) 5.5 (X73-BB10) C1-A (X73-BB109)	abierta, mediante la incomunicación de ain en sarvicio, La toma de díalos se realizará P40-RR053 (1° cana) P40-NN006 P40-NN003A P40-NN003A P40-NN010 P40-RR056 E12-RR007A P42-RR089 P40-RR124 (2° cana) P40-RR124 (1° cana) P40-RR124 (2° cana) P40-RR124 (2° cana) P40-RR124 (2° cana)	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR123 P40 R19 P40RR124 P40 R19 P40RR124	sistema estable durante al m	enos una hora, el Unidades m3h kg/cm2 man m3h m3h m3h m3h m3h m3h m3h m3h m3h m3h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
6 Nota: Pare 7 fenerotodi 9 1 10 1 11 P39 ALINE 12 CAUDAL II 13 PRESIÓN II 14 PRESIÓN II 15 NIVEL UHS 16 GOL (R43- 11 17 RHR A. C1 18 GH A.C. Q1 19 CONDENS: 20 OI P39-ZZ 21 CONDENS: 22 DF4A. (P5 23 SALA LPC 24 SALA LPC 27 SALARC	n de la unidad de enfriamiento que esté se los equipos alineados al P40. Equipo EADO: EAD	abierta, mediante la incomunicación de ain en servicio, La toma de datos se realizará Instrumento P40-RR053 (1° cana) P40-NN005 P40-NN005 P40-NN005 P40-RR058 E E12-RR007A P42-RR089 P40-RR056 P40-RR124 (1° cana) P40-RR124 (1° cana) P54-RR006 P40-RR124 (2° cana)	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR129 P40 R19 P40RR124 P40 R19 P40RR124 B P40 R19 P40RR124 B P40 R19 P40RR124 B P40 R19 P40RR124 B	sistema estable durante al m	enos una hora, el Unidades m3/h kg/em2 man kg/em2 man m3/h m3/h m3/h m3/h m3/h m3/h m3/h m3/h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras
en funció 7 tener toda 910 11 11 P39 ALINE 12 CAUDAL L 13 PRESIÓN FI 14 PRESIÓN FI 15 NIVEL UNS 16 GOL (R43) 17 RHR-A (E1 18 G41 A-C (C1 19 CONDENS. 20 OI P39-ZZ 20 OI P39-ZZ 21 PSA-4 (P5 - 20) 21 CONDENS. 22 OI P39-ZZ 23 PSA-4 (P5 - 20) 24 SALA LPC 25 SALA LPC	n de la unidad de enfinamiento que esté as los equipos alineados al P40. Equipo alineados al P40. Equipo alineados al P40. EscarGA BOMBA (P40-CC001A) DESCARGA BOMBA (P40-CC001A) S ESCARGA BOMBA (P40-CC001A) S BB001AB) 2.8001C(A) G41.8001A BB001C) ADOR P39-ZZ001A 2001A ADOR P39-ZZ001A 2001A ADOR P39-ZZ001C 2001C 4.CC001A) 5.5 (X73-BB10) C1-A (X73-BB109)	abierta, mediante la incomunicación de ain en sarvicio, La toma de díalos se realizará P40-RR053 (1° cana) P40-NN006 P40-NN003A P40-NN003A P40-NN010 P40-RR056 E12-RR007A P42-RR089 P40-RR124 (2° cana) P40-RR124 (1° cana) P40-RR124 (2° cana) P40-RR124 (2° cana) P40-RR124 (2° cana)	a de P52. la válvula FF307 ó FF308 (válvula neu con la bomba P40-CC001A arrancada y con el s Punto de lectura en SCD P40 R19 aux, punto P40 R19 aux, punto P40 R19 P40RR053 A P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR056 P40 R19 P40RR123 P40 R19 P40RR124 P40 R19 P40RR124	sistema estable durante al m	enos una hora, el Unidades m3h kg/cm2 man m3h m3h m3h m3h m3h m3h m3h m3h m3h m3h	ite P39-ZZ001A ó C) sistema deberá H ^o Muestras

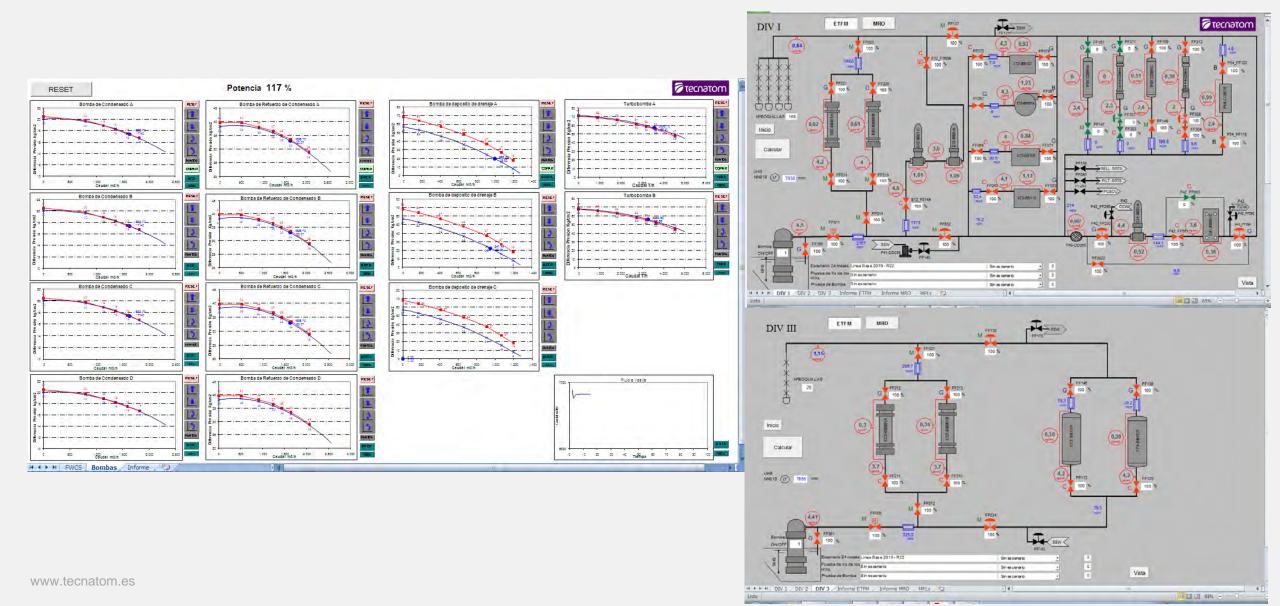


Project DT P40 : Digital Twin





Project DT P40 : Digital Twin





Project DT P40 : Digital Twin

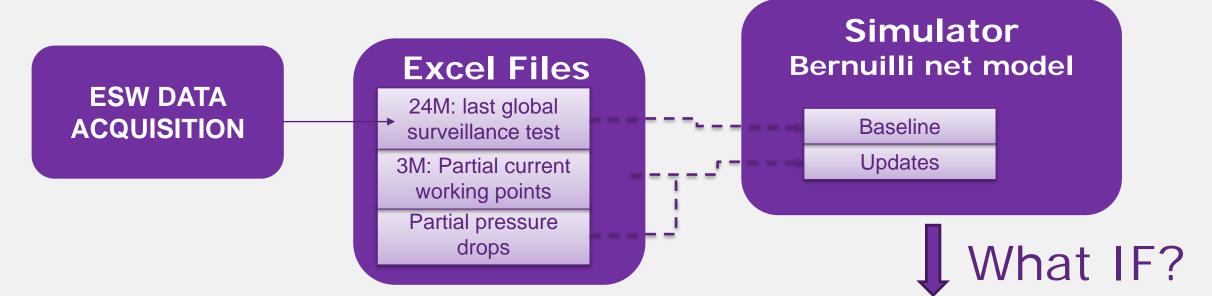


a	DIVISION								DIVI	SION I				DIVISION III							
	inea Base	2019-R22						Linea Base	2019 - R22						Linea Base	2019 - R22	2				
S of	Sin escenai	io.						Sin escena	rio				_	-	Sin escena	rio					-
	Sin escenari	o				1		Sin escenar	rio					9	Sin escenar	io			-		-
ŀ			Caudal	Caudal de	Caudal	Calculad	: Unidades	-		Caudal	Caudal de	Caudal	Calculad	k Unidades	-		Caudal		Caudal	Calculad	o Unidades
-	OMBA	1554YN/2	de ETF	Intervenció	de	Galeerad	c on houses	BOMBA	1554YN/3	de ETF	Intervenció	de	Guidenad	c officiales	BOMBA	TC-N0042-		Intervenció	de	Galectiad	o onsoudes
P	descarga Pretorno Caudal Nivel UHS	NN003A NN006 RR053 (1/2) NN010	0,8 2042 7239		-	0,84	kg/cm2 kg/cm2 m3/h mm	P descarga P retorno Caudal Nivel UHS	NN002 NN005 RR053 (2/2) NN010	0,8 2035 7239	1		4,58 0,88 2250 7993	kg/om2 kg/om2 m3/h mm	P descarga P retorno Caudal Nivel UHS	NN001 NN004 RR054 NN010	0,8 180,9 7239		2		kglom2 kglom2 m3/h mm
0	6D	RR096	258	276,4	285,7	349,6	m3/h	GD	RR097	258	283,8	296,8	387,4	m3/h	GD	RR110	130,4	161,2	176,7	285,7	m3/h
F	HR-A	E12-RR007A	1174	1213,8	1233,7	1373	m3/h	RHR-B	E12-RR007B	1174	1221,8	1245,7	1414	m3/h							
C	541-A	P42-RR089	112,7	119	122,1	144,1	m3/h	G41-B	P42-RR090	112,7	119,2	122,5	145,3	m3/h	X73-88119 X73-88103	BR127 BR111	6,4 9,6	9 11,7	10,2 12,8	19,3 20,2	m3/h m3/h
2	LPCS SELLOS A LPCI A RCIC	RR122 RR128 RR084 RB123	12,1 4,5 7,2 1,7	16.2 4.8 11.9 2.8	18,2 5 14,2 3,4	32,4 6,0 30,5 7,3	m3/h m3/h m3/h m3/h	SELLOS B LPCI B SELLOS C LPCI C	RR085	4,5 7,2 4,5 7,2	7,2 11,7 6,0 10,9	8,5 14 6,8 12,7	17,8 29,9 12,2 25,5	m3/h m3/h m3/h m3/h							
1	P39-A-COND 39-A-ENFR	RR056 RR129	118,9	133,7	141,2	0,0 0,0	m3/h m3/h	P3 P39-B-COND P3 P39-B-ENFR	RR057 RR131	118,9	135,8	144,4	196,7 7,0	m3/h							
	939-0-00ND 38-C-ENFR	RR124 (1/2) RR124 (2/2) TOTAL P39	118,9	137	145,9	199,5 9,6	m3/h m3/h	P3 P39-D-COND P39-D-ENFR	RR126 (1/2) RR126 (2/2) TOTAL P39	118,9	133,9	141,4	0,0 0,0	m3/h m3/h m3/h							
		(P39 en servicio)	118,9	137	145,9		100	the second	(P39 en servicio)	118,9	135,8	144,4	203,7								
	954-A	P54-RR008	1,6	2,3	2,6	4,9	m3/h	P54-B	P54-RR009	1,6	2,2	2,5	4,7	m3/h	100.0						
l	gd-Ia Gd-Ia Gd-Ib	RR058 dPi RR113 RR060 dPi RR112				4,2 0,62 4,0 0,61	kg/cm2 dPi kg/cm2 kg/cm2 dPi kg/cm2	CD ID	RR062 dPLRR117 RR064 dPLRR116				4,0 0,75 3,8 0,71	kglom2 dPi kglom2 kglom2 dPi kglom2	CD IID	RR066 api RR120 RR068 dPI RR121				3,7 0,30 3,7 0,36	kg/cm2 dPi kg/cm2 kg/cm2 dPi kg/cm3
10	HR-A 12-8001C	RR133				4,6	kg/cm2	RHR-B	RB142				4,7	kg/cm2	UE's	BR141				4,2	kg/cm2
1	12-8001A	dPI RR102 RR134 dPI RR103				1,01 3,6 1,06	dPi kg/cm2 kg/cm2 dPi kg/cm2		dPI RR104 RR143 dPI RR105				1,00 3,7 1,03	dPi kg/cm2 kg/cm2 dPi kg/cm2		dPI BR034				0,38 4,2	dPi kg/cm/
	641-A FILTRO 641-B001A	P40-dPI-RR15 P42-RR091 dPI P42-RR093	51			0,007 4,4	dPi kg/cm2 kg/cm2 dPi kg/cm2	G41-B FILTRO	P40-dPI-RR15 P42-RR092 dPIP42-RR093				0,005 4,3	dPi kg/cm2 kg/om2 dPi kg/cm2	K73-BB103	dPI RR095				0,39	dPi kg/cmi

GD	RR110	155	182.2	195.7	291.8	m3/h
	TOTAL	30			41.5	m3/h
X73-BB119 X73-BB103	RR127 RR111	11 18.6	13 19.4	13.9 19.8	21.8 19.7	m3/h m3/h



Project DT P40 : Digital Twin



Prediction

Quantitative result of eventual system use or complete test

Best system configuration

Simulate valves and pumps operation

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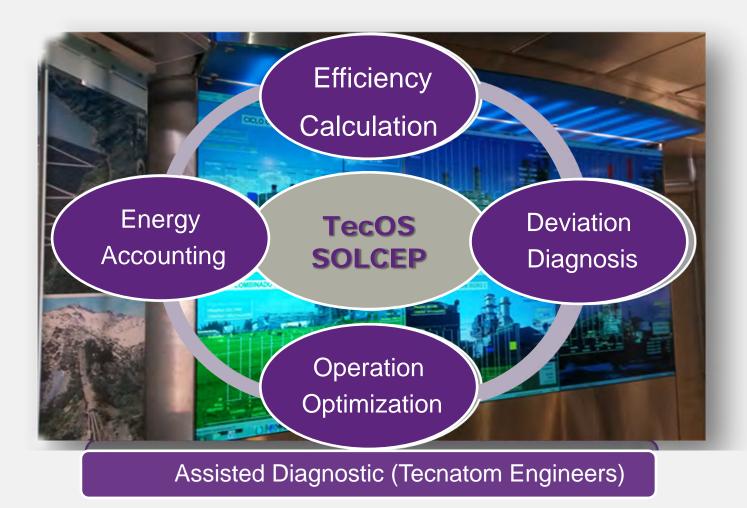
All right reserved [®] 2017



Other success stories



Project DT TecOS SOLCEP



ASME PTC PM 1993(2010). Performance Monitoring Guidelines for Power Plants



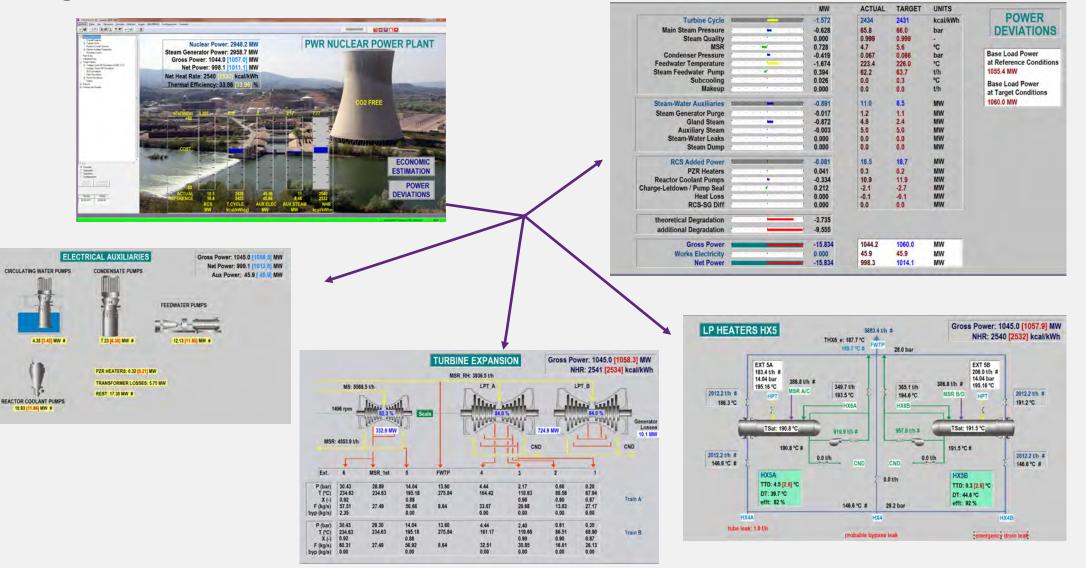








Project DT TecOS SOLCEP









Conclusions



Īre		

Benefits:

- Maximizing operation increasing safety margins
 - No need for maintenance shutdown
- Eases the evaluation of blockages and soiling
 - On-line information (digitalization)
 - Optimizes preventive maintenance tasks (cleaning)
 - Optimizes operator workload:
 - Automates calculations and surveillance and test requirements reports

Key Factors:

- Integrated approach for design modifications
 - Simulation, I&C, HFE, Operation



www.tecnatom.es



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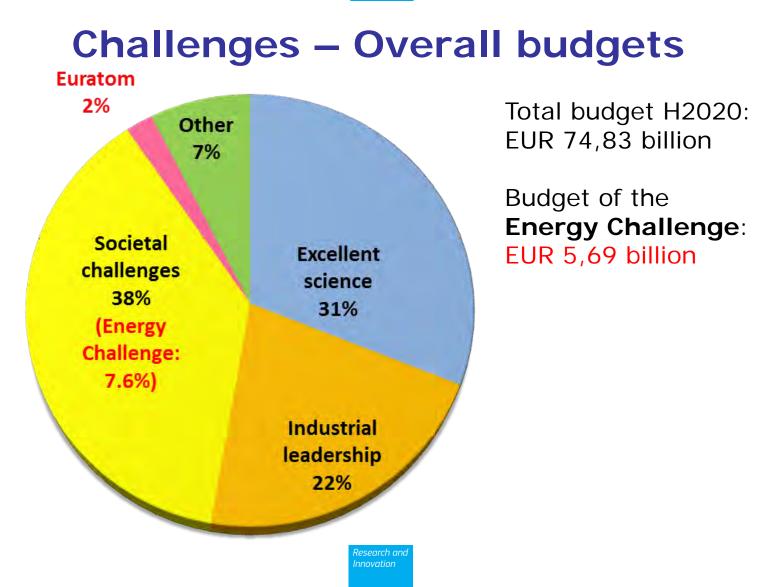
THE EU FRAMEWORK PROGRAMME FOR RESEARCH AND INNOVATION

EURATOM RESEARCH AND TRAINING PROGRAMME FISSION RESEARCH

Panagiotis MANOLATOS DG RTD Clean Planet panagiotis.manolatos@ec.europa.eu

DIGITAL TWIN 4-5 December 2020

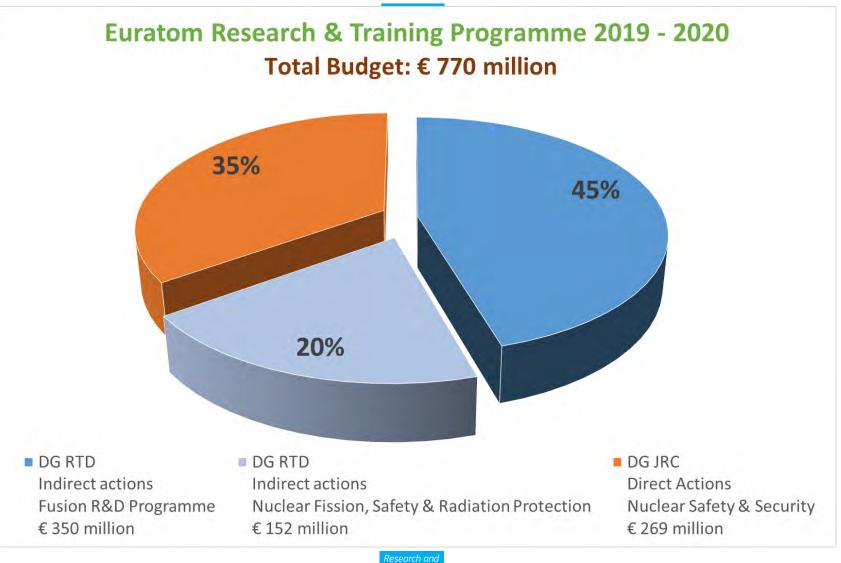








EURATOM





Council Regulation

Euratom indirect actions specific objectives:

(a) supporting **safety** of nuclear systems;

(b) contributing to the development of safe, longer term solutions for the management of ultimate **nuclear waste**, including final geological disposal as well as partitioning and transmutation;

(c) supporting the development and sustainability of **nuclear expertise** and excellence in the Union;

(d) supporting **radiation protection** and development of medical applications of radiation, including, inter alia, the secure and safe supply and use of radioisotopes;



Council Regulation

Euratom indirect actions specific objectives:

(e) moving towards **demonstration of feasibility of fusion** as a power source by exploiting existing and future fusion facilities;

(f) laying the **foundations for future fusion** power plants by developing materials, technologies and conceptual design;

(g) promoting **innovation** and industrial competitiveness;

(h) ensuring availability and use of **research infrastructures** of pan-European relevance.



Current Euratom Nuclear Fission and Radiation Protection budget share

	Reactor systems
	Safety of existing nuclear installation (Gen-II-III)
	Safety of Advanced nuclear systems (Gen-IV)
~ 40%	Partitioning, Transmutation and closing the fuel cycle
	Cross-cutting aspects (e.g. fuels, materials, simulation, nuclear data)
	 Other applications (e.g. cogeneration, support to Research Reactors)
~ 20%	Radiation protection
~ 20%	Geological disposal
~ 20%	Research infrastructures; Training and mobility; Cross-cutting

Research and Innovation

Research and Innovation Actions

They are actions with Research and Development activities as the core of the project intending to establish new scientific and technical knowledge and/or explore the feasibility of a new or improved technology, product, process, service or solution

- may include basic and applied research, technology development and integration, testing and validation on a small-scale prototype in a laboratory or simulated environment

- may contain closely connected but limited demonstration or pilot activities aiming to show technical feasibility in a near to operational environment

• 100% funding rate

"Pure" Innovation Actions

"'Innovation action' means an action primarily consisting of activities **directly aiming** at producing plans and arrangements or designs for new, altered or improved products, processes or services. For this purpose they may include prototyping, testing, demonstrating, piloting, large-scale product validation and market replication"

Research and

• 70% funding rate (100% for non-profit legal entities)



Types of Actions – Coordination and Support

Coordination and Support Action

Actions consisting primarily of accompanying measures such as standardisation, dissemination, awareness-raising and communication, networking, coordination or support services, policy dialogues and mutual learning exercises and studies, including design studies for new infrastructure and may also include complementary activities of strategic planning, networking and coordination between programmes in different countries.





Nuclear Fission & Radiation Protection Research (NRFP) Call 2019-2020 Calendar

WP Adoption:	14 December 2018
Call Open:	15 May 2019

Submission deadline: Evaluation: 25 September 2019 November 2019

62 proposals received

EC requested : EUR 265 million EC budget : EUR 134 million

Signature of GAs:

esearch an

9



Research and Innovation Actions (RIA)

Торіс	Budgets
	(EUR million)
Nuclear safety - NFRP-01: Ageing phenomena of	16
components and structures and operational issues	
Nuclear safety - NFRP-02: Safety assessments for LTO	12
upgrades of Generation II and III reactors	
Nuclear safety - NFRP-03: Safety margins determination	8
for design basis-exceeding external hazards	
Nuclear safety - NFRP-05: Support for safety research of	8
Small Modular Reactors	
Nuclear safety - NFRP-06: Safety Research and	7.6
Innovation for advanced nuclear systems	
Nuclear safety - NFRP-07: Safety Research and	6
Innovation for Partitioning and/or Transmutation	





Coordination and Support Actions (CSA)

Торіс	Budgets
	(EUR million)
Nuclear safety - NFRP-08: Towards joint European effort in	1.1
area of nuclear materials	
Education and Training - NFRP-11: Advancing nuclear	5
education	
Research Infrastructure - NFRP-16: Roadmap for use of	1.1
Euratom access rights to JHR experimental capacity	
Research Infrastructure - NFRP 17: Optimised use of	1.1
European research reactors	

Innovation Action (IA)

Nuclear safety - NFRP-04: Innovation for Generation II and	12
III reactors	



Nuclear safety



Topics NRFP - 1, 2

Торіс	Acronym	Title	Duration (Months)	Max EC contribution (M€)	Total cost (M€)
NFRP-01	ACES	Towards improved assessment of safety performance for long-term operation of nuclear civil engineering structures	48	4	5,5
	ENTENTE	European database for multiscale modelling of radiation damage	48	4	5
	INCEFA-SCALE	Increasing safety in npps by covering gaps in environmental fatigue assessment - focusing on gaps between laboratory data and component SCALE	60	4	6,8
	STRUMAT-LTO	Structural materials research for safe long term operation of LWR npps	48	4	4,8
NFRP-02	АМНҮСО	Towards an enhanced accident management of the hydrogen/co combustion risk	48	4	4
	APAL	Advanced PTS analysis for LTO	48	4	4,6
	CAMIVVER	Codes and methods improvements for VVER comprehensive safety assessment	36	4	4
		Research and Innovation			



Abstracts, coordinator, and further info is published as soon as the Grant Agreements are signed and can be found at :

https://cordis.europa.eu/projects/en





International Cooperation

Multilateral

- International Energy Agency (IEA)
- Nuclear Energy Agency (OECD-NEA)
- International Atomic Energy Agency (IAEA)

Bilateral

- Association Agreements with Switzerland and Ukraine
- Cooperation with Japan, Canada, US, China, Korea, Brazil, Argentina...



International Cooperation

Participation

Open for all legal entities established in third countries and for international organisations.

Restrictions only possible if introduced in the work programme.

- ✓ For reciprocity reasons
- ✓ For security reasons

Funding

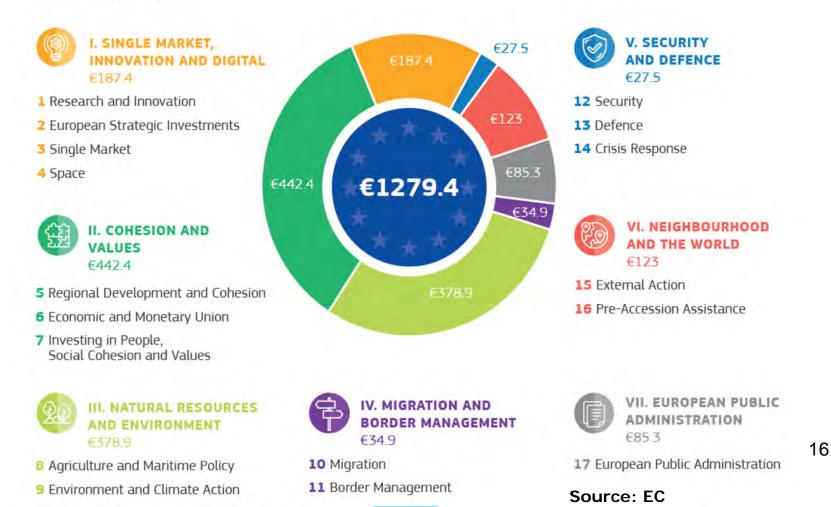
- Third country identified in the Work Programme or
- ✓ participation deemed by the Commission essential in the action or
- when provided under a bilateral scientific and technological agreement





EU priorities: 2021-2027 MFF proposal

In billion euro, current prices

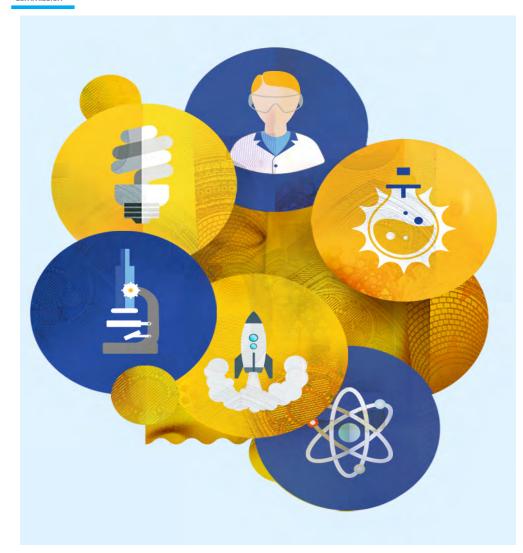




Research and

Commission proposal for Horizon Europe

THE NEXT EU RESEARCH & INNOVATION PROGRAMME (2021 – 2027)





Horizon Europe budget proposal (2021-2027)



€ billion In current prices

Open Science

Global Challenges & Ind.
 Competitiveness
 Open Innovation

Strengthening ERA

Euratom

Research and

Innovation

€ 100 B including € 3.5 from InvestEU

18



Thank you!

#HorizonEU

http://ec.europa.eu/horizon-europe

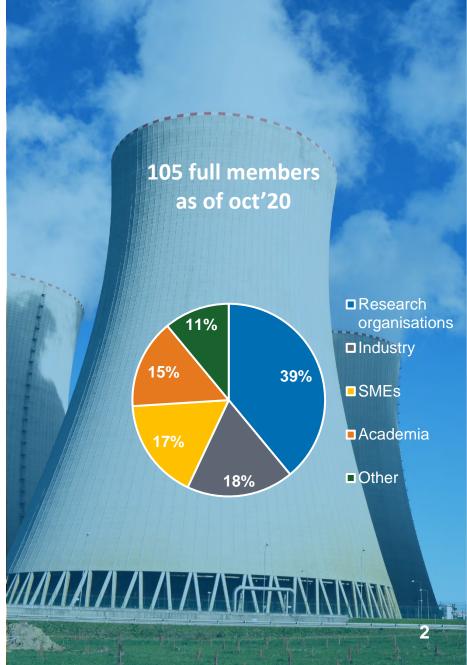


European R&D&I towards Digital Twins

A. Al Mazouzi (General Secretariat)

SNETP in a nutshell

- SNETP was set up in 2007 under the auspices of the European Commission with the goal to support technological development for enhancing safe and competitive nuclear fission in a climate-neutral and sustainable energy mix.
- In line with the objectives of the SET-Plan, SNETP aims to contribute to:
 - Lowering European greenhouse gas emissions
 - > Assuring security of energy supply for Europe
 - Stabilizing electricity prices in Europe
- The association gathers various types of stakeholders: industry, research centres, safety organisations, universities, non-governmental organisations, SMEs, etc.





Objectives

Promoting Scientific Excellence

Agree on, implement and promote common R&I priorities within the SNETP community representing the three pillars

Boosting Innovation

Facilitate industrial-driven and intersectoral innovation (digital, robotics, materials, etc.) in nuclear for current and new applications (nonpower, hydrogen, etc.)

Representing nuclear fission R&D in European Affairs

Promote SNETP expertise and research priorities towards European institutions

Strenghtening International Relations

Promote SNETP expertise and research priorities towards international nuclear institutions (IAEA, OECD/NEA, GIF, etc.)

Providing solutions to Industry

Foster industrial-driven research addressing the needs of SNETP industrial members in particular regarding safety, supply chain, licensing and costcompetitiveness

Cooperating closely with Regulators

Reinforce cooperation between SNETP and the different regulatory and standardization bodies.

Supporting R&D infrastructures

Support projects and initiatives aiming at maintaining/refurbishing/building the needed infrastructure to perform R&D&I in the nuclear field.

Sharing Experience with European Associations

Fostering and coordinating interactions with European associations in the field of nuclear, and any other sector with potential mutual interests with nuclear.

Engaging with Civil Society



Engage with civil society and non-nuclear stakeholders to rationalize the debate on the European energy mix and enhance the acceptability of nuclear.

SNETP-Strategic Research and Innovation agenda

- Establishes long-term research priorities for its members
- Provides a clear research plan for industry, policy makers and research centers
- Provides state of the art analysis on nuclear research & innovation topics in line with European foreseen electricity mix in 2050 and the Green deal
- Prioritizes the topics of added value to the end users
- Create a synergy between various industrial sectors: cross-sectorial innovation (digital, material, space, ocean, robotics, etc.)
- Establish win-win relationship with national/European and international stakeholders
- Initiate and disseminate innovation within the nuclear sector











Current state of Digital Reactor



Different uses in Reactor Simulations

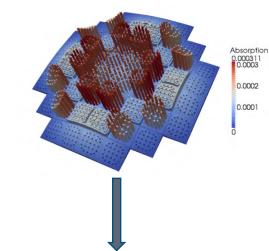
Higher representativity





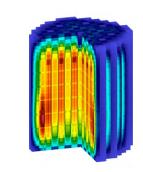
- Operators training
- Driver assistance systems
- Operations studies

Best estimate



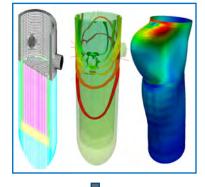
- Design Studies
- Reactor Design
- Accidents and safety studies

Best efforts



- Quantification of simulation biases
- Reference for safety studies

High Fidelity



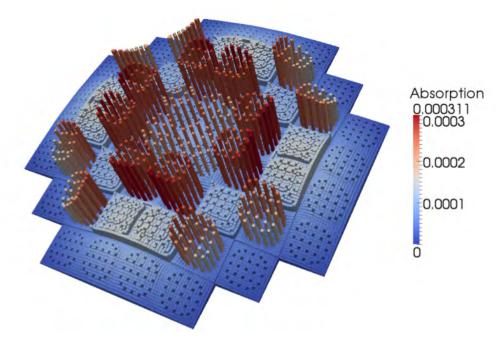
- Reference calculations
- Studies in extreme situations (accidents...)
- Substitute for experiments where no data are available



The European Nuclear sector has a long standing experience in developing a lot of physics codes including state of the art thanks to the EURATOM support and international collaboration

7

Emphasis on codes – Neutronics



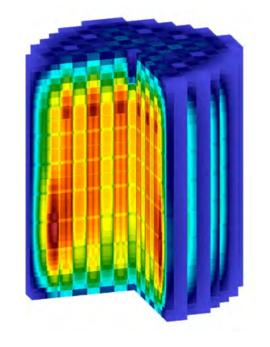
H2020 projects:

- ARIEL (2019-2023)
- SANDA (2020-2024)

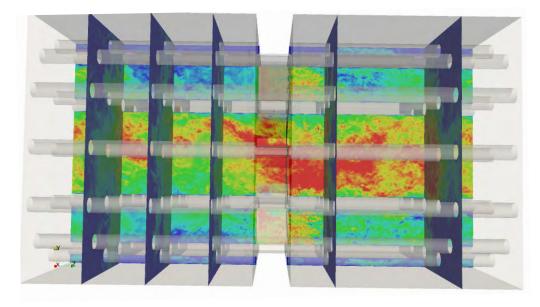


neutronics code

- Both lattice and core calculations
- Transport solvers on unstructured meshes
- Parallelization on thousands of nodes
- Depletion chain with more than a thousand isotopes
- Allows advanced calculation such as direct calculation (on going work)



Emphasis on codes – Thermalhydraulics



- Single and multiphase flows
- Based on the porous media assumption
- Used for Cores, Steam Generators, Heat exchangers
- used for Safety analysis, Core refueling operations ad R&D studies

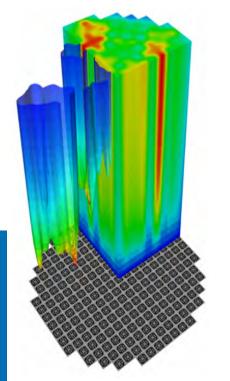


H2020 projects (exemples):

- McSafe (2017-2020); McSafer (2020-2024)
- Cortex (2017_2021)
- PIACE (2020-2024)
- CAMVVER (2020-2024)

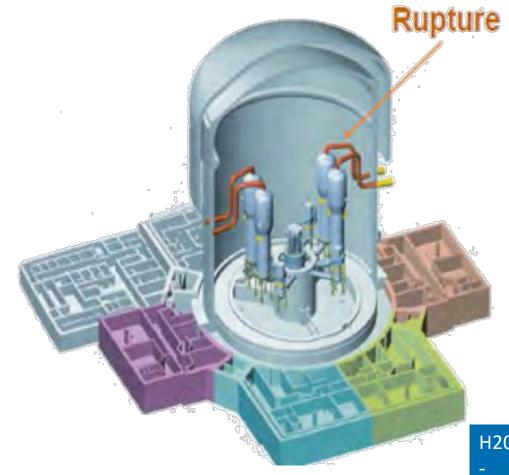
CFD code

- Single and multiphase flows
- RANS and LES turbulent models
- Unstructured meshes and parallelization on tens of thousands of nodes
- Multiphysics: Fire, Severe Accidents, turbomachinery, ground water flows, ...



Advanced Modeling Applications

Typical use case : Steam-line break accident (SLB)



SNETP Sustainable Nuclear Energy Technology Platform

- This transient has been studied for decades by with different simulation tools.
- Very complex situation with <u>strong physics coupling and 3D effects</u>: good candidate for <u>advanced simulation codes</u> (CFD, neutronics transport with unstructured meshes...)
- Allows benchmarking between legacy and new generation of codes (test for code interchangeability)
- Possibility benchmarking with other international software (VERA from CASL,...)
- Good candidate <u>for advanced visualization techniques</u> to help understand the physics.

H2020 projects (examples):

- INCEFA+ & INCEFA-SCALE (2015-2025)
- MEACTOS (2017-2021)
- MUSA (2020-2024)
- APAL (2020-2024)

Improvements in Reactor Simulations

Representativity/quality

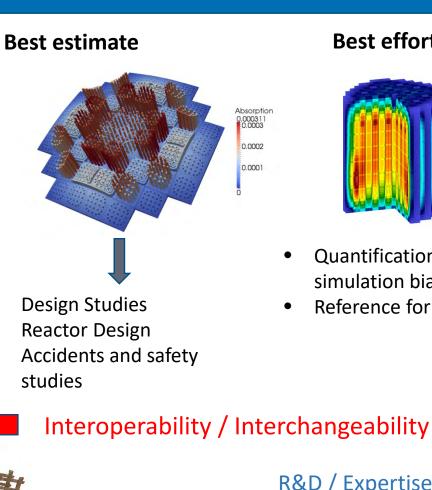
Simulators



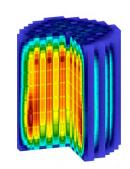
- **Operators training**
- Driver assistance system:

Exploitation

Operations studies

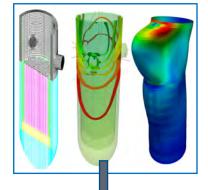


Best efforts



- Quantification of simulation biases
- Reference for safety studies ۲

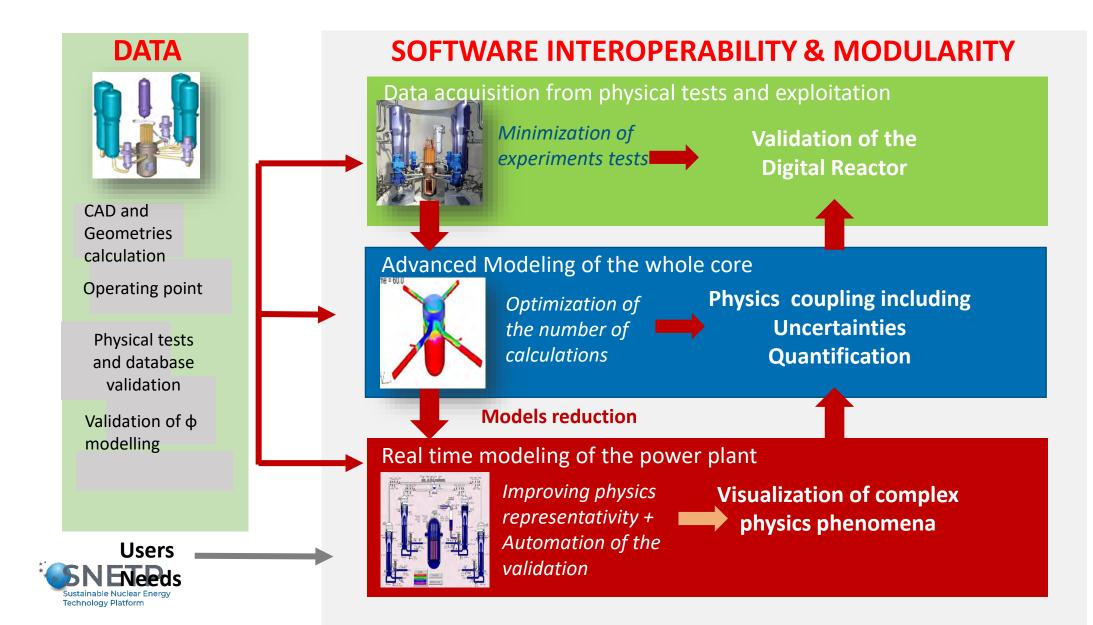
High Fidelity



- **Reference** calculations •
- Studies in extreme situations (accidents...)
- Substitute for experiments ۲ where no data are available



Codes and Data integration



Some Scientific and technical challenges

Need

of

collaboration

(European

and

international)

Goals / Challenges

- Building a <u>multi-physics</u> (interoperability) and <u>multi-scale</u> (interchangeability) platform where all relevant physics codes should be able to plug in seamlessly.
- Being able to come together with a common standard (API, data model exchange) for both <u>new and legacy codes</u>.
- Building bridges to allow *advanced* codes to be used in simulators as well.
- $\hfill\square$ Using reduction models techniques for at least real-time simulation .
- □ Taking into account, from the ground up, the possibility to quantify uncertainties.
- Developing the right methodology for propagating uncertainties when doing multiphysics.
- Being able to understand the physics involved as complexity increases dramatically.
- Using advanced, ergonomic, visualization techniques (metaphors, AR, VR...) as a helping tool.
- □ V&V of the whole platform when using strongly coupled physics.



SNETP added value

• SNETP is the only European wide association dedicated to collaborative nuclear research.

- > All major European R&D organisations involved in nuclear are members of the association.
- Various events are organised and online tools are deployed to facilitate collaboration of the community on new projects proposals. Since its creation in 2007, SNETP has supported discussions on approximately 300 project ideas.
- The specific European Technology & Innovation Platform (ETIP) status provides an important visibility to SNETP and its members, with privileged access to relevant high-level managers within EU institutions, international organisations, and member states.
- SNETP and its members contribute to the shaping of European energy policies, by exchanging with peers on research priority topics, by producing reference documents (e.g. SRIA) on the state of R&D&I in Europe, by publishing position papers, etc.



Contact us



www.snetp.eu



secretariat@snetp.eu



www.linkedin.com/company/snetp





October 18, 2020

Chris Spirito Nuclear Cybersecurity Specialist

Digital Twins and Cyber Capability Development





War Operations Plan Response WOPR (circa 1983)

2

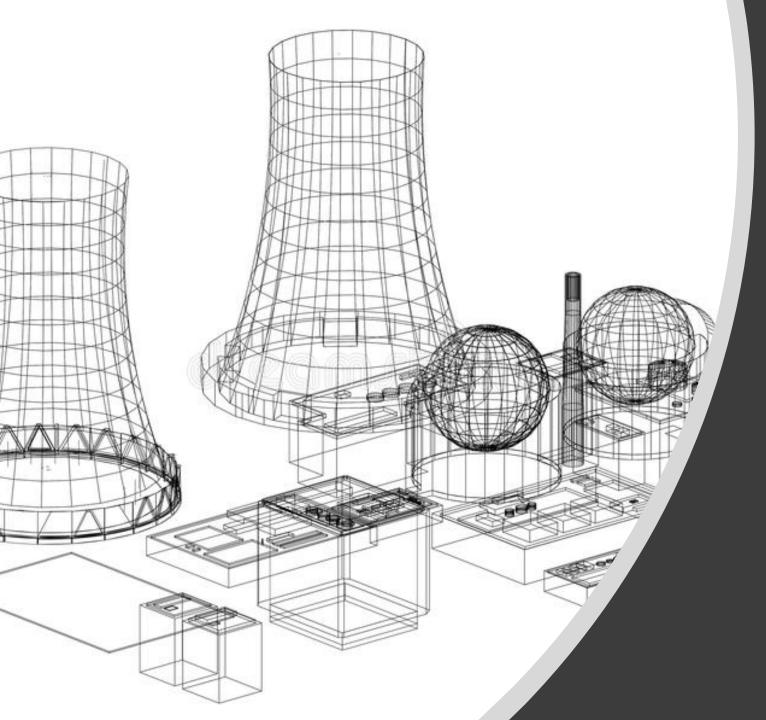
Simulation Software / AI:
- Joshua

Simulation Models:

- Basic Strategy (Tic-Tac-Toe)
- Complex Strategy (Chess)
- Basic Warfare (Air-to-Ground Actions)
- Tactical Warfare (Theaterwide Biotoxic and Chem)
- Digital Twin

(Global Thermonuclear War)

IDAHO NATIONAL LABORATORY



Cyber Capability Development (Digital Twins // Systems)

3

Reactor Simulators: - IAEA Asherah, GSE GPWR, ••

Digital Twin Targets:

- Systems (Pressurizer, Condenser, ...)
- Components (PLCs, FPGAs, ···)
- Comm Mediums (Analog, Digital, ...)
- Functional Targets (Diodes, Proto. Converters)



Cyber Capability Development (Digital Twins / Humans)

Personality Characteristics:
- Curiosity & Relentlessness
- Novelty & Creativity,

Motivation and Ethics:

- Mercenary & Ideology

Strategies

- Weakness Exploitation
- Denial & Deception

Enumerate Interaction Pathways

IDAHO NATIONAL LABORATORY



Cyber Capability Development (Digital Twins / APTs)

5

Interactive Test Ranges:

- Integrated AI
- Infrastructure Modeling

Attack Library:

- Validation of Capabilities
- Validation of Processes
- Theoretical Testbed

Idaho National Laboratory

WWW.INL.GOV

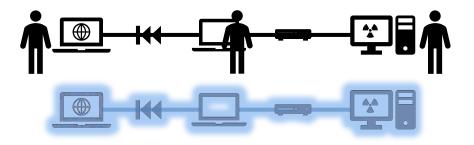
Cyber Security for Digital Twins

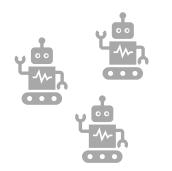
Cynthia DeBisschop and Alan (AI) Konkal Senior Cyber Security Analysts NRC Contractors, Cyber Security Branch (CSB) Division of Physical and Cyber Security Policy (DPCP) Office of Nuclear Security and Incident Response (NSIR)



Overview

- Background
- Motivation
- Considerations for Entire Life Cycle of Digital Assets
- Cyber Security Vulnerabilities and Protective Strategies
- Summary





ICON LEGEND → Real World → Virtual Model → Technology Gap

2

United States Nuclear Regulato

Protecting People and the Environment

Collaborative Review

INL DICE Glossary¹ of Terms: What Does Cyber Analyst Hear?

- **Digital Twin.** The computational simulation of a physical process or system that has a live link to the physical system, enabling enhanced verification of the simulation, control of the physical system, and analysis of trends via artificial intelligence and machine learning.
- Artificial Intelligence (AI). The simulation of human intelligence in computers or computercontrolled robots, allowing them to perform tasks commonly associated with intelligent beings.
- **Machine Learning.** The application of AI that provides systems the ability to automatically learn and improve from experience without being explicitly programmed.
- **Operational AI.** The application of artificial intelligence (AI) in energy systems to automate expensive and manual human activities and improve the efficiency of asset operations.
- Next Gen AI. The simulation of human intelligence in computers or computer-controlled robots, allowing them to perform tasks commonly associated with intelligent beings.

¹ Source: Idaho National Laboratory (INL) Digital Innovation Center of Excellence (DICE) at https://dice.inl.gov/glossary-of-terms



Technology Gap Must Close Safely, If at All



INL DICE Operational Artificial Intelligence² Description



The use of artificial intelligence (AI) in energy applications has a game-changing potential in automating expensive and manual human activities in various types of industries. In the energy industry, power plants (especially nuclear) rely on staff performing several types of manual activities on a regular basis. Future energy plants, including advanced nuclear reactors, are designed to reduce the dependence on people for the operations, maintenance, and support activities of a plant. A light water nuclear power plant is typically full of analog gauges and manual actuators. By comparing a nuclear power plant control room to a modern airplane cockpit where the plane can fly itself and the pilot's role can be reduced to simply monitoring the airplane, it is obvious that a significant technology gap exists that needs to be closed. Human intelligence needs to be replaced by machine intelligence in various forms of AI if this vision is to be realized.

- Comparison of nuclear power plant (NPP) control room to modern self-flying-airplane cockpit
- Mention of need to close significant technology gap if this Vision is to be realized for NPPs
- Cyber Security Analyst: Technology gap must close safely, if at all

² Source: Idaho National Laboratory (INL) Digital Innovation Center of Excellence (DICE) at https://dice.inl.gov/operational-artificial-intelligence



Motivation

Pop Quiz³ from October 2019 Forbes Article "How to Protect Your Digital Twin"

Q. "Which of the following is more valuable: a Boeing 777 or the digital twin of a Boeing 777?"



A. "The first option, the physical plane, is an expensive item – buying a new one will cost you around \$344 million. Yet, the digital twin of a 777 is far more valuable. It's the digital simulation of the plane that constantly collects situational awareness data and is used to understand and improve the ongoing performance of various parts and systems. If you control the digital twin, you control every 777 on (and above) the planet."

BONUS. Fill in the Blank.

Cyber Security Analyst: To effectively <u>protect</u>, think carefully throughout the evolution. Keep the vision in mind. Evaluate protections with every step along the road!



U.S.NRC United States Nuclear Regulatory Commission Protecting People and the Environment

Need to Understand/Consider

(Now, Throughout Evolution, and Before Procurement or Use)

- Technology Itself
- Security Gaps
- Threat You Are Designing Against
- Changes to Environment of Digital Assets
- Attack Surfaces of Digital Assets
- Cyber Risk
- Consequences of Cyber Attacks
- Defense-in-Depth Protective Strategies



Protecting People and the Environment

Cyber Security Vulnerabilities

- **Data Exfiltration.** Plant sensors and data streams need to be connected to virtual model to realize concept. Digital twin is intended to be near-perfect blueprint of its real twin. Potential exists for monitoring and exfiltration of information about types of systems and sensors used by plant.
- Man in the Middle Attack. Early component failures may result due to alternated maintenance cycles based on faulty data after a compromise, if data is used for predictive maintenance. Scenarios that involved predictive component failure were used in the now famous Stuxnet attack. Untimely failure of a key component could be used as an element of a kinetic attack.
- Supply Chain Attack. Digital twins can be used to model new components, testing how they will
 perform under real-world conditions. Data obtained from components of digital twin models can be
 used in manufacturing. Compromised components data could lead to manufacturing faulty
 components.



Fully Implement a Sound Cyber Security Framework

- NEI 08-09, RG 5.71, NIST 800-53, and NIST 800-82
- Implement security patches and remediate vulnerabilities quickly
- Harden digital twin platforms
 - Utilize hash code-based allowlisting
 - Remove all unnecessary files and services
 - Implement Anti-Virus and Host Intrusion Detection Systems
- Identify security and remediate gaps between the twin and the physical hardware



Secure Software Development Environments

- Develop in secure isolated environment
- Verify all Third Party and Open Source Code
- Test for language conformance, known vulnerabilities and flaws
- Conduct peer code reviews
- Use a secure repository control
- Utilize security testing techniques, fuzzing and penetration testing



Implement Software Hardening

- Harden software to make the binary resistant to hacking
- Use coded cyclic redundancy checks (CRC) or embedded hash codes checks, binary runtime encryption
- Utilize inline coding and merge functions to minimize modular code, and altered code flow to make reverse engineering difficult
- Implement glass box techniques (Binary code can only run on designated hardware)



Supply Chain and Intellectual Property

- Supply Chain Protection
 - Audit vendors for compliance with cyber security best practices
 - Test and verify third-party code releases prior to introducing them into your environment
 - Purchase hardware and software from trusted sources
- Protection of All Intellectual Property
 - Secure all digital twin artifacts, documents, schematics, etc.
 - Protect all information flow to and from the digital twin platform
 - Minimize access to source code and critical design elements



Summary

- This presentation offers considerations from a regulatory perspective while digital twin technology is in development.
- Before procurement or use of technology and throughout its evolution, there is a need to understand the attack surfaces and environments associated with digital assets.
- Nuclear power plant operators maintain the following throughout the life cycle of digital assets: a security defensive architecture to address the attack surfaces and environments, and multiple layers of cyber security protections to establish sufficient defense-in depth. Defense-in-depth protective strategies are maintained to ensure the capability to detect, respond, and recover from cyber attacks.

Protecting People and the Environment

• Such an objective depends on understanding and careful consideration of technology before procurement or use.



Asherah NPP Simulator Cybersecurity in a Digital Closed-Loop Environment

The 2020 Workshop on Digital Twin Applications in the Nuclear Industry

Rodney Busquim e Silva December 3, 2020

Nuclear Power Plants Rely on Digital-based Systems

- NPPs are among the most complex energy systems ever built.
- NPP functions and processes rely on a myriad of digital IT and I&C systems.



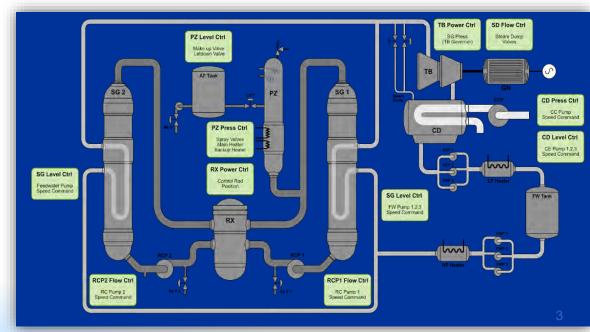


- NPPs' capital cost and the radioactive nature of nuclear fuel demand computational tools for licensing, operation and accident analysis.
- NPPs are among the most emblematic examples of critical infrastructure cyber-targets.

The Challenge

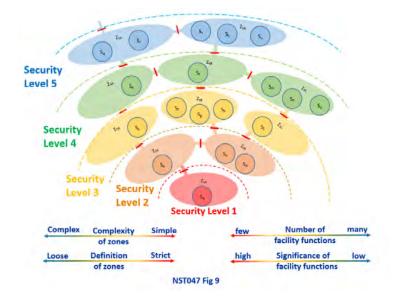


How can we improve cybersecurity capabilities, conduct IT and I&C research, increase awareness, and perform training and hands-on exercises in an integrated nuclear power plant environment?



In a NPP environment, how do we:

- assess the facility impact of a system being compromised?
- evaluate the effectiveness of segregating facility functions?
- assess computer security (CS) vulnerabilities in the systems that perform functions?
- evaluate the use of de-coupling mechanisms?
- test the effectiveness of firewall rules?
- ... and assess many other CS related issues?

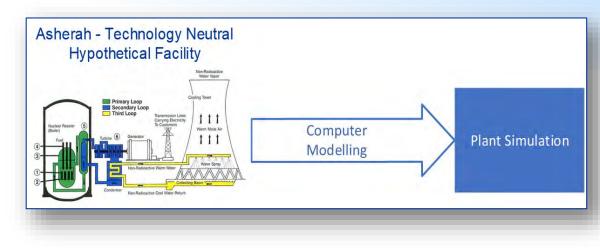




IAEA CRP: Asherah NPP Simulator (ANS)



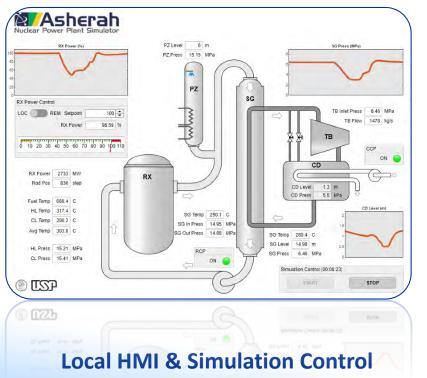
A hypothetical/neutral PWR named "Asherah" was defined based upon several NPP existing designs.



- The results were combined to produce a technological neutral facility.
- USP developed ANS model to be the heart of a cyber security assessment test bed.
- The simulator was designed specifically for the simulation of cyber-attacks.

IAEA CRP: Asherah NPP Simulator (ANS)





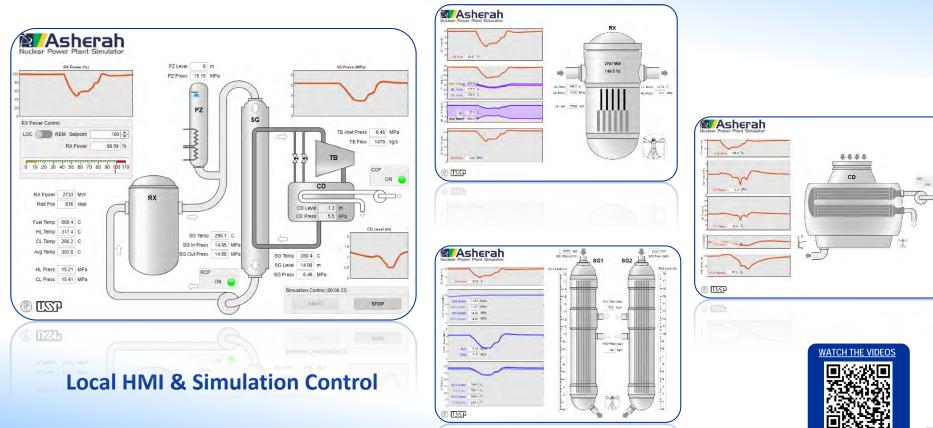
- ANS reproduces the Asherah NPP behavior using dynamic models.
- It is based on:

The TMI core. Typical industry systems and equipment. Standard control logic.

- It has been implemented using the Matlab/Simulink environment.
- It has the capability to interface with IT/OT equipment for cyber security assessment.

IAEA CRP: Asherah NPP Simulator (ANS)





ANS Interfaces

- ANS has been connected to physical and virtual controllers and other equipment.
- USP developed I/O interfaces for Modbus and OPC-UA & DA communication.
- USP has also developed a light OPC UA Server & client, i4BrSrv, for ANS communication.



ANS Deployment Modes

		Abstract &	
Standalone	ANS (plant processes & controllers) running in one VM	Poi	rtable
Model-Based	ANS plant processes & controllers running in many VMs		
CLDT-Based	ANS plant processes & controllers running in a Closed-Loop Digital Twin (CLDT) test bed		
HIL-Based	ANS plant processes & controllers running in a Hardware-In-the-Loop (HIL) test bed		

Concrete & Complex

IAEA ICONS 2020 DEMO: HIL and Model-based Run





Easy to run by any user

Model & HIL based setups

4 Virtual machines per setup

Easy to analyze the network

ATTACK

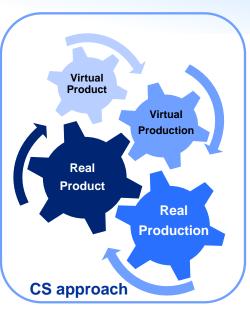
Press the ATTACK button to start the attack

Closed-Loop Digital Twin (CLDT)

• A DT is a simulated/emulated device/system that replicates in detail their physical counterparts on the logic and network layer.

- A DT may be leveraged for CS purposes in two ways:
 - Simulation mode
 - Replication mode

- CS can be introduced during the **product design** and production phases.
- CS can be seamlessly integrated in the entire digital-based systems lifecycle.





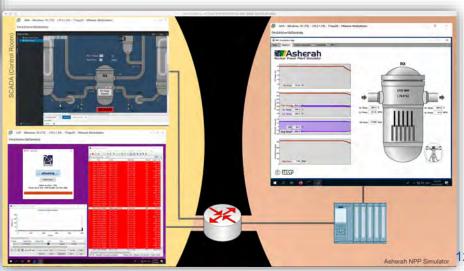
ANS PLC CLDT: Successful Attack Example







- Attack scenario where a PLC is compromised from outside the I&C controllers network.
- A PLC DT integrated with the ANS CLDT test bed allows for assessment of the network indicator of compromise and of the facility impact.



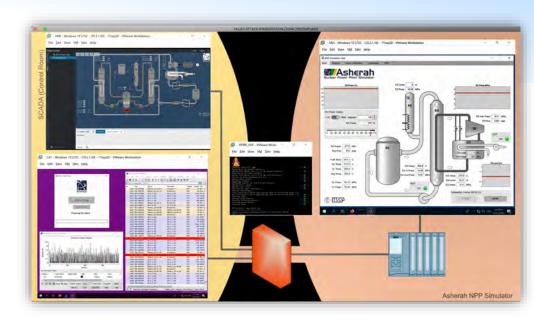
ANS PLC CLDT: Unsuccessful Attack Example



The ANS PLC CLDT Simulation mode test bed allowed for:

- Monitoring of NPP facility functions.
- Assessment of the effectiveness of a computer security strategy.
- Monitoring I/O tags at the PLC (process).
- Checking integrity and availability of PLC I/O tags and HMI tags (network).







14

Final Remarks

- DTs create new possibilities for monitoring, simulating, estimating and assessing states of real systems.
- ANS was developed for an IAEA CRP (17 teams of 13 MS) for computer security research and it has been supporting graduate and postdoctoral studies.
- ANS has been integrated in test beds, applied in CS exercises and demonstration in Austria, Brazil, Canada, China, Germany, ROK and USA.
- DTs can be leveraged for computer security purposes when integrated with nuclear simulators like the ANS.















Thank you!





Advanced Modeling and Simulation and its Future Role in Nuclear Systems Digital Twin Technology

Dave Kropaczek Oak Ridge National Laboratory

Technical Session: Multiphysics Modeling Digital Twin Applications for Advanced Nuclear Technologies December 1-4, 2020

ORNL is managed by UT-Battelle, LLC for the US Department of Energy



Digital Twin – Role of Modeling and Simulation

"The digital twin is the virtual representation of a physical object or system across its life-cycle. It uses real-time data and other sources to enable learning, reasoning, and dynamically recalibrating for improved decision making."

In Nuclear Systems, the digital twin may be characterized by:

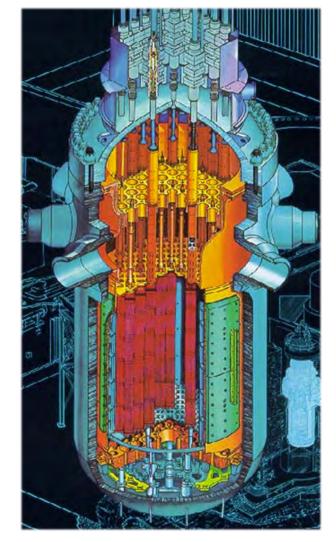
- Virtual simulator for the plant systems and subsystems, including the reactor core and fuel
- Use of a wide-range of sensors (in-core/ex-core detectors, thermocouples, pressure, flow, etc.)
- Mapping of sensor data onto the virtual model through update of the simulator model parameters
- Recalibration of the virtual simulator based on real-time data
- Use of the virtual simulator to monitor operational limits (e.g. core, fuel)
- Use of the virtual simulator to make future projections regarding reactor behavior under "what if" scenarios
- Use of the virtual simulator as part of the reactor control system (human or autonomous)

By these definitions, the digital twin for nuclear systems has existed for decades in the form of online core monitoring systems. What has changed are the advances in modeling, sensors, calibration techniques, and predictive analytics to enable a step change in decision-making capabilities for reactor operation.



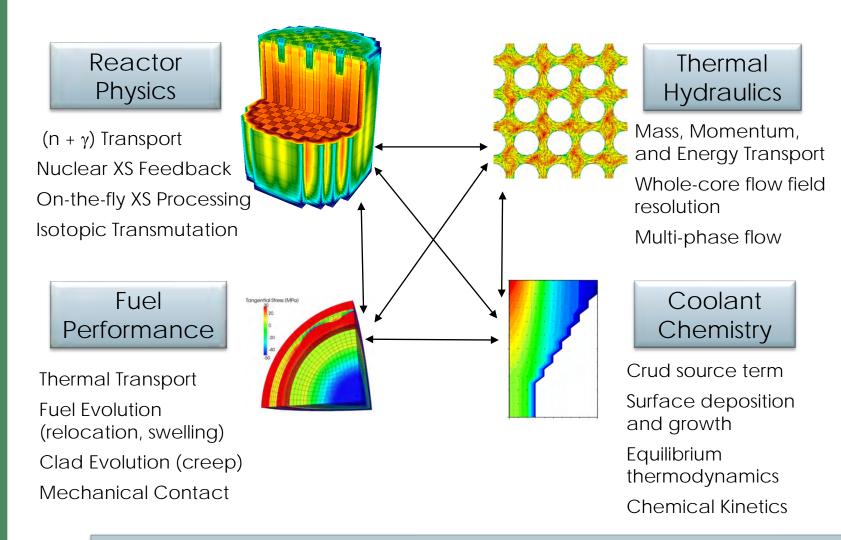
Digital Twin - Virtual Simulator

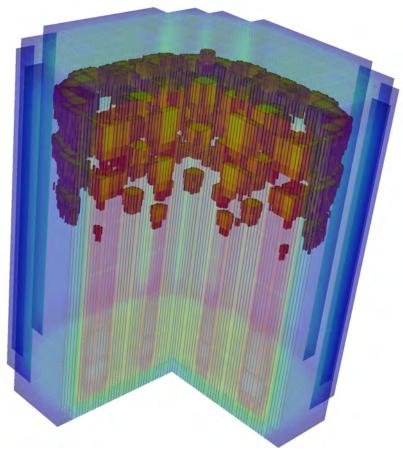
- High fidelity predictive simulation for quantities of interest
 - Safety parameters (temperatures, power deposition)
 - Operational parameters (power response, energy output)
 - Component behavior (lifetime analysis)
- Physics-based modeling for key phenomena
 - First-principles based (elimination of correlations)
 - Multi-physics response for coupled physics
 - Includes neutronics, thermal-hydraulics, chemistry, and materials modeling
- High geometrical resolution
 - Sufficient resolution to make use of real-time sensor data
 - Modeling across length scales atomistic to engineering scale
- Comprehensive, usable and extensible software system
 - Verified software code and solution verification
 - Validated software single and integral effects tests
 - Quantified uncertainties for model parameters and input data





Multi-Physics Coupled Simulation





Crud distribution in a PWR

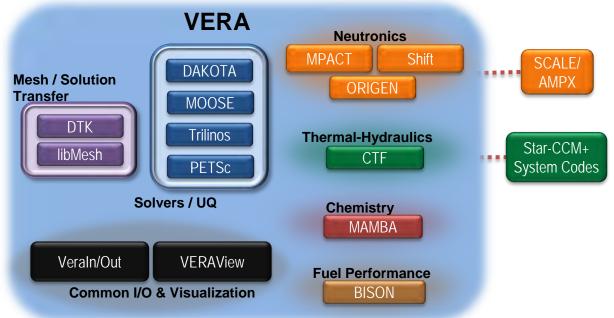
Physics Phenomena of Interest are Common to All Reactor Types

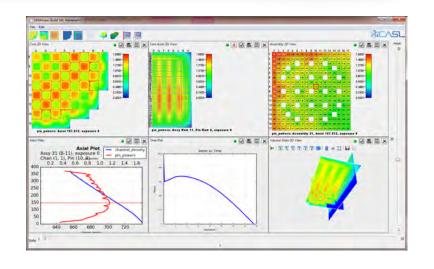


VERA – A Fully Integrated Capability for Reactor Analysis

Virtual Environment for Reactor Applications

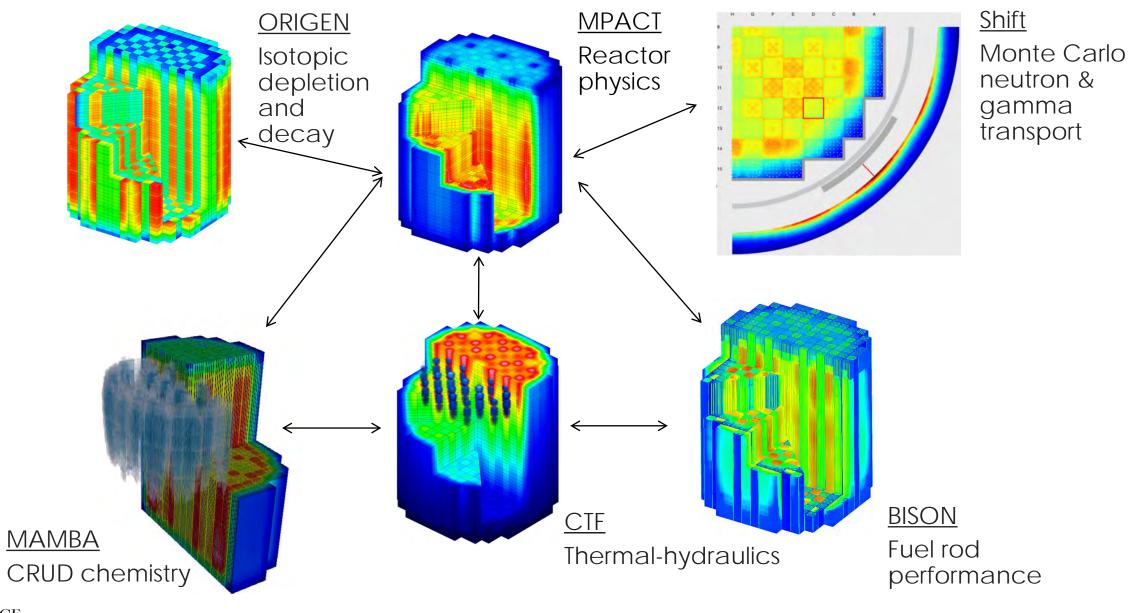
- High Resolution:
 - Fully coupled and pin-resolved neutronic, T/H, and crud growth physics
 - Detailed rod-wise fuel performance analyses
- Integrated Applications:
 - Modeling in-core and ex-core detector prediction of axial offset (AO) due to CRUD deposition
 - Identification of PCI failure risk during load follow operation with accident tolerance fuel and cladding
 - Accumulation of radiation damage in the reactor vessel due to neutron fluence
 - Prediction of cladding integrity during reactivityinitiated transient using coupled neutronics and T/H with offline fuels analysis
- Performance & Usability:
 - User-friendly I/O (e.g. automated mesh generation and data transfers)
 - Integrated visualization tools







VERA Key Physics Codes

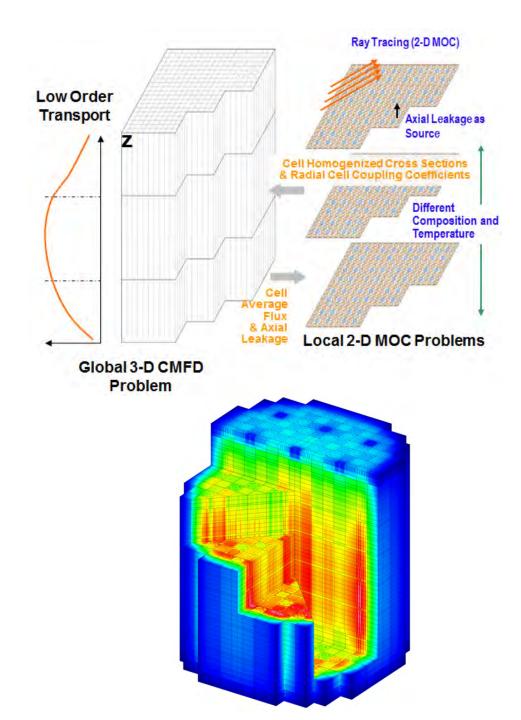


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6

MPACT

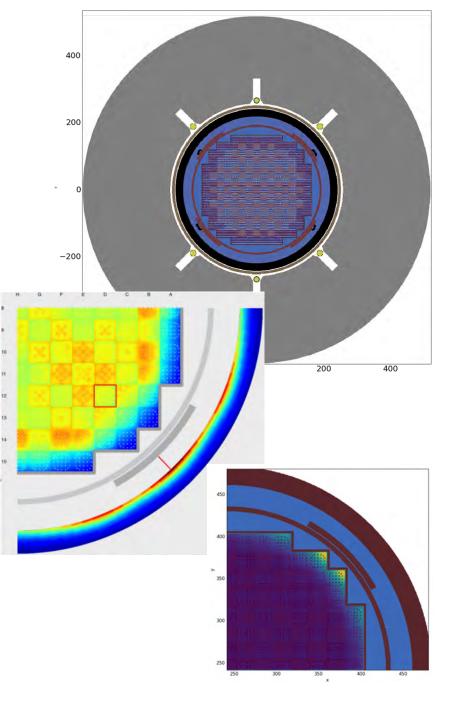
- Advanced 3D Neutronics
 - Method-of-Characteristics
 - 51 energy group nuclear data library
 - Whole pin-wise resolution, including intra-pellet power and isotopic distributions
- Steady-state and transient capability
- Integrated explicit isotopic depletion and decay with ORIGEN
- 3D accuracy comparable to continuousenergy Monte Carlo methods, including Shift and MCNP
- Core shuffling and control rod movement
- In-core detector responses
- Validated against critical experiments and over 150 fuel cycles





Shift

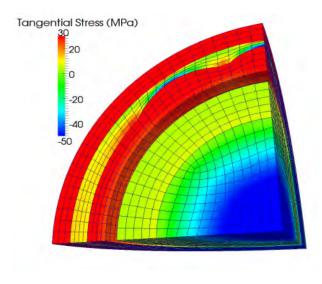
- Accurate and efficient neutron and gamma transport
 - Continuous-energy Monte Carlo neutron & gamma transport to any region outside of the reactor core
 - State-of-the-art hybrid methods focus particles toward the regions of interest
- General geometry capability for ex-vessel region
- MPACT provides accurate 3D fission source & isotopics
- Enables best-estimate vessel fluence analysis and coupon irradiation
- Ex-core detector response calculations and weighting factor generation
- Coupling with materials models allows for calculation of concrete degradation and core structure embrittlement

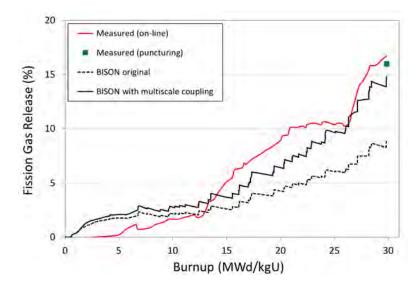




BISON

- VERA can be used to perform detailed fuel rod performance analysis with BISON
 - Finite element-based engineering scale fuel performance code
 - Solves the fully-coupled thermo-mechanics and species diffusion equations in 1D symmetric, 1.5D, 2D axisymmetric or generalized plane strain, or 3D
- Lower length scale and mechanistic models for key physics phenomenon (e.g. fission gas release, thermal conductivity) applicable to existing and future ATF fuel forms and clad
- Fuel rod geometry and power histories used to automatically create BISON inputs for any or all fuel rods in a reactor core
- BISON results are collected into VERAOut format for wholecore fuel rod performance analysis or screening





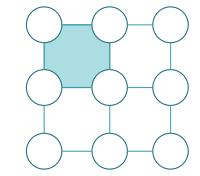
Ref. K. Gamble, G. Pastore, M. Cooper, D. Andersson, ATF material model development and validation for priority fuel concepts, CASL-U-2019-1870-000, July 2019.

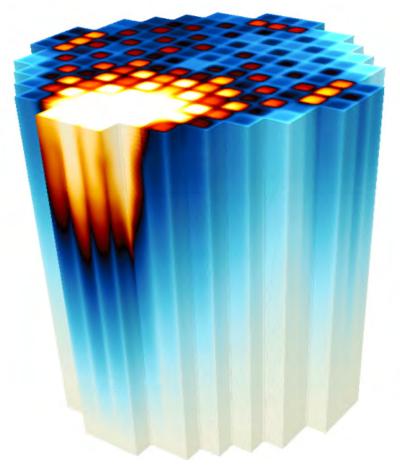


CTF

- Whole-Core Two-Phase Subchannel Thermal-Hydraulics
 - Three-field representation of two-phase flow
 - Continuous vapor (mass, momentum and energy)
 - Continuous liquid (mass, momentum and energy)
 - Entrained liquid drops (mass and momentum)
 - Non-condensable gas mixture (mass)
 - Native, transient fuel temperature model
- Cross flow between channels
- Coupling with Systems Codes (TRACE, RELAP) via inlet and exit boundary conditions
- Spacer grid pressure loses and blockages and intra-grid form losses
- Use of higher resolution computational fluid dynamics (CFD) simulation to improve the subchannel modeling





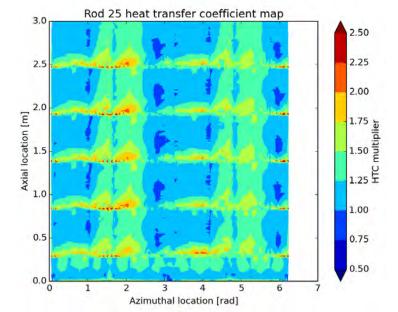




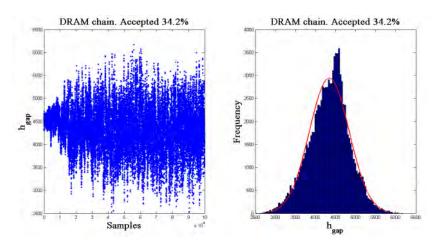
Addressing Modeling Gaps

- Use of high resolution, high fidelity methods to improve lower resolution model for key System Response Quantities (SRQs)
 - STAR-CCM+ informs CTF
 - SRQs include azimuthal heat flux and TKE

- Use of integral experiments and system level data to calibrate fundamental model parameters where Single Effects Tests (SETs) data does not exist
 - Bayesian calibration allows for establishment of uncertainty bounds on calibrated parameters



Ref. Salko, R., S. Slattery, T. Lange, M. Delchini, W. Gurecky, E. Tatli, and B. Collins, Development of Preliminary VERA-CS Crud-Induced Localized Corrosion Modeling Capability, CASL-U- 2018-1617-000, June 2018.



Ref. B. Kuwaileh and P. Turinsky, Data Assimilation and Uncertainty Quantification Using VERA-CS for a Core Wide LWR Problem with Depletion, CASL-U-2016-1054-000, April 2016.



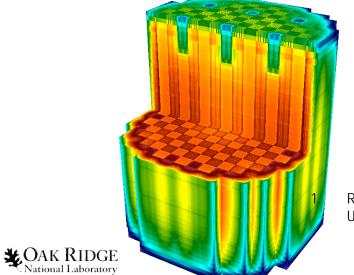
Watts Bar Unit 2 Power Ascension

First US reactor startup in over two decades modeled in near real time as a 'blind prediction'

- 4,130 hourly state-points
- 13.5 days of runtime on 2,784 cores
- 892,837 core-hours
- 16,605 fully-coupled neutronics/TH iterations

Accurate comparison to measurement, including a new Vanadium-wire, in-core flux map system (± 2.4%)

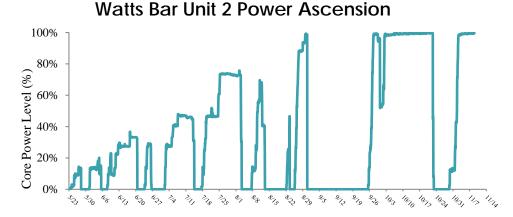
Pin-by-pin spatial detail of 'non-measurable' quantities of interest (e.g. Xe-135)

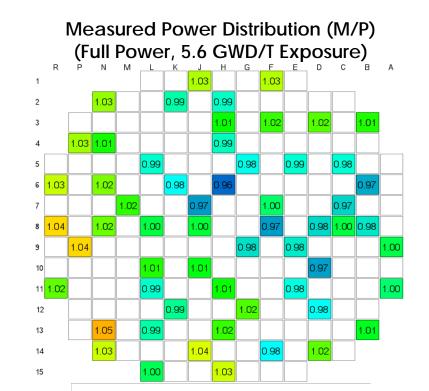


Watts Bar 2 predicted, transient Xenon-135 distribution at 28% power level

Ref. A. Godfrey, B. Collins, C. Gentry, S. Stimpson, J. Ritchie, Watts Bar Unit 2 Startup Results with VERA, CASL-U-2017-1306-000, March 2017.





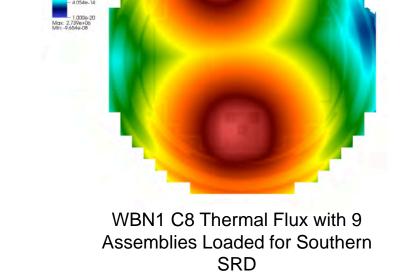


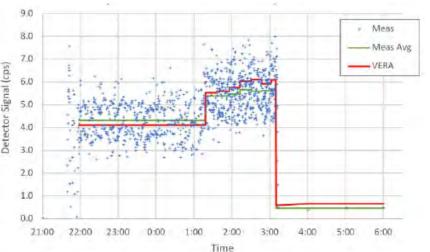
VERA Simulation of Signal Response

- First-of-a-kind capability demonstrated for VERA-Shift applied to coupled incore/ex-core calculations
- Addresses a concern over secondary source signal strength as seen by the source range detectors (SRD) during refueling
- In this application, virtual detector signals were generated for the refueling shuffle sequence with direct comparison against measured count rates
- Excellent agreement between measured and predicted signal

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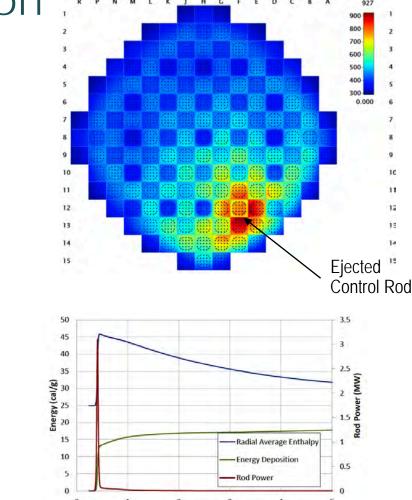


Ref. Godfrey, E. Davidson, G. Wolfram, B. Collins, C. Gentry, G. Ilas, S. Palmtag, T. Pandya, K. Royston, Watts Bar Unit 1 Source Range Detector Response Validation During Refueling, CASL-U-2018-1561-000, December 2018.



DOE-NE Advanced Modeling Simulation Light Water Reactors - Near-term focus

- Provide support for advanced LWR nuclear technologies and target areas for which current LWR modeling and simulation capabilities cannot be used
- Areas include:
 - Accident tolerant fuels
 - High burnup, high enrichment fuel
 - Materials fabrication and performance, including advanced manufacturing
 - Two-phase fluid flow, including flow regime transitions
 - Reactor operational performance
 - Reactor safety performance



AP1000[®] RIA fuel rod enthalpy and energy deposition evolution (ATF fuel form)

Transient Time (s)

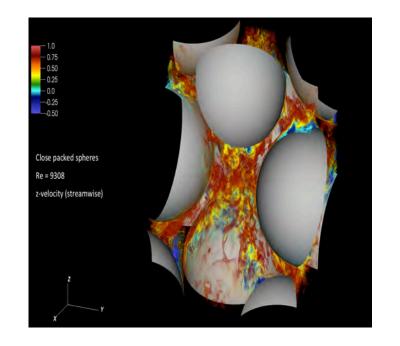


DOE-NE Advanced Modeling Simulation Advanced Non-Light Water Reactors - Near-term focus

- Target areas identified by industry, GAIN Technical Working Groups, and the US NRC to support their activities including molten salt, HTGR, and fast reactor technologies
- Support industry and the NRC for the rapid development and demonstration of microreactors in the 3-5 years time frame
- Areas include:
 - Fuels

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- Materials fabrication and performance, including advanced manufacturing
- Chemistry
- Reactor systems



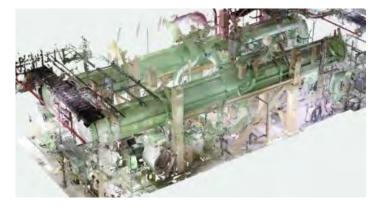
Turbulent Heat Flux – Nek5000/BISON

Summary

- The virtual reactor simulator is one aspect of the Digital Twin for Nuclear Systems
- High fidelity, high resolution virtual simulator technology has rapidly evolved to the level of high predictability for reactor quantities of interest based on coupled, multi-physics modeling
 - First principles combined with multi-scale approach can capture the relevant physics phenomena
- Uncertainties in input parameters and closure relations may nevertheless be an issue for a particular reactor configuration (fuel form, coolant).
 - Model gaps can be addressed through use of formal calibration methods
 - Such methods benefit from availability of measured data required for calibration
- Integration of high fidelity, high resolution simulation with advanced sensors result in unprecedent detailed of reactor behavior



Modeling and Simulation to Support Digital Twins



Dr. Jeffrey W. Lane Chief Engineer and Principal Consultant Zachry Nuclear Engineering <u>lanejw@zachrynuclear.com</u> 919-903-6763

Digital Twin Applications for Advanced Nuclear Technologies Online Workshop December 1-4, 2020



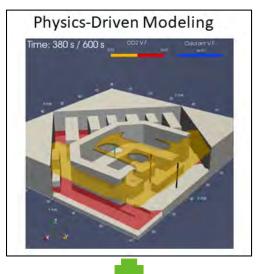
INTRODUCTION

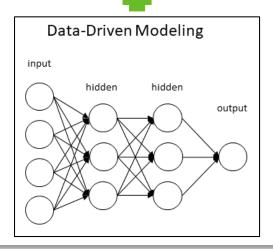
Digital Twin is a virtual replica of a physical asset

- Can be a plant, system or specific component
- Enhanced understanding of physical asset integrating data + simulation
- Can use Machine Learning (ML) & Artificial Intelligence (AI) to identify causal relationships and produce reduced-order models (ROM)
- Predict performance and expected response of the asset
- Identify vulnerabilities

Types of Digital Twins

- Design identify issues before construction and optimize system
- Construction support scheduling and evaluate as-built deviations
- Operations monitor performance degradation and maintenance
- Others







EXAMPLE - DESIGN DIGITAL TWIN

Design by simulation

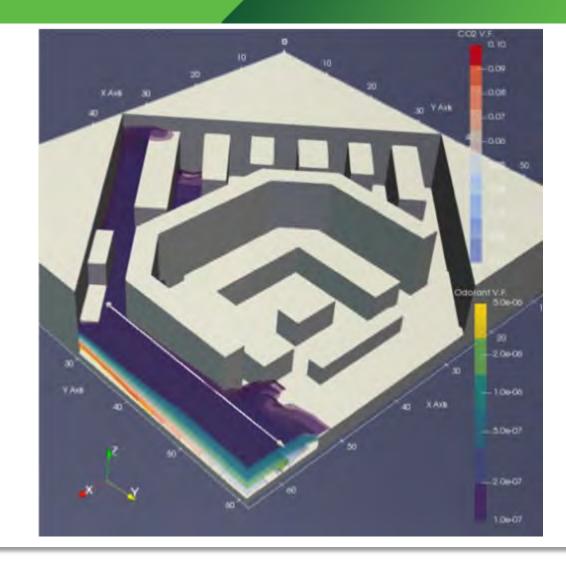
- Physics based modeling provides access to a wealth of data, including unmeasurable quantities
- Can be more cost effective than testing

Attributes:

- Identify system faults before the system is built
- System optimization

Example – Ventilation System

- Toxic gas and room habitability assessments
- Location and sizing of HVAC and filtration
- Complex geometry and recirculation patterns
- Simulation identified local pockets of higher concentration for original design





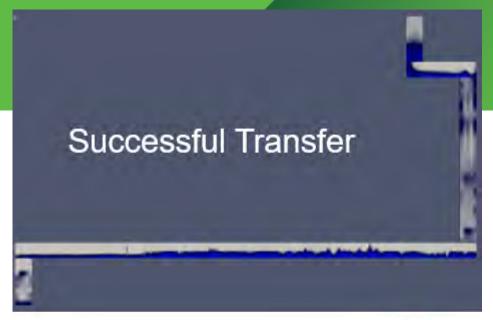
EXAMPLE - OPERATIONS DIGITAL TWIN

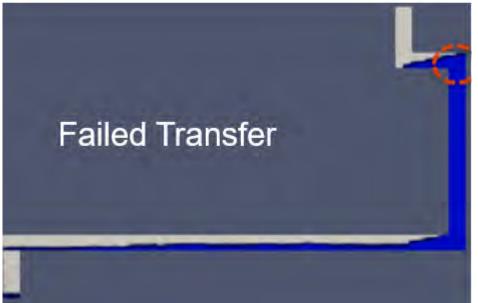
Attributes:

- Connected to the physical asset by continuously monitoring and collecting information
- Continuously learning and dynamically updating
- Simulation used to fill in knowledge gaps (e.g., non-existent data for fault scenarios)

Example - Vacuum transfer system

- Time critical transfer of fluid
- Performance degradation of seals and vacuum system
- Elongate time between maintenance and minimize downtime
- Pre-emptively schedule maintenance before failure

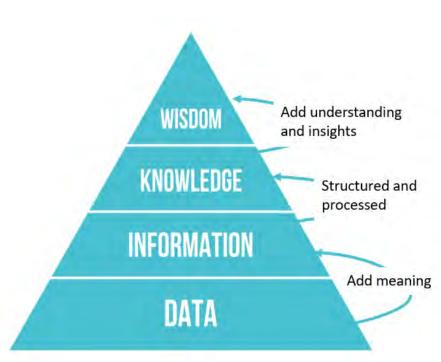






REQUIREMENTS

- Modeling & Simulation (M&S) plays an important role in digital twins to:
 - Fill in knowledge gaps or lack of available data
 - Provide access to unmeasurable quantities
- However, the M&S results must be **obtainable** and **meaningful**
 - Design by simulation requires VVUQ of M&S tool
 - Will need to assimilate M&S results with information obtained directly from the asset (I&C signals) and resolve discrepancies
- The following must be considered with respect to applying M&S for digital twins
 - Establish Applicability
 - Data Assessment
 - Software Requirements
 - Computational Requirements



From - J. Rowley, "The Wisdom Hierarchy: Representations of the DIKW hierarchy", Journal of information Science, pp. 163-180, 2006



APPLICABILITY OF M&S TOOLS

- Just like any other application of M&S, one must establish the credibility and applicability of the evaluation model (code + inputs) for the intended application
 - Provides confidence that the simulation includes the necessary physics and produces accurate results throughout the application domain
 - Establish the **uncertainty** and **trustworthiness** of the results
- Established methods for assessment
 - US NRC Code Scaling, Applicability and Uncertainty (CSAU) using Phenomena Identification and Ranking Table (PIRT)
 - Evaluation Model Development and Assessment Process (EMDAP) from Reg. Guide 1.203
- Provide good frameworks for evaluating the adequacy and sufficiency of a result; however, in practice many applications of these methods tend to rely heavily on engineering judgement.

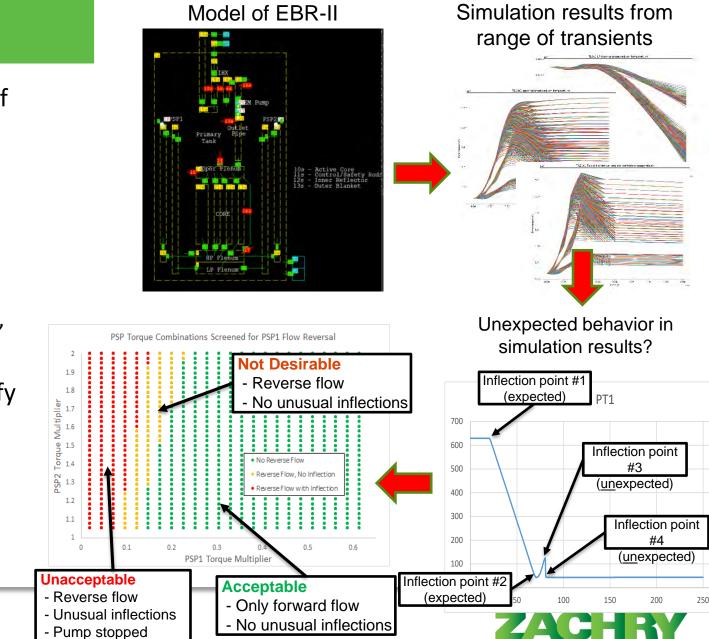
APPLICABILITY OF M&S TOOLS

- Need quantitative approaches to assess
 - Accuracy of results, including uncertainty and effects of scale
 - Domain coverage where are the holes?
 - Adequacy what level of agreement is sufficient?
- Quantitative approaches
 - Reduce reliance on engineering judgement
 - Rank and prioritize areas for improvement both in the simulation and experimental needs
- Example:
 - Predictive Capability Maturity Quantification (PCMQ)



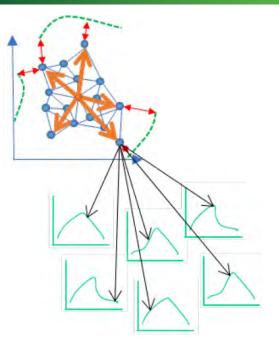
DATA ASSESSMENT

- Must assess the quality and trustworthiness of M&S data prior to training ML algorithms.
 - Establish confidence or identify unexpected results
- Automated tool to parse results
 - Search against multiple criteria and types
 - Limits (>,<), logical (AND/OR), inflection points, etc.
- Scanning tool and criteria developed to identify unexpected or anomalous behavior in simulation results
 - Reinforces that samples and training data cannot be treated as a "black-box"



DIGITAL TWINS AS SOFTWARE

- Digital twin functionality is similar to M&S
 - Stores prototypical conditions, physics, and closures to make inferences and predictions for real systems
 - But, generally able to provide results much faster than M&S
- Digital twins include different types of software
 - Computational Engine or simulator
 - Training of ML/AI algorithms
 - Digital twin itself
- ML/AI are new paradigms relative to existing nuclear SQA standards (e.g., 10 CFR 50 Appendix B, ASME NQA-1)
 - Not static and continuously learning, but must be able to verify results
 - Must provide transparency & traceability to gain confidence in these technologies
- Depending on functionality or role of the digital twin, may also need to consider software reliability, hazard analysis and cybersecurity.





COMPUTATIONAL REQUIREMENTS

- Several different considerations for "computational performance"
 - Effectiveness of process for generating training data
 - Adaptive sampling and coverage assessment
 - Assisted using other available knowledge bases
 - Digital Twin Training Process
 - Balance accuracy with potential for overfitting
 - Hyperparameters represent an additional sensitivity/uncertainty
 - Execution time for Digital Twin
 - Depends on the time scale for the event, but initial response must be real-time
 - Potential for recommendations to change during processing time
- Need a general purpose, validated and robust simulation engine to generate training data
 - Can involve O(10⁴-10⁶) or more simulations, so even a small fraction of simulations that fail to run to completion can be problematic.
 - Requires a 3-D, coarse-grid CFD code that can model all facets of the plant (reactor vessel, piping systems and containment) using a variable mesh and is applicable to both LWR and non-LWRs
 - GOTHIC is an industry trusted multi-physics, multi-scale M&S tool that supports digital twin development



CONCLUSIONS

Digital twin solutions support decision making and provide a variety of benefits.

Modeling & simulation plays an important role in digital twins

- Must establish the credibility of M&S results as it directly impacts the credibility of digital twins
- Therefore, this is a critical element to the adoption, and regulatory approval, of ML based technologies for nuclear applications





WORKSHOP ON DIGITAL TWIN APPLICATIONS FOR ADVANCED NUCLEAR TECHNOLOGIES



December 1-4, 2020

MULTI-PHYSICS MODELING FOR ADVANCED REACTOR SAFETY



RUI HU Nuclear Science and Engineering Division Argonne National Laboratory



Safety Characteristics of Advanced Reactors

Pursuing high levels of inherent (walk-away) safety

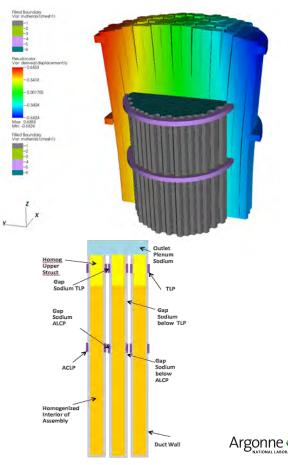
- Inherent reactivity feedback
- Passive decay heat removal
- Ultimate heat sink (ambient air)
- Advanced fuel
 - TRISO, metallic, liquid
- SMR and Micro-Reactor
 - Small nuclear fuel inventories
 - Large surface to volume ratio

- Multi-physics calculation for unprotected transients?
- Accurate modeling of in-vessel heat transport (from the core to vessel wall)
- Detailed simulation vs. lumped parameter approach
- Integrated modeling of reactor cavity cooling system or vessel cooling system



Needs for Multi-scale Multi-physics Capability (1)

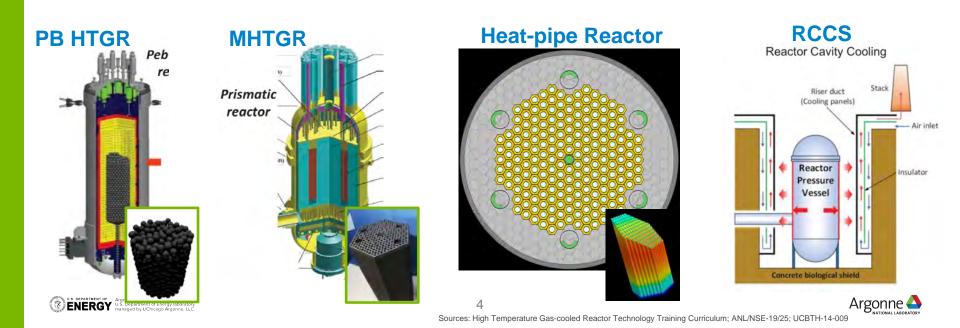
- Analysis of the transient behavior of a nuclear reactor requires coupled simulation of reactor kinetics and thermal-hydraulics of the reactor core
- In advanced nuclear reactors, e.g. Sodium-cooled fast reactor, the reactivity feedback due to core radial and axial thermal expansion are important
- The coupled simulation of thermal-hydraulics and thermal-mechanics is important for the multiphysics simulations of advance reactors for accurate prediction of thermal reactivity feedbacks





Needs for Multi-scale Multi-physics Capability (2)

- Decay heat removal
 - Most advanced reactor designs rely on passive safety system, such as RCCS
 - Decay heat must be conducted from core to surface: fuels/structures are strongly thermally-coupled, and requires multi-dimensional modeling and simulation capabilities



MULTI-PHYSICS SIMULATION OF HEAT PIPE MICRO-REACTOR



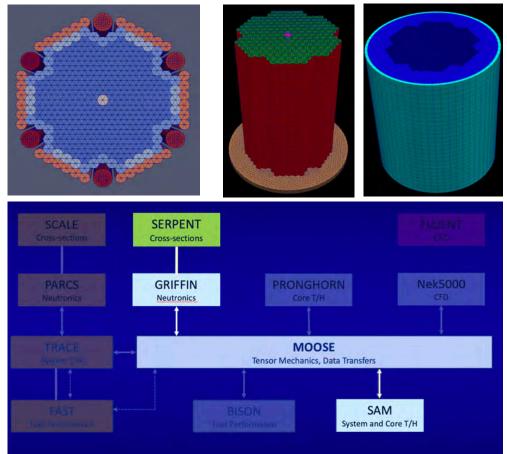
 Argonne National Laboratory is a
 U.S. Department of Energy laboratory managed by UChicago Argonne, LLC.



COUPLED CODE SIMULATIONS

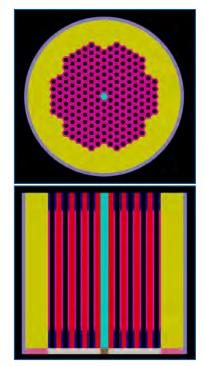
- Joint Argonne-INL-NRC efforts using BlueCRAB
- Coupled codes in reference heat pipe microreactor model
 - Reactor Kinetics (MAMMOTH/Rattlesnake)
 - Thermomechanics (MOOSE Tensor Mechanics)
 - 3D Heat Transfer (SAM)
 - Heat Pipe Heat Exchanger (SAM)
 - Reactor Cavity Cooling System (SAM)
- MOOSE: multi-physics framework
- MAMMOTH: INL neutronics code
- SAM: ANL system code

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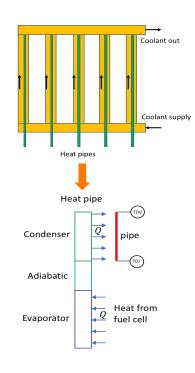


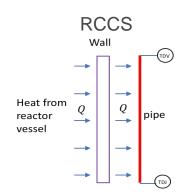
SAM MODELS

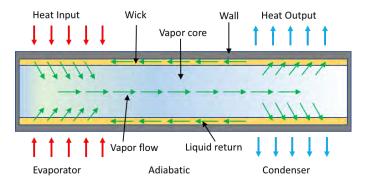
Reactor core



Heat pipe heat exchanger



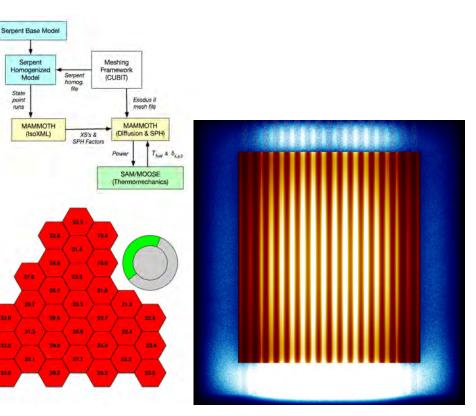






MAMMOTH REACTOR PHYSICS MODEL

- Beginning-of-life (BOL) conditions
- Multi-group diffusion solver with MAMMOTH/Rattlesnake
- Correction with the super-homogenization (SPH) equivalence scheme
- Cross-section preparation with SERPENT Monte Carlo code
- Reactivity feedback effects
 - Doppler effect: fuel temperature
 - Radial expansion: radial core mesh deformation
 - Axial expansion: fuel axial mesh deformation

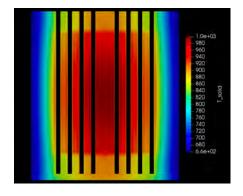




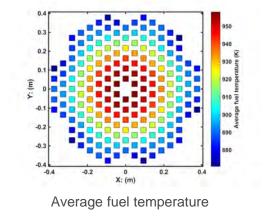
STEADY STATE

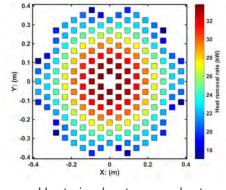
- The model works very well for the steady state operation analysis
- Average fuel temperature keeps very well the symmetry of the reactor core
- Heat pipe near the center removes roughly 1.5 times heat compared with heat pipe near the periphery of the core (average 26 kW)

Parameters	Value
Eigenvalue	0.99990492
Total power	5.0 MW
Power to heat pipes	4.8942 MW
Power to RCCS	0.05291 MW
Average fuel temperature	914.7 K
Average hex can temperature	912.8 K
Average bottom/top reflector temperature	866.9 K
Average side reflector temperature	765.6 K
Average plate temperature	803.6 K
Average vessel wall temperature	674.5 K



Reactor core temperature



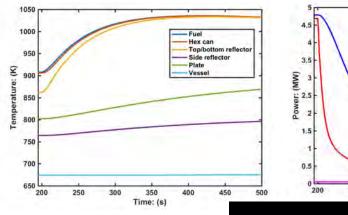


Heat pipe heat removal rate

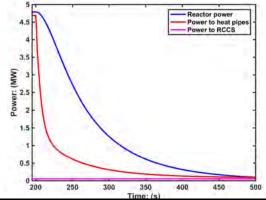


LOSS OF HEAT SINK

- Heat pipe heat removal rate drops quickly to a lower level
 - Flow rate drops to 0.1% of steady-state value
 - Slow decrease due to the thermal inertial of the heat pipes
- Reactor power drops quickly due to the strong negative reactivity feedback
- Decay power was not considered yet in the reactor physics model



Average solid temperature



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MULTI-PHYSICS MODELING FOR DIGITAL TWIN DEVELOPMENT



GY Argonne National Laboratory is a U.S. Department of Energy laboratory managed by UChicago Argonne, LLC.



SCALABLE DIGITAL TWIN IN SAFARI PROJECT

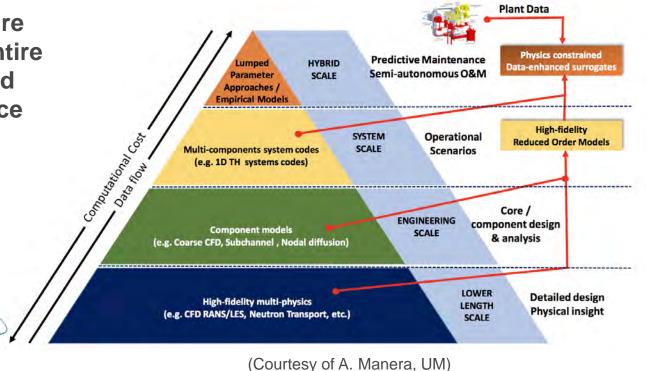
Physics-based to ensure robustness over the entire range of operations and data-enabled to enhance predictive capabilities.

SAFARI: Secure Automation for Advanced Reactor Innovation, ARPA-E GEMINA Award

Argonne

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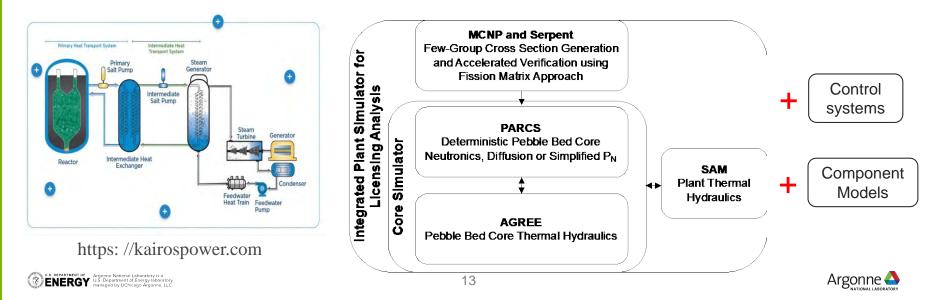
Kairos Power CURTISS -





MULTI-PHYSICS MODELING FOR DIGITAL TWIN DEVELOPMENT

- Multi-physics simulations including plant control and protection systems
- To build the ML-augmented, physics-based reduced order models of the FHR
- To demonstrate the accuracy of the digital twin and the commercial benefit



MULTI-SCALE MULTI-PHYSICS MODELING CAPABILITY NEEDED FOR ADVANCED REACTOR SAFETY AND SCALABLE DIGITAL TWIN DEVELOPMENT



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UNIVERSITY OF CENTRAL FLORIDA

Hybrid Physics-Informed Neural Networks, Cumulative Damage Models, and Digital Twins

Felipe A. C. Viana, PhD PML Principal Investigator Assistant Professor

E-mail: viana@ucf.edu

Probabilistic Mechanics Laboratory pml-ucf.github.io

Mechanical and Aerospace Engineering University of Central Florida

Prognosis and digital twins

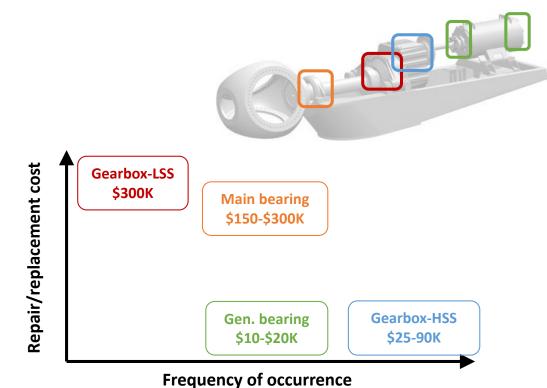
Maintenance costs

- Production lost
- Component
- Equipment rent, labor, etc.

Prognosis and digital twin challenges:

- Physics not fully understood
- Data is highly unstructured
 - Operation/controls vastly available (?)
 - Poor inspection and failure data
- Digital twins must bridge the gap between model predictions (understanding) and observations (reality)
- 2. Hybrid models can be really helpful

(a) Onshore wind turbine example



Sethuraman, L., Guo, Y., & Sheng, S. (2015). Main bearing dynamics in threepoint suspension drivetrains for wind turbines. American Wind Energy Association Conference & Exhibition, May 18–21, Orlando, FL.



Physics-informed neural networks are not new...

JOURNAL OF COMPUTATIONAL PHYSICS 91, 110-131 (1990)

Neural Algorithm for Solving Differential Equations

HYUK LEE

Department of Electrical Engineering, Polytechnic Institute of New York, Brooklyn, New York 11201

AND

IN SEOK KANG

Department of Chemical Engineering, California Institute of Technology, Pasadena, California 91125

Received August 17, 1988; revised October 6, 1989

Finite difference equations are considered to solve differential equations numerically by utilizing minimization algorithms. Neural minimization algorithms for solving the finite difference equations are presented. Results of numerical simulation are described to demonstrate the method. Methods of implementing the algorithms are discussed. General features of the neural algorithms are discussed. © 1990 Academic Press, Inc.

2018 Advances in Neural Information Processing Systems (NeurIPS 2018 – Best paper)

Neural Ordinary Differential Equations

Ricky T. Q. Chen*, Yulia Rubanova*, Jesse Bettencourt*, David Duvenaud University of Toronto, Vector Institute {rtqichen, rubanova, jessebett, duvenaud}@cs.toronto.edu

Abstract

We introduce a new family of deep neural network models. Instead of specifying a discrete sequence of hidden layers, we parameterize the derivative of the hidden state using a neural network. The output of the network is computed using a blackbox differential equation solver. These continuous-depth models have constant memory cost, adapt their evaluation strategy to each input, and can explicitly trade numerical precision for speed. We demonstrate these properties in continuous-depth residual networks and continuous-time latent variable models. We also construct continuous normalizing flows, a generative model that can train by maximum likelihood, without partitioning or ordering the data dimensions. For training, we show how to scalably backpropagate through any ODE solver, without access to its internal operations. This allows end-to-end training of ODEs within larger models.

How can we leverage this concept to build digital twins?



Cumulative damage models and uncertainty quantification

Fatigue crack growth

$$\frac{da}{dN} = C\Delta K^m$$

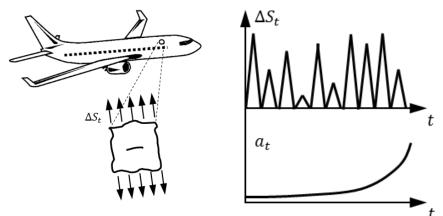
where:

- *N*: number of cycles
- *C* and *m*: material properties (coupon tests)
- $\Delta K = F \Delta S \sqrt{\pi a}$
- ΔS : engineering analysis (e.g., FEM)

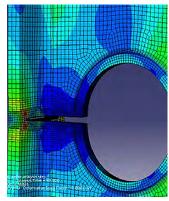
What if ΔK or ΔS are not accurate?

We propose using hybrid models for uncertainty quantification

(a) Fatigue crack growth at fuselage panel



(b) Finite element modeling





Physics-informed neural networks are perfect for prognosis digital twin

years

years

Use case:

- Very few output observations
- Inputs observed throughout
- Sequences are VERY long
- Cell models transition never observed

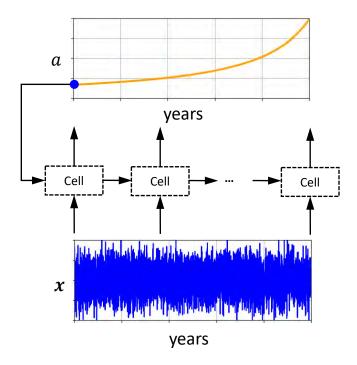
If output is observed throughout, data-driven recurrent neural networks (LSTM, GRU, etc.) might be useful, otherwise...

(a) Typical training

а

x

(b) Typical prediction



Very hard (impossible) without physics

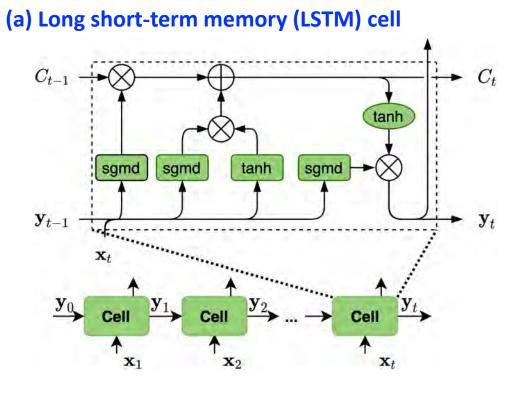
R. G. Nascimento and F. A. C. Viana, "Cumulative damage modeling with recurrent neural networks," AIAA Journal, Online First, 13 pages, 2020, DOI: 10.2514/1.J059250.

Blue: observed data

Cell

Gray: desired output (never fully observed) **Orange:** Recurrent neural network prediction

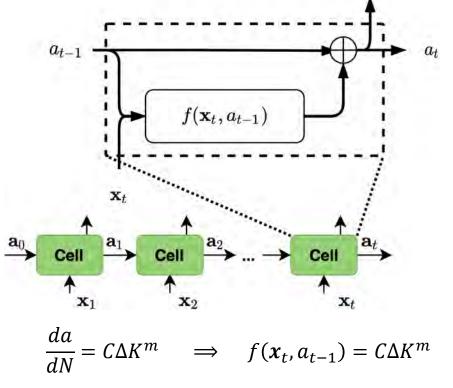
Cumulative damage model with recurrent neural networks



• RNNs are perfect fit for damage accumulation,

• $f(x_t, a_{t-1})$ can be customized.





R. G. Nascimento, K. Fricke, and F. A. C. Viana, "A tutorial on solving ordinary differential equations using Python and hybrid physics-informed neural network," Engineering Applications of Artificial Intelligence, Vol. 96, 2020, 103996, DOI: 10.1016/j.engappai.2020.103996.



Wind turbine main bearing fatigue

Model-form uncertainty:

- Bearing fatigue: relatively well-understood
- Grease degradation: difficult to model with physics

Damage inspection:

- Bearing: not always measurable
- Grease:
 - Laboratory: accurate but expensive
 - Visual: large uncertainty but affordable

Unbalanced data:

- Supervisory control and data acquisition (SCADA) system (per 10 mins)
- Inspection depends on operator inspection policy

Y. A. Yucesan and F. A. C. Viana, "A physics-informed neural network for wind turbine main bearing fatigue," International Journal of Prognostics and Health Management, Vol. 11 (1), 2020.

Y. A. Yucesan and F. A. C. Viana, "Hybrid physics-informed neural networks for main bearing fatigue prognosis with visual grease inspection," Computers in Industry, Accepted.

(b) Visual grease inspection ranking (high variability)



(a) SCADA data

Example of ranking

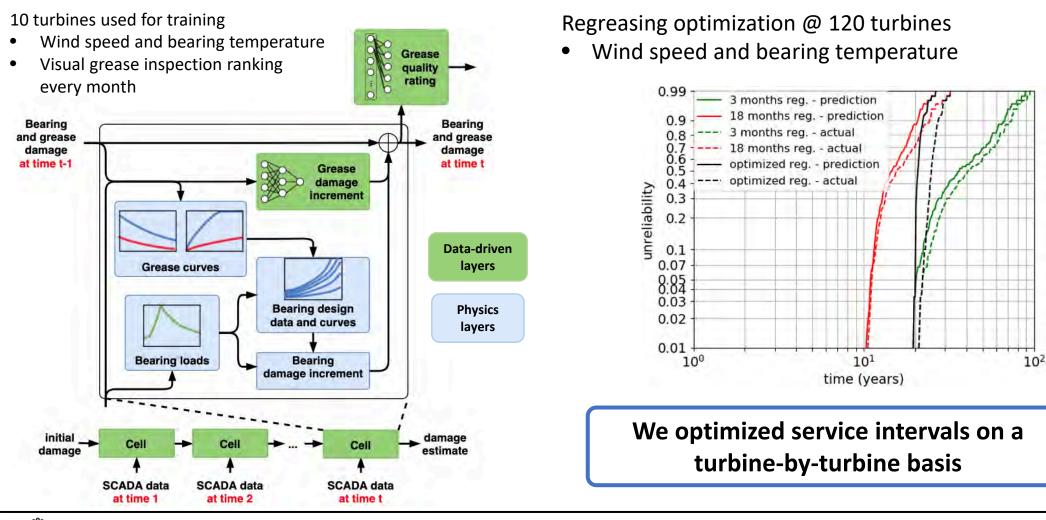






Hybrid physics-informed neural network

(a) Hybrid model





Lithium-ion battery aging modeling

Key technology for electric vehicles



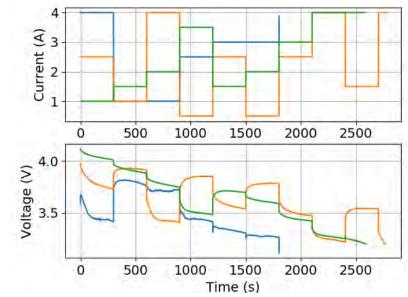


Challenges:

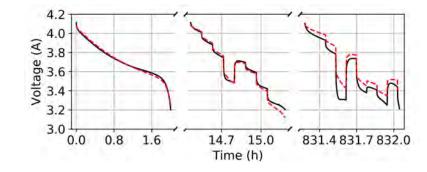
- Prognosis models depend on a number of empirically adjusted factors
- Hard to account for aging

R. G. Nascimento, M. Corbetta, C. S. Kulkarni, and F. A. C. Viana, "Hybrid Physics-Informed Neural Networks for Lithium-Ion Battery Modeling and Prognosis," Applied Energy, submitted.

(a) Example of random loading conditions



(b) Aging can cause models to diverge from observations



Hybrid physics-informed neural network

(b) Aging model (a) Hybrid model 1.3 1e4 Forecast 8 batteries used for training 1.2 Current and voltage time histories 1.1 Internal voltage adjusted with constant discharge . 1.0 Battery aging is a probabilistic model adjusted using hundreds of . 0.9 000 0 6.0 d hours worth of data Voltage 0.7 at time t Fleet prior 0.6 ---- Observations only Battery Battery 0.5 states states Observations + fleet prior time t-1 time t Li ion Physics 0.4 1.5 2.5 0.5 1.0 2.0 0.0 concentration kernels Cumulative Energy (kWh) increment (c) Probabilistic forecast data Equilibrium Voltage increment Data-driven (based on Butlerpotential kernels Volmer model) Nernst model) 4.2 Measurement ---- Observations only 4.0 Battery aging Variational Interna Observations + fleet prior Voltage kernels parameters Voltage (V) Current ······ time t Battery Battery 3.4 states Cell states Cell Cell time 0 time t 3.2 Current Current Current time 1 time 2 time t 3.0 500 0 1000 1500

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3.0

3.5

2000

Time (s)

Model-form uncertainty in corrosion fatigue

Challenge

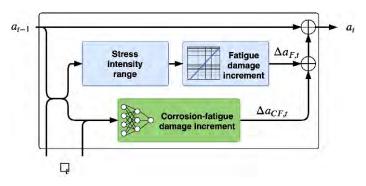
- Assumed: pure mechanical fatigue
- After 5 years: corrosion-fatigue

Data

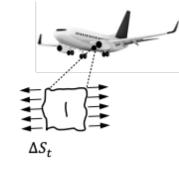
- Load history of 5 years: 150 aircraft
- Crack length: 15 aircraft at end of 5th year.

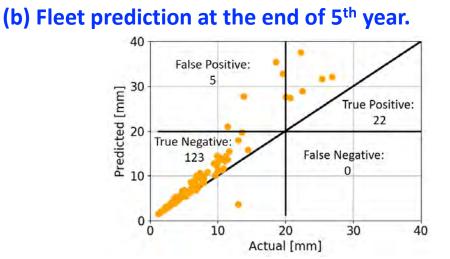
Damage accumulation grossly underestimated!!!

(a) Hybrid physics-informed neural network cell

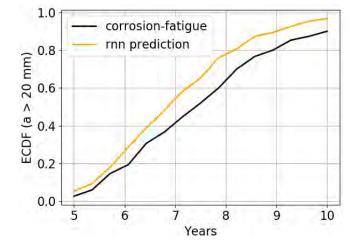


A. Dourado and F. A. C. Viana, "Physics-informed neural networks for missing physics estimation in cumulative damage models: a case study in corrosion fatigue," ASME Journal of Computing and Information Science in Engineering, Vol. 20 (6), 10 pages, 2020.





(c) Probability of failure forecast



Probabilistic Mechanics Laboratory

Publications:

pml-ucf.github.io/publications

🗘 GitHub

Physics-informed neural networks package github.com/PML-UCF/pinn

Ordinary differential equation solver: https://github.com/PML-UCF/pinn_ode_tutorial

Wind turbine main bearing fatigue github.com/PML-UCF/pinn_wind_bearing

Corrosion-fatigue prognosis github.com/PML-UCF/pinn corrosion fatigue

Credit really goes to my PhD students







Kajetan Fricke



Renato Nascimento



Yigit Yucesan

Sponsors and Collaborators













UNIVERSITY OF MASSACHUSETTS DARTMOUTH

A Quantitative Framework to Assess Tradeoffs in Alternative Models and Algorithms for Prognostics and Health Management

Saikath Bhattacharya and Lance Fiondella



Introduction

• Prognostics and health management

Dartmouth

UMass |

- Modernizing system reliability engineering with sensing, models, and algorithms to accurately estimate remaining useful life
- Promotes nonfunctional RAM+C (reliability, availability, maintainability, and cost) requirements

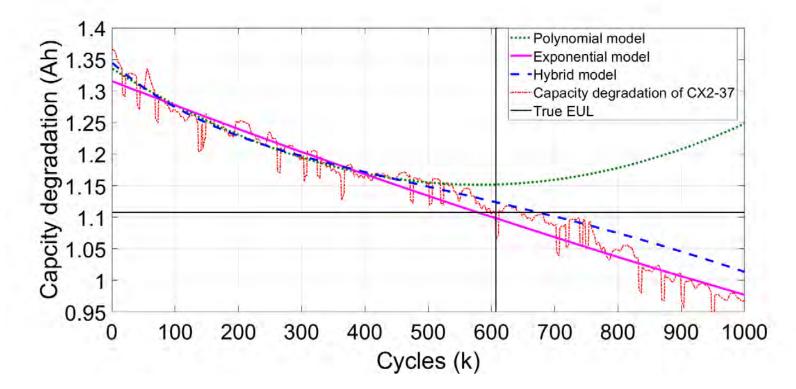


Motivation

- Previous studies
 - Emphasize development of
 - Degradation models
 - Algorithms to estimate model parameters
 - Typically
 - Restricted to single maintenance cycle and focused on enhancing prediction
 - Do not assess long term performance of competing methods



Limitations of Academic Modeling Studies



Number of cycles used to fit models (500) Often hand-picked to make a proposed model appear favorable



Motivation (2)

- Fewer studies
 - Assess impact of PHM decisions on cost and other derived reliability measures
 - Restricted to simulation and analytical techniques (not data-driven)



Proposed Approach

- Objective framework to assess
 - Performance decisions made by alternative combinations of models and algorithms
 - Adapts analytical methods from maintenance theory to data-driven approach
 - Average cost per unit time
 - Utilization
 - Safety
 - Availability

(1)

(2)

(3)

Capacity (Battery) Degradation Models

Some parametric models

• Polynomial model

$$- y_k = x_1 k^2 + x_2 k + x_3$$

• Exponential model

$$- y_k = x_1 e^{x_2 k} + x_3 e^{x_4 k}$$

• Hybrid model

$$- y_k = x_1 e^{x_2 k} + x_3 k^2 + x_4$$

Filtering for Battery Degradation Models

- Unscented Kalman filter
 - Recursively updates degradation model parameters (**x**) based on capacity in past and present cycles (y_k) to estimate RUL
- Particle Filtering
 - Based on Bayesian Monte Carlo simulation with importance sampling to update parameters



Preventive Maintenance

- Based on present model parameter estimates
- Recommends maintenance
 - If remaining useful life (RUL) prediction less than prognostic distance
- Continues operation otherwise



Reliability, Availability, and Maintainability Measures

• Given unit lifetime τ and maintenance interval T, inter-renewal time $Z = \min(\tau, T)$ such that

$$E[Z] = \int_0^T (1 - F(t))dt$$

- R(t) = 1 - F(t) - Unit reliability (complement of CDF)



Age Replacement Maintenance Model

• Average cost per unit time

$$\eta_{age}(T) = \frac{F(T)C_{ER} + (1 - F(T))C_{PM}}{\int_0^T (1 - F(t))dt}$$

- F(T) Probability of failure before maintenance
- C_{ER} Cost of emergency repair
- C_{PM} Cost of preventive maintenance



Age Replacement Maintenance Model (2)

• Average cost per cycle

$$C(\theta) = \frac{\sum_{i=1}^{l} \{C_{PM}I(k_i) + C_{ER}[1 - I(k_i)]\}}{\sum_{i=1}^{l} \{k_i^{\theta}I(k_i) + EUL_i[1 - I(k_i)]\}}$$

- θ Prognostic distance
- -l Number of units
- $I(k_i)$ Indicator function of i^{th} unit
- k_i^{θ} Cycle at which preventive maintenance performed on *i*th unit with prognostic distance θ



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ILLUSTRATIONS



Data and Methodology

- Utilized Li-ion battery data set (n = 4)
 - Performed least squares estimation on battery exhibiting most cycles prior to failure and used as initial estimates for UKF and PF (also considered battery with fewest cycles)



Data and Methodology (2)

• Ratio of emergency and preventive repair costs

$$\frac{C_{ER}}{C_{PM}} = 1,000$$

• Mean times to repair

 $MTTR_{PM} = 3$ and $MTTR_{ER} = 8$



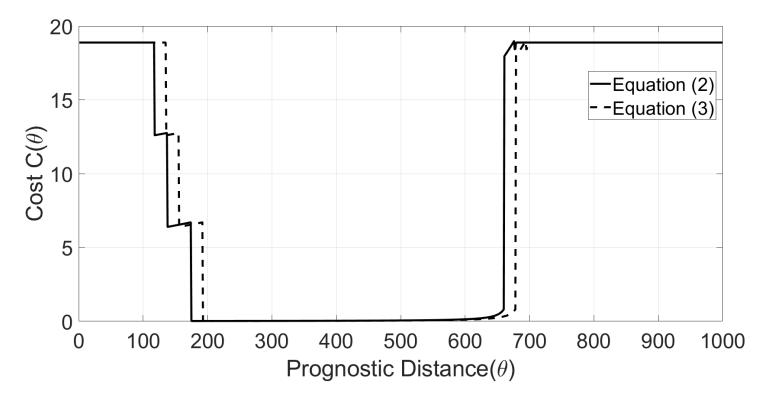
Point Example: Equation (2) under UKF with $\theta = 150$

Measure	CX2-34	CX2-36	CX2-38
True EUL (cycles)	505	560	524
Maintenance (k_i^{θ})	505	527	511
Predicted EUL (cycles)	679	677	661
Unused life (cycles)	0	33	13
Cost $C(\theta)$	10,000	10	10
Safety $S(\theta)$	0	1	1
Time to repair (cycles)	8	3	3

 $C(150) = \frac{10,020}{1.543} = 6.494, \qquad U(150) = \frac{1,543}{1.589} = 97.11\%, \qquad S(150) = \frac{2}{3}, \qquad A(150) = 99.1\%$



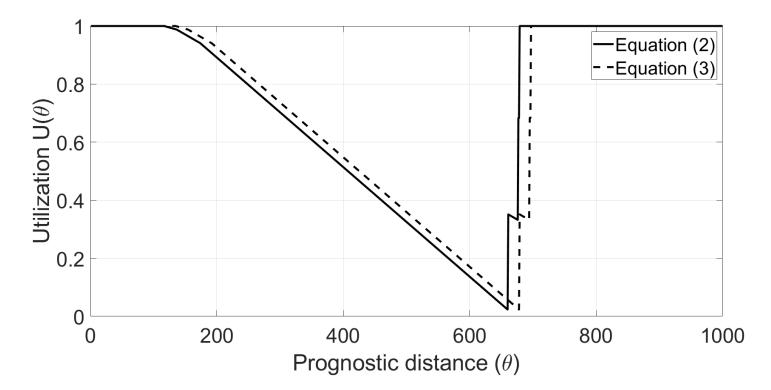
Average Cost per Cycle (UKF)



Prognostic distance $\theta \in (175,678)$ minimizes cost



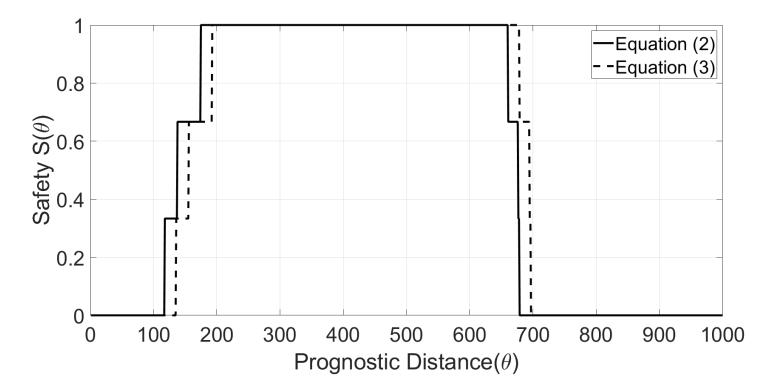
Utilization (UKF)



Utilization decreases monotonically as larger prognostic distance initiates earlier maintenance



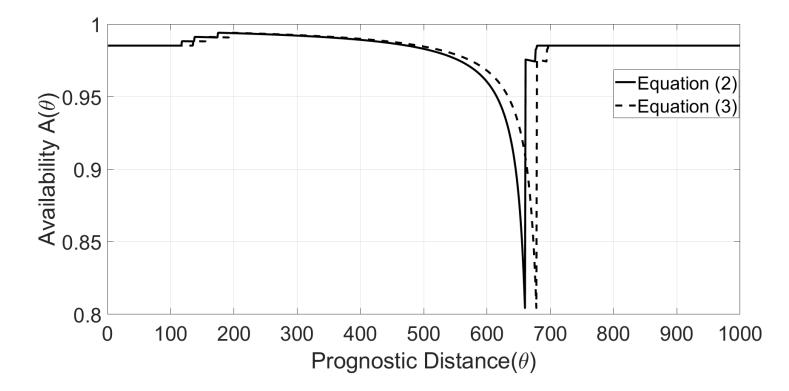
Safety (UKF)



Safety and average cost per cycle exhibit inverse trends



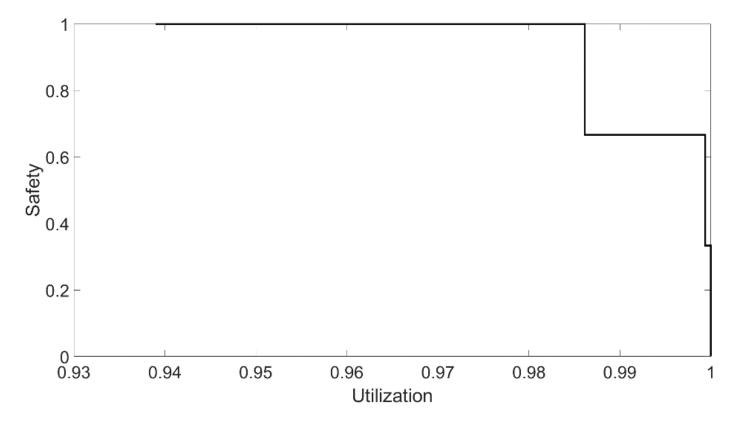
Availability (UKF)



Low utilization corresponds to low availability



Example tradeoff (UKF)



Safety and utilization competing constraints



Summary and Conclusions

- Proposed framework to assess
 - Quantitative performance of PHM decisions made by alternative combinations of models and algorithms
- Developed RAM+C measures for PHM
 - Average cost per unit time, utilization, safety, and availability

Summary and Conclusions (2)

 Applied to combinations of three degradation models and two filtering methods with Li-ion data

set

- Proposed approach
 - Offers method to select prognostic distance to balance stakeholder needs
 - Can be applied to other domains, degradation models (physics of failure), and algorithms (deep learning)



Future work

- Open source framework
 - Crowdsource contribution of

Dartmouth

• Models

UMass

- Algorithm
- Datasets/Challenges
- Raise academic standards for comparison
- Promote collaboration between academic, industry, and government stakeholders



- Formulation of additional quantitative measures
- Performance of particle filtering on quantitative measures
- Comparison of quantitative measures in window minimizing cost

BACK UP SLIDES



Additional Measures

• Utilization

$$U(\theta) = \frac{\sum_{i=1}^{l} \left(k_i^{\theta} I(k_i) + EUL_i \left(1 - I(k_i) \right) \right)}{\sum_{i=1}^{l} EUL_i}$$

- EUL_i End of useful life of i^{th} unit
- Can take values in interval (0,1)
- Poses competing objective with cost and safety



Additional Measures (2)

• Safety

$$S(\theta) = \frac{\sum_{i=1}^{l} I(k_i)}{l}$$

- Fraction of units that undergo preventive maintenance
- Minimizing cost corresponds to maximizing safety



Additional Measures (3)

• Availability

$$A(\theta) = \frac{MTTF}{MTTF + MTTR}$$

• Mean time to failure (MTTF)

$$MTTF = \frac{1}{l} \sum_{i=1}^{l} \left(k_i^{\theta} I(k_i) + EUL_i (1 - I(k_i)) \right)$$



Additional Measures (4)

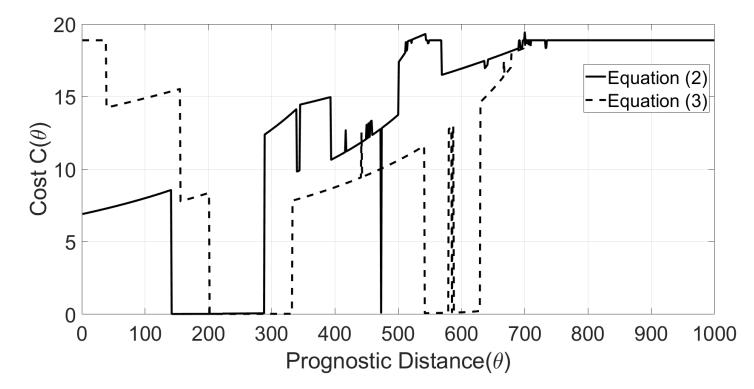
• Mean time to repair (MTTR)

$$MTTR = \frac{1}{l} \left(l_{PM} MTTR_{PM} + l_{ER} MTTR_{ER} \right)$$

- l_x Number of units subject to $x \in (PM, ER)$
- $MTTR_x$ Mean time to repair given that unit underwent $x \in (PM, ER)$



Average Cost per Cycle (PF)

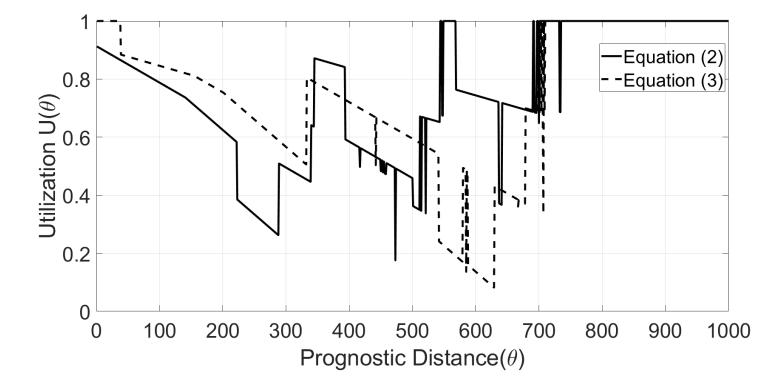


Prognostic distance $\theta \in (202,288)$ minimizes cost



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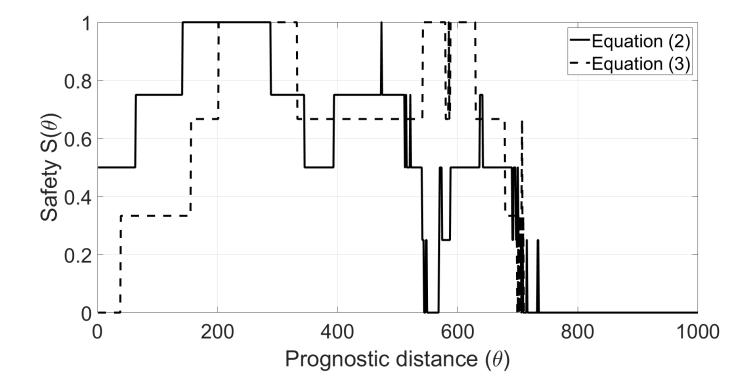
Utilization (PF)



Prognostic distances that produce low utilization correspond to low cost



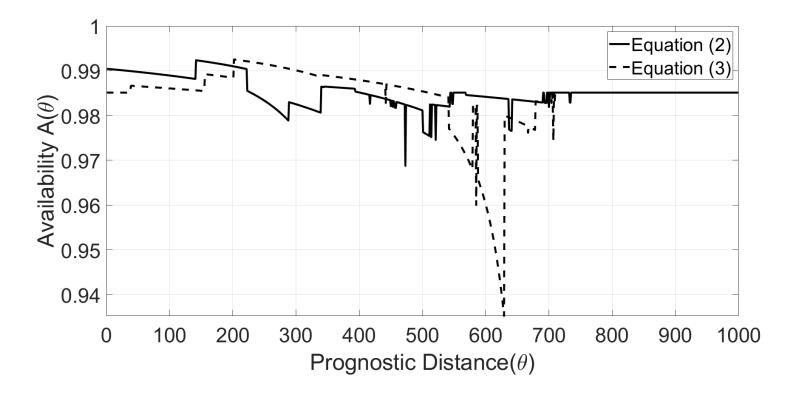
Safety (PF)



Safety and average cost per cycle exhibit inverse trends



Availability (PF)



Low utilization corresponds to low availability



Measures in Prognostic Window with Cx2-37 (most cycles to failure)

Unscented Kalman Filter	Particle Filter
----------------------------	-----------------

θ	Left	Mid	Right	Left	Mid	Right
Eq (2)	175	412	660	142	215	288
$C(\theta)$	0.020	0.039	0.810	0.025	0.031	0.071
$U(\theta)$	0.939	0.482	0.023	0.734	0.596	0.262
$A(\theta)$	0.994	0.988	0.804	0.992	0.990	0.978

Eq (3)	193	435	678	202	267	332
$C(\theta)$	0.020	0.039	0.810	0.025	0.030	0.037
$U(\theta)$	0.939	0.482	0.023	0.751	0.628	0.500
$A(\theta)$	0.994	0.988	0.804	0.992	0.991	0.988

Conservative strategy selects θ at midpoint



Measures in Prognostic Window with Cx2-34 (most cycles to failure)

Unscented	Particle Filter
Kalman Filter	

θ	Left	Mid	Right	Left	Mid	Right
Eq (1)	101	343	586	330	361	392
$C(\theta)$	0.019	0.035	0.267	0.035	0.039	0.044
$U(\theta)$	0.937	0.503	0.067	0.512	0.456	0.4013
$A(\theta)$	0.994	0.989	0.925	0.989	0.988	0.986

Eq (3)	136	374	613	160	261	362
$C(\theta)$	0.020	0.039	0.576	0.022	0.028	0.044
$U(\theta)$	0.887	0.459	0.031	0.815	0.634	0.401
$A(\theta)$	0.994	0.988	0.852	0.993	0.991	0.986

Equation (3) with UKF stable in both scenarios

Digital Twins in a Nearly Autonomous Management and Control System for Advanced Reactors

Nam Dinh, Linyu Lin

Department of Nuclear Engineering

North Carolina State University

12/08/2020





ENGINEERING

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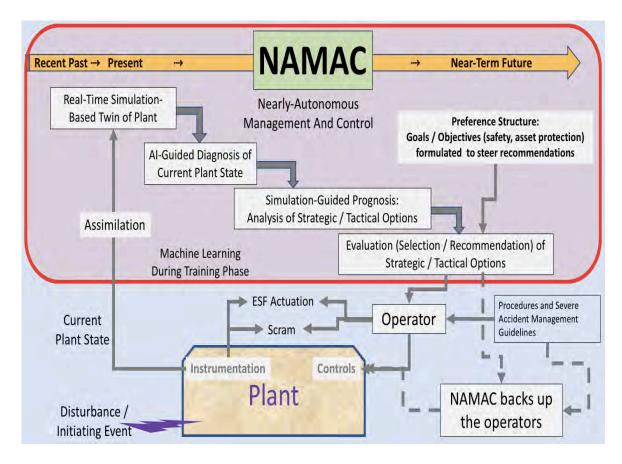
Nearly Autonomous Management and Control (NAMAC)

- A comprehensive control system to assist plant operations
 - Knowledge integration
 - Scenario-based model of plant (systems, success paths)
 - plant operating procedures, tech. specs., etc.
 - Real-time measurements
 - Digital twin technology
 - Power of AI/ML
- NAMAC

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- Diagnoses the plant state
- Searches for all available mitigation strategies
- Projects the effects of actions and uncertainties into the future behavior
- Determines the best strategy considering plant safety, performance, and cost.



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Guiding Principles and Development Philosophy

- High-level requirements
 - Technology neutral
 - Accurate representation (twin) of the plant
 - Dynamic and real-time: diagnosis, prognosis, and evaluation during operations
 - Adaptive (or continuously learning)
 - Explainable: outputs are traceable and justifiable
- Design principles for an intelligent autonomous control system
 - Three-Level Architecture
 - Knowledge Base
 - Digital Twin
 - Digital Twin Development and Assessment Process
 - Trustworthiness Assessment



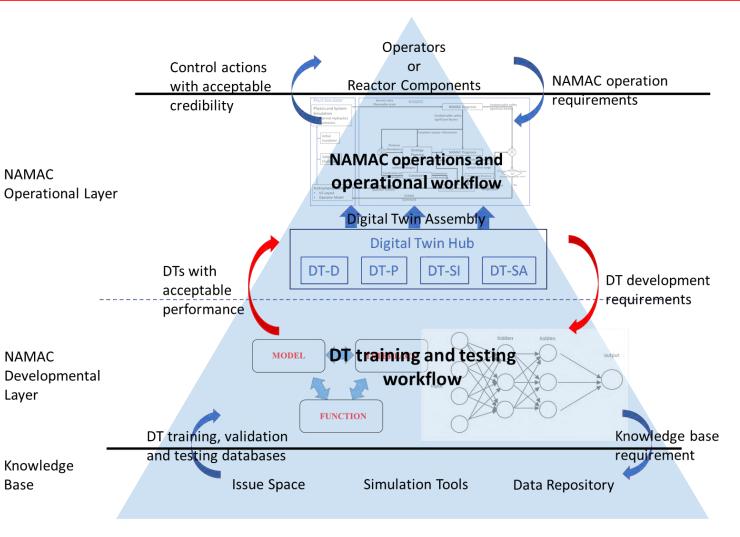
Three-Layer Architecture

- Individual Digital Twins (DT) are assembled into a DT-Hub to support decisions in operation, maintenance, safety management, etc. in the **Operational Layer**
- Each Digital Twin (DT) is a knowledge acquisition system to support specific functions
 - Digital Twin for Diagnosis (DT-D) ۲
 - Digital Twin for Strategy Inventory (DT-SI) •
 - Digital Twin for Prognosis (DT-P) •
 - Digital Twin for Strategy Assessment (DT-SA) •
- Developmental Layer extracts useful information from the knowledge base and creates Digital Twins (DT)
- Knowledge base stores data from simulations, operations, documents, procedures, etc.

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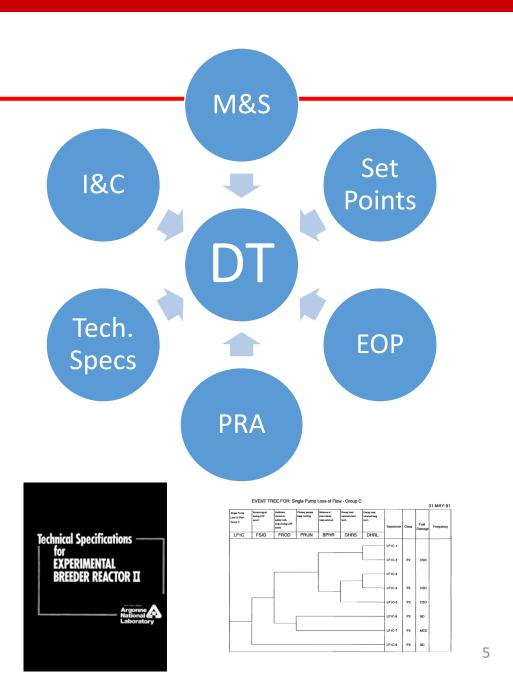
Base



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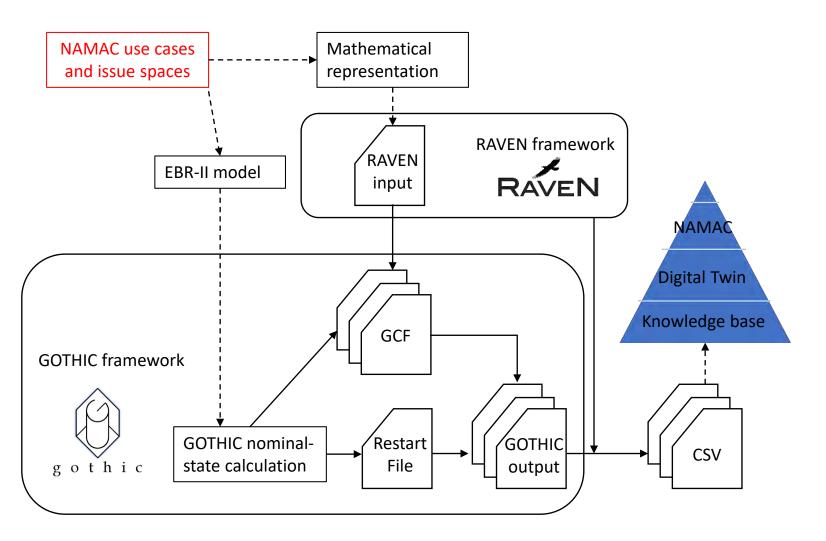
Knowledge Base

- Knowledge base is the foundation of DTs and NAMAC
- Integrate knowledge from a variety of sources
 - Plant monitoring systems
 - Scenario based modeling & simulation (M&S)
 - Operating limits and control procedures
 - Probabilistic assessment of the risk
 - Emergency Operating Procedures (EOP)
- Knowledge base will transit from simulation-based data to assimilating sensor data as a new plant comes on-line and operating history becomes available
 - M&S will always be a key contributor to the knowledge base, particularly for accidents and other low frequency events where actual plant data may not be available.
- Not just "raw" data signals, but these sources are vital knowledge bases
 - Leverage existing information
 - Minimize propensity to treat ML and DTs as "black-box" Department of NUCLEAR ENGINEERING



Database Generation in Knowledge Base

- NAMAC Database generation:
 - Training databases are generated by sampling scenarios to populate information in the application domain
 - The Digital Twin are constructed according to the databases for supporting diagnosis, prognosis, etc.

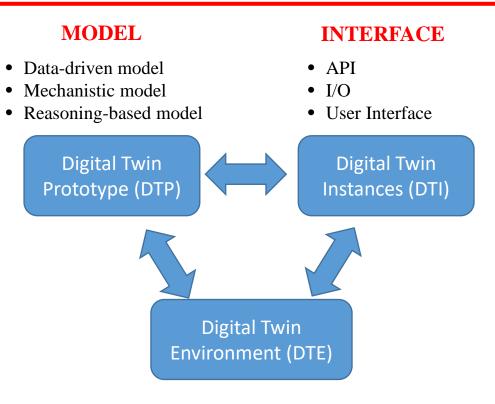




Digital Twin

Definitions for DTs [1]

- Digital Twin technology construct a digital replica (twin) for the real reactors and transients
- DTs must provide insights equivalent to Modeling and Simulation (M&S), but need to learn and provide those insights much faster than the development and uses of M&S
- But DTs are tightly coupled with operation
 - Assimilating and adapting to real-time information from the operating environment
 - Interacting with user for specific objectives



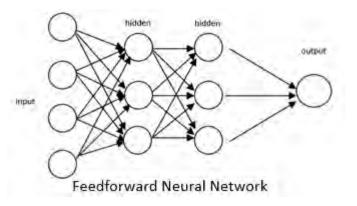
FUNCTION

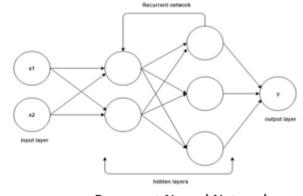
- Use cases
- Objectives
- Output types

Department of [1] F. Ka **Ne6LeAR** . A fransdisciplinary perspectives on complex systems - new findings and approaches", Springer, 2017

Digital Twin Training and Algorithms

- Artificial Neural Network (ANNs) is currently the major technology in constructing Digital Twins and NAMAC system.
- As complexity of NAMAC case studies increases, advanced algorithms are required to support DTs
 - Modular framework allows for multi-tiered implementation
 - Do not need a single, monolithic solution to cover all conditions
- Two classes of advanced algorithms are being investigated:
 - 1. Knowledge/reasoning-based methods
 - Provide explainability and transparency
 - 2. Model free methods
 - Deep learning capability that is needed for diversity and complexity
- Need both types for NAMAC





Recurrent Neural Network



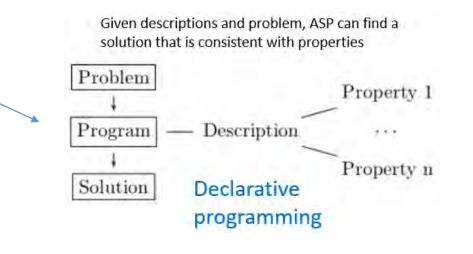
Digital Twin Training and Algorithms

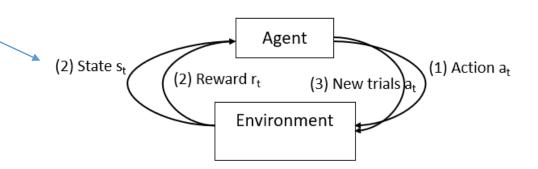
• Advanced Algorithms

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- Answer Set Programming (ASP) is a form of declarative programming oriented towards difficult search problems
 - Discrepancy Checker (DC)
- Ensemble modeling employs a voting technique to aggregate/select predictions from a set of base models
 - Digital twin for diagnosis (DT-D)
- **Reinforcement Learning (RL)** interacts with the environment and is time aware
 - Wholistic NAMAC for furnishing recommendations
- Adaptive sampling techniques for data generation
 - Efficient process to support Strategy Inventory (DT-SI)
- Meta-Learning to accelerate and optimize development

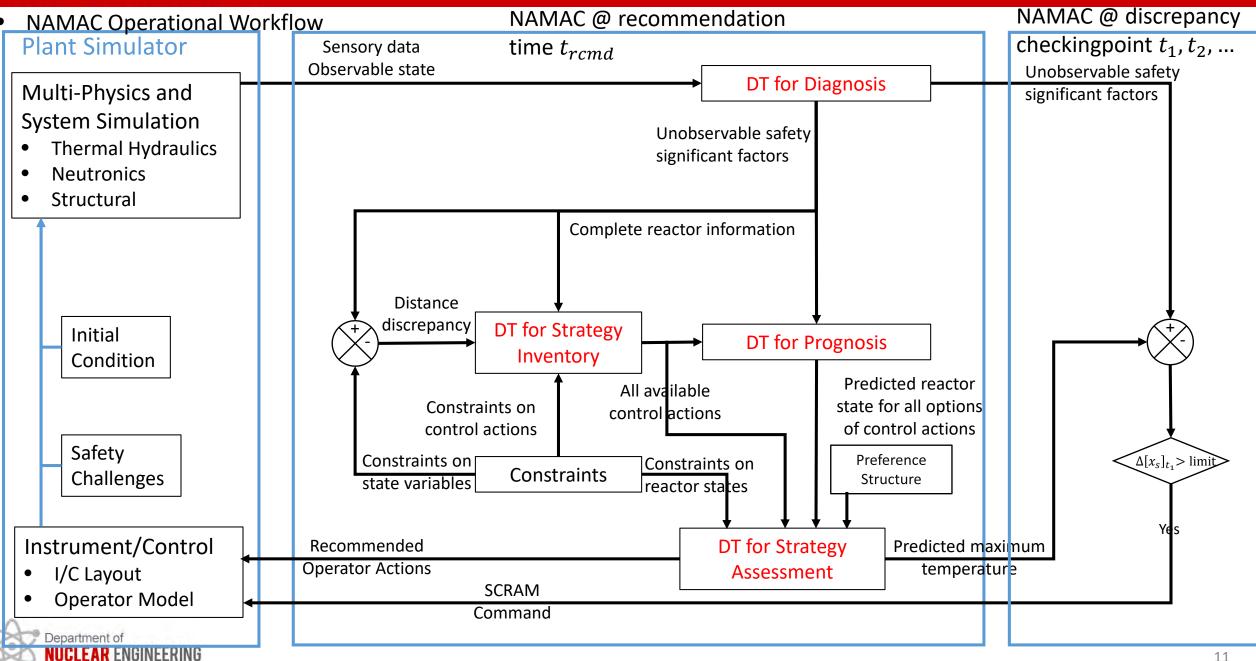




Modular Framework

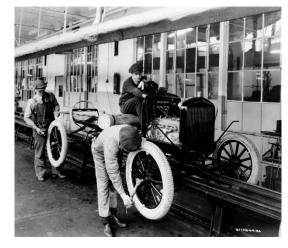
	Function	Modeling	
Diagnosis	Recover full reactor states by assimilating plant sensor data with the knowledge base	Neural nets (feedforward & recurrent); Logic programming (Answer Set Programming)	
Strategy Inventory	Find all available control/mitigation strategies	Uniform sampling Reinforcement Learning	
Prognosis	Predict the transients of state variables over a time range	Neural nets (feedforward & recurrent)	
Strategy Assessment	Rank possible mitigations strategies and make recommendations considering preference structure	Safety margin/limiting surface; Expected utility;	
Discrepancy Checker	Detect unexpected transient during operations considering DT trustworthiness for current conditions	Distance metrics; Logic programming (Answer Set Programming)	





The Development and Assessment Process (DAP)

 Instead of claiming to have a perfect autonomous system for a specific reactor during a specific scenario, our objective is to have a "smart" Development and Assessment Process (DAP) that produces NAMAC systems for generic types of reactors based on requirements from all stakeholders.



1924 – Ford assembly line



1965 – Ford assembly line



2019 – Tesla smart factory

Evolution of "Development and Assessment Process (DAP)" for Automobile

[1] [2] Picture by Ford, "The evolution of assembly lines: A brief history", <u>https://robohub.org/the-evolution-of-assembly-lines-a-brief-history/</u>, 2014
 [3] Picture from "Popular Mechanics", <u>https://ottomotors.com/blog/what-is-the-smart-factory-manufacturing</u>, 2019
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Digital Twin Development and Assessment Process

- DT-DAP for a scalable and robust application of digital twins and NAMAC concept to generic types of use cases and advanced reactors.
- The DAP is conducted iteratively to deliver a reliable NAMAC with a set of credible DTs

Element 1: Refined requirements

<u>Element 2</u>: More complex and realistic knowledge base

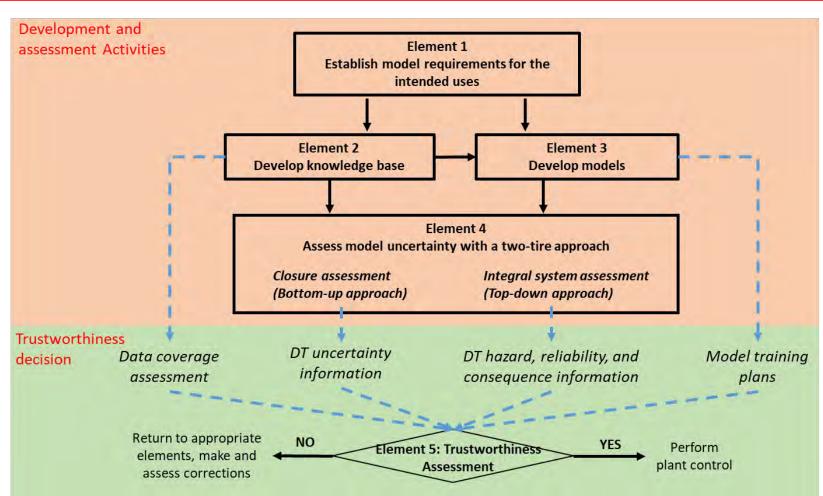
<u>Element 3</u>: Different machine-learning algorithms

Element 4: ML uncertainty quantification, software reliability analysis

<u>Element 5</u>: Digital twin trustworthiness assessment

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Adopted from U.S. NRC RG 1.203 "Transient and Accident Analysis Methods"

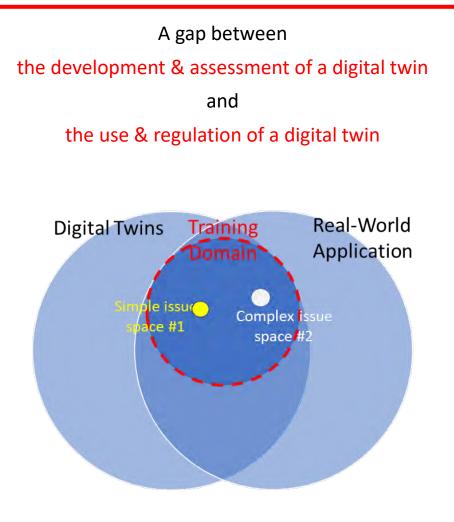
Digital Twin Trustworthiness

- In fundamental, the NAMAC make recommendations by extracting knowledge from knowledge base and assimilating them with realtime sensor signals – Digital Twin
 - Considering the complexity and heterogeneity of knowledge base, we investigate data-driven models and machine-learning algorithms for
- However, for complex systems and difficult tasks, the uncertainty of the DTs in NAMAC, if being overlooked, could introduce additional risks and degrade the trustworthiness of NAMAC recommendations, especially when the DT itself is complicated and black-box
- As a result, we need a trustworthiness assessment framework for DTs in NAMAC (ongoing)

(1) monitor uncertainty that could complicate the determination of mitigation strategies

(2) make uses of information from the DT development and assessment process





Summary

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- Implementation of digital twins for extracting and assimilating the knowledge base with real-time information
- Proof-of-concept of NAMAC for one class of transients
 - Pump malfunction ranging from flow anomaly to complete loss of flow accident
 - NAMAC provides recommendations during the event consistent to human operator norm
- The design of a digital twin development and assessment process (DT-DAP) for implementing, improving, and collecting evidence of a generic types of digital twins in autonomous systems
 - DT-DAP at scoping stage that is driven by user experiences and sensitivity analysis
 - Informed by EMDAP, but necessarily seeks to provide quantitative basis to support NAMAC decision making
 - Next steps the trustworthiness and robustness of DTs based on both intrinsic (i.e., uncertainty quantification, reliability) and contextual properties (i.e., confidence, safety-related vs. non-safety related).



Development and assessment of a nearly autonomous management and control system for advanced reactors

Linyu Lin^a», Paridhi Athe^a, Pascal Rouxelin^a, Maria Avramova^a, Abhinav Gupta^b, Robert Youngblood^c, Jeffrey Lane^d, Nam Dinh^a

¹Department of Nuclear Engineering, North Carolina State University, Roleigh, NC, United States ¹Department of Civil, Construction, Environmented Engineering, North Carolina State University, Baleigh, NC, United States ⁴Adob National Indonemy, Idahe Falls, 10, United States. ⁴Zachry Nuclear Engineering, Inc., Cary, NC, United States.

ARTICLE INFO ARSTRACT Article history: This paper develops a Nearly Autonomous Management and Control (NAMAC) system for advanced read Received 13 May 2020 tors. The development process of NAMAC is characterized by a three layer-layer architecture: knowledge Received in revised form 29 July 2020 base, the Digital Twin (DT) developmental layer, and the NAMAC operational layer. The DT is described as Accepted 7 September 2020 a knowledge acquisition system from the knowledge base for intended uses in the NAMAC system. A set of DTs with different functions is developed with acceptable performance and assembled according to the NAMAC operational workflow to furnish recommendations to operators. To demonstrate the capability of Keywords: the NAMAC system, a case study is designed, where a baseline NAMAC is implemented for operating a Autonomous control simulator of the Experimental Breeder Reactor II during a single loss of flow accident. When NAMAC is Digital twin operated in the training domain, it can provide reasonable recommendations that prevent the peak fue Diamonis. centerline temperature from exceeding a safety criterion. Tremesis @ 2020 Elsevier Ltd. All rights reserved

Introduction

With the advancement in computer performance, machine learning, and digital systems, interest in development of autono-

Adversations: AI, Artificial Intelligence; UT, Digital Twin; DTL: Digital Twin favironment; UTP, Digital Twin Prostuppe; UTD, Lingial Twin Interace; DT-D, Digital Twin for Diagnosis; UT-P. Digital Twin for Strategy Investory; EBH-L Equition of the Diagnosis; UT-P. Digital Twin for Strategy Investory; EBH-L Equimental Brender Reactor II: PCJ, Fuel Contentino: Temperature; FDD, Fauk Detection and Diagnosis; UT-P, Function-Study Hierarchical Fourieve; FDR, False Negative; PNN Feel Forward: NNR False Negative; RAI: PT, False Negative; PNN Feel Forward: NNR False Negative; Antoneworse Management and Contrib, HPF, Nufater Postar Biot, Roady Antoneworse Management and Contrib, HPF, Nufater Postar Diat, FSCI, Pak False Contention: Temperature; ISP, Trimary Solium Pomp; PIA, Probabilisti; Risk Assessment; Q-O, Guantity of Enterent; SMR E, Rott Mena Signar error; SSC, Supervising Control System; SSF, Safety Significant Factor; TR, True Negative; TMR, True Negative False; TP, True Positive; TPR, True Positive; TR, True Positive; Reit; Diatester; RMR.

* Corresponding author.

E-mail addresses: Nin9weauxilu (L. Lin); pathe9tnesa.idu (P. Athe), primiteof9tecsu.etu (P. Roucelin), matercano9tecsu.nlu (M. Avramova), aguptal 9tecsu.edu (A. Gupta), inderty-genegloscielling/ov (R. Youngblood), Land/W@rachrygoup.com (J. Lane), ntdinh6tecsu.edu (N. Dinh).

https://doi.org/10.1016/j.anucene.2020.107861 0306-4540/o 2020 Elsevier Ltd. All rights reserved. mous control systems has increased in a variety of fields from industrial manufacturing to unmanned space, ground vehicles, and nuclear reactors. Autonomous control systems are intelligent systems with self-governance ability to perform and execute control functions in the presence of uncertainty for an extended time (Antsaklis et al., 1991). The degree of autonomy of an autonomous control system depends upon the extent to which it can perform fault diagnosis, planning, forecasting, and decision-making under uncertainty, without human intervention (Wood et al., 2017).

Owing to the inherent risk and uncertainty associated with the operation of nuclear reactor systems, the design of autonomous control systems is a challenging task. Over the past several years, different techniques have been adopted to develop functions related to autonomous control and operation of nuclear reactor systems. Upadhyaya et al. (Upadhyaya et al., 2007) (Na et al., 2006) developed an autonomous control system for a space reactor system (Fast spectrum Lithium cooled reactor) with Model Predictive Control (MPC) using a Genetic Algorithm for optimization, Fault detection in this system is performed using Principal Component analysis. Cetiner et al. (Cetiner et al., 2016) developed a Supervisory Control System (SCS) that uses a probabilistic decisionmaking approach using fault tree and event tree in conjunction

Acknowledgement

- This work is performed with the support of ARPA-E MEITNER program under the project entitled:" Development of a Nearly Autonomous Management and Control System for Advanced Reactors"
- The NAMAC project team
 - Nam Dinh (PI), Abhinav Gupta, Maria Avramova, Min Chi (NCSU)
 - Linyu Lin, Pascal Rouxelin, Paridhi Athe, Joomyung Lee, Anil Gurgen, Edward Chen, Longcong Wang, Harleen Sandhu, Yeojin Kim (NCSU)
 - Son Tran, Botros Hanna (NMSU)
 - Carol Smidts, Xiaoxu Diao, Yunfei Zhao, Boyuan Li (OSU)
 - Robert Youngblood, Cristian Rabiti (INL)
 - David Pointer, Sacit Cetiner , Birdy Phathanapirom (ORNL)
 - Jeff Lane, John Link (Zachry)
 - Olu Omotowa, Eric Williams (TerraPower)
 - Richard Vilim (ANL), Andrea Alfonsi (INL), Askin Yigitoglu (ORNL) [Resource Team]
- GOTHIC license is provided by Zachry Nuclear Engineering, Inc. GOTHIC incorporates technology developed for the electric power industry under the sponsorship of EPRI, the Electric Power Research Institute.



Structural Condition Monitoring with a Digital Twin: Explorations on a Nuclear Containment Vessel Model

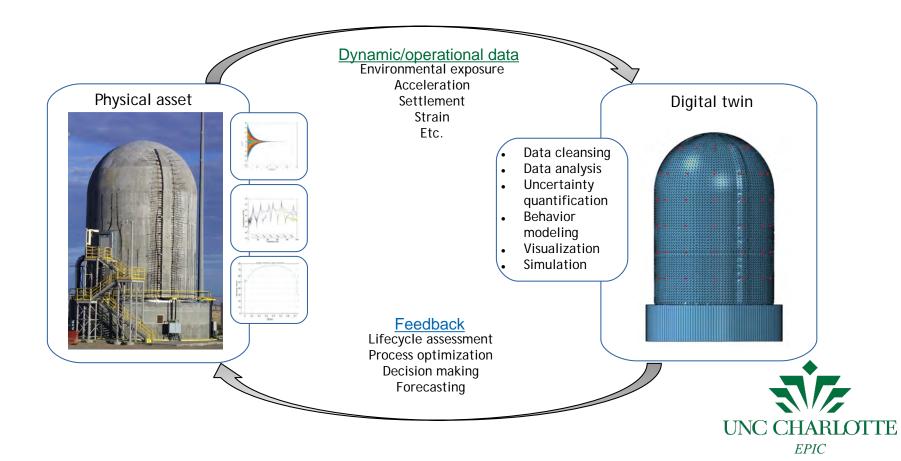
Presenter: Dr. Timothy Kernicky, EPIC Research Assistant Professor of Civil Engineering, University of North Carolina Charlotte

Contributors: Dr. Matthew Whelan



Digital Twin

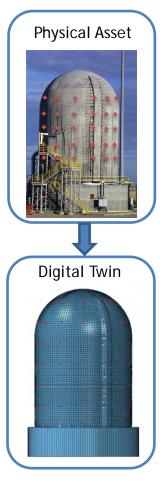
- A digital twin is more than a digital model that faithfully represents physical assets and processes
- The primary distinguishing feature of a digital twin is its connection to the real-world asset with the ability to inform the state of the physical asset



First Step: Structural Identification

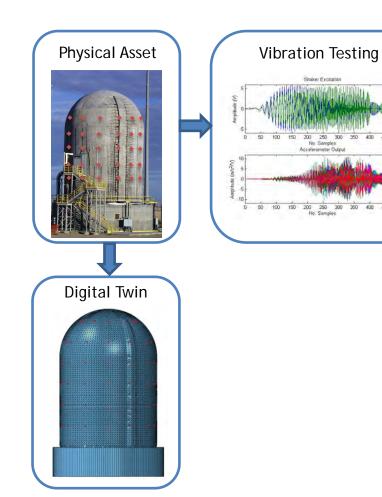
- The first step towards successful digital twin deployment is the development of a "trusted" model, which faithfully replicates the performance of the physical asset
- This simple study leverages vibration-based structural identification to calibrate a set of uncertain material parameters of the digital twin using synthetic measurement data





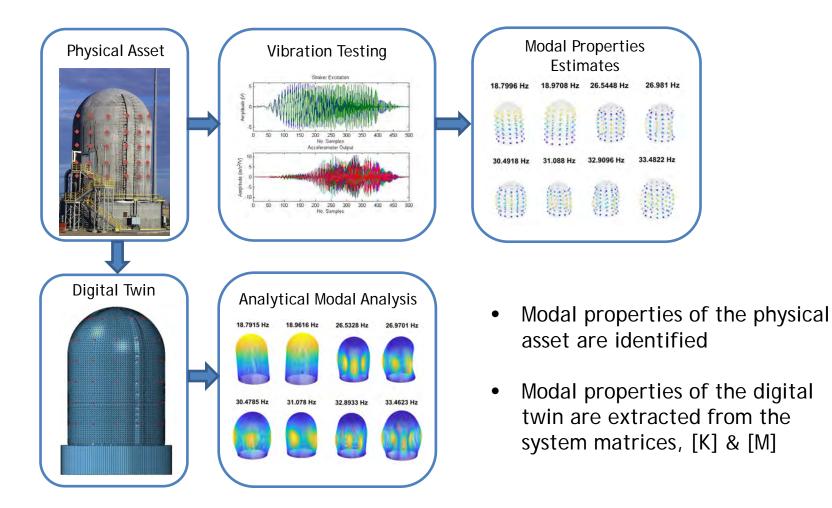
- Finite element (FE) model of physical asset created
- Initial model suffers from
 - Parameter uncertainties
 - Geometries
 - Material properties
 - Idealization errors
 - Discretization errors
- FE model may be leveraged to develop appropriate sensor array for physical structure



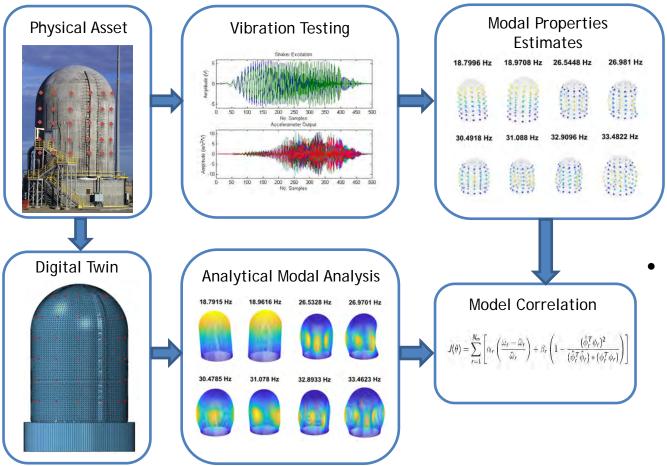


- Vibration data is acquired from sensor array on the physical structure
- Structure excitation can be ambient (operational) or forced



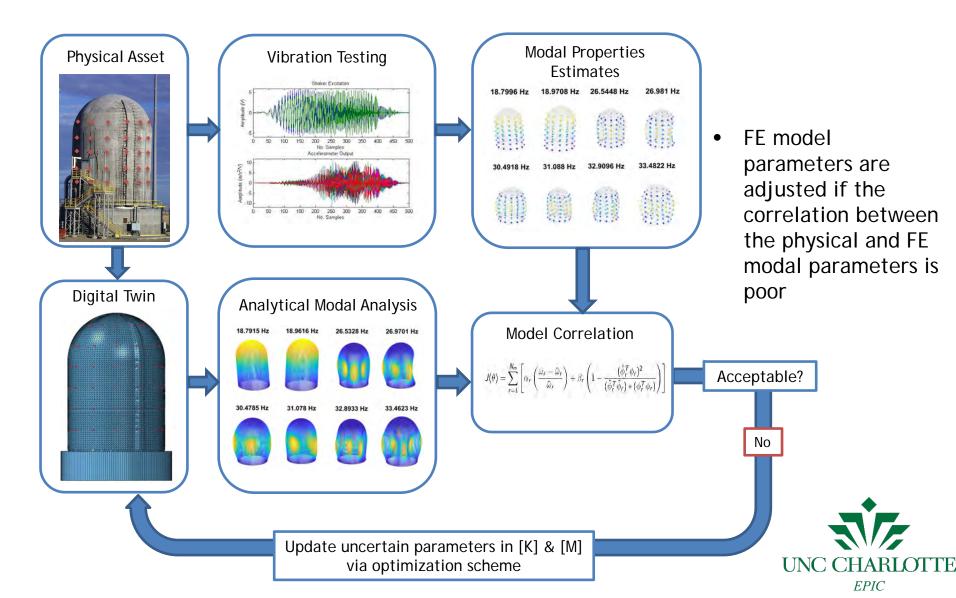


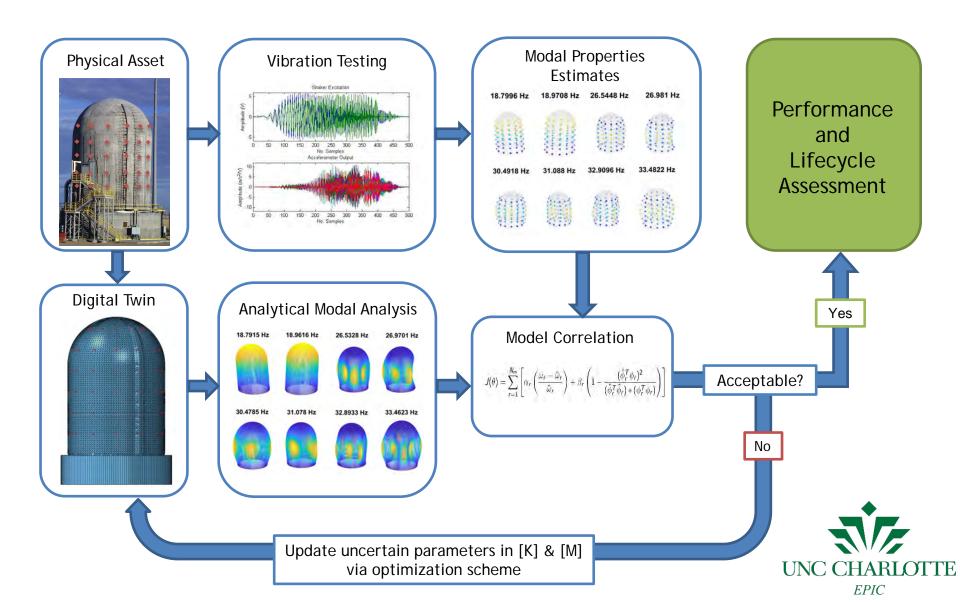




Correlation between
the physical and FE
modal properties is
determined by a modal
measure of fit, which
accepts natural
frequencies and mode
shapes as input.

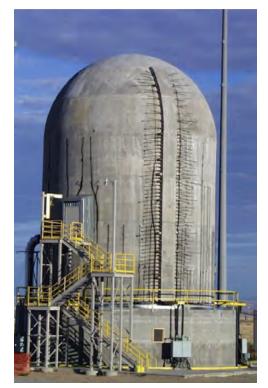






Digital Twin Study of a Containment Vessel

- This study will demonstrate a potential capability to track changes/deterioration in a concrete containment vessel
- The structure explored is based on the 1:4 scale model of the Ohi-3 containment vessel in Japan, which was funded by NUPEC and the NRC and tested by SNL [NUREG/CR-6810, SAND2003-0840P]
- Two FE models will be utilized in this study.
 - One represents the physical asset from which "in-service", synthetic measurements are obtained
 - The second will serve as the digital twin to be updated
- Measurements of dynamic properties (modal parameters) will be used by the digital twin to inform changes in the structural condition, while synthetic response measurements obtained from the "physical" structure will be used to correct the digital twin



[NUREG/CR-6810, SAND2003-0840P]



Concrete Containment Model

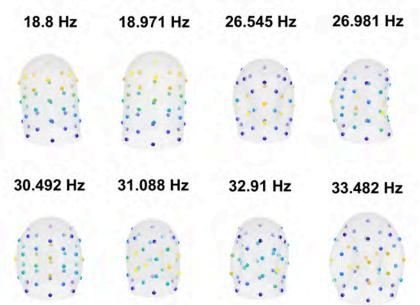
- A simplified finite element model was created using ABAQUS
- Four locations of interest where penetrations exist in the vessel were chosen as uncertain parameters to identify
 - main steam penetration (M/S)
 - feed water penetration (F/W)
 - equipment hatch (E/H)
 - air lock (A/L)
- The modulus of elasticity of each section of elements was used as the uncertain parameter to be updated
- Four deterioration scenarios were examined to demonstrate the ability of the methodology to identify material degradation

Case	Parameter	Stiffness Change (%)
1	$E_{M/S}$	0
2	$E_{M/S}$	-5
3	$E_{M/S}$	-15
4	$E_{M/S}$	-25



Development of Synthetic Dataset

- Synthetic measurement data was extracted from the finite element model in the form of natural frequencies and mode shapes from 42 biaxial sensors
- Noise was added to the synthetic measurements by adding 0.5% Gaussian noise to generate 10 sets of data



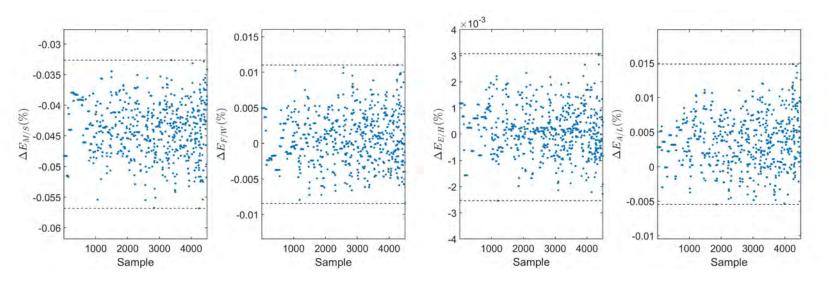
Synthetic natural frequencies and mode shapes for Case 1

	Case 1	Case 2		Case 3		Case 4	
Mode	f_n	$\Delta f(\%)$	MAC	$\Delta f(\%)$	MAC	$\Delta f(\%)$	MAC
1	18.800	-0.033	1.000	-0.108	1.000	-0.200	1.000
2	18.971	-0.041	1.000	-0.135	0.999	-0.250	0.999
3	26.545	-0.038	1.000	-0.122	0.999	-0.218	0.999
4	26.981	-0.031	1.000	-0.100	0.999	-0.183	0.999
5	30.492	-0.035	1.000	-0.115	0.999	-0.212	0.999
6	31.088	-0.025	1.000	-0.080	0.999	-0.148	0.999
7	32.910	-0.045	1.000	-0.141	0.999	-0.248	0.999
8	33.482	-0.048	1.000	-0.149	0.999	-0.258	0.999



Bayesian Model Updating

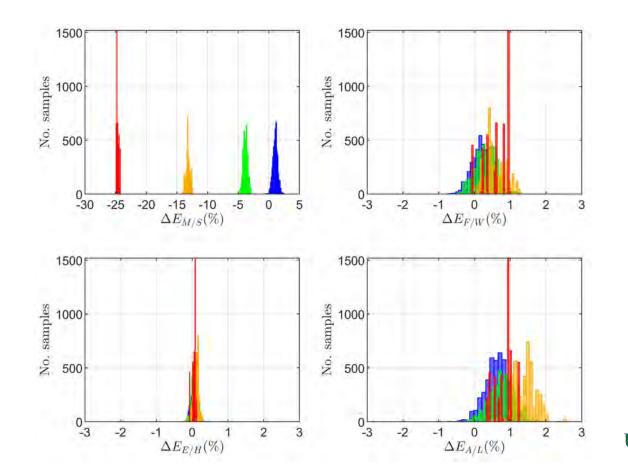
- Probabilistic updating was utilized as the model updating method, which accounts for measurement and modeling uncertainties
- Each uncertain parameter was assigned lower and upper bounds to which an adaptive Markov Chain Monte Carlo sampling method was used to generate 5000 posterior probability distributions





Bayesian Model Updating

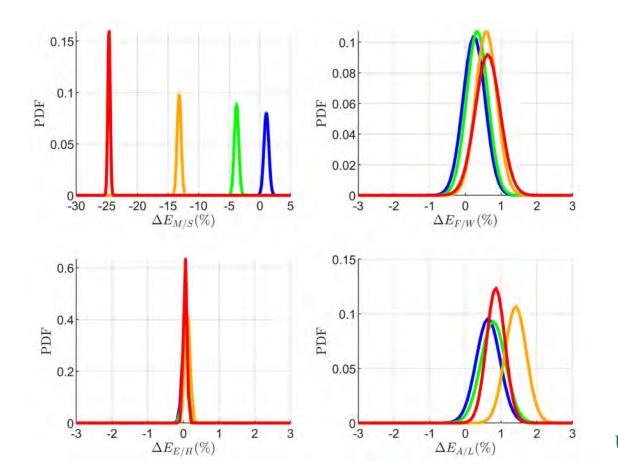
 The discrete distributions of the points samples clearly indicate a successful identification of deterioration in the modulus of elasticity of the M/S elements, with negligible changes identified in the other parameters





Bayesian Model Updating

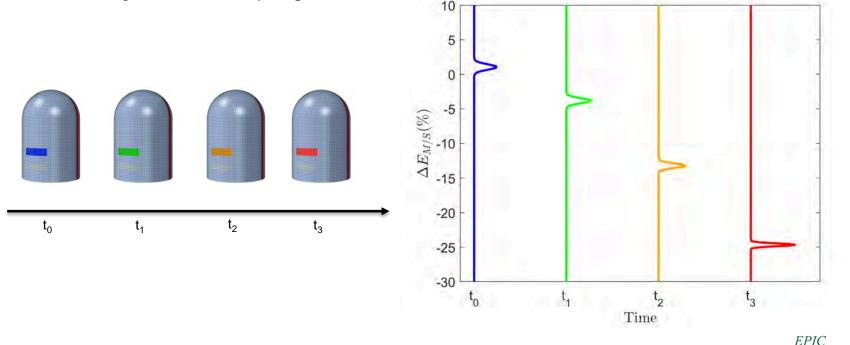
• Posterior probability density functions may be analyzed from which confidence bounds may be placed on the parameter identification





Trusted Model...Now What?

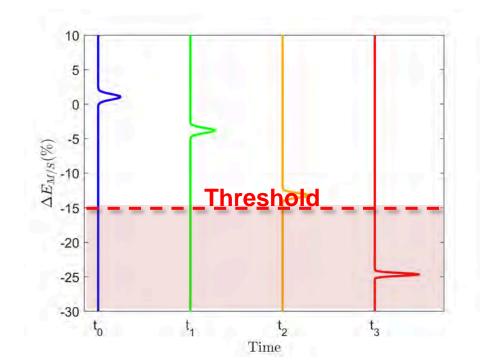
- Once a faithful digital representation of a physical structure has been realized:
 - Performance of the structure may be monitored by using historical and present-day data streams
 - Critical limit states may be evaluated in a digital environment
 - Lifecycle analyses may be performed to inform maintenance outside of routinely scheduled programs



ГТЕ

Trusted Model...Now What?

- Once a faithful digital representation of a physical structure has been realized:
 - Performance of the structure may be monitored by using historical and present-day data streams
 - Critical limit states may be evaluated in a digital environment
 - Lifecycle analyses may be performed to inform maintenance outside of routinely scheduled programs





Challenges

- <u>Physical Asset</u>
 - Development of appropriate performance metrics
 - Deployment of suitable sensor net to capture relevant physical phenomena

- <u>Digital Twin</u>
 - Development of data pipeline to connect physical sensors to digital twin
 - Creation of routines to process and interpret operational data
- Development of end-user application of methodology
- Instruction of end-user knowledgebase



Advantages of Methodology

- May provide near real-time assessment
- Not inhibited by outages as other periodic inspections
- Can incorporate data from periodic inspections
- Capable of identifying hidden/local deterioration
- Identifies potential areas of preventative maintenance





Digital Twins for Prognostic Health Management (PHM) in Nuclear Energy: Opportunities and Challenges

Pradeep Ramuhalli Distinguished Scientist

Virtual Workshop on Digital Twin Applications for Advanced Nuclear Technologies December 3, 2020

ORNL is managed by UT-Battelle, LLC for the US Department of Energy

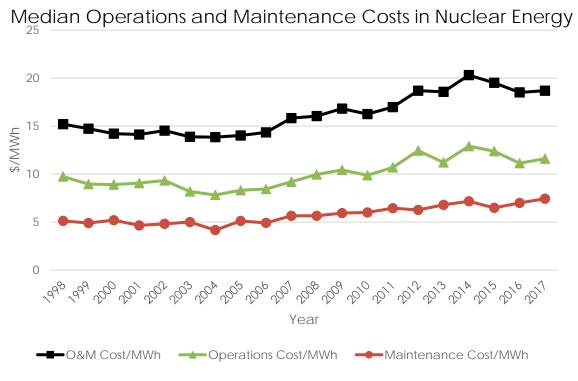


Outline

- Background drivers for prognostics health management in nuclear power
- Diagnostics, prognostics and decision making An integrated solution using intelligent digital twins
- Examples
- Research Needs and Summary

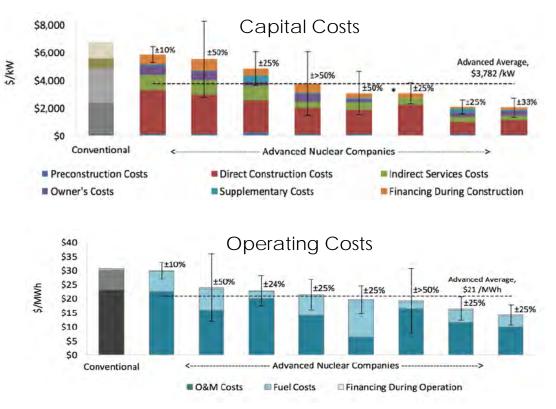


The Big Picture



Data from: "Broken: Costs to Operate, Maintain Electricity Generation Have Soared Over Two Decades" (uptake.com/energy)

Operating Plants



Energy Options Network Report (2019) "What Will Advanced Nuclear Power Plants Cost? A Standardized Cost Analysis of Advanced Nuclear Technologies in Commercial Development"

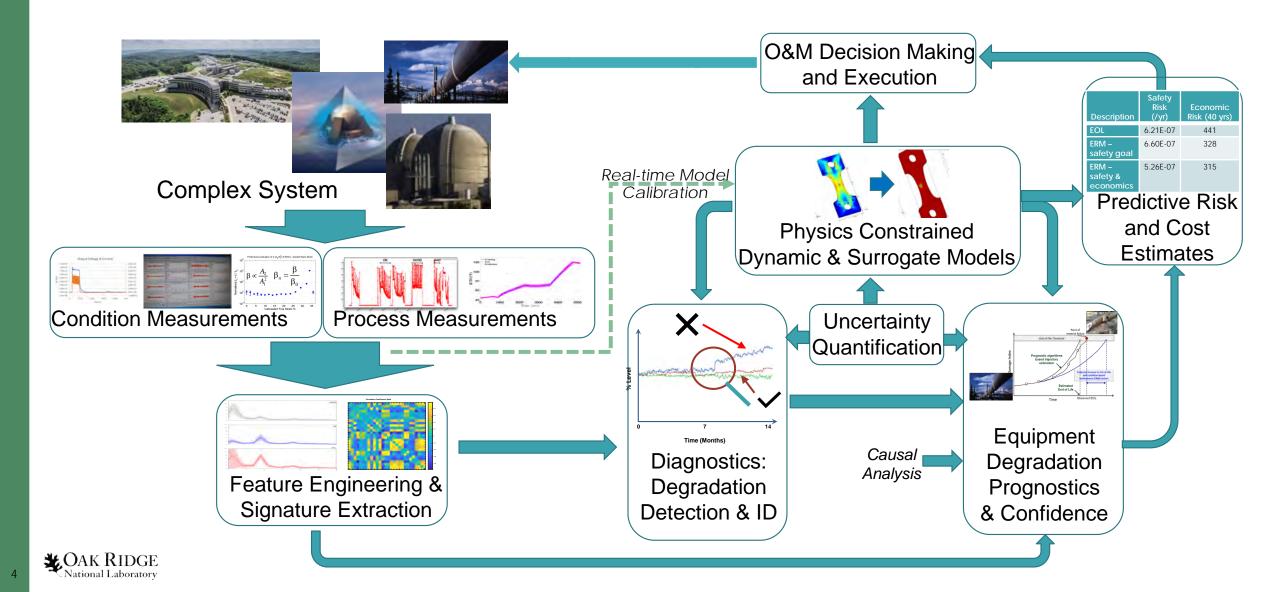
Advanced Reactors

Need: Information-driven Asset Management Technologies and best practices to lower operating and maintenance costs while maintaining safety and reliability

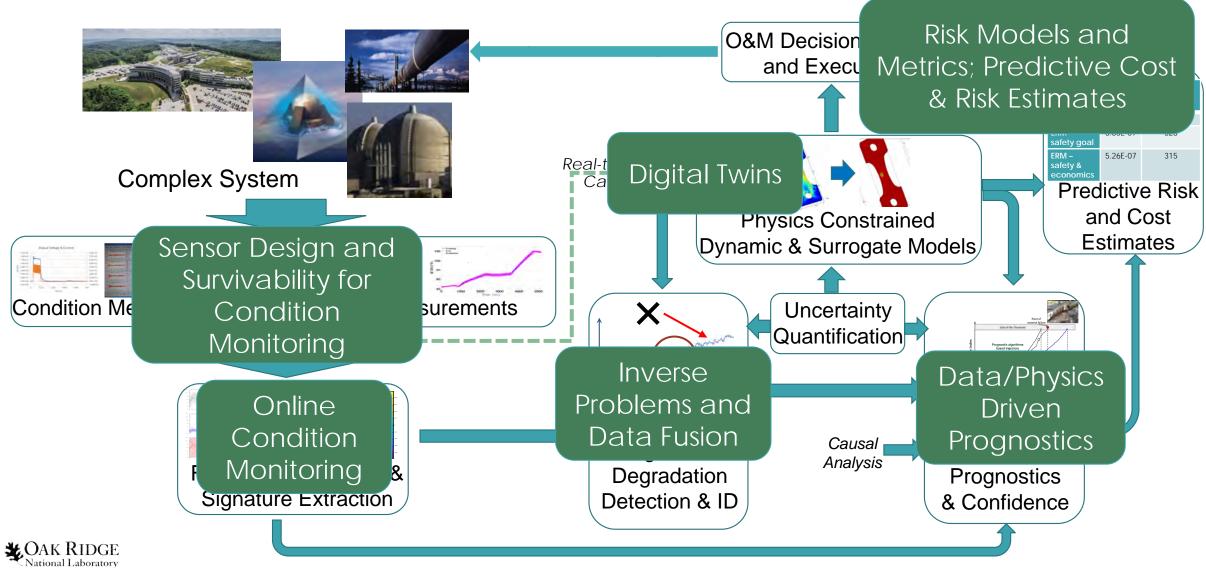


3

Diagnostics and Prognostics Enable Information-Driven Asset Management

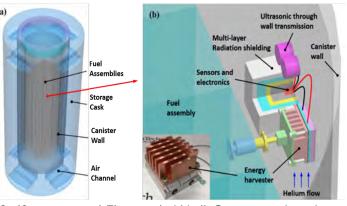


Intelligent Digital Twins Enable PHM

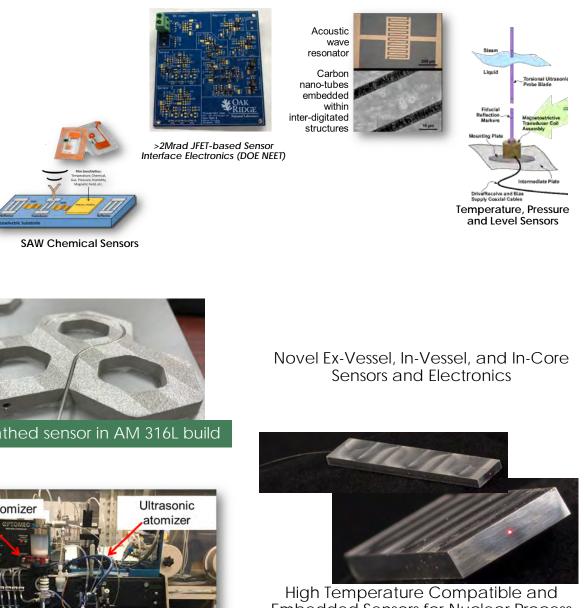


Together with Advances in...

- Sensors and instrumentation
- Modeling and simulation methods and high performance computing
- Data analytics, especially domain-aware data analytics
- Communication technologies
- Advanced manufacturing

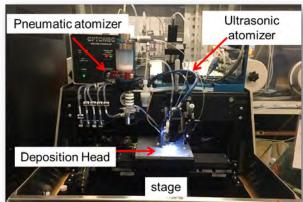


Self-powered Through-Wall Communication **CAK RIDGE** National Laboratory





316L sheathed sensor in AM 316L build

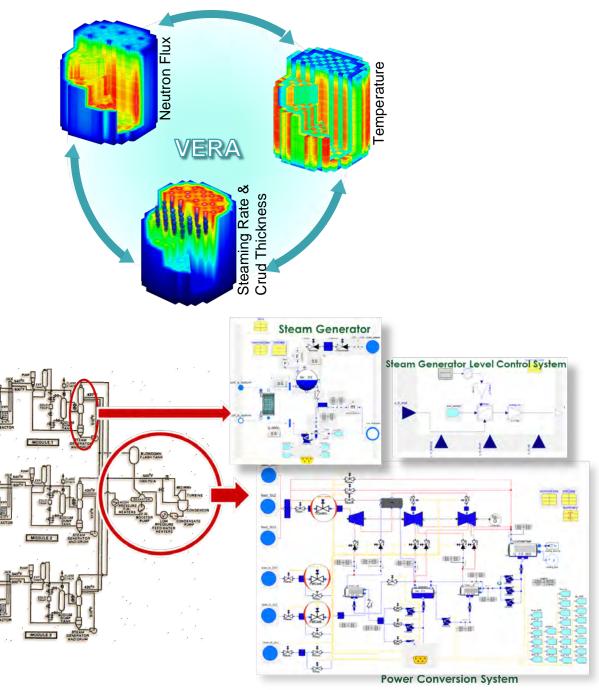


3D Printing Passive Wireless Sensors

Embedded Sensors for Nuclear Process and Component Health Monitoring

Digital Twin

- A software design pattern that represents a physical object with the objective of understanding the asset's state, responding to changes, improving business operations and adding value (Gartner)
- Potential for different levels of fidelity and for different uses, and spanning the range from fully data-driven to physics-based
 - What is "good enough" for the problem?



CAK RIDGE

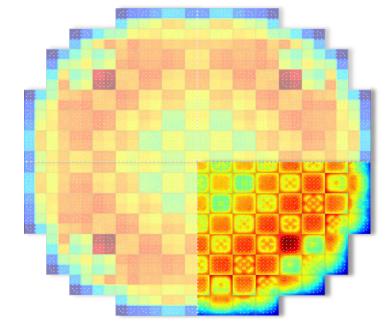
ALMR PRISM Power Block

Intelligent Digital Twins for Diagnostics and Prognostics

- Hybrid (Data-driven, with domain information) can serve as digital twins for diagnostics and prognostics
- Reliability assessment and prediction

- Sensors

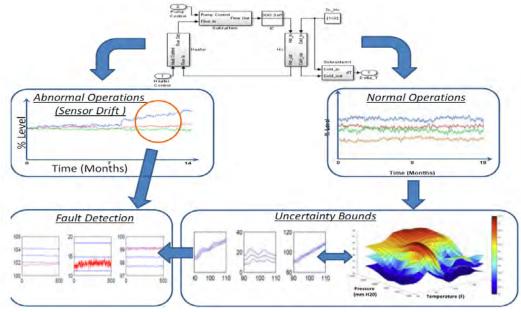
- Active components (pumps, valves, etc.)
- Passive components (piping, vessel, etc.)
- Sub-system (power conversion unit, etc.)
- Risk-informed operational decision making for autonomous operations
- Risk-informed maintenance decision making for cost reduction

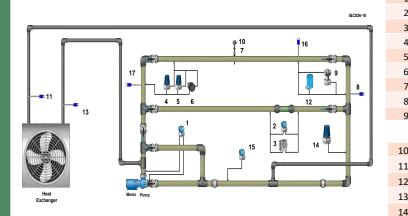


Physics Informed Machine Learning Reduced Order Model



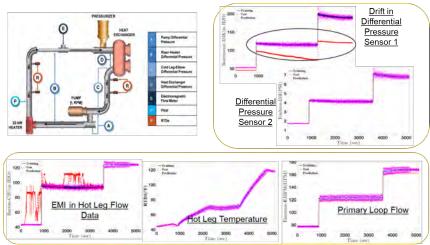
Robust Virtual Sensor Models Can Improve Sensor Drift Detection and Compensation Performance



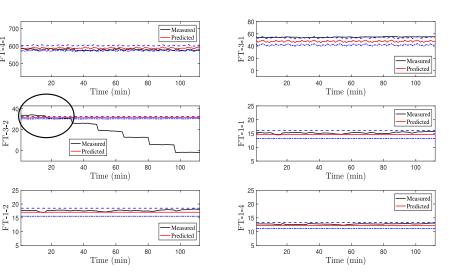


EM	ID	SENSOR TYPE	MANUFACTURER
1	FT-4-1	DIFFERENTIAL PRESSURE	ROSEMOUNT
2	FT-3-1	DIFFERENTIAL PRESSURE (SMART)	ROSEMOUNT
3	FT-3-2	DIFFERENTIAL PRESSURE	BARTON
4	FT-1-1	DIFFERENTIAL PRESSURE	FOXBORO
5	FT-1-2	DIFFERENTIAL PRESSURE	FOXBORO
6	FT-1-4	DIFFERENTIAL PRESSURE (SMART)	BARTON
7	TE-1-2	RTD (SMART)	ROSEMOUNT
8	TC-2-1	THERMOCOUPLE TYPE-J (SMART)	ROSEMOUNT
9	FT-2-1	DIFFERENTIAL PRESSURE	SCHLUMBERGER

10	CTRL-TEMP	RTD (SMART)	ROSEMOUNT
11	TC-HX-OUT	THERMOCOUPLE TYPE-J	OMEGA
12	FT-2-3	DIFFERENTIAL PRESSURE	HONEYWELL
13	TC-HX-IN	THERMOCOUPLE TYPE-J	OMEGA
14	CTRL-PSR	GAUGE PRESSURE	FOXBORO
15	PT-2	GAUGE PRESSURE	ROSEMOUNT
16	TC-LOOP-FAR	THERMOCOUPLE TYPE-E	OMEGA
17	TC-PUMP-OUT	THERMOCOUPLE TYPE-K	OMEGA



Example of Sensor Calibration Drift Detection and Compensation

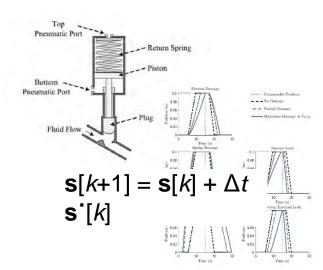


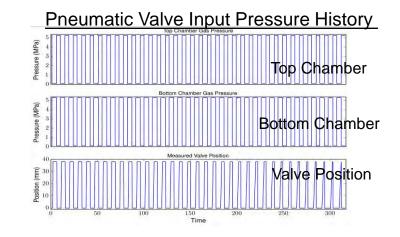
Tipireddy, Ramuhalli et al, ANS NPIC-HMIT 2017

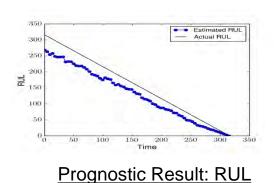
CAK RIDGE

9

Data-driven, Physics-Inspired Models for Diagnostics and Predictive Maintenance





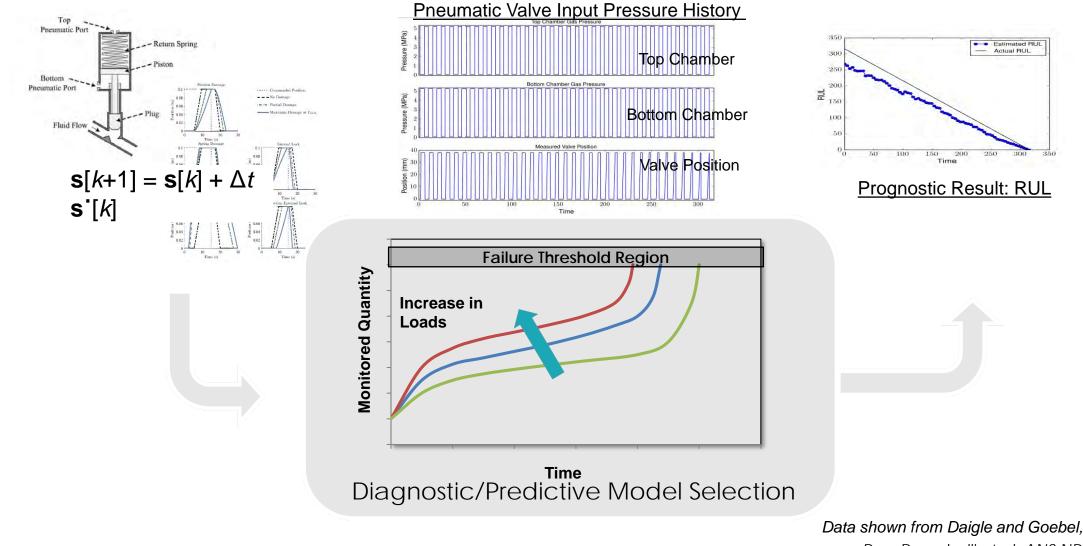




10

Data shown from Daigle and Goebel, IJPHM, 2008 Roy, Ramuhalli et al, ANS NPIC-HMIT 2015 Dib, Roy, et al, ANS NPIC-HMIT 2017

Data-driven, Physics-Inspired Models for Diagnostics and Predictive Maintenance

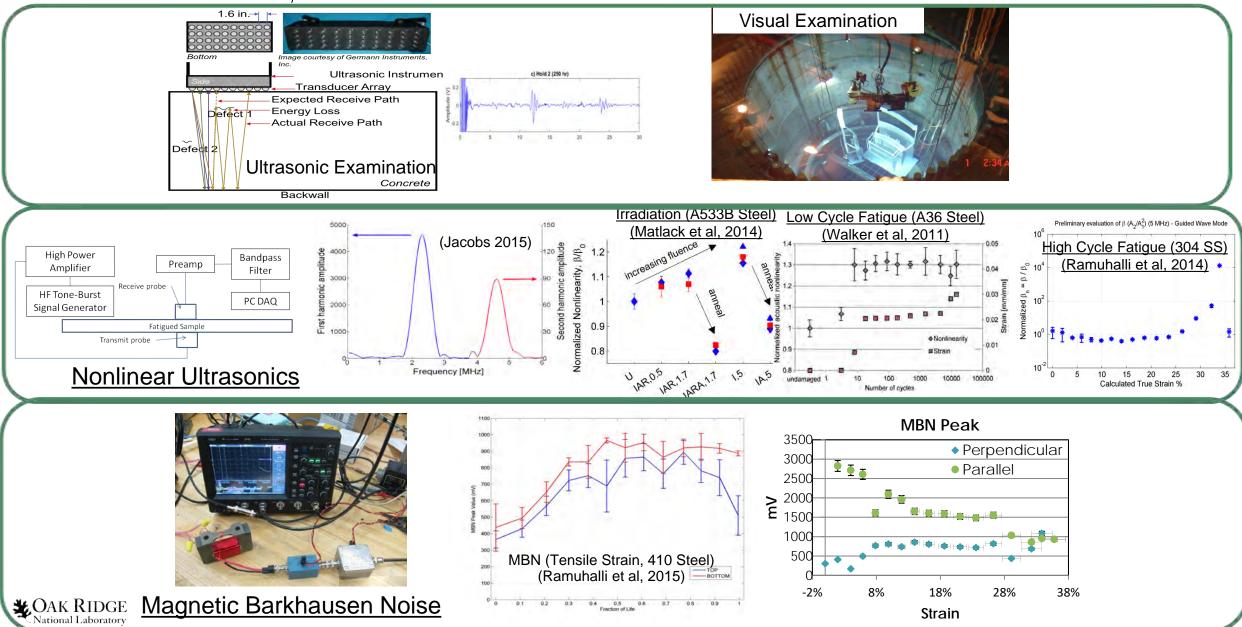


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11

Data shown from Daigle and Goebel, IJPHM, 2008 Roy, Ramuhalli et al, ANS NPIC-HMIT 2015 Dib, Roy, et al, ANS NPIC-HMIT 2017

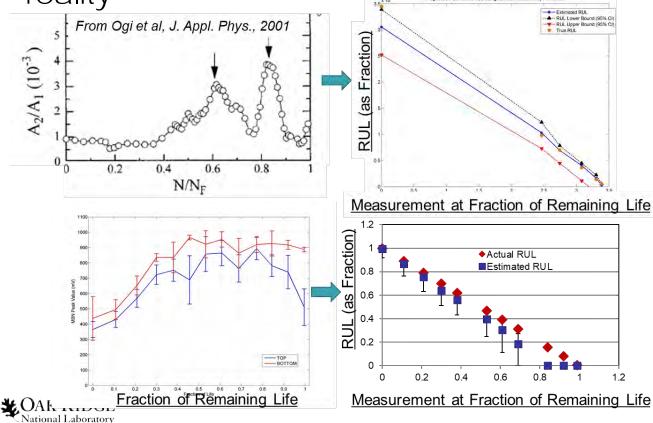
Complex Multi-scale Physics of Failure Models Challenge PHM for Materials Failure; Data-driven Models Show Promise

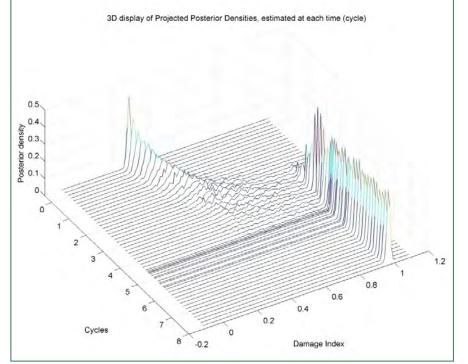


12

Bayesian Methods Allow Integration of Failure Physics Information, Condition Data, and enable Uncertainty Quantification

- Underlying models can be at desired level of fidelity
- Prediction updates with new measurements
- Model updates over time also possible to reflect reality

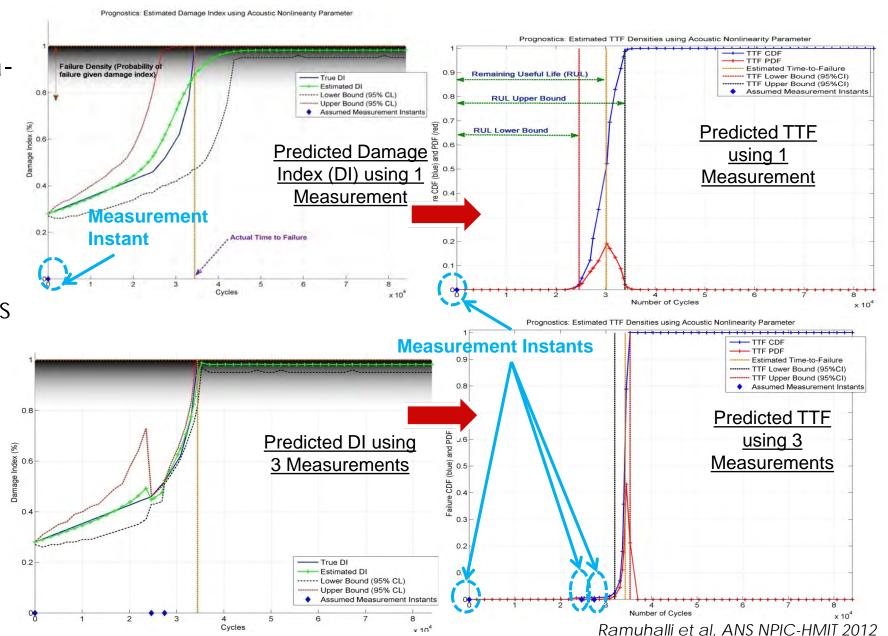




Evolution of Posterior Probability Density with Time

Example: Predicted Time-to-Failure (TTF) for Fatigue Crack Initiation

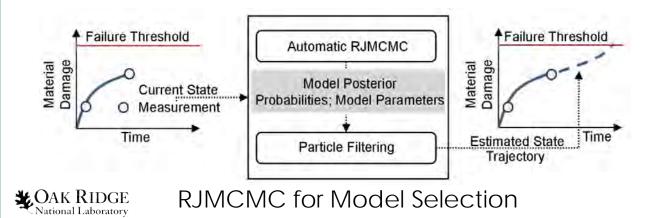
- Diagnostics and prognostics using datadriven models of
 - Damage growth
 - Measurement
- Necessary data may be difficult to acquire
- Physics-inspired models (damage growth and measurement) have been used in other instances with good accuracy



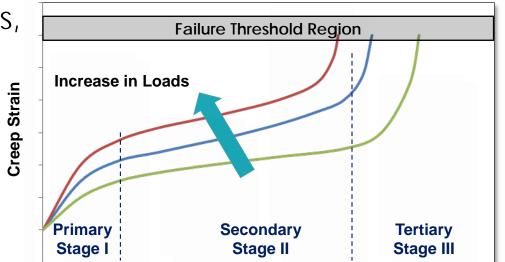


Digital Twin Model Updates are Essential for Many Applications

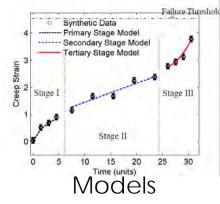
- Limited examples for certain fault conditions, and limited data
- Operational conditions may vary over time
- System or component condition may vary, grading different models
- New failure modes with longer term
 operation
- Continuous learning, with model selection, will be necessary

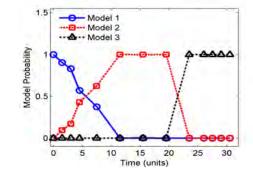


15



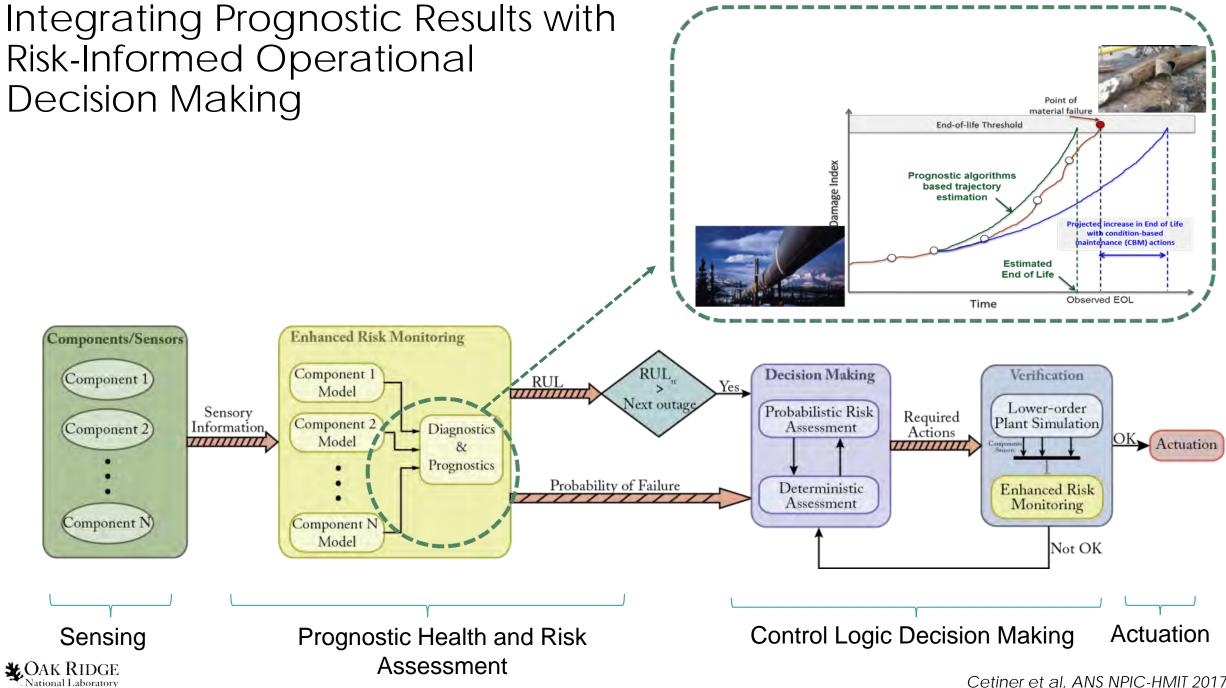
Degradation Growth Characteristics – Function of Time and Load





Model Likelihood

Roy, Ramuhalli et al, ANS NPIC-HMIT 2015



16

Cetiner et al, ANS NPIC-HMIT 2017

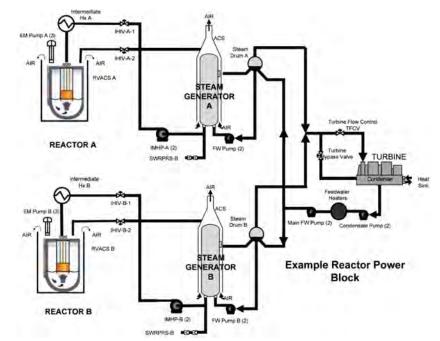
Integrating PHM and Risk Monitors with Plant Control Logic

- Risk: Measure of probability of some undesirable consequence
 - Core damage frequency, large early release frequency, health consequences to the public

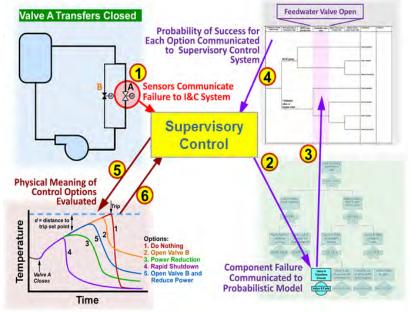
EE	SGL	RVACS	SEQUENCE	SEQ. FROB	SEQ NAM
External Event - Plug/Failure of RVAC	Steam Generator Louver	Reactor Vessel Auxiliary Cooling	OUTCOM		
Œ			no CD	0.00E+00	
	SGLV		no CD	0.00E+00	
		OP-RVACS	CD	0.00E+00	10
Sin	nplified	Reacto	r PRA	Even	Tree

- Elements of PHM integration with risk monitors
 - Equipment condition assessment (ECA) and prognostics for predictive health assessment
 - Predictive risk assessment (safety and economic)
 - Uncertainty quantification

CAK RIDGE



Simplified Diagram of Multimodule Reactor



Risk-informed Decisions: Economics and Safety

- Methodology for using cost metrics for component replacement scheduling
- Hypothetical cost and failure rates used in analysis
- Assessment computes safety related risk metric (CDF) and normalized cost over 40 years for three cases
 - <u>Case A</u>: Run to end-of-service-life; replace during scheduled outage.
 - <u>Case B</u>: Use diagnostics/prognostics; replace equipment just prior to plant exceeding safety limit.
 - <u>Case C</u>: Use diagnostics/prognostics; replace equipment if risk of unplanned outage at a future time. Schedule based on optimizing cost metric.

า	Case #	Description	Expected CDF (/yr)	Reduction in Economic Risk Over 40 yrs (Relative to Case A)
)	A	Expected end- of-life replacement	6.21E-07	-
	В	ERM – safety goal based maintenance	6.60E-07	25.6%
	С	ERM – safety and economics based maintenance	5.26E-07	28.6%



Summary

- Digital twin solutions for intelligent asset management and autonomous operations
 - Enabled by technology advances in sensing, data analysis, modeling and simulation, and machine learning
- Technical challenges still exist and are targets for ongoing research
 - Research leveraging advances in machine learning
- Resulting technologies enable sustainable nuclear power by improving the reliability and economics of nuclear plants



19

Looking Forward: Some Challenges

Data

- Data access and data quality
- Optimal sensor type and placement
- Testbeds for data generation and verification and validation (V&V)
- Technology Development
 - Robust digital twin development
 - Model selection and model updates
 - Robust diagnostics and prognostics in the presence of concurrent mechanisms, influencing factors, interacting subsystems, and measurement drift
 - Methods for semi-autonomous decision making
- Deployment
 - V&V approaches
 - Uncertainty quantification
 - Cybersecurity



Acknowledgments

- A number of collaborators have contributed to the work presented here, and include staff from National Laboratories (ORNL, PNNL, ANL, Bettis, INL), Universities (UT-Knoxville, PSU, WSU, ISU, CSU-LB, WUSTL, Ajou University), and Industry (AMS Corp.)
- A portion of the research presented here was supported by the USDOE Office of Nuclear Energy through the Advanced Reactor Technologies (ART), Nuclear Energy Enabling Technologies (NEET), and the National Scientific User Facility (ATR-NSUF) programs. A portion of the research was supported by the NNSA Office of Defense Nuclear Nonproliferation (NA22). Parts of this work were supported by Ajou University (S. Korea).
- Oak Ridge National Laboratory is operated by UT-Battelle for the US Department of Energy.



21

Questions?



22



Managing Regulated Change: An Enterprise-Level Digital Twin for the Nuclear Industry

Michael Mazzola, Robert Cox, Jeffrey Hawkins Energy Production and Infrastructure Center (EPIC) UNC Charlotte, NC, USA mmazzola@uncc.edu EPIC.UNCC.EDU



Outline

- Introduction to EPIC
- Consensus on Nuclear Energy has Changed
- Construction Best Practice has Yet to Change (Enough)
- Translating Enterprise Digital Twin Culture to Construction
- ► Leveraging the Single Source of Truth for JIT Regulation



EPIC's Mission

- Education for Engineers in Energy
- Research and Development
- Economic Development

www.epic.uncc.edu



Albert and Freeman Energy Production and Infrastructure Center



Practical Nuclear Experience through Partnerships with SMR Developers

- EPIC expertise is being applied to inspection techniques and construction sequencing for SMR's
- EPIC is supporting the development of a construction-related LTR for the NRC
- Digital Twin Pilot Projects include
 - Structural health monitoring (Dr. Tim Kernicky presented on Thursday)
 - Adapting Enterprise DT's from Advanced Manufacturing to Construction (partnership with Siemens)
- Single Source of Truth" for structural and geotechnical models during construction (partnership with EPRI and Purdue)
- EPIC nuclear industry advisor is Mr. Jeff Hawkins, retired Vice President – Project Director Fluor Nuclear Power



Consensus on Nuclear Energy has Changed



The message is we need new nuclear...



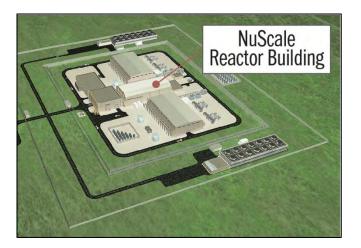
A carbon-free future is a nuclear future by Rep Michael Burgess & Bud Albright

"Contrary to what Hollywood might have you believe, nuclear power is one of the safest and most reliable sources of energy in the world, producing approximately 20% of our nation's electrical power, and more than half of our nation's carbon-free energy."

- The environmental emergency of the second decade is climate change.
- Zero emission sources that work 24 hours a day are important again.



If it is affordable and in time...



Is next generation nuclear technology destined to serve Utah? by Amy Joi O'Donoghue, DeseretNews, Nov. 11, 2020

"UAMPS spokesman LaVarr Webb said the power association will not move forward with the project unless costs per megawatt hour remains at \$55 or lower and the current timeline for licensing and permitting is preserved."

EPIC

• Acceptance for new nuclear energy depends on cost and schedule.

"More than 50% of costs are civil works."

Tim Schmitt, Engineering Supervisor for Civil Analysis, and Carl Fisher, VP for Products and Engineering, Framatome, meeting at EPIC, Nov. 1, 2018

Construction Best Practice has Yet to Change "Enough"



What do experienced nuclear construction professionals say?

January 28, 2019 Workshop on Large Energy Plant Construction hosted by EPIC. Industry contributors: EPRI, Framatome, Atkins SNC Lavalin and Duke Energy

- <u>Planning</u>, scheduling, and sourcing optimized by and connected to the design.
- Fully 3D digital representation of complete design that remains <u>trusted</u>.
- Avoid "over the wall" design strategy artificial separation between designer and construction—<u>Reduce</u> the cascade of ECO's!
- Consider how the regulator will interpret as-built construction—Is it to <u>license</u> or not?

What does the Construction Industry Institute say?

CII analyzed the performance of 975 light and heavy industrial projects.*

- Only 5.4% met "best in class" predictability in cost and schedule.
- Owners and contractors constructing large capital projects have resisted full-scale adoption of integrated digital tools and platforms to drive project performance.
- Nuclear plant construction is the most expensive example.

*www.pwc.com/us/en/industries/capital-projects-infrastructure/library/digital-twin-platform-capital-projects.html





NEI 20-08

Strategic Project Management Lessons Learned & Best Practices for New Nuclear Power Construction

Prepared by the Nuclear Energy Institute April 2020 Rev 0

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Translating Enterprise Digital Twin Culture to Construction



Aerospace and Nuclear - Both regulated for safety and economics

Commercial Aerospace Regulatory Hierarchy

- Accountable entity for the Type Design Certificate: Aircraft manufacturer (e.g., Boeing)
- Safety Regulator: Civil Aviation Authority (e.g., Federal Aviation Administration)

Economic Regulator: Owner/operator (e.g., Airlines) Nuclear Energy Regulatory Hierarchy

- Accountable entity for the License: Owner/operator (e.g., Utility)¹
- Safety Regulator: Nuclear Safety Authority (e.g., Nuclear Regulatory Commission)
- Economic Regulator: Rate Setting Authority (e.g., Public Utility Commissions)
- In the Commercial Aerospace Industry, airliner OEM leads the integrated product team for the entire life cycle of the product (airliner).
- In the Nuclear Energy Industry, the owner of the plant should lead the integrated project team during the construction project¹ (planning, construction, handoff) and thence for the life cycle of the plant (operation, maintenance, and decommissioning).

¹NEI Technical Report 20-08 Strategic Project Management Lessons Learned & Best Practices for New Nuclear Power Construction, pp. 15-16.



Build the airplane, not the airport.

Characterized by:

- All digital design
- Automated component production by supply chain
- Repeatable component dimensions
- Reliable assembly by OEM
- Cost competitive
- High production rates
- Achieved in a regulated environment designed for safety



Characterized by:

- Single design (no two airports alike)
- Local fabrication
- No complete digital design
- Diminishing supply chain



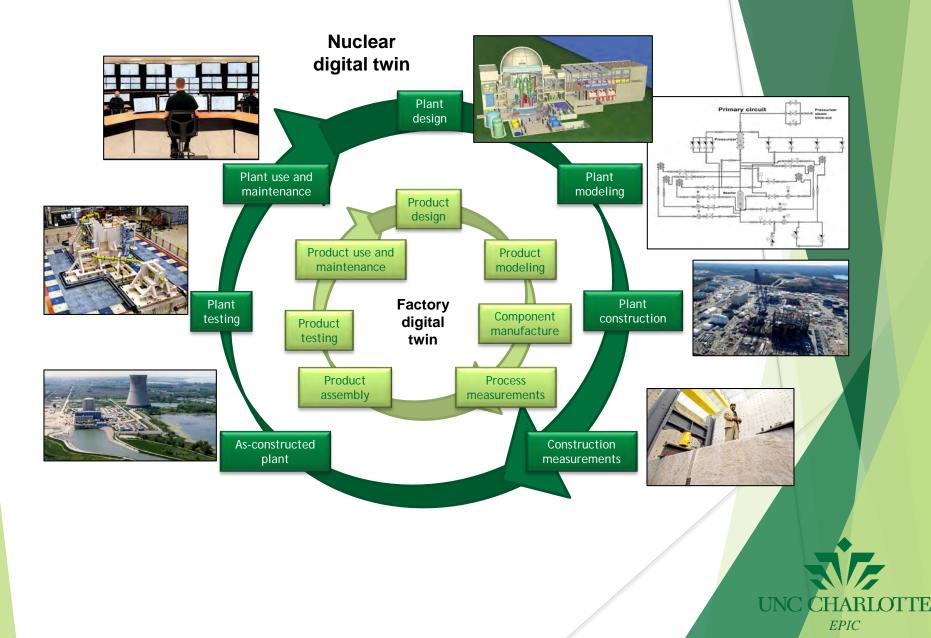
CHARLO EPIC Methods to improve the work culture to one of "finish on-time and within budget"

Digital Twin for New Nuclear Construction

- Includes enterprise level program management support throughout construction.
 - A programmatic approach to cost saving with clear leadership, responsibility, and organization for project management.
 - Information technology and data analytics leveraged to simplify workflow, to make shared information current and consistent, and to automate administrative tasks.
 - IT based actionable spend analytics for improved decision-making strategies.
 - IT based actionable data analytics for improved procurement.
- Trusted model of what is built or modified for the life of the project.
- Design for construction and assembly. (Similar to DFMA)
- Incorporate details about fabrication processes so components can be produced by any vendor in the supply chain.



Analogy to Advanced Manufacturing



Leveraging the Single Source of Truth for Just In Time Regulation



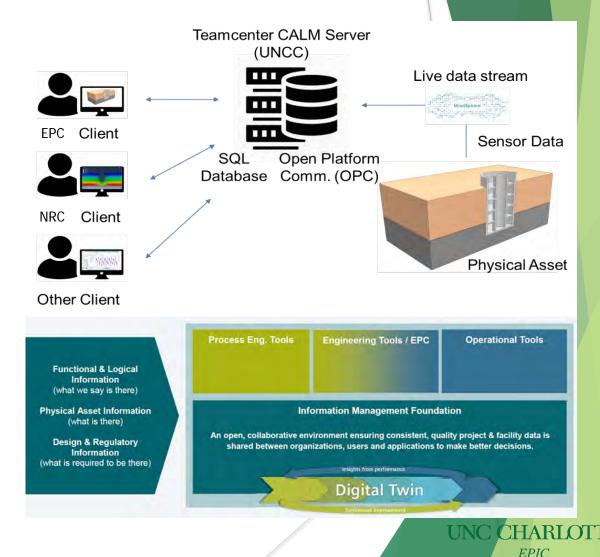
Part 52 Current Rules

- Part 52 licensees may proceed with construction departing from licensing basis only after:
- 1) the licensee, with collaboration from the NRC, determines that a License Amendment Request (LAR) is not required; or
- 2) the licensee submits a LAR and the NRC reviews and approves it; or
- 3) the licensee receives Preliminary Amendment Request (PAR) "no objection" letter from NRC.
- The PAR "no objection" letter is provided only after the associated LAR is developed, submitted and accepted for review by the NRC.



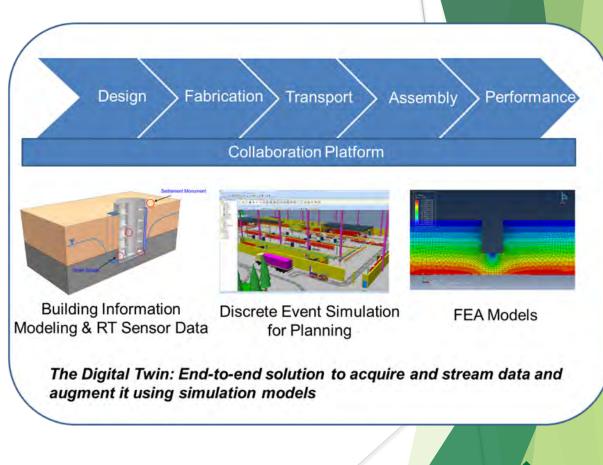
Project Management and Regulator Have Access to the Same Single Source of Truth

- Interfaces to different clients and project, technical, or regulatory software systems available in Teamcenter.
- Consideration of phases of the project and how the Part 52 restrictions should be applied.
- Projects could be allowed to advance specified phases prior to the final COL being issued.
- The phased approach allows parallel project execution to occur.
- Risk should be well defined and understood by management and regulator.



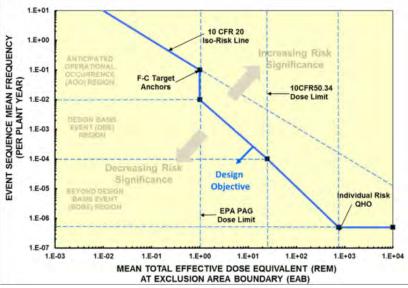
Maintaining Single Source of Truth during Construction - Example

- Structures constructed using innovative technologies
- Structures tracked through design, fabrication, transport, assembly, and in-service life
- E.g., Finite-element structural and geotechnical models can be integrated application domains



EPIC







December 4, 2020

Mike Calley Department Manager, Regulatory Support



Including Risk in Digital Twins



Nuclear Safety and Regulatory Research Division

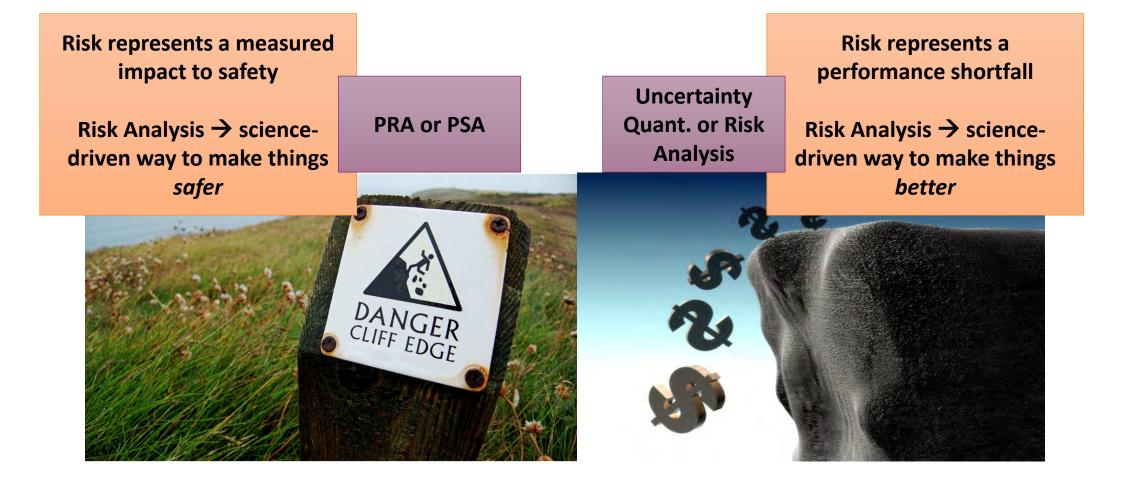
Let's start with the punchlines

- Many next generation reactors will use digital twin technology (DT) for design and operation
- "Risk" in terms of performance shortfalls is a powerful way to characterize and understand complex systems
- Risk, in terms of a "public health" frequency-consequence idea, is a key part of the next-gen risk-informed approach (e.g., in NRC's SECY-19-0117)
- For completeness in design and operation, we must consider uncertainties
- When we put these together, we can realize a major efficiency if we design, operate, and license advanced reactors using a digital twin approach that includes a risk element

Outline

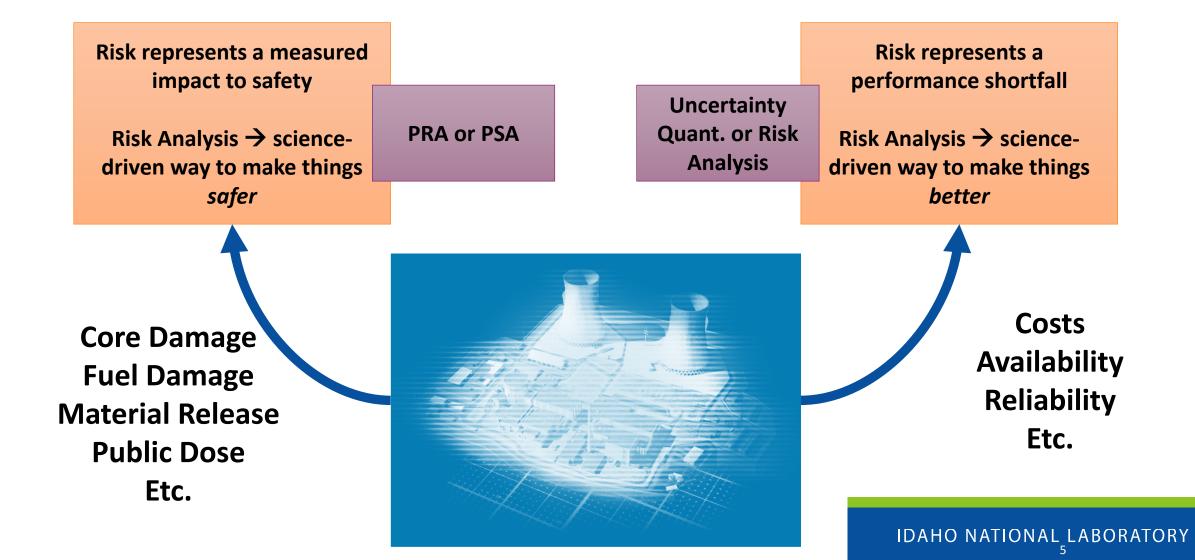
- Risk
 - What does it mean to use the word "risk" in the context of a DT?
- Context
 - Why is context important for operation of a reactor?
- Framework
 - How would we include risk when using a DT?

"Risk" tends to be used to describe one of two contexts

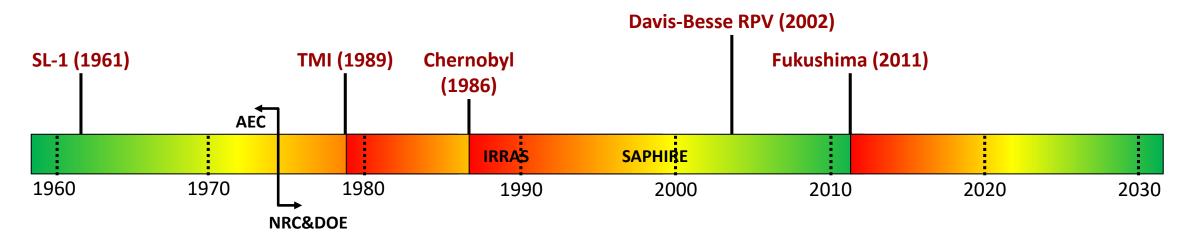


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For a DT, risk can be addressed for different metrics



Context is important to understand off-normal events



Positive

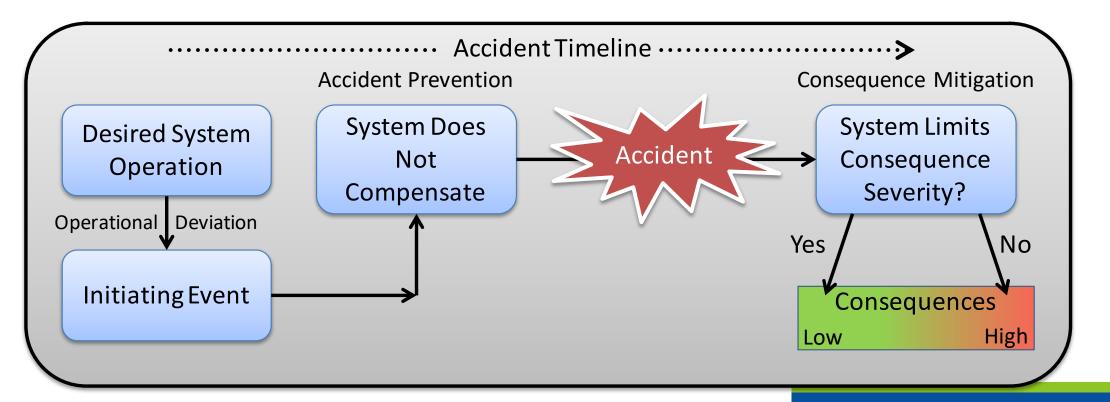
Negative

Nuclear Power Outlook

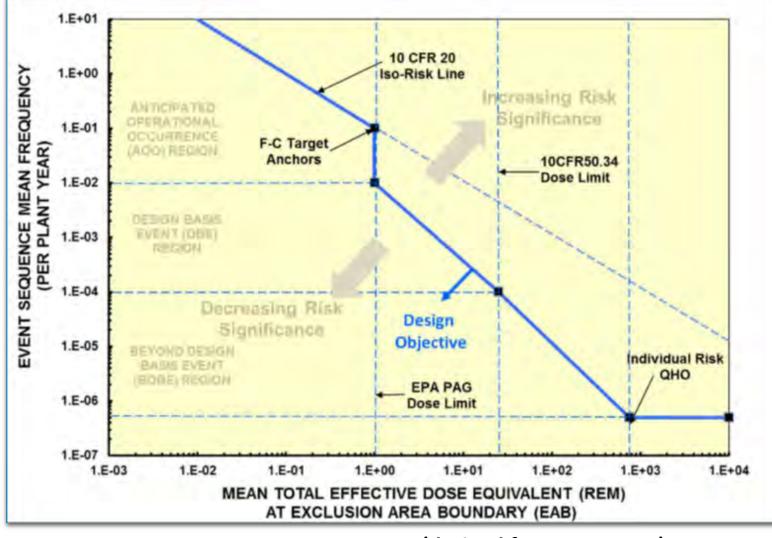
- Understanding what is going on with, and around, a nuclear reactor is vital to capturing possible off-normal scenarios
 - Off-normal events can be triggered by human actions, plant degradations, external events, or …
- Having a model of the reactor physics (e.g., thermal-hydraulics, neutronics, materials) only captures part of the scenario story

A scenario depicts off-normal behavior

- Context for a facility includes and understanding of possible hazards
- For hazards that may impact a DT, context sets the scenario
 - Scenario = initiating event + enabling conditions + undesired events/actions

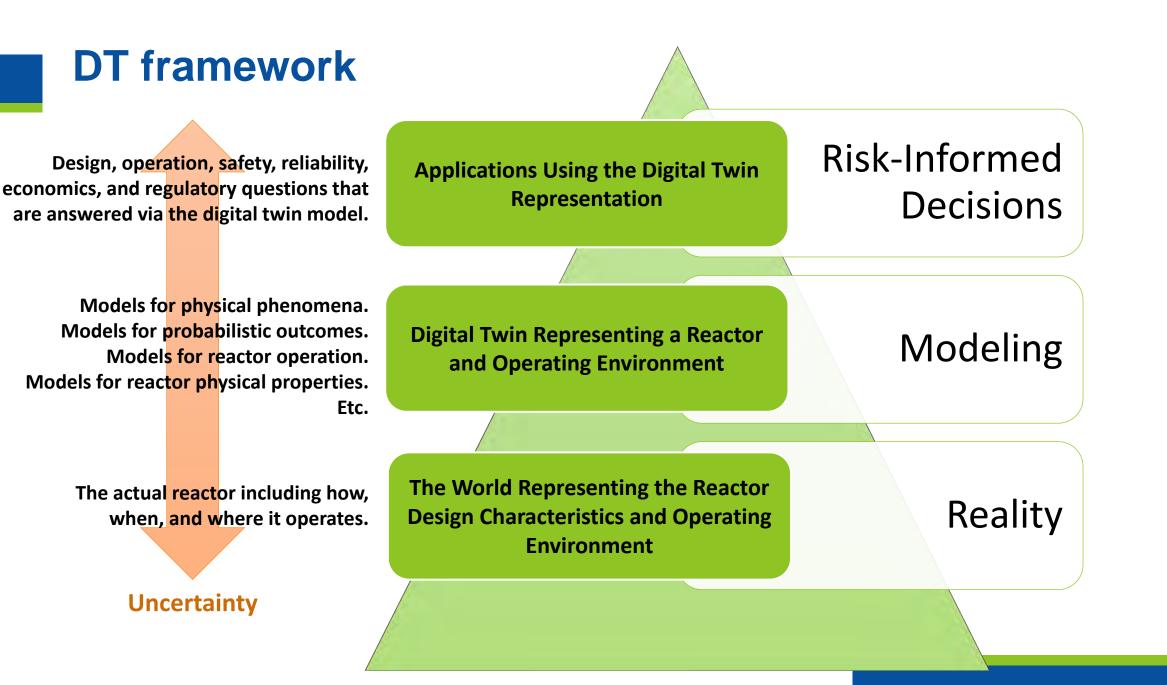


From risk concepts & models comes risk-informed decisions



Frequency-consequence target (derived from NEI 18-04)

IDAHO NATIONAL LABORATOR

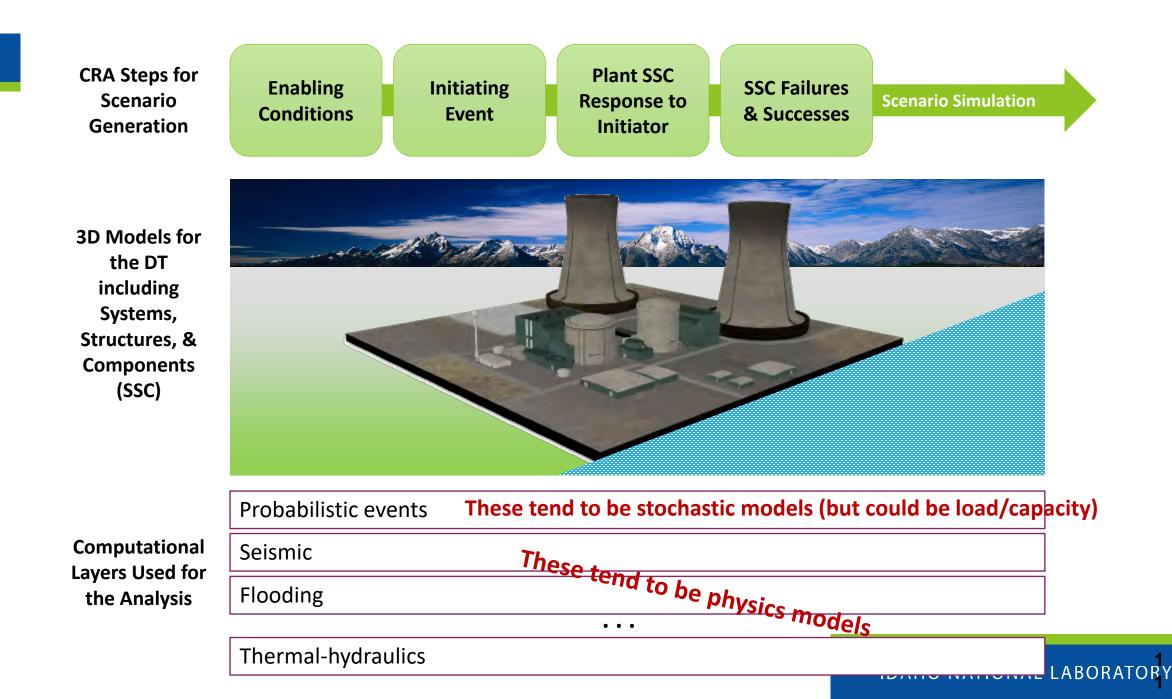


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DT modeling, including risk, implies usage of computational risk assessment

- Computational Risk Assessment integrates risk and physics models
- CRA is a combination of
 - Probabilistic scenario creation where scenarios unfold (in the computer) and are not defined a priori
 - Mechanistic analysis representing physics of the unfolding scenarios
- CRA is not simply solving traditional PRA models faster or with higher precision
 - It is a different way of approaching a safety analysis or a performance-shortfall evaluation

Integrating the worlds of physics and probability leads us to predictions based upon an approach called "computational risk assessment"



In summary

• DT for design and operation

- Helps us understand the facility, and how it will operate, before and during actual operation
- Performance shortfall to characterize and understand complex systems
 - Helps us to focus on the strengths and weakness of these systems
- Public health risk is a part of the next-gen licensing
 - Helps us license advanced reactors in an efficient manner
- We must consider uncertainties
 - Helps us characterize our knowledge about the operation- and safety-cases
- These points imply we should include risk for our advanced reactor DTs







Defense Nuclear Nonproliferation Research & Development

Towards a Digital Twin to Detect Nuclear Proliferation Activities

NA-22 Office of Data Science

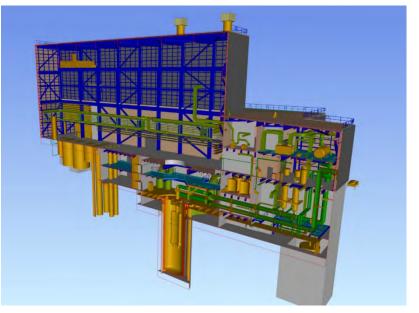
Christopher Ritter (PI), Sam Bays, Eric Bohney, Ross Hays, John Koudelka, Ross Kunz, Gustavo Reyes, Mark Schanfein



Current Limits of Practice



- State of the Art:
 - Safeguards analysis is typically
 SME based without models
 - When models exist, they are disconnected, have no AI/ML integration, and no digital twin capabilities
- Problem: Development of new advanced reactors (Gen IV) increases importance of new methods to understand diversion and misuse scenarios and determine mitigation pathways



Versatile Test Reactor: Gen IV Sodium Cooled Fast Reactor (scheduled for operation in 2026)





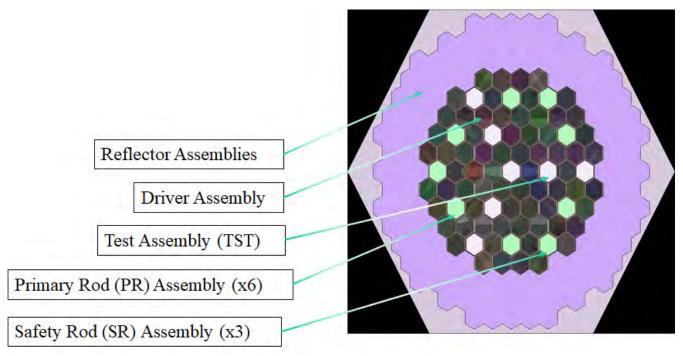
- **Diversion**: The special fissionable nuclear material (Pu239, U233, U enriched in U233/235) that has been declared to the International Atomic Energy Agency is removed surreptitiously either by taking small amounts of nuclear material over a long time (known as protracted diversion) or large amounts in a short time (known as abrupt diversion)
- **Misuse**: The undeclared source material (material that can be transmuted into special fissionable nuclear material: depleted uranium, natural uranium, and thorium) is placed in the core uses the neutron flux for the transmutation



Nuclear Physics Overview



- Thermal Output: 300 MWth
- Cycle Length: 400 days
- Outage Length: 20 days
- Three batches of fuel in the core
 - (Fresh, 1st burned, 2nd burned)







- **Target**: Obtain 1 Significant Quantity (SQ) of Plutonium (Pu = 8 kg) for a clandestine weapons program, ideally in 1 year
- <u>Diversion</u> diverting 1, 2, 4, 8, or 12 fuel pins per 217 pin *declared* assembly and substituting with either lead (Pb), stainless steel (SS), or natural uranium pins. Thereby immediately obtaining the fissionable Pu intended for the fresh fuel.
- <u>Misuse</u> placement of a whole *undeclared* assembly(s) (referred to as a target) of fertile natural uranium in an experiment test location(s) within the reactor. Thereby transmuting the NU to Pu over time.



Key Technologies



DIAMOND Ontology

- Project Objective(s): This ontology allows for a generic, common framework to enable digital engineering programs. Like previous successful Idaho National Labs initiatives (ex. MOOSE), this data ontology will allow for a common framework to be shared, allowing for more complex energy projects to be undertaken and utilize a plug and play model.
- Technical Challenges: (1) Ontological compatibility with other domain ontologies: Mitigated through BFO use (2) Right sized ontology development to ensure the ontology is deep enough to be useful but flexible enough to support multiple designs (3) Verification of the ontology to ensure that functional specifications are executable; this is mitigated by the use of the Monterey Phoenix event trace system
- Approach: (1) analysis and selection of top level meta models (BFO/LML) (2) development of lower ontological decompositions for nuclear design using subject matter input to create an easily extendable ontology framework (3) validation and verification of the DIAMOND ontology for nuclear reactor behavior models using Monterey Phoenix (MP)



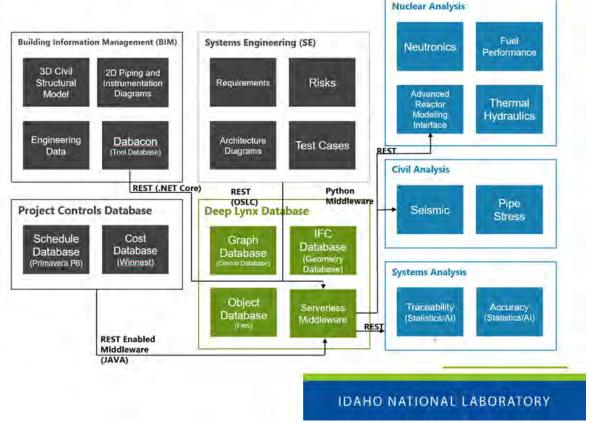


Key Technologies



Deep Lynx Datawarehouse

- Ontology: Utilizes ontology for a standardized, common data model to enable a generic framework independent of tool/solution
- Central Software Framework: This allows for a common software framework to be shared, allowing for code reuse and minimal point-to-point integrations
- Central Datastore: This is utilizing the Microsoft Azure Postgres Hyperscale Database which allows a balance between scalability and historical stability







- Model Input:
 - List of selected parameters (fuel design, location, operations, etc.)
 - SERPENT input generating scripts (utilize above parameters)
 - SERPENT version, installation, runtime data
 - Output data extraction script

• Model Output:

- SERPENT data is passed through extraction script to yield selected parameters
- Parameters stored in HDF5 archive, passed to DeepLynx.
- DeepLynx extracts archive, converts to DIAMOND type





- Current Status
 - Defining and converting input parameters within generation scripts
 - Defining output parameters of current and future interest, creating DIAMOND type classes and relationships
 - Establishing automated linkages, authentication methods between cloud data host, local SERPENT installation

• Future Work

- Expand DIAMOND type mappings
- Automate creation, execution of SERPENT inputs
- Automate extraction, ingestion of data to DeepLynx.





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• INL HPC (High Performance Computing) hosting:

- Serpent: burnup calculations
- Serpent output adapter and HDF5 to JSON converter (python) for ingestion to Deep Lynx



Deep Lynx data warehouse

- NodeJS
- PostgreSQL
- DIAMOND data model

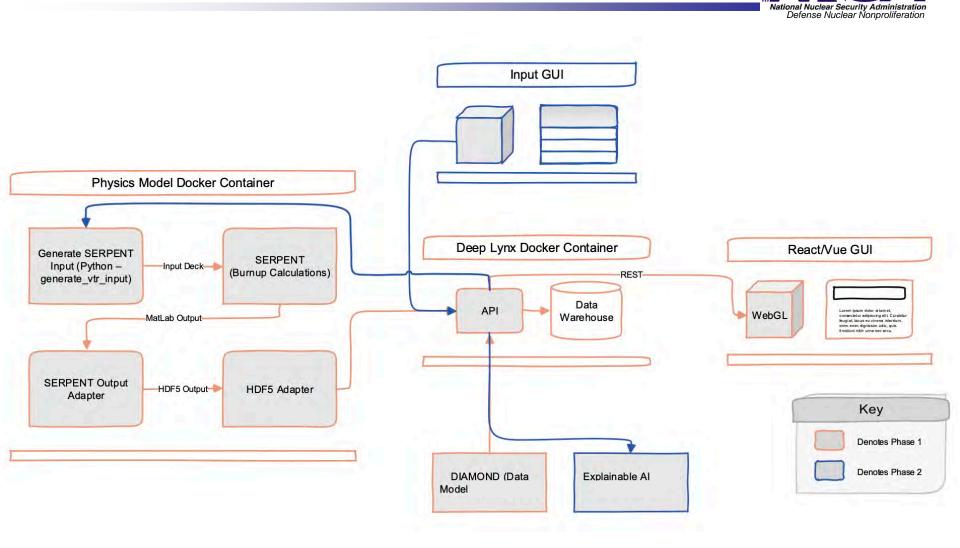


Input and Output GUIs

- Input²: Visual selection to create input to Serpent
- Output: WebGL app that provides 3D model of reactor



Digital Twin Data Pipeline



Questions

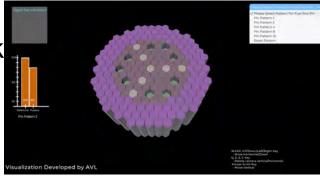
Office of Defense Nuclear Nonproliferation R&D

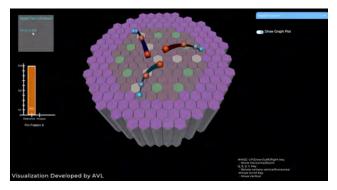


Digital Twin Visualization Technology



- Web-based, Virtual Reality environment
 - Reactor model developed in Autodesk Inventor and 3DS Max
 - WebGL 3D environment
 - Visual Analytics
 - Integration of 2D & 3D
 - RESTful API calls for data
 - Scalable (desktop, laptop, tablet)
 - Dynamic Interaction
- Data Connection
 - Calls to Deep Lynx for data
 - Digital Twin is controlled by the result





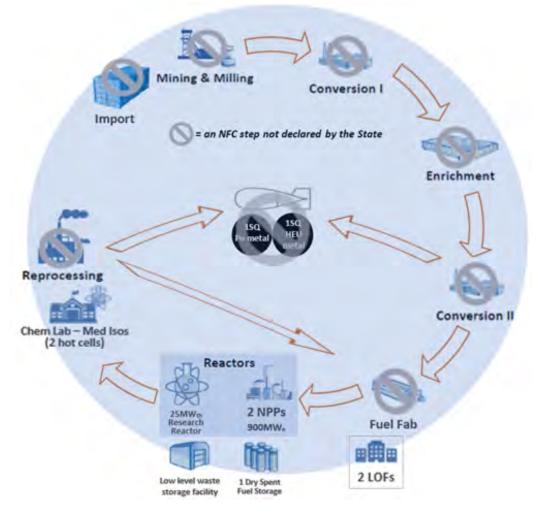
Questions



Realizing DT's Full Potential



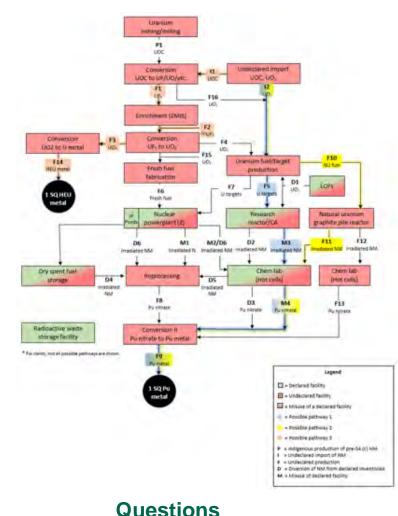
- Diversion & Misuse at <u>declared</u> facilities may indicate <u>undeclared</u> facilities & activities
- NPP a great place to start...
- The value of DT applies to the entire NFC





Using DT in IAEA's Maturing State Level Concept





- SLC is applied holistically in a State based on its declared NFC, technical capabilities, etc.
- Safeguards activities driven by acquisition path analysis (APA)
- DT can point to misuse/diversion at other NFC steps in the State
- DT informs IAEA's AP Complementary Access activities
- This can lead to a more effective International Safeguards program



Expected Results



- Complete and demonstrated digital twin framework for safeguards by design
- Opportunity for comprehensive understanding of nuclear fuel cycle facility operations to significantly strengthen nuclear safeguards and nonproliferation regime
- Future opportunity to support diversion/misuse detection for both item (LWR) and bulk (MSR) type advanced reactors. As well as indicators for clandestine reactors

