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ATTN: Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555-0001

SUBJECT: Perry Nuclear Power Plant Docket No. 50-440 Annual Environmental and Effluent Release Report

Enclosed is the Annual Environmental and Effluent Release Report for the Perry Nuclear Power Plant (PNPP) for the period of January 1, 2015 through December 31, 2015. This document includes the radiological environmental operating report, radioactive effluent release report, and the non-radiological environmental operating report which satisfies the requirements of the PNPP Technical Specifications (TS), the PNPP Offsite Dose Calculation Manual (ODCM), and the Environmental Protection Plan, Appendix B of the PNPP Operating License. Also enclosed are two corrected pages to 2014 Annual Environmental and Effluent Release Report.

There are no regulatory commitments contained in this letter. If there are any questions or if additional information is required, please contact Mr. Thomas Veitch, Chemistry Manager at (440) 280-5188.

Sincerely,

David Hamilton Vice President

Enclosures:

- A PNPP 2015 Annual Environmental and Effluent Release Report
- B Corrections to the 2014 PNPP Annual Environmental and Effluent Release Report
- cc: NRC Project Manager NRC Resident Inspector NRC Region III

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PNPP 2015 Annual Environmental and Effluent Release Report

2015

ANNUAL ENVIRONMENTAL AND EFFLUENT RELEASE REPORT

for the Perry Nuclear Power Plant

PREPARED BY: CHEMISTRY SECTION PERRY NUCLEAR POWER PLANT FIRSTENERGY NUCLEAR OPERATING COMPANY PERRY, OHIO APRIL, 2015

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Appendices

Appendix A: 2015 Inter-Laboratory Cross Check Comparison Program Results

Appendix B: 2015 REMP Data Summary Reports

Appendix C: 2015 REMP Detailed Data Report

Appendix D: Corrections to Previous AEERR

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Appendix G: ODCM Changes

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EXECUTIVE SUMMARY

The Annual Environmental and Effluent Release Report (AEERR) details the results of environmental and effluent monitoring programs conducted at the Perry Nuclear Power Plant (PNPP) from January 01 through December 31, 2015. This report meets all of the requirements in PNPP Technical Specifications, the Offsite Dose Calculation Manual (ODCM), the Environmental Protection Plan (EPP) and Regulatory Guide 1.21. It incorporates the requirements of the Annual Radioactive Effluent Release Report (ARERR), the Annual Radiological Environmental Operating Report (AREOR) and the Annual Environmental Operating Report (AEOR). Report topics include radioactive effluent releases, radiological environmental monitoring, land use census, clam/mussel monitoring, herbicide use, and special reports. The results of the environmental and effluent programs indicate that the operations of the PNPP did not result in any significant environmental impact.

RADIOACTIVE EFFLUENT RELEASES

During the normal operation of a nuclear power plant, small quantities of radioactivity are released to the environment in liquid and gaseous effluents. Radioactive material is also shipped offsite as solid waste. PNPP maintains a comprehensive program to control and monitor the release of radioactive materials from the site in accordance with Nuclear Regulatory Commission (NRC) release regulations.

The dose to the general public from the plant's liquid and gaseous effluents was below regulatory limits. The calculated maximum individual whole body dose potentially received by an individual resulting from PNPP liquid effluents was 1.02E-03 mrem (0.03% of the regulatory limit). The calculated maximum individual whole body dose potentially received by an individual resulting from PNPP gaseous effluents (excluding C-14) was 8.21E-03 mrem (0.16% of the regulatory limit).

Radioactivity released to the environment in the form of gaseous Carbon-14 (C-14) was estimated based on plant type and power production. The calculation is based on an industry initiative supported by the Nuclear Energy Institute (NEI), the Electric Power Research Institute (EPRI) and the NRC. The calculated hypothetical maximum individual whole body dose potentially received by an individual resulting from PNPP gaseous effluents including C-14 is 2.6E-01 mrem (5.2% of the limit). Refer to page 21 for additional C-14 information.

The summation of the hypothetical maximum individual dose from effluents is less than 1% of the total dose an individual living in the PNPP area receives from all sources of manmade and background radiation.

Shipments of solid waste consisted of waste generated during water treatment, radioactive material generated during normal daily operations and maintenance, and irradiated components. PNPP complied with regulations governing radioactive shipments of solid radioactive waste.

RADIOLOGICAL ENVIRONMENTAL MONITORING

The Radiological Environmental Monitoring Program (REMP) was established in 1981 to monitor the radiological conditions in the environment around PNPP. The REMP is conducted in accordance with PNPP ODCM. This program includes the collection and analysis of environmental samples and evaluation of results.

The REMP was established at PNPP six (6) years before the plant became operational. This pre-operational program was designed to provide data on background radiation and radioactivity normally present in the area. PNPP has continued to monitor the environment during plant operation by collecting and analyzing samples of air, milk, fish, vegetation, water and sediment, as well as by measuring radiation directly. The results of the REMP program indicate adequate control of radioactivity released from PNPP effluents. These results also demonstrate that PNPP complies with federal regulations.

Air samples were collected to monitor the radioactivity in the atmosphere; the results showed normal background radionuclide concentrations.

Terrestrial monitoring included the analysis of milk and vegetation; the results indicated concentrations of radioactivity similar to that found in previous years. Analyses of vegetation samples detected only natural radioactivity similar to those observed in previous years and indicated no radioactivity attributable to the operation of PNPP.

Aquatic monitoring included the collection and analyses of water, fish, and shoreline sediments. The analytical results for water, fish and sediment samples showed normal background radionuclide concentrations.

Direct radiation measurements showed no discernible change from previous years. The indicator locations averaged 12.5 mrem/quarter and control locations averaged 12.3 mrem/quarter. Radiation dose in the area of PNPP was similar to the radiation dose measured at locations greater than ten (10) miles away from PNPP.

Based on these results, the operation of the PNPP resulted in no measureable increase in the radionuclide concentrations observed in the environment.

LAND USE CENSUS

In order to estimate radiation dose attributable to the operation of PNPP, the potential pathways through which public exposure can occur must be known. To identify these exposure pathways, an Annual Land Use Census is performed as part of the REMP. During the census, PNPP personnel travel public roads within a five (5) mile radius of the plant to locate key radiological exposure pathways. These key pathways include the nearest resident, garden, and milk animal in each of the ten meteorological land sectors that surround the plant. The information obtained from the census is entered into a computer program, that is used to assess the hypothetical dose to members of the public. The predominant land use within the census area continues to be rural and/or agricultural.

CLAM/MUSSEL MONITORING

Clam and mussel shells can clog plant piping and components that use water from Lake Erie. For this reason, sampling for clams and mussels has been conducted in Lake Erie near PNPP since 1971. The monitoring is specifically for Corbicula (Asiatic clams) since their introduction into the Great Lakes in 1981, and for Dreissena (zebra mussels) since

their discovery in Lake Erie in 1989. Since no Corbicula have ever been found at PNPP, routine Corbicula monitoring will provide early detection capability if this pest species arrives at PNPP. The Dreissena program includes both monitoring and control and is directed at minimizing the mussel's impact on plant operation. As in past years, this program has successfully prevented Dreissena from causing any significant operational problems at PNPP.

HERBICIDE USE

The use of herbicides on the PNPP site is monitored to ensure compliance with Ohio Environmental Protection Agency (OEPA) requirements and to protect the site's natural areas. Based on weekly inspections, herbicide use has not had a negative impact on the environment around the plant.

SPECIAL ENVIRONMENTAL REPORTS

Significant environmental events (e.g. spills, releases), noncompliance with environmental regulations (e.g., OEPA discharge limits), and changes in plant design or operation that affect the environment are reported to regulatory agencies as they occur. Three (3) reports were submitted in 2015. See page 43 for details.

INTRODUCTION

Nuclear energy provides an alternative energy source, which is readily available and has very limited impact upon the environment. To more fully understand nuclear energy as a source of generating electricity, it is helpful to understand basic radiation concepts and its occurrence of radioactivity in nature.

RADIATION FUNDAMENTALS

Atoms are the basic building blocks of all matter. Simply described, atoms are made up of positively and negatively charged particles and particles which are neutral. These particles are called protons, electrons, and neutrons, respectively. The relatively large protons and neutrons are packed together in the center of the atom called the nucleus. Orbiting around the nucleus are one or more smaller electrons. In an electrically neutral atom, the positively charged protons in the nucleus balance the negatively charged electrons. Due to their dissimilar charges, the protons and electrons have a strong attraction for each other, which helps hold the atom together. Other attractive forces between the protons and neutrons keep the densely packed protons from repelling each other and prevent the nucleus from breaking apart.

Atoms with the same number of protons in their nuclei make up an element. The number of neutrons in the nuclei of an element may vary. Atoms with the same number of protons but different numbers of neutrons are called isotopes. All isotopes of the same element have the same chemical properties and many are stable or non-radioactive. An unstable or radioactive isotope of an element is called a radionuclide. Radionuclides contain an excess amount of energy in the nucleus, which is usually due to an excess number of neutrons.

Radioactive atoms attempt to reach a stable, non-radioactive state through a process known as radioactive decay. Radioactive decay is the release of energy from an atom's nucleus through the emission of alpha and beta particles and gamma rays. Radionuclides vary greatly in the rate in which they decay. The length of time an atom remains radioactive is defined in terms of its half-life. Half-life is defined as the time required for a radioactive substance to lose half its activity through the process of radioactive decay. Half-lives vary from millionths of a second to millions of years.

RADIATION AND RADIOACTIVITY

Radioactive decay is a process in which the nucleus of an unstable atom becomes more stable by spontaneously emitting energy. Radiation refers to the energy that is released when radioactive decay occurs within the nucleus. This section includes a discussion on the three (3) primary forms of radiation produced by radioactive decay.

Alpha Particles

Alpha particles consist of two protons and two neutrons and have a positive charge. Because of their charge and large size, alpha particles do not travel very far when released (less than 4 inches in air). They are unable to penetrate any solid material, such as paper or skin, to any significant depth. However, if alpha particles are released inside the body, they can damage the soft internal tissues because they deposit all their energy in a small area.

Beta Particles

Beta particles have the same characteristics as electrons but originate from the nucleus. They are much smaller than alpha particles and travel at nearly the speed of light, thus they travel longer distances than alpha particles. External beta radiation primarily affects the skin. Because of their electrical charge, beta particles are stopped by paper, plastic or thin metal.

Gamma Rays

Gamma rays are bundles of electromagnetic energy called photons. They are similar to visible light, but at a much higher energy. Gamma rays can travel long distances in air and are often released during radioactive decay along with alpha and beta particles. Potassium-40 is an example of a naturally occurring radionuclide found in all humans that emits a gamma ray when it decays.

Interaction with Matter

When radiation interacts with other materials, it affects the atoms of those materials principally by removing the negatively charged electrons out of their orbits. This causes an atom to lose its electrical neutrality and become positively charged. An atom that is charged, either positively or negatively, is called an ion, and the radiation is called ionizing radiation.

UNITS OF MEASURE

Some of the units of measure used in this report require explanation.

Activity

Activity is the number of atoms in a material that decay per unit of time. Each time an atom decays, radiation is emitted. A curie (Ci) is the unit used to describe the activity of a material and indicates the rate at which the atoms are decaying. One curie of activity indicates the decay of 37 billion atoms per second. Smaller units of the curie are often used in this report. Two common units are the microcurie (μ Ci), one millionth of a curie, and the picocurie (pCi), one trillionth of a curie. The mass, or weight, of radioactive material, which would result in one (1) curie of activity, depends on the disintegration rate. For example, one gram of radium-226 is equivalent to one (1) curie of activity. It would require about 1.5 million grams of natural uranium, however, to equal one (1) curie.

Dose

Biological damage due to alpha, beta, and gamma radiation may result from the ionization caused by these types of radiation. Some types of radiation, especially alpha particles that cause dense local ionization, can result in much more biological damage for the same energy imparted than does gamma or beta radiation. A quality factor therefore must be applied to account for the different ionizing capabilities of various types of ionizing radiation. When the quality factor is multiplied by the absorbed dose (as measured in rads), the result is the dose equivalent, which is an estimate of the possible biological damage resulting from exposure to any type of ionizing radiation. The dose equivalent is measured in terms of the Roentgen Equivalent Man (rem). When discussing environmental radiation effects, the rem is a large unit; therefore, a smaller unit, the millirem (mrem) is often used. One mrem is equivalent to 1/1000 of a rem.

LOWER LIMIT OF DETECTION

Sample results are often reported as below the Lower Limit of Detection (LLD). The LLD for an analysis is the smallest amount of radioactive material that will show a positive result for which there can be a 95% confidence that radioactivity is present. This statistical parameter is used as a measure of the sensitivity of a sample analysis. When a measurement is reported as less than the LLD (<LLD), it means that no radioactivity was detected. Had radioactivity been present at (or above) the stated LLD value, it statistically would have been detected. The NRC has established the required LLD values for environmental and effluent sample analyses.

OTHER SOURCES OF RADIATION DOSE TO THE U.S. POPULATION

This section discusses the doses that the average American typically receives each year from naturally occurring background radiation and all other sources of radiation. With the information presented in this section, the reader can compare the doses received from Nuclear Power Plant (NPP) effluents with the doses received from natural, medical, and other sources of radiation. This comparison provides some context to the concept of radiation dose effects.

In March 2009, the National Council on Radiation Protection and Measurements (NCRP) published Report No. 160 as an update to the 1987 NCRP Report No. 93, Ionizing Radiation Exposure of the Population of the United States (Refs. 30, 31). Report No. 160 describes the doses to the U.S. population from all sources of ionizing radiation for 2006, the most recent data available at the time the NCRP report was written. The NCRP report also includes information on the variability of those doses from one individual to another. The NCRP estimated that the average person in the United States receives about 620 mrem of radiation dose each year. NCRP Report No. 160 describes each of the sources of radiation that contribute to this dose, including:

- Naturally occurring sources (natural background) such as cosmic radiation from space, terrestrial radiation from radioactive materials in the earth, and naturally occurring radioactive materials in the food people eat and in the air people breathe;
- Medical sources from diagnosis and treatment of health disorders using radioactive pharmaceuticals and radiation-producing equipment;
- Consumer products (such as household smoke detectors);
- Industrial processes, security devices, educational tools, and research activities; and
- Exposures of workers that result from their occupations.

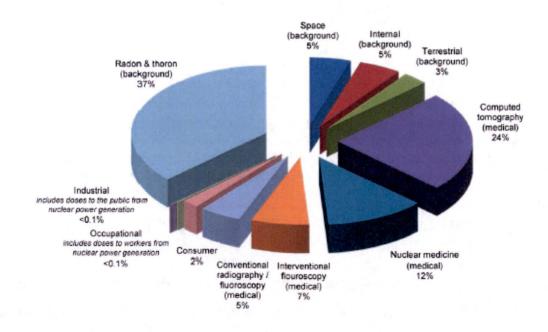


Figure 1

Sources of Radiation Exposure to the U.S. Population

Figure 1 shows the contribution of various sources of exposure to the total collective effective dose and the total effective dose per individual in the U.S. population for 2006. Larger contributors to dose are represented by proportionally larger slices of the pie. Doses to the public from NPPs are included in the industrial category; doses to workers from nuclear power generation are included in the category of occupational dose. Doses to the public due to effluents from NPPs are less than 0.1% (one-tenth of one percent) of what the average person receives each year from all other sources of radiation.

ENVIRONMENTAL RADIONUCLIDES

Many radionuclides are present in the environment due to sources such as cosmic radiation and fallout from nuclear weapons testing. These radionuclides are expected to be present in many of the environmental samples collected in the vicinity of PNPP. Some of the radionuclides normally present include:

Beryllium-7, present as a result of the interaction of cosmic radiation with the upper atmosphere,

Potassium-40, a naturally occurring radionuclide normally found in humans and throughout the environment, and

Radionuclides from nuclear weapons testing fallout, including tritium and Cesium-137.

Tritium due to the interaction of Nitrogen in the air and cosmic rays.

Beryllium-7 and Potassium-40 are common in REMP samples. Since they are naturally occurring and are expected to be present, positive results for these radionuclides are not discussed in the section for the Sampling Program results. These radionuclides are included; however, in Appendix A, 2015 Inter-Laboratory Cross Check Comparison Program Results.

RADIOACTIVE EFFLUENT RELEASES

INTRODUCTION

The source of radioactive material in a nuclear power plant is the generation of fission products (e.g., noble gas, iodine, and particulate) or neutron activation of water and corrosion products (e.g., tritium and cobalt). The majority of the fission products generated remain within the nuclear fuel pellet and fuel cladding. Most fission products that escape from the fuel cladding, as well as the majority of the activated corrosion products, are removed by plant processing equipment.

During the normal operation of a nuclear power plant, small amounts of radioactive material are released in the form of solids, liquids, and gases. PNPP was designed, and is operated in such a manner as to control and monitor these effluent releases. Effluents are controlled to ensure any radioactivity released to the environment is minimal and within regulatory limits. Effluent release programs include the operation of monitoring systems, in-plant sampling and analysis, quality assurance, and detailed procedures covering all aspects of effluent monitoring.

The liquid and gaseous radioactive waste treatment systems at PNPP are designed to collect and process these wastes in order to remove most of the radioactivity. Effluent monitoring systems are used to provide continuous indication of the radioactivity present

and are sensitive enough to measure several orders of magnitude lower than the release limits. This monitoring equipment is equipped with alarms and indicators in the plant control room. The alarms are set to provide warnings to alert plant operators when radioactivity levels reach a small fraction of the limits. The waste streams are sampled and analyzed to identify and quantify the radionuclides being released to the environment.

Gaseous effluent release data is coupled with on-site meteorological data in order to calculate the dose to the general public. Devices are maintained at various locations around PNPP to continuously sample the air in the surrounding environment. Frequent samples of other environmental media are also taken to determine if any radioactive material deposition has occurred. The Radiological Environmental Monitoring Program (REMP) is described in detail later in this report.

Generation of solid waste is controlled to identify opportunities for minimization. Limiting the amount of material taken into the plant, sorting material as radioactive or non-radioactive waste, and incinerating waste help to lower the volume of radioactive solid waste generated. After vendor processing, solid waste is shipped to a licensed burial site.

REGULATORY LIMITS

The Nuclear Regulatory Commission has established limits for liquid and gaseous effluents that comply with:

Title 10 of the Code of Federal Regulations, Part 20, Standards for Protection Against Radiation, Appendix B;

Title 10 of the Code of Federal Regulations, Part 50, Domestic Licensing of Production and Utilization Facilities, Appendix I;

Title 10 of the Code of Federal Regulations, Part 72.104, Criteria for Radioactive Materials in Effluents and Direct Radiation from an ISFSI or MRS

Title 40 of the Code of Federal Regulations, Part 190, Environmental Radiation Protection Standards for Nuclear Power Operations

These limits were incorporated into the PNPP Technical Specifications, and subsequently into the PNPP ODCM. The ODCM prescribes the maximum doses and dose rates due to radioactive effluents resulting from the operation of PNPP. These limits are defined in several ways to limit the overall impact on persons living near the plant. Since there are no other fuel sources near the PNPP, the 40CFR190 limits, which are described below, were not exceeded.

The 40CFR190 limit for whole body dose is 25 mrem. For 2015, the total whole body dose to a member of the general public, considering all sectors, was 0.26 millirem. This value was determined by summing the annual whole body doses from liquid and gaseous radioactive effluents and the annual gaseous Carbon-14 dose. Since the direct radiation dose, as determined by TLD, was indistinguishable from natural background (see Figure 8, page 35), it was not included in the calculation.

LIQUID EFFLUENTS

The concentration of radioactive material released in liquid effluents to unrestricted areas shall be limited to the concentrations specified in 10CFR20, Appendix B, Table 2, Column 2 for radionuclides other than dissolved or entrained noble gases, as required by the ODCM.

For dissolved or entrained noble gases, the concentration is limited to a concentration of 2.0E-04 μ Ci/ml. These values are the maximum effluent concentrations.

The dose or dose commitment to a member of the public from radioactive materials in liquid effluents released to unrestricted areas shall be limited to the following:

During any calendar quarter:

Less than or equal to 1.5 mrem to the whole body, and

Less than or equal to 5 mrem to any organ

During any calendar year:

Less than or equal to 3 mrem to the whole body, and

Less than or equal to 10 mrem to any organ

GASEOUS EFFLUENTS

The dose rate due to radioactive materials released in gaseous effluents from the site to areas at and beyond the site boundary shall be limited to the following as required by the ODCM:

Noble gases:

Less than or equal to 500 mrem per year to the whole body, and

Less than or equal to 3000 mrem per year to the skin

lodine-131, lodine-133, Tritium, and all radionuclides in particulate form with half lives greater than eight days:

Less than or equal to 1500 mrem per year to any organ

Air dose due to noble gases to areas at, and beyond the site boundary, shall be limited to the following:

During any calendar quarter:

Less than or equal to 5 mrad for gamma radiation, and

Less than or equal to 10 mrad for beta radiation

During any calendar year:

Less than or equal to 10 mrad for gamma radiation, and

Less than or equal to 20 mrad for beta radiation

Dose to a member of the public from lodine-131, lodine-133, Tritium, and all radionuclides in particulate form with half-lives greater than eight days in gaseous effluents released to areas at and beyond the site boundary shall be limited to the following:

Less than or equal to 7.5 mrem to any organ per any calendar quarter, and

Less than or equal to 15 mrem to any organ per any calendar year

The PNPP ODCM does not contain a concentration limit for gaseous effluents. For this reason, effluent concentrations are not used to calculate maximum release rates for gaseous effluents.

INDEPENDENT SPENT FUEL STORAGE INSTALLATION

During any calendar year:

Less than or equal to 25 mrem whole body dose; Less than or equal to 75 mrem thyroid dose; and Less than or equal to 25 mrem to any other critical organ.

Release Summary

Effluents are sampled and analyzed to identify both the type and quantity of radionuclides present. This information is combined with effluent path flow measurements to determine the composition, concentration, and dose contribution of the radioactive effluents.

Liquid Effluents

The PNPP liquid radioactive waste system is designed to collect and treat all radioactive liquid waste produced in the plant. The treatment process used for radioactive liquid waste depends on its physical and chemical properties. It is designed to reduce the concentration of radioactive material in the liquid by filtration to remove suspended solids and demineralization to remove dissolved solids. Normally, the effluent from the liquid radioactive waste system is returned to plant systems. To reduce the volume of water stored in plant systems; however, the processed liquid effluent may be discharged from the plant via a controlled release. In this case, effluent activity and dose calculations are performed prior to and after discharging this processed water to Lake Erie to ensure regulatory compliance and dose minimization principles are maintained.

Liquid radioactive waste system effluents may be intermittently released, which are considered to be "batch" releases. Table 1 provides information on the number and duration of these releases.

	Quarter 1	Quarter 2	Quarter 3	Quarter 4
Number of batch releases	13	14	· 0	0
Total time period for batch releases, min	3.09E+03	3.45E+03	NA	NA
Maximum time for a batch release, min	3.61E+02	3.45E+02	NA	NA
Average time period for a batch release, min	2.38E+02	2.46E+02	NA	NA
Minimum time for a batch release, min	1.60E+01	2.20E+02	NA	NA

Table 1: Liquid Batch Releases

Table 2 provides information on the nuclide composition for the liquid radioactive effluent system releases. If a radionuclide was not present at a level "greater than or equal to the LLD" (\geq LLD), then the value is expressed as "less than the LLD" (<LLD). In each case, LLDs were met, or were below the levels required by the ODCM. Table 2a provides information specific to radioactive effluent batch releases and Table 2b provides information specific to continuous radioactive effluent releases.

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	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error, (%)
A. Fission and Activation Products					
1. Total Released, Ci (excluding tritium, gases, alpha)	5.83E-03	8.91E-03	2.81E-05	8.79E-05	1.00E+01
2 Average Diluted Concentration, µCi/mL *	3.15E-10	4.78E-10	1.05E-12	4.62E-12	
3. Percent of Applicable Limit, %	7.06E-03	1.17E-02	3.16E-05	1.21E-04	
B. Tritium		-		,	
1. Total Released, Ci	2.76E+00	2.53E+00	1.27E-02	1.55E-01	1.00E+01
2. Average Diluted Concentration, µCi/mL	1.49E-07	1.36E-07	4.75E-10	8.15E-09	
3. Percent of Applicable Limit, %	1.49E-02	1.36E-02	4.75E-05	8.15E-04	
C. Dissolved and Entrained Gases					
1. Total Released, Ci	3.93E-03	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<>	<lld< td=""><td>1.00E+01</td></lld<>	1.00E+01
2. Average Diluted Concentration, µCi/mL	2.12E-10	NA	NA	NA	
3. Percent of Applicable Limit, %	1.06E-04	NA	NA	NA	,
D. Gross Alpha Activity, Ci	2.10E-07	<lld< td=""><td>4.27E-07</td><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<>	4.27E-07	<lld< td=""><td>1.00E+01</td></lld<>	1.00E+01
E. Waste Volume Released, Liters (prior to dilution)	2.44E+07	2.46E+07	2.31E+07	2.30E+07	
F. Dilution Water Volume Used, Liters	1.85E+10	1.86E+10	2.68E+10	1.90E+10	

Table 2: Summation of All Liquid Effluent Releases

<LLD – Less than the lower limit of detection

*Average diluted concentrations are based on total volume of water released during quarter.

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	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error, (%)
A. Fission and Activation Products					
Total Released, Ci (excluding tritium, gases, alpha)	5.51E-03	8.86E-03	-	-	1.00E+01
B. Tritium		<u>. </u>	,	.	
Total Released, Ci	2.75E+00	2.52E+00	<u> </u>	<u> </u>	1.00E+01
C. Dissolved and Entrained Gases		•	-	-	
Total Released, Ci	3.93E-03	<lld< td=""><td> ·</td><td></td><td>1.00E+01</td></lld<>	·		1.00E+01
D. Gross Alpha Activity, Ci	<lld< td=""><td><lld< td=""><td></td><td></td><td>1.00E+01</td></lld<></td></lld<>	<lld< td=""><td></td><td></td><td>1.00E+01</td></lld<>			1.00E+01
E. Waste Volume Released, Liters (prior to dilution)	1.72E+06	1.80E+06	_ ·	_	

Table 2a: Summation of Batch Liquid Effluent Releases

<LLD – Less than the lower limit of detection

Table 2b: Summation of Continuous Liquid Effluent Releases

	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error, (%)
A. Fission and Activation Products	_				
Total Released, Ci (excluding tritium, gases, alpha)	3.23E-04	4.59E-05	2.81E-05	8.79E-05	1.00E+01
B. Tritium					
Total Released, Ci	1.46E-02	1.25E-02	1.27E-02	1.55E-01	1.00E+01
C. Dissolved and Entrained Gases					
Total Released, Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<>	<lld< td=""><td>1.00E+01</td></lld<>	1.00E+01
D. Gross Alpha Activity, Ci	2.10E-07	<lld< td=""><td>4.27E-07</td><td><lld< td=""><td>1.00E+01</td></lld<></td></lld<>	4.27E-07	<lld< td=""><td>1.00E+01</td></lld<>	1.00E+01
E. Waste Volume Released, Liters (prior to dilution)	2.26E+07	2.28E+07	2.31E+07	2.30E+07	

<LLD -- Less than the lower limit of detection

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Table 3 lists the total number of curies (Ci) of each radionuclide present in liquid effluent releases for each quarter. If a radionuclide was not present at a level "greater than or equal to the LLD" (\geq LLD), then the value is expressed as "less than the LLD" (\leq LLD). In each case, the LLDs were either met, or were below the levels required by the ODCM.

Table o Radiouotive Elquia		1		I		<u>,</u>
	Units	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Annual
Tritium	Ci	2.76E+00	2.53E+00	1.27E-02	1.55E-01	5.46E+00
Chromium-51	Ci	1.16E-03	1.78E-04	<lld< td=""><td><lld< td=""><td>1.34E-03</td></lld<></td></lld<>	<lld< td=""><td>1.34E-03</td></lld<>	1.34E-03
Manganese-54	Ci	5.18E-04	1.79E-03	3.04E-06	2.90E-06	2.31E-03
Iron-55	Ci	<lld< td=""><td><lld< td=""><td><ĽLD</td><td><lì.d< td=""><td><lld_< td=""></lld_<></td></lì.d<></td></lld<></td></lld<>	<lld< td=""><td><ĽLD</td><td><lì.d< td=""><td><lld_< td=""></lld_<></td></lì.d<></td></lld<>	<ĽLD	<lì.d< td=""><td><lld_< td=""></lld_<></td></lì.d<>	<lld_< td=""></lld_<>
Iron-59	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cobalt-58	Ci	2.86E-04	4.31E-04	<lld< td=""><td>1.94E-05</td><td>7.37E-04</td></lld<>	1.94E-05	7.37E-04
Cobalt-60	Ci	3.58E-03	6.01E-03	2.51E-05	6.56E-05	9.67E-03
Zinc-65	Ci	<lld< td=""><td>8.42E-05</td><td><lld< td=""><td><l`ld< td=""><td>8.42E-05</td></l`ld<></td></lld<></td></lld<>	8.42E-05	<lld< td=""><td><l`ld< td=""><td>8.42E-05</td></l`ld<></td></lld<>	<l`ld< td=""><td>8.42E-05</td></l`ld<>	8.42E-05
Strontium-89	Ci	_ <lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Strontium-90	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Molybdenum-99	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Silver-110m	Ci	2.57E-04	4.15E-04	<lld< td=""><td><lld< td=""><td>6.72E-04</td></lld<></td></lld<>	<lld< td=""><td>6.72E-04</td></lld<>	6.72E-04
Tin-113	Ci	1.86E-08	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.86E-08</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.86E-08</td></lld<></td></lld<>	<lld< td=""><td>1.86E-08</td></lld<>	1.86E-08
lodine-131	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cesium-134	Ci .	8.96E-06	<lld<sup>,</lld<sup>	<lld< td=""><td><lld< td=""><td>8.96E-06</td></lld<></td></lld<>	<lld< td=""><td>8.96E-06</td></lld<>	8.96E-06
Cesium-137	Ci	2.78E-05	<lld< td=""><td><lld< td=""><td><lld< td=""><td>2.78E-05</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>2.78E-05</td></lld<></td></lld<>	<lld< td=""><td>2.78E-05</td></lld<>	2.78E-05
Cerium-141	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cerium-144	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Xenon-133	Ci	3.93E-03	<lld< td=""><td><lld< td=""><td><lld< td=""><td>3.93E-03</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>3.93E-03</td></lld<></td></lld<>	<lld< td=""><td>3.93E-03</td></lld<>	3.93E-03
Gross Alpha	Ci	2.10E-07	<lld< td=""><td>4.27E-07</td><td><lld< td=""><td>6.37E-07</td></lld<></td></lld<>	4.27E-07	<lld< td=""><td>6.37E-07</td></lld<>	6.37E-07

Table 3 Radioactive	Liquid Effluent	Nuclide Composition
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<LLD – Less than the lower limit of detection

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Gaseous Effluents

Gaseous effluents are made up of fission and activation gases, iodine and particulate releases. Gaseous effluents from PNPP exit the plant via one of four effluent vents. Each of these four effluent vents contains radiation detectors that continuously monitor the air to ensure that the levels of radioactivity released are below regulatory limits. Samples are also collected and analyzed on a periodic basis to ensure regulatory compliance. Gaseous effluents released from PNPP are considered continuous and at ground level.

In 2013 PNPP increased the volume of air captured when sampling for tritium in gaseous effluents. This has increased the detection capability (LLD) by a factor of 20. Gaseous effluent tritium releases are now being detected where before they were too dilute to measure. This has resulted in a reported increase in tritium released over previous years. A summation of all gaseous radioactive effluent releases is given in Table 4.

Table 4. Outlination of An Gaseous Endents					
	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Est. Total Error, %
A. Fission and Activation Products			-		<u> </u>
1. Total Released, Ci	5.58E+01	0.00E+00	0.00E+00	0.00E+00	1.00E+01
2. Average Release Rate, µCi/sec	7.18E+00	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit, %	N/A	Ň/A	N/A	N/A	
B. lodine					
1. Total lodine-131 Released, Ci	1.27E-04	0.00E+00	0.00E+00	0.00E+00	1.00E+01
2. Average Release Rate, µCi/sec	1.63E-05	0.00E+00	0.00E+00	0.00E+00	
3. Percent of Applicable Limit, %	N/A	N/A	N/A	N/A	
C. Particulates with Half-Lives > 8 days			-		
1. Total Released, Ci	0.00E+00	9.58E-05	0.00E+00	0.00E+00	1.00E+01
2. Average Release Rate, µCi/sec	0.00E+00	1.22E-05	0.00E+00	0.00E+00	
3. Percent of Applicable Limit, %	N/A	N/A	N/A	N/A	
D. Alpha Activity, Ci	4.82E-07	2.70E-06	6.69E-07	1.59E-07	1.00E+01
E. Tritium					,
1. Total Released, Ci	1.68E+00	1.79E-01	1.30E+00	6.42E-01	1.00E+01
2. Average Release Rate, µCi/sec	2.16E-01	2.28E-02	1.63E-01	8.08E-02	
3. Percent of Applicable Limit, %	N/A	N/A	N/A	N/A	
F. Carbon-14, Ci	3.25E+00	3.40E+00	4.74E+00	4.76E+00	1.00E+01

Table 4: Summation of All Gaseous Effluents

<LLD -- Less than the lower limit of detection

N/A – Not Applicable, the ODCM does not have a release rate limit for gaseous effluents.

Carbon-14 activity was calculated based on power production and using the EPRI provided Spreadsheet.

The radionuclide composition of all gaseous radioactive effluents for a continuous-mode, ground-level release is given in Table 5. If a radionuclide was not present at a level "greater than or equal to the LLD," then the value is expressed as "less than the LLD" (<LLD). In each case, LLDs were met or were below the levels required by the ODCM. Discussion of C-14 doses is listed on page 21, Carbon-14 supplemental information.

Die 5. Rauloactive Gaseou	5 Lilluch	L HUGHUC	oomposid	<u> </u>		
	Unit	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Annual
1. Fission and Activation Gase	s		.			
Tritium	Ci	1.68E+00	1.79E-01	1.30E+00	6.42E-01	3.80E+00
Krypton-85m	Ci	2.85E+00	<lld< td=""><td><lld< td=""><td><lld< td=""><td>2.85E+00</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>2.85E+00</td></lld<></td></lld<>	<lld< td=""><td>2.85E+00</td></lld<>	2.85E+00
Krypton-87	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Krypton-88	Ci	1.38E+00	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.38E+00</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.38E+00</td></lld<></td></lld<>	<lld< td=""><td>1.38E+00</td></lld<>	1.38E+00
Xenon-133m	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Xenon-133	Ci	5.01E+01	<lld< td=""><td><lld< td=""><td><lld< td=""><td>5.01E+01</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>5.01E+01</td></lld<></td></lld<>	<lld< td=""><td>5.01E+01</td></lld<>	5.01E+01
Xenon-135	Ci	1.43E+00	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.43E+00</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.43E+00</td></lld<></td></lld<>	<lld< td=""><td>1.43E+00</td></lld<>	1.43E+00
Xenon-138	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Total for Period	Ci	5.75E+01	1.79E-01	1.30E+00	6.42E-01	5.96E+01
2. Iodine/Halogens						
lodine-131	Ci	1.27E-04	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.27E-04</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.27E-04</td></lld<></td></lld<>	<lld< td=""><td>1.27E-04</td></lld<>	1.27E-04
lodine-133	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Total for Period	Ci	1.27E-04	<lld< td=""><td><lld< td=""><td><lld< td=""><td>1.27E-04</td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td>1.27E-04</td></lld<></td></lld<>	<lld< td=""><td>1.27E-04</td></lld<>	1.27E-04
3. Particulates						
Manganese-54	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Iron-59	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cobalt-58	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cobalt-60	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Zinc-65	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Strontium-89	Ci	<lld< td=""><td>9.58E-05</td><td><lld< td=""><td><lld< td=""><td>9.58E-05</td></lld<></td></lld<></td></lld<>	9.58E-05	<lld< td=""><td><lld< td=""><td>9.58E-05</td></lld<></td></lld<>	<lld< td=""><td>9.58E-05</td></lld<>	9.58E-05
Molybdenum-99	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cesium-134	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cesium-137	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cerium-141	Ci	<lld< td=""><td><u><</u>LLD</td><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<u><</u> LLD	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Cerium-144	Ci	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""><td><lld< td=""></lld<></td></lld<></td></lld<>	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
Total for Period	Ci	<lld< td=""><td>9.58E-05</td><td><lld< td=""><td><lld< td=""><td>9.58E-05</td></lld<></td></lld<></td></lld<>	9.58E-05	<lld< td=""><td><lld< td=""><td>9.58E-05</td></lld<></td></lld<>	<lld< td=""><td>9.58E-05</td></lld<>	9.58E-05

Table 5: Radioactive Gaseous Effluent Nuclide Composition

1

<LLD – Less than the lower limit of detection

Solid Waste

All solid radioactive waste from PNPP was processed and combined with waste from several other utilities by intermediate vendors (Energy Solutions and Erwin Resin Solutions). This waste was ultimately sent to Clive, Utah disposal facilities for burial.

Table 6: Solid Waste Shipped	d Offsite for Burial or Disposal
------------------------------	----------------------------------

1. Type of Solid Waste Shipped	Volume (m³)	Activity (Ci)	Est. Total Error (%)
a. Resins, Filters and Evaporator Bottoms	1.19E+02	4.58E+02	+/- 25
b. Dry Active Waste	1.69E+03	1.85E+00	+/- 25
c. Irradiated components, control rods, etc.	0.00E+00	0.00E+00	+/- 25
d. Other Waste	0.00E+00	0.00E+00	+/- 25

2. Estimate of Major (1) Nuclide Composition (by type of waste)	Radionuclide	Abundance (%)	Est. Total Error, (%)
a. Resins, Filters and Evaporator Bottoms	Mn-54	4.17	+/- 25
· ·	Fe-55	20.39	
	Co-58	1.84	
	Co-60	64.45	
	Zn-65	6.56	
b. Dry Active Waste	Mn-54	2.34	+/- 25
	Fe-55	32.31	
	Co-60	61.79	
	Ni-63	1.21	
c. Irradiated Components, Control Rods, etc.	N/A	N/A	<u>N/A</u>
d. Other Waste	N/A	N/A	N/A

(1) - "Major" is defined as any individual radionuclide identified as >1% of the waste type abundance.

3. Sol	id Waste Disposition	
Number of Shipments	Mode of Transportation	Destination
65	Hittman Transport	Energy Solutions Bear Creek Operations
2	Hittman Transport	Energy Solutions Gallaher Operations
3	Hittman Transport	Erwin Resin Solutions, LLC 151 T.C. Runnion Road
2	Miller Transfer and Rigging	Energy Solutions Bear Creek Operations
1	Specialty Transport Inc.	Energy Solutions Bear Creek Operations
1	Tri State Motor Transit	Energy Solutions Bear Creek Operations

METEOROLOGICAL DATA

The Meteorological Monitoring System at PNPP consists of a 60-meter tower equipped with two independent systems for measuring wind speed, wind direction, and temperature at both 10-meter and 60-meter heights. The tower also has instrumentation to measure dew point and barometric pressure. Data is logged from the tower through separate data loggers, and transmitted to a common plant computer. This system compiles the data and calculates a variety of atmospheric parameters, communicates with the Meteorological Information Dose Assessment System (MIDAS), and sends data over communication links to the plant Control Room.

A detailed report of the monthly and annual operation of the PNPP Meteorological Monitoring Program is produced under separate cover. For 2015, the report substantiates the quality and quantity of meteorological data collected in accordance with applicable regulatory guidance.

DOSE ASSESSMENT

The maximum concentration for any radioactive release is controlled by the limits set forth in Title 10 of the Code of Federal Regulations, Part 20 (10CFR20). Sampling, analyzing, processing, and monitoring the effluent stream ensures compliance with these concentration limits. Dose limit compliance is verified through periodic dose assessment calculations. Some dose calculations are conservatively performed for a hypothetical individual who is assumed to reside on the site boundary at the highest potential dose location all year. This person, called the "maximum individual", would incur the maximum potential dose from direct exposure (air plus ground plus water), inhalation, and ingestion of water, milk, vegetation, and fish. Because no one actually meets these criteria, the actual dose received by a real member of the public is significantly less than what is calculated for this hypothetical individual.

Dose calculations for this maximum individual at the site boundary are performed for two cases. First, they are performed using data for a 360 degree radius around the plant site (land and water based meteorological sectors); even though some of these sectors are over Lake Erie, which has no permanent residents. The second calculation is performed considering only those sectors around the plant in which people reside (land-based meteorological sectors).

The calculated hypothetical, maximum individual dose values at the site boundary are provided in Table 7. This table considers all meteorological sectors around PNPP and provides either the whole body or worst-case, organ dose values.

Type of Dose	Organ	Estimated Dose, (mrem)	Limit	% of Limit
Liquid Effluent	Whole body	1.02E-03	3.0E+00	3.4E-02
	GI Tract	1.50E-03	1.0E+01	1.5E-02
Noble Gas	Air Dose	5.75E-02	1.0E+01	5.7E-01
	Gamma – mrad			
	Air Dose	5.81E-02	2.0E+01	2.9E-01
	Beta – mrad			
Noble Gas	Whole body	8.21E-03	5.0E+00	1.6E-01
	Skin	1.76E-02	1.5E+01	1.2E-01
Particulate & lodine	Thyroid	2.00E-03	1.5E+01	1.3E-02
Carbon-14 *	Whole Body	2.59E-01	5.0E+00	5.2E+00

Table 7: Maximum Yearly Individual Site Boundary Dose, Considering All Sectors

*C-14 dose calculated at nearest garden.

The calculated hypothetical, maximum 50-mile radius population dose values at the site / boundary are provided in Table 8. This table considers all meteorological sectors around PNPP and provides either the whole body or worst-case, organ dose values.

ble 0. i opulation reali	ly Dose, Considering	All dectors out to
	Organ	Estimated Dose (person-rem)
Liquid Effluent	Whole body	1.4E-01
	Thyroid	6.4E-02
Gaseous Effluent	Whole body	8.4E-04
	Thyroid	8.8E-04

Table 8: Population Yearly Dose, Considering All Sectors out to 50 miles.

Table 9 provides the calculated hypothetical maximum site boundary dose values considering only the land-based sectors.

Type of Dose	Organ	Estimated Dose, (mrem)	Limit	% of Limit
Liquid Effluent	Whole Body	1.02E-03	3.0E+00	3.4E-02
	GI Tract	1.50E-03	1.0E+01	1.5E-02
Noble Gas	Air Dose Gamma – mrad	2.51E-03	1.0E+01	2.5E-02
	Air Dose Beta`– mrad	2.65E-03	2.0E+01	1.3E-02
Noble Gas	Whole Body	1.92E-04	5.0E+00	3.8E-03
	Skin	4.01E-04	1.5E+01	2.7E-03
Particulate & lodine	Thyroid	1.79E-04	1.5E+01	1.2E-03
Carbon-14 *	Whole Body	2.59E-01	5.0E+00	5.2E+00

 Table 9: Maximum Yearly Individual Site Boundary Dose, Considering Only Land

 Sectors

*C-14 dose calculated at nearest garden.

Other dose calculations are performed for a hypothetical individual who is assumed to be inside the site boundary for some specified amount of time. This person would receive the maximum dose during the time spent inside site boundary. Because no one actually meets the criteria established for these conservative calculations, the actual dose received by a member of the public is significantly less than what is calculated for this hypothetical individual. This dose is assessed relative to the offsite dose, and considers dilution, dispersion, and occupancy factors.

The highest hypothetical dose from liquid effluents to a member of the public inside the site boundary is to a person who is fishing on Lake Erie from the shore on PNPP property. The calculations assume that this person will spend 60 hours per year fishing, with a liquid dilution factor of 10. The ratio of the exposure pathway to the doses calculated for offsite locations yields the dose values shown in Table 10.

ible to: Maximum one Dose nom Elquid Ellidents				
	Whole Body Dose, (mrem)	Organ Dose (mrem)		
First Quarter	2.6E-04	3.0E-04		
Second Quarter	5.2E-04	6.1E-04		
Third Quarter	1.5E-06	1.7E-06		
Fourth Quarter	5.2E-06	6.1E-06		
Annual	8.1E-04	9.4E-04		

Table 10: Maximum Site Dose from Liquid Effluents

Although several cases were evaluated to determine the highest hypothetical dose from gaseous effluents to members of the public inside site boundary, the activity inside the site boundary with the highest dose potential is also shoreline fishing. The cases evaluated included traversing a public road within the site boundary, shoreline fishing (assuming fishing 60 hours per year), non-plant related training, car-pooling, and job interviews. The maximum on-site gaseous doses generated are shown in Table 11.

	Whole Body Dose, (mrem)	Organ Dose (mrem)
First Quarter	9.6E-04	2.0E-03
Second Quarter	5.9E-06	1.2E-05
Third Quarter	5.9E-05	5.9E-05
Fourth Quarter	3.6E-05	3.6E-05
Annual	1.1E-03	2.1E-03

Table 11: Maximum Site Dose from Gaseous Effluents

An average whole body dose to individual members of the public at or beyond the site boundary is then determined by combining the dose from gaseous and liquid radiological effluents. The dose from gaseous radiological effluents is based upon the population that lives within 50 miles of PNPP. The dose from liquid radiological effluents is determined for the population that receives drinking water from intakes within 50 miles of PNPP. The results of this calculation are provided in Table 12.

	Liquid Effluents (mrem)	Gaseous Effluents (mrem)
First Quarter	3.2E-05	2.2E-07
Second Quarter	2.4E-05	1.0E-08
Third Quarter	3.5E-08	9.6E-08
Fourth Quarter	1.5E-06	1.9E-08
Annual	5.8E-05	3.5E-07

 Table 12: Average Individual Whole Body Dose

CARBON-14 SUPPLEMENTAL INFORMATION

Carbon-14 (C-14), with a half-life of 5730 years, is a naturally occurring isotope of carbon produced by cosmic ray interactions in the atmosphere. Nuclear weapons testing in the 1950s and 1960s significantly increased the amount of C-14 in the atmosphere. C-14 is also produced in commercial nuclear reactors, but the amounts produced are much less than those produced naturally or from weapons testing. C-14 is released primarily from BWRs through the off-gas system in the form of carbon dioxide (CO₂). The quantity of gaseous C-14 released to the environment can be estimated using a C-14 source term scaling factor based on power generation.

The U.S. Nuclear Regulatory Commission (NRC) requires an assessment of gaseous C-14 dose impact to a member of the public resulting from routine releases in radiological effluents. Prior to 2011, the industry did not estimate the dose impact of C-14 releases. Since the dose contribution had been considered negligible compared to the dose impact from effluent releases of noble gases, tritium, particulates and radioiodines. At PNPP, improvements over the years in effluent management practices and fuel performance have resulted in a decrease in the concentration and changes in the distribution of gaseous radionuclides released to the environment.

This report contains estimates of the gaseous C-14 radioactivity released and the resulting public dose resulting from this release. This calculation is done using a spreadsheet provided by EPRI and is based on power production. This method for estimating C-14 release has been endorsed by the NRC. Because the dose contribution of C-14 from liquid radioactive waste is much less than that contributed by gaseous radioactive waste, evaluation of C-14 in liquid radioactive waste at PNPP is not required. Refer to Table 4 and Table 9 for C-14 estimated release values and doses.

GROUNDWATER MONITORING PROGRAM

Based on the Environmental Resource Management hydrogeology study, twelve (12) monitoring wells were recommended for the site. Since most groundwater flow drains north towards Lake Erie, the majority of wells are drilled there. A set of control wells was drilled south of the plant, to assess what a typical groundwater profile would be.

There are 4 sets of triplet wells installed at each location. Each triplet has a shallow well of approximately 25 feet, a mid-depth well of approximately 50 feet, and a deep well of approximately 75 feet. These 3 depths are designated A, B and C, from shallowest to deepest, respectively. Refer to Figure 2 for locations of Groundwater wells 1A through 4C. These wells encompass the groundwater monitoring locations at PNPP.

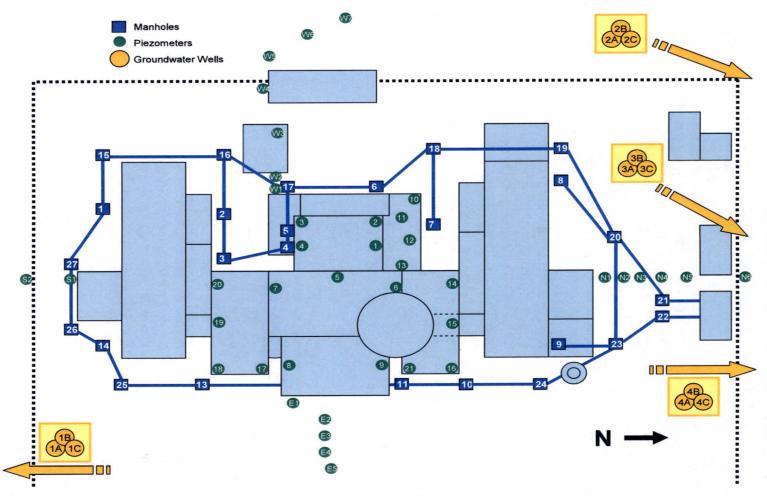


Figure 2: Underdrain System and Groundwater Monitoring Wells

The monitoring wells are sampled twice annually, in spring and fall. The sampling is done twice yearly by personnel from FirstEnergy's BETA Laboratories. The samples are shipped to Midwest Laboratories in Illinois. Midwest analyzes the sample for gamma isotopic and tritium. Any positive result less than 500 pCi/L is considered as background activity and not due to plant operations. The ODCM reporting level for tritium in an environmental water sample is 20,000 pCi/L. There was no indication of any effluent releases via groundwater.

Monitoring Well	Spring	Fall
	H-3, pCi/L	H-3, pCi/L
1A	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
1B	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
1C	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
2A	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
2B	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
2C	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
3A	189	<lld< td=""></lld<>
3B	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
3C	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
4A	<lld< td=""><td>231</td></lld<>	231
4B	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>
4C	<lld< td=""><td><lld< td=""></lld<></td></lld<>	<lld< td=""></lld<>

Table 13: Summary of Onsite Groundwater Samples

RADIOLOGICAL ENVIRONMENTAL MONITORING INTRODUCTION

The Radiological Environmental Monitoring Program (REMP) was established at PNPP for several reasons. First, it verifies the adequacy of plant design and operation to control radioactive materials and limit effluent releases. Second, it assesses the radiological impact, if any, that the plant has had on the surrounding environment. Third, it ensures compliance with regulatory guidelines. The REMP is conducted in accordance with the PNPP Operating License, Appendix B, Technical Specifications and the ODCM. The Nuclear Regulatory Commission (NRC) established the REMP requirements.

A variety of samples are collected as part of the PNPP REMP. The selection of sample types, locations, and collection frequency are based on many things. Potential pathways for the transfer of radionuclides through the environment to humans, sample availability, local meteorology, population characteristics, land use and NRC requirements are all factors.

To ensure that the REMP data are meaningful and useful, detailed sampling methods and procedures are followed. This ensures that samples are collected in the same manner and from the same locations each time. All samples are packaged on site, and then shipped to an independent vendor laboratory for analysis. The vendor laboratory analyzes the samples and reports results to the PNPP Chemistry Unit staff, the Lake County General Health District, and the State of Ohio Department of Health. Additionally the Lake County General Health District obtains monthly "split" samples of milk, water and vegetation. This permits an independent verification of PNPP's radiological environmental monitoring program.

SAMPLING LOCATIONS

REMP samples are collected at numerous locations, both on site and up to 20.6 miles away from the plant. Sampling locations are divided into two general categories: indicator and control. Indicator locations are those that monitor for any environmental impact due to plant operations. They are relatively close to the plant. Control locations are those that are unaffected by plant operation; they are a greater distance from the plant and in the least prevalent wind directions. Data obtained from the indicator locations are compared with data from the control locations. This comparison allows naturally occurring background radiation to be taken into account when evaluating any radiological impact PNPP may have had on the environment. Table 14, Figure 3, Figure 4 and Figure 5 identify the PNPP REMP sampling locations.

Many REMP samples are collected in addition to those required by the PNPP ODCM. The ODCM requirements for each sample type are discussed in more detail later in the report.

Location #	Description	Miles	Direction	Media (1)
	Chapel Road	3.2	ENE	TLD, AIP
2	Kanda Garden	2.0	ENE	Food Products
3	Meteorological Tower	1.0	SE	TLD, AIP
4	Site Boundary	0.7	S	TLD, AIP
5	Quincy Substation	0.6	SW	TLD, AIP
6	Concord Service Center	11.1	SSW	TLD, AIP
7	Site Boundary	0.6	NE	TLD, AIP
8	Site Boundary	0.7	E	TLD
9	Site Boundary	0.7	ESE	TLD
10	Site Boundary	0.6	SSE	TLD
11	Parmly Rd.	0.6	SSW	TLD
12	Site Boundary	0.6	wsw	TLD
13	Madison-on-the-Lake	4.6	ENE	TLD
14	Hubbard Rd.	4.9	E	TLD
15	Eagle St. Substation	5.1	ESE	TLD
16	Eubank Garden	0.9	S	Food Products
18	Kijauskas Farm (goat)	2.6	E	Food Products Milk
19	Goodfield Dairy	9.2	S	Milk
20	Rainbow Farms	1.9	E	Food Products
21	Hardy Rd.	5.1	wsw	TLD
23	High St. Substation	7.9	WSW	TLD
24	St. Clair Ave.	15.0	SW	TLD
25	Offshore - PNPP discharge	2.0	NNW	Fish
28	CEI Ashtabula Plant Intake	20.6	ENE	Water
29	River Rd.	4.5	SSE	TLD
30	Lane Rd.	4.9	SSW	TLD
31	Wood and River Rd.	4.9	SE	TLD
32	Offshore - Mentor	15.8	wsw	Fish
33	River Rd.	4.7	s	TLD
34	PNPP Intake	0.2	NW	Water

Table 14: REMP Sampling Locations

Location #	Description	Miles	Direction	Media (1)
35	Site Boundary	0.7	E	TLD, AIP
36	Lake County Water Plant	4.0	wsw	TLD, Water
37	Gerlica Farm	1.6	ENE	Food Products
39	Painesville Purification Plant	8.3	W	Water
51	Rettger Milk Farm (cow)	9.7	s	Milk
53	Great Lakes Nuclear Services	0.7	WSW	TLD
54	Hale Rd. School	4.7	SW	TLD
55	Center Rd.	2.5	S	TLD
56	Madison High School	4.0	ESE	TLD
57	Perry High School	1.7	S	TLD
58	Antioch Rd.	0.8	ENE	TLD
59	Lake Shoreline at Green Rd.	4.0	ENE	Water
60	Lake Shoreline at Perry Park	1.0	wsw	Water
64	Northwest Drain Mouth	0.4	WNW	Sediment
66	Lake Shore, Metropolitan Park	1.4	NE	Sediment
70	H&H Farm Stand	17.1	SSW	Food Products

(1) AIP = Air, Iodine and Particulate TLD = Thermoluminescent Dosimeter



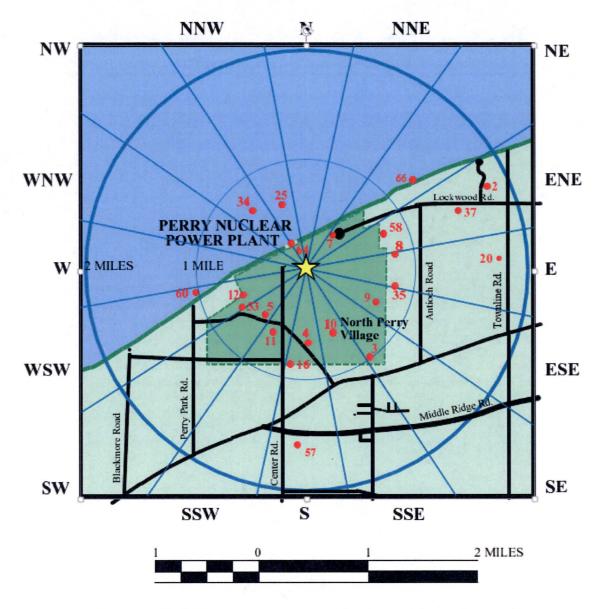


Figure 3: REMP Sampling Locations within Two Miles of Plant Site

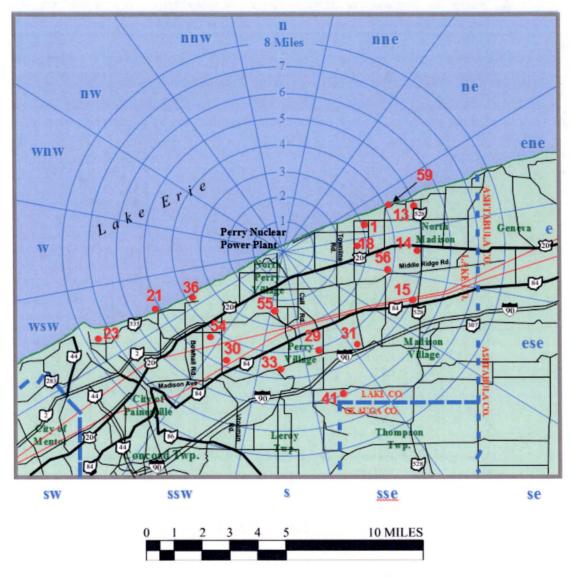


Figure 4: REMP Sampling Locations between Two and Eight Miles of the Plant Site

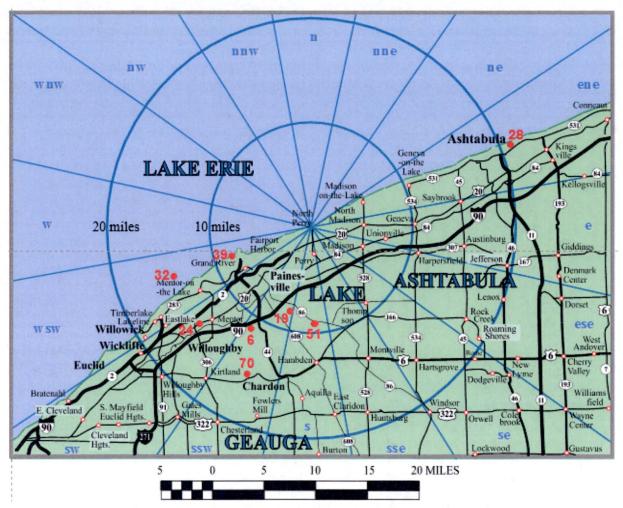


Figure 5: REMP Sampling Locations Greater Than Eight Miles from the Plant Site

SAMPLE ANALYSIS

When environmental samples are analyzed for radioactivity, several types of measurements are performed to provide information about the types of radiation and radionuclides present. The major analyses that are performed are discussed below.

Gross beta activity measures the total amount of beta-emitting radioactivity present in a sample, and acts as a tool to identify samples that may require further analysis. Beta radiation may be released by many different radionuclides. Since beta decay results in a continuous energy spectrum rather than the discrete energy levels, or "peaks", associated with gamma radiation, identification of specific beta-emitting nuclides is more difficult. Therefore, gross beta activity only indicates whether the sample contains normal or abnormal amounts of beta-emitting radioactivity; it does not specifically identify the radionuclides present.

Gamma spectral analysis provides more specific information than does the analysis for gross beta activity. Gamma spectral analysis identifies each radionuclide, and the amount of radioactivity, present in the sample emitting gamma radiation. Each radionuclide has a very specific "fingerprint" that allows for accurate identification and quantification.

lodine activity analysis measures the amount of radioactive iodine present in a sample. Some media (e.g. air sample charcoal cartridges) are analyzed directly by gamma spectral analysis. With other media (e.g. milk), the radioiodines are extracted by chemical separation before being analyzed by gamma spectral analysis.

Tritium activity analysis measures the amount of the radionuclide tritium (H-3) present in a sample. Tritium is an isotope of hydrogen that emits low-energy beta particles. Tritium occurs naturally and is also man-made.

Gamma doses received by Thermoluminescent Dosimeters (TLD) while in the field are determined by a special laboratory procedure. Thermoluminescence is a process by which ionizing radiation interacts with the sensitive phosphor material in the TLD. Energy is trapped in the TLD material and can be stored for months or years. This capability provides a method to measure the dose received over long periods of time. The amount of energy that was stored in the TLD as a result of interaction with radiation is released by a controlled heating process and measured in a calibrated reading system. As the TLD is heated, the phosphor releases the stored energy as light. The amount of light is directly proportional to the amount of radiation to which the TLD was exposed. Table 15 provides a list of the analyses performed on environmental samples collected for the PNPP REMP.

Sample results are often reported as less than the lower limit of detection (< LLD), which is defined as the smallest amount of radioactive material that will show a positive result for which there can be confidence that radioactivity is present. This statistical parameter is used as a measure of the sensitivity of a sample analysis. When a measurement is reported as < LLD, it means that no radioactivity was detected. The required detection limits for samples is determined by the sample media and the radionuclide that is being analyzed for and is listed in the ODCM. The NRC has established LLD values for REMP sample analyses. The vendor laboratory for REMP sample analyses complied with those values.

Туре	Sample	Frequency	Analysis
Atmospheric Monitoring	Airborne Particulates	Weekly & Quarterly	Gross Beta Activity & Gamma Spectral Analysis
	Airborne Radioiodine	Weekly	lodine-131
Terrestrial Monitoring	Milk	Monthly & Bi-Monthly when cows are on pasture	Gamma Spectral Analysis & lodine-131
	Broadleaf Vegetation	Monthly during growing season	Gamma Spectral Analysis
Aquatic Monitoring	Water	Monthly	Gross Beta Activity & Gamma Spectral Analysis
		Quarterly	Tritium Activity
	Fish	Annually	Gamma Spectral Analysis
	Sediment	Biannually	Gamma Spectral Analysis
Direct Radiation	TLD	Quarterly &	Gamma Dose
Monitoring		Annually	Gamma Dose

Table 15: REMP Sample Analyses

SAMPLING PROGRAM

The contribution of radionuclides to the environment resulting from PNPP operation is assessed by comparing results from the environmental monitoring program with preoperational data (i.e., data from before 1986), operational data from previous years, and control location data. The results for each sample type are discussed below and compared to historical data to determine if there are any observable trends. All results are expressed as concentrations. Refer to Appendix B, 2015 REMP Data Summary Reports for a detailed listing of these results. The NRC requires special reporting whenever sample analysis results exceed set limits. No values exceeded those limits.

PROGRAM CHANGES

Due to shutdown of the Ashtabula power plant (location 28), the water control location was shifted to the Painesville Ohio Purification Plant (location 39).

The milking animal (goat) at location 18 died during the summer and the owner chose not to replace the animal. This removes the one milk sample that was in the vicinity of the PNPP. There are no other milking animals in the vicinity of PNPP to use as a replacement.

ATMOSPHERIC MONITORING

Air

Air sampling is conducted to detect any increase in the concentration of airborne radionuclides. The PNPP REMP maintains an additional two (2) air sampling locations

above the five (5) locations (four indicators and one control) required by the ODCM. Six (6) of these locations are within four miles of the plant site; the seventh is used as a control location and is eleven miles from PNPP. Air sampling pumps draw continuous samples at a rate of approximately two cubic feet per minute. The air is drawn through glass fiber filters (to collect particulate material) and a charcoal cartridge (to adsorb iodine). The samples are collected on a weekly basis, 52 weeks a year, from each of the seven (7) air sampling stations.

Air samples are analyzed weekly for gross beta activity and radioiodine activity. The air samples are also analyzed by gamma spectral analysis quarterly. A total of 361 air particulate and 361 air radioiodine samples were collected and analyzed. Three samples were not collected, see Appendix F for explanation.

Gross beta activity was detected in all the air samples. The average gross beta activity for the indicator locations was 0.027 pCi/m³ and the controls was 0.028 pCi/m³. Historically, the concentration of gross beta in air has been essentially identical at indicator and control locations. Figure 6 reflects the average gross beta activity for 2015 and the previous years. All radioiodine samples were less than the lower limit of detection for lodine-131.

Except for naturally occurring Beryllium-7, no radionuclides were identified in the quarterly gamma spectral analysis above the LLD values.

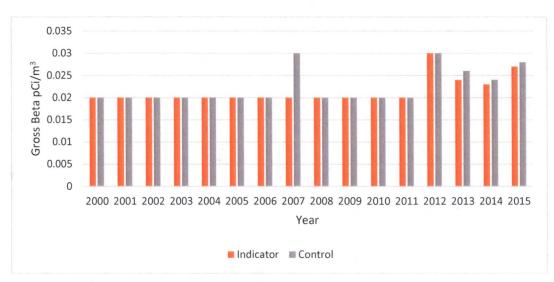


Figure 6: Annual Average Gross Beta Activity, in Air

TERRESTRIAL MONITORING

Collecting and analyzing samples of milk and food products provides data to assess the build-up of radionuclides that may be ingested by humans. The historical data from soil and vegetation samples provides information on atmospheric radionuclide deposition.

Milk

Samples of milk are collected once each month from November through March, and twice each month from April through October. Sampling is increased during the summer because animals usually feed outside on pasture and not on stored feed. The PNPP REMP includes three (3) milk locations.

Since the milk sampling locations do not meet the requirements of the ODCM (only one milkproducing animal is located within the required distance vs. two required), broadleaf vegetation sampling (discussed below) is performed. Milk is collected from the available location to augment vegetation sampling.

Milk samples are analyzed by gamma spectral analysis for radioiodines and other radionuclides. A total of forty-six (46) milk samples were collected. With the exception of naturally occurring Potassium-40, no other radionuclides were detected.

Broadleaf Vegetation

Because there are not a sufficient number of milk sampling locations, the PNPP REMP samples broadleaf vegetation. These samples are collected monthly during the growing season from six (6) gardens in the vicinity of PNPP and one control location 17.1 miles SSW from PNPP.

Sixty-nine (69) samples were collected and analyzed by gamma spectral analysis.

Four (4) vegetation types were grown and collected: mustard, collard greens, turnip greens and Swiss chard. Beryllium-7 and Potassium-40, naturally-occurring radionuclides, were found in the samples, which is expected. No other radionuclides were detected.

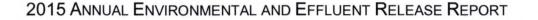
AQUATIC MONITORING

Radionuclides may be present in Lake Erie from many sources other than the PNPP. These sources include atmospheric deposition, run-off/soil erosion, and releases of radioactivity in liquid effluents from hospitals, universities or other industrial facilities. These sources provide two forms of potential radiation exposure, external and internal. External exposure can occur from contact with water or shoreline sediments, while internal exposure can occur from either direct ingestion of radionuclides or the transfer of radionuclides through the aquatic food chain. Direct ingestion can occur from drinking the water, while the transfer via the aquatic food chain occurs from the eventual consumption of aquatic organisms, such as fish. To monitor these pathways, PNPP samples water, shoreline sediments, and fish.

Water

Water is sampled from five (5) locations along Lake Erie in the vicinity of the PNPP as required by the PNPP ODCM. Fifty-four (54) water samples were collected and analyzed for gross beta activity and gamma spectral analysis. From these monthly samples, eighteen (18) quarterly composite samples were analyzed for tritium activity.

Gross beta activity was detected in forty-seven (47) of the fifty-four (54) samples collected. The indicator average gross beta activity was 1.5 pCi/L and the control average gross beta activity was 1.5 pCi/L. Refer to Figure 7 for the annual average gross beta activity for both indicator and control locations. No tritium or gamma activity was detected.



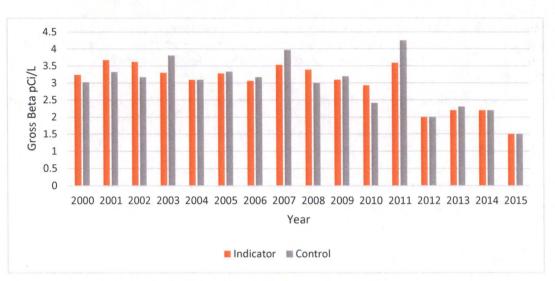


Figure 7: Annual Average Gross Beta Activity, in Water

Sediment

Sampling shoreline sediments provides an indication of the accumulation of particulate radionuclides which may lead to an external radiation source to fishermen and swimmers from shoreline exposure. Sediment was sampled from two (2) locations.

Five (5) sediment samples were collected in 2015 and analyzed by gamma spectroscopy. The only radionuclide detected was naturally occurring Potassium-40.

Fish

Fish are analyzed primarily to quantify the radionuclide intake by humans, and secondarily to serve as indicators of radioactivity in the aquatic ecosystem. Fish are collected from two (2) locations annually during the fishing season as required by the ODCM. An important sport or commercial species is targeted, and only the fillets are sent to the laboratory for analysis.

Fourteen (14) fish samples were collected and analyzed – six (6) indicator and eight (8) control. The species were smallmouth bass, white perch, walleye, channel catfish, freshwater drum, white bass and tiger musky. Only naturally occurring Potassium-40 was detected in the samples.

DIRECT RADIATION MONITORING

Thermoluminescent Dosimeter (TLD)

Environmental radiation is measured directly at twenty seven (27) locations around the PNPP site and two (2) control locations. The locations are positioned in two rings around the plant as well as at the site boundary. The inner ring is within a one-mile radius of the plant site; the outer ring is four miles to five miles from the plant. The control locations are over ten miles from the plant in the two least prevalent wind directions. Each location has three TLDs, two of which are changed quarterly and one is changed annually.

A total of 261 TLDs were collected and analyzed. This includes 232 collected on a quarterly basis and twenty nine (29) collected annually. Annual TLDs are not required per the ODCM and are used for supplemental data only.

The annual average dose for all indicator locations was 57.1 mrem, with 59.1 mrem for the control locations.

The average quarterly dose for the indicator locations was 12.5 mrem, and 12.3 mrem for the control locations. Refer to Figure 8.

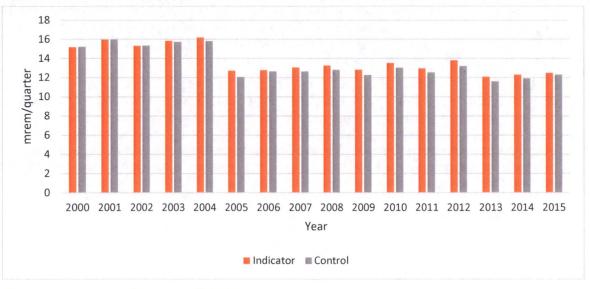


Figure 8: Average Quarterly TLD Dose

CONCLUSION

There is no detectable radiological effect on the surrounding environment due to operation of the Perry Nuclear Power Plant.

INTER-LABORATORY CROSS-CHECK COMPARISON PROGRAM

Introduction

The purpose of the Inter-laboratory Cross-Check Comparison Program is to provide an independent check on the vendor laboratory's analytical procedures. Samples with a known concentration of specific radionuclides are provided to the vendor laboratory. The vendor laboratory measures and reports the concentration of specified radionuclides. The known values are then compared to the vendor results. Results consistently outside established acceptance criteria indicate a need to check instruments or procedures. Regulatory Guide 4.15 specifically required that contractor laboratories that performed environmental measurement participate in the EPA's Environmental Radioactivity Laboratory Inter-Comparison Studies Program, or an equivalent program.

The EPA's program is no longer funded or offered. The reason that the EPA program was referenced in the regulatory guide is that the EPA standards were traceable to National Bureau of Standards (now known as National Institute Standard Technology). In response to this problem, ATI Environmental, Inc. (PNPP vendor lab) incorporated a program offered by Environmental Resource Associates (ERA Company), which covered the same analyses in the same matrix at the same frequency as the EPA program. The ERA Company has received NIST accreditation as an equivalent program. In addition to comparison cross checks performed with the ERA Company, the vendor laboratory routinely monitors the quality of their analyses by:

Analyzing "spiked" samples (samples with a specific quantity of radioactive material present in them) and

Participating in the Department of Energy's Mixed Analyte Performance Program (MAPEP).

See Appendix A, for the vendor Inter-Laboratory Cross-Check Comparison Program Results.

LAND USE CENSUS

Introduction

Each year a Land Use Census is conducted to identify the locations of the nearest milk animal, garden (of greater than 500 square feet), and residence in each of the meteorological sectors that is over land. Information gathered during the Land Use Census is used for offsite dose assessment and to update sampling locations for the REMP. The census is conducted by traveling all roads within a five-mile radius of the plant site, and recording and mapping the location of the nearest resident, milk animal and vegetable garden. The Land Use Census was conducted on September 26 2015. The census identified the garden, residence and milk animal locations tabulated in Tables 17, 18 and 19 and depicted in Figure 9. Note that the W, WNW, NW, NNW, N, and NNE sectors extend over Lake Erie, and are not included in the survey.

Discussions and Results

In general, the predominant land use within the census area continues to be rural/ agricultural. In recent years however, it has been noted that tracts of land once used for farming are now being developed as mini-industrial parks and residential housing. This is reflected in the loss of available milking animals within a five mile radius of PNPP to support the REMP.

Table 17 identifies the nearest residences, by sector, to the PNPP. There were no changes from last year's Land Use Census.

Sector	Location Address	Miles from PNPP	Map Locator Number
NE	4384 Lockwood	0.7	1
ENE	4602 Lockwood	1.1	2
E	2626 Antioch	1.0	3
ESE	2836 Antioch	1.1	4
SE	4495 North Ridge	1.3	5
SSE	3119 Parmly	0.9	6
S	3121 Center	0.9	7
SSW	3850 Clark	0.9	8
SW	2997 Perry Park	1.2	9
WSW	3460 Parmly	1.0	10

Table 17: Nearest Residence, By Sector

Table 18 identifies the nearest milking animal by sector, to the PNPP. There were no changes from last year's Land Use Census.

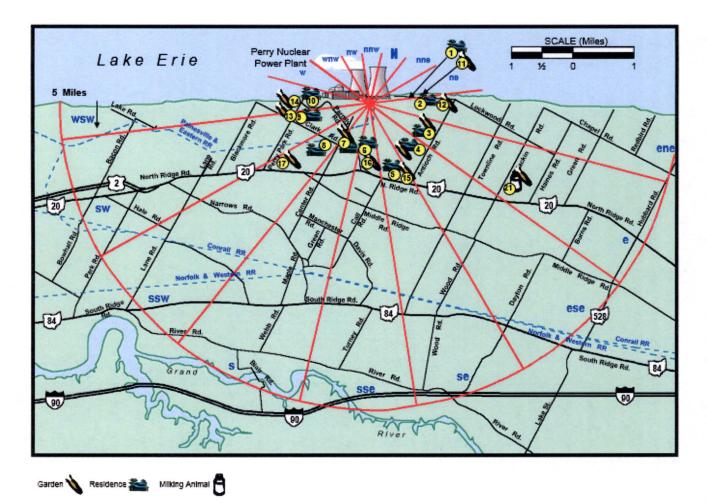
Sector	Location Address	Miles from PNPP	Map Locator	
E	2591 McMackin Rd.	2.6	21	

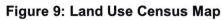
Table 18: Nearest Milk Animal, By Sector

Table 19 lists the nearest gardens by sector to the PNPP consisting of at least 500 square feet. There were no changes from last year's Land Use Census.

Sector	Location Address	Miles from PNPP	Map Locator Number
NE	2340 Hemlock	0.9	11
ENE	4630 Lockwood	1.1	12
Е	2626 Antioch	1.0	3
ESE	2836 Antioch	1.1	4
SE	4671 North Ridge	1.3	15
SSE	4225 Red Mill Valley	1.1	16
S	3121 Center Rd.	0.9	7
SSW	3431 Perry Park	1.9	17
SW	3021 Perry Park	1.3	13
WSW	3460 Parmly	1.0	14

Table 19: Nearest Garden, By Sector





CLAM/MUSSEL MONITORING

INTRODUCTION

Sampling for macro-invertebrates (clams and mussels) has been conducted in Lake Erie in the vicinity of PNPP, since 1971. The clam/mussel program currently focuses on two species: Corbicula fluminea (Asiatic clam) and Dreissena polymorpha (zebra mussel).

CORBICULA PROGRAM

Monitoring specifically for Corbicula was initiated in response to an NRC bulletin and concerns of the Atomic Safety and Licensing Board. The monitoring was done as part of the Environmental Protection Plan (Operating License, Appendix B). The program consists of visually inspecting the raw water systems, when they are opened for maintenance. The purpose of this program is to detect Corbicula, should it appear at PNPP.

Monitoring

Samples were collected from raw water systems and examined for shells and fragments. In addition to sample collections, plant components that use raw water are inspected when opened for maintenance or repair. Sample collection/inspection dates are listed in Table 20.

Date	Sample Location	
3/12/2015	N34 – Turbine Lube Oil Clr.	
3/15/2015	N71 – Circulating Water	
3/15/2015	HP Condenser, North Box "C"	
3/15/2015	HP Condenser, North Box "B"	
3/15/2015	HP Condenser, North Box "A"	
3/15/2015	HP Condenser, North Box "D"	
3/16/2015	Div. 2 Diesel Generator Heat Exchanger, Inlet	
3/16/2015	P42- Emergency Closed Cooling	
3/16/2015	R46 – Div. 2 Diesel Generator Jacket Water Heat Exchanger	
3/16/2015	N34 –Turbine Lube Oil Cooler. 'A"	
3/16/2015	Div.2 Diesel Generator Jacket Water. Heat Exchanger (West)	
3/16/2015	Circulating Water Basin & Flumes	
3/17/2015	N71 – Circulating Water	
3/17/2015	Emergency Closed Cooling Heater "B"	
6/15/2015	Lube Oil Cooler 1N34-B0001A	
7/20/2015	Fire Protection System	

Table 20: Corbicula Monitoring

Page 40

7/20/2015	Hydrant No. 30
7/24/2015	Lube Oil Cooler 1N34-B0001B
8/10/2015	Lube Oil Cooler N34B1B
8/10/2015	Lube Oil Cooler N34B1B
8/11/2015	Lube Oil Cooler 1N34-B0001A
8/13/2015	Fire Hydrant No. 28
9/1/2015	Lube Oil Cooler 1N34D0001A
9/8/2015	Training Center Yard Piping, Order- 200648596
9/14/2015	Lube Oil Cooler 1N34-B001 – "B"
10/7/2015	Fire Protection System P54 in area of 54F5065 East side of P&R Building

Conclusions

Although the presence of Corbicula was detected at the Eastlake Power Plant, it has not been demonstrated that their presence has created any operational problems at PNPP. As in the past, the monitoring program did not identify Corbicula in any sample collected.

DREISSENA PROGRAM

Dreissena or zebra mussels were first discovered at PNPP in September 1988. The initial collection of nineteen (19) mussels was made as part of the Corbicula monitoring program. The Dreissena monitoring program began in 1989, with monitoring and testing. The current control program was designed and implemented in 1990.

Monitoring

In addition to visually inspecting the plant's raw water systems when they are opened for maintenance or repair, monitoring methods include the use of commercial divers and side-stream monitors. Commercial divers monitor mussel infestation during the inspection of forebays, basins, and the intake and discharge structures. Divers have also been used to take underwater videotapes of the water basins and intake tunnel. Side-stream monitors are flow-through containers that receive water diverted from plant systems and are set up at two in-plant locations during the mussel season.

Treatment

Chemicals used for mussel control included sodium hypochlorite and a commercial molluscicide. The chlorine is intermittently injected into the plant service water, emergency service water, and circulating water systems by metering sodium hypochlorite into each system's influent. Sodium bisulfite is added at the plant discharge structure for dechlorination prior to return into Lake Erie.

The Ohio Environmental Protection Agency (OEPA) has approved the use of a commercial molluscicide. The chemical selected for use at the PNPP was alkyl-dimethyl-benzyl-ammonium chloride. Treatment was applied once in 2015. The active ingredients were detoxified by adsorption using bentonite clay prior to discharge into Lake Erie.

Results

The effectiveness of the intermittent biocide treatment has been determined in several ways. First, visual inspections of raw water system components are conducted when systems are open during maintenance or repair. In addition, settlement monitors were inspected for new settlement. No live settlement has been found in any plant component to date.

The effectiveness of the application of the commercial molluscicide was measured by observing mortality of mussels placed in a flow-through container installed in plant service water and subjected to the chemical treatment. The observed mortality rate utilizing the flow-through container was 100%. To date, PNPP has had no significant problems related to zebra mussels.

Conclusions

PNPP has taken the approach that the best method for avoiding problems with zebra mussels is preventive treatment of plant water systems. The current program of monitoring and chemical treatment will be continued to minimize the possibility that PNPP will experience future problems due to zebra mussels.

HERBICIDE APPLICATIONS

Herbicides are used sparingly on the PNPP site. A request must be made to and approved by the PNPP Chemistry Unit prior to spraying to ensure that only approved chemicals are used, and only in approved areas. Each application was in compliance with the Ohio Environmental Protection Agency's rules and regulations. There were no adverse environmental impacts observed during weekly site environmental inspections as a result of these applications. The herbicides used were Round-Up Quick Pro, Brushmaster, Trimec, Finale and Pronto. For each application, the type of weed to be treated dictated the herbicide and concentration to be used. Table 21 provides quantity for each chemical used. The quantity represents the amount of herbicide applied, prior to any dilution.

Chemical	Amount (gal)			
Round-Up Quick Pro	16.0			
Brushmaster	6.1			
Trimec 992	27.1			
Finale	25.8			
Pronto	39.5			

Table 21: Herbicide Applications

SPECIAL REPORTS NPDES PERMIT EXCEEDANCES

The Ohio Environmental Protection Agency (OEPA) issues the National Pollutant Discharge Elimination System (NPDES) permit. It establishes monitoring requirements and limits for discharges from the PNPP. It also specifies the locations from which the plant is allowed to discharge.

There were three NPDES exceedances at PNPP in 2015. Notifications were made to the Ohio EPA for each occurrence.

- 1. On 8/12/15, prior to commencing zebra mussel treatment, a feed hose ruptured spilling no more than 55 gallons of biocide (NALCO H150M) into Lake Erie. The cause for the hose rupture could not be determined.
- 2. On 8/22/15, an excessive sodium hypochlorite feed rate into the Service Water system resulted in an NPDES violation for excessive Total Residual Chlorine at the Outfall structure. The cause for the event was an inadequate procedure and inaccurate flow rate information. The corrective action was to revise the procedure and to use more accurate feed instrumentation.
- 3. On 12/15/15, the Service Water chlorination skid drain valve SW-2 was opened which drained more than 100 gallons of sodium hypochlorite to the Service Water intake structure that resulted in an NPDES violation. The cause for this event was operating a valve outside of procedure guidance. The corrective action was to lock closed the improperly opened valve.

ENVIRONMENTAL PROTECTION PLAN

The Environmental Protection Plan (EPP), which is Appendix B of the PNPP Operating License, requires a non-radiological environmental monitoring and reporting program be established at the PNPP.

Other than the non-compliance NPDES report (mentioned above) no other reports were submitted in 2015.

ENVIRONMENTAL IMPACT EVALUATIONS

All proposed changes to the PNPP design or operation, as well as tests or experiments, must be evaluated for potential environmental impacts in accordance with the EPP and administrative quality assurance procedures.

Approval was obtained from regulatory agencies for minor stream modifications. The modifications were done for flood control purposes as a part of the required changes due to the Fukushima event.

Appendix A

Inter-Laboratory Cross Check Comparison Program Results



APPENDIX A

INTERLABORATORY COMPARISON PROGRAM RESULTS

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NOTE: Environmental Inc., Midwest Laboratory participates in intercomparison studies administered by Environmental Resources Associates, and serves as a replacement for studies conducted previously by the U.S. EPA Environmental Monitoring Systems Laboratory, Las Vegas, Nevada. Results are reported in Appendix A. TLD Intercomparison results, in-house spikes, blanks, duplicates and mixed analyte performance evaluation program results are also reported. Appendix A is updated four times a year; the complete Appendix is included in March, June, September and December monthly progress reports only.

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January, 2015 through December, 2015

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Appendix A

Interlaboratory Comparison Program Results

Environmental, Inc., Midwest Laboratory has participated in interlaboratory comparison (crosscheck) programs since the formulation of it's quality control program in December 1971. These programs are operated by agencies which supply environmental type samples containing concentrations of radionuclides known to the issuing agency but not to participant laboratories. The purpose of such a program is to provide an independent check on a laboratory's analytical procedures and to alert it of any possible problems.

Participant laboratories measure the concentration of specified radionuclides and report them to the issuing agency. Several months later, the agency reports the known values to the participant laboratories and specifies control limits. Results consistently higher or lower than the known values or outside the control limits indicate a need to check the instruments or procedures used.

Results in Table A-1 were obtained through participation in the environmental sample crosscheck program administered by Environmental Resources Associates, serving as a replacement for studies conducted previously by the U.S. EPA Environmental Monitoring Systems Laboratory, Las Vegas, Nevada.

Table A-2 lists results for thermoluminescent dosimeters (TLDs), via internal laboratory testing and by irradiation and evaluation by the University of Wisconsin-Madison Radiation Calibration Laboratory at the University of Wisconsin Medical Radiation Research Center.

Table A-3 lists results of the analyses on in-house "spiked" samples for the past twelve months. All samples are prepared using NIST traceable sources. Data for previous years available upon request.

Table A-4 lists results of the analyses on in-house "blank" samples for the past twelve months. Data for previous years available upon request.

Table A-5 lists REMP specific analytical results from the in-house "duplicate" program for the past twelve months. Acceptance is based on the difference of the results being less than the sum of the errors. Complete analytical data for duplicate analyses is available upon request.

The results in Table A-6 were obtained through participation in the Mixed Analyte Performance Evaluation Program.

Results in Table A-7 were obtained through participation in the environmental sample crosscheck program administered by Environmental Resources Associates, serving as a replacement for studies conducted previously by the Environmental Measurement Laboratory Quality Assessment Program (EML).

Attachment A lists the laboratory precision at the 1 sigma level for various analyses. The acceptance criteria in Table A-3 is set at ± 2 sigma.

Out-of-limit results are explained directly below the result.

Attachment A

ACCEPTANCE CRITERIA FOR "SPIKED" SAMPLES

LABORATORY PRECISION: ONE STANDARD DEVIATION VALUES FOR VARIOUS ANALYSES

Analysis	Level	One standard deviation for single determination
Gamma Emitters	5 to 100 pCi/liter or kg > 100 pCi/liter or kg	5.0 pCi/liter 5% of known value
Strontium-89 ^b	5 to 50 pCi/liter or kg > 50 pCi/liter or kg	5.0 pCi/liter 10% of known value
Strontium-90 ^b	2 to 30 pCi/liter or kg > 30 pCi/liter or kg	5.0 pCi/liter 10% of known value
Potassium-40	≥ 0.1 g/liter or kg	5% of known value
Gross alpha	≤ 20 pCi/liter > 20 pCi/liter	5.0 pCi/liter 25% of known value
Gross beta	≤ 100 pCi/liter > 100 pCi/liter	5.0 pCi/liter 5% of known value
Tritium	≤ 4,000 pCi/liter	$\pm 1\sigma =$ 169.85 x (known) ^{0.0933}
	> 4,000 pCi/liter	10% of known value
Radium-226,-228	≥ 0.1 pCi/liter	15% of known value
Plutonium	≥ 0.1 pCi/liter, gram, or sample	10% of known value
lodine-131, lodine-129 ^b	≤ 55 pCi/liter > 55 pCi/liter	6 pCi/liter 10% of known value
Uranium-238, Nickel-63 ^b Technetium-99 ^b	≤ 35 pCi/liter > 35 pCi/liter	6 pCi/liter 15% of known value
Iron-55 ^b	50 to 100 pCi/liter > 100 pCi/liter	10 pCi/liter 10% of known value
Other Analyses ^b		20% of known value

^a From EPA publication, "Environmental Radioactivity Laboratory Intercomparison Studies

Program, Fiscal Year, 1981-1982, EPA-600/4-81-004.

Laboratory limit.

Lab Code	Date	Analysis	Laboratory	entration (pCi/L) ERA	Control	
	Date	Analysis	Result ^b	• Result °		Assentana
			Result	• Result	Limits	Acceptanc
ERW-1444	4/6/2015	Sr-89	59.71 ± 5.44	63.20	51.10 - 71.20	Pass
ERW-1444	4/6/2015	Sr-90	43.41 ± 2.43	41.90	30.80 - 48.10	Pass
ERW-1448	4/6/2015	Ba-133	77.75 ± 4.69	82.50	69.30 - 90.80	Pass
ERW-1448	4/6/2015	Cs-134	68.82 ± 3.08	75.70	61.80 - 83.30	Pass
ERW-1448	4/6/2015	Cs-137	191.9 ± 5.9	189.0	170.0 - 210.0	Pass
ERW-1448	4/6/2015	Co-60	85.05 ± 4.59	84.50	76.00 - 95.30	Pass
ERW-1448	4/6/2015	Zn-65	196.0 ± 12.0	203.0	183.0 - 238.0	Pass
ERW-1450	4/6/2015	Gr. Alpha	34.05 ± 1.90	42.60	22.10 - 54.00	Pass
ERW-1450	4/6/2015	G. Beta	26.93 ± 1.12	32.90	21.30 - 40.60	Pass
ERW-1453	4/6/2015	I-131	22.47 ± 0.83	23.80	19.70 - 28.30	Pass
ERW-1456	4/6/2015	Ra-226	8.20 ± 0.56	8.43	6.33 - 9.90	Pass
ERW-1456	4/6/2015	Ra-228	5.00 ± 0.67	4.39	2.56 - 6.01	Pass
ERW-1456	4/6/2015	Uranium	5.98 ± 0.31	6.59	4.99 - 7.83	Pass
ERW-1461	4/6/2015	H-3	3,254 ± 180	3280	2,770 - 3,620	Pass
ERW-5528	10/5/2015	Sr-89	34.76 ± 0.06	35.70	26.70 - 42.50	Pass
ERW-5528	10/5/2015	Sr-90	29.23 ± 0.06	31.10	22.70 - 36.10	Pass
ERW-5531	10/5/2015	Ba-133	30.91 ± 0.53	32.50	25.90 - 36.70	Pass
ERW-5531	10/5/2015	Cs-134	57.40 ± 2.57	62.30	50.69 - 68.50	Pass
ERW-5531	10/5/2015	Cs-137	163.1 ± 4.8	157.0	141.0 - 175.0	Pass
ERW-5531	10/5/2015	Co-60	73.41 ± 1.72	71.10	64.00 - 80.70	Pass
ERW-5531	10/5/2015	Zn-65	138.9 ± 5.7	126.0	113.0 - 149.0	Pass
ERW-5534	10/5/2015	Gr. Alpha	29.99 ± 0.08	51.60	26.90 - 64.70	Pass
ERW-5534	10/5/2015	G. Beta	27.52 ± 0.04	36.60	24.10 - 44.20	Pass
ERW-5537	10/5/2015	l-131 _/	25.54 ± 0.60	26.30	21.90 - 31.00	Pass
ERW-5540	10/5/2015	Ra-226	7.32 ± 0.37	7.29	5.49 - 8.63	Pass
ERW-5540 ^d	10/5/2015	Ra-228	7.80 ± 0.02	4.25	2.46 - 5.85	Fail
ERW-5540 °	10/5/2015	Ra-228	4.45 ± 0.96	4.25	2.46 - 5.85	Pass
ERW-5540	10/5/2015	Uranium	53.30 ± 0.55	56.20	45.70 - 62.40	Pass
ERW-5543	10/5/2015	H-3	21,260 ± 351	21,300	18,700 - 23,400	Pass

TABLE A-1. Interlaboratory Comparison Crosscheck program, Environmental Resource Associates (ERA)^a.

^a Results obtained by Environmental, Inc., Midwest Laboratory as a participant in the crosscheck program for proficiency testing in drinking water conducted by Environmental Resources Associates (ERA).

^b Unless otherwise indicated, the laboratory result is given as the mean ± standard deviation for three determinations.

^c Results are presented as the known values, expected laboratory precision (1 sigma, 1 determination) and control limits as provided by ERA.

^d Ra-228 spike was at a level close to the detection level. The high result was likely caused by interference from short-lived Rn-222 daughters.

^e The result of reanalysis (Compare to original result, footnoted "e" above).

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Lab Code	Irradiation		Known	Lab	Control	
	Date	Description	Value	Result	Limits	Acceptance
Environment	<u>al, Inc.</u>					
2015-1	6/24/2015	30 cm.	98.81	103.67 ± 6.05	69.20 - 128.50	Pass
2015-1	6/24/2015	30 cm.	98.81	111.32 ± 15.97	69.20 - 128.50	Pass
2015-1	6/24/2015	60 cm.	24.70	27.23 ± 1.33	17.30 - 32.10	Pass
2015-1	6/24/2015	60 cm.	24.70	26.98 ± 4.98	17.30 - 32.10	Pass
2015-1	6/24/2015	120 cm.	6.18	6.71 ± 1.77	4.30 - 8.00	Pass
2015-1	6/24/2015	120 cm.	6.18	6.78 ± 0.38	4.30 - 8.00	Pass
2015-1	6/24/2015	120 cm.	6.18	6.43 ± 2.00	4.30 - 8.00	Pass
2015-1	6/24/2015	150 cm.	3.95	4.13 ± 0.72	2.80 - 5.10	Pass
2015-1	6/24/2015	150 cm.	3.95	4.12 ± 1.36	2.80 - 5.10	Pass
2015-1	6/24/2015	150 cm.	3.95	4.50 ± 1.51	2.80 - 5.10	Pass
2015-1	6/24/2015	180 cm.	2.74	3.27 ± 0.28	1.90 - 3.60	Pass
2015-1	6/24/2015	180 cm.	2.74	3.05 ± 1.11	1.90 - 3.60	Pass
2015-1	6/24/2015	180 cm.	2.74	3.14 ± 0.18	1.90 - 3.60	Pass

TABLE A-2.1. Thermoluminescent Dosimetry, (TLD, CaSO₄: Dy Cards). ^a

.

Lab Code	Irradiation		Delivered	Reported	Performance ^c	
	Date	Description	Dose	Dose	Quotient (P)	Acceptance ^c
<u>Environment</u>	<u>al, Inc.</u>					
2015-2	12/15/2015	Spike 1	138.0	118.5 ± 2.1	-0.14	Pass
2015-2	12/15/2015	Spike 2	138.0	120.0 ± 1.6	-0.13	Pass
2015-2	12/15/2015	Spike 3	138.0	121.9 ± 1.9	-0.12	Pass
2015-2	12/15/2015	Spike 4	138.0	124.5 ± 3.3	-0.10	Pass
2015-2	12/15/2015	Spike 5	138.0	126.5 ± 3.2	-0.08	Pass
2015-2	12/15/2015	Spike 6	138.0	140.0 ± 4.2	0.01	Pass
2015-2	12/15/2015	Spike 7	138.0	128.2 ± 1.2	-0.07	Pass
2015-2	12/15/2015	Spike 8	138.0	128.0 ± 4.0	-0.07	Pass
2015-2	12/15/2015	Spike 9	138.0	124.9 ± 5.1	-0.09	Pass
2015-2	12/15/2015	Spike 10	138.0	122.9 ± 3.0	-0.11	Pass
2015-2	12/15/2015	Spike 11	138.0	123.3 ± 3.0	-0.11	Pass
2015-2	12/15/2015	Spike 12	138.0	119.0 ± 3.4	-0.14	Pass
2015-2	12/15/2015	Spike 13	138.0	123.0 ± 2.7	-0.11	Pass
2015-2	12/15/2015	Spike 14	138.0	125.4 ± 2.0	-0.09	Pass
2015-2	12/15/2015	Spike 15	138.0	122.0 ± 3.1	-0.12	Pass
2015-2	12/15/2015	Spike 16	138.0	120.8 ± 2.0	-0.12	Pass
2015-2	12/15/2015	Spike 17	138.0	118.8 ± 1.1	-0.14	Pass
2015-2	12/15/2015	Spike 18	138.0	117.0 ± 2.3	-0.15	Pass
2015-2	12/15/2015	Spike 19	138.0	120.8 ± 2.6	-0.12	Pass
2015-2	12/15/2015	Spike 20	138.0	122.6 ± 3.0	-0.11	Pass
Mean (Spike	1-20)			123.4	0.11	Pass
Standard De	viation (Spike 1	-20)		5.0	0.04	Pass

TABLE A-2.2 Thermoluminescent Dosimetry, (TLD, CaSO₄: Dy Cards). ^b

^a TLD's were irradiated at Environmental Inc. Midwest Laboratory. (Table A-2.1)

^b TLD's were irradiated by the University of Wisconsin-Madison Radiation Calibration Laboratory following ANSI N13.37 protocol from a known air kerma rate. TLD's were read and the results were submitted by Environmental Inc. to the University of Wisconsin-Madison Radiation Calibration Laboratory for comparison to the delivered dose.(Table A-2.2)

^c Performance Quotient (P) is calculated as ((reported dose - conventially true value) ÷ conventially true value) where the conventially true value is the delivered dose.

^d Acceptance is achieved when neither the absolute value of mean of the P values, nor the standard deviation of the P values exceed 0.15.

e Tables A2.1 and A2.2 assume 1 roentgen = 1 rem (per NRC -Health Physics Positions Based on 10 CFR Part 20 - Question 96 - Page Last Reviewed/Updated Thursday, October 01, 2015).

TABLE A-3.	In-House "Spiked" Samples	

			Concentration (pCi/L) ^a				
Lab Code ^b	Date		Laboratory results	Known	Control		
	-		2s, n=1 °	Activity	Limits ^d	Acceptance	
W-020315	2/3/2015	Ra-226	16.19 ± 0.42	16.70	13.36 - 20.04	Pass	
W-021215	2/12/2015	Gr. Alpha	18.38 ± 0.39	20.10	16.08 - 24.12	Pass	
W-021215	2/12/2015	Gr. Beta	27.98 ± 0.32	30.90	24.72 - 37.08	Pass	
SPW-687	2/27/2015	Ni-63	239.6 ± 3.5	202.4	161.9 - 242.9	Pass	
SPAP-689	3/2/2015	Gr. Beta	42.37 ± 3.50	43.61	34.89 - 52.33	Pass	
SPAP-691	3/2/2015	Cs-134	1.77 ± 0.61	1.90	1.52 - 2.28	Pass	
SPAP-691	3/2/2015	Cs-137	83.02 ± 2.60	97.20	77.76 - 116.64	Pass	
SPW-693	3/2/2015	Cs-134	44.30 ± 2.53	53.40	42.72 - 64.08	Pass	
SPW-693	3/2/2015	Cs-137	74.82 ± 3.50	73.80	59.04 - 88.56	Pass	
SPW-693	3/2/2015	Sr-89	87.45 ± 3.62	87.48	69.98 - 104.98	Pass	
SPW-693	3/25/2015	Sr-90	37.22 ± 1.55	38.10	30.48 - 45.72	Pass	
SPMI-697	3/2/2015	Cs-134	96.67 ± 7.74	107.00	85.60 - 128.40	Pass	
SPMI-697	3/2/2015	Cs-137	78.51 ± 7.02	73.84	59.07 - 88.61	Pass	
SPMI-697	3/2/2015	Sr-89	72.98 ± 4.86	87.48	69.98 - 104.98	Pass	
SPMI-697	3/2/2015	Sr-90	39.17 ± 1.51	38.10	30.48 - 45.72	Pass	
SPW-699	3/2/2015	H-3	59,592 ± 703	58,445	46,756 - 70,134	Pass	
W-031115	3/11/2015	Ra-226	13.73 ± 0.35	16.70	13.36 - 20.04	Pass	
N-030215	3/2/2015	Ra-228	32.79 ± 2.31	31.44	25.15 - 37.73	Pass	
SPF-1040	3/16/2015	Cs-134	787.5 ± 9.2	840.0	672.0 - 1,008.0	Pass	
SPF-1040	3/16/2015	Cs-137	2,599 ± 24	2,360	1,888 - 2,832	Pass	
SPW-1036	3/25/2015	Fe-55	1,792 ± 63	1961	1,569 - 2,353	Pass	
SPW-1374	4/6/2015	U-238	46.03 ± 2.25	41.70	25.02 - 58.38	Pass	
W-040815	4/8/2015	Gr. Alpha	20.18 ± 0.42	20.10	16.08 - 24.12	Pass	
W-040815	4/8/2015	Gr. Beta	29.70 ± 0.33	30.90	24.72 - 37.08	Pass	
SPW-1038	4/13/2015	C-14	3,497 ± 9	4,734	2,840 - 6,628	Pass	
W- 2165	4/20/2015	H-3	5550 ± 226	5,780	3,468 - 8,092	Pass	
W-2165	4/20/2015	Sr-89	90.70 ± 8.20	108.70	65.22 - 152.18	Pass	
W-2165	4/20/2015	Sr-90	76.80 ± 2.00	75.90	45.54 - 106.26	Pass	
W-2165	4/20/2015	Cs-134	62.40 ± 6.40	57.30	34.38 - 80.22	Pass	
W- 2165	4/20/2015	Cs-137	91.30 ± 7.70	84.00	50.40 - 117.60	Pass	
N-2392	4/13/2015	H-3	5032 ± 214	5780	3468 - 8092	Pass	
W-2392	4/13/2015	Ni-63	222.4 ± 3.8	202.0	121.2 - 282.8	Pass	
W-2392	4/13/2015	Cs-134	53.26 ± 5.01	57.30	34.38 - 80.22	Pass	
W-2392	4/13/2015	Cs-137	91.90 ± 7.76	84.20	50.52 - 117.88	Pass	
W-042415	4/24/2015	Ra-226	12.52 ± 0.39	16.70	10.02 - 23.38	Pass	
N-050715	5/7/2015	Gr. Alpha	19.05 ± 0.41	20.10	12.06 - 28.14	Pass	
N-050715	5/7/2015	Gr. Beta	27.30 ± 0.32	30.90	18.54 - 43.26	Pass	
N-061215	6/12/2015	Gr. Alpha	20.72 ± 0.44	20.10	12.06 - 28.14	Pass	
N-061215	6/12/2015	Gr. Beta	28.51 ± 0.33	30.90	18.54 - 43.26	Pass	
U-2982	6/9/2015	Gr. Beta	500.1 ± 5.1	604.0	362.4 - 845.6	Pass	
U-3200	6/9/2015	H-3	2229 ± 424	2346	1408 - 3284	Pass	
W-70915	7/9/2015	Gr. Alpha	18.76 ± 0.40	20.10	12.1 - 28.1	Pass	
W-70915	7/9/2015	Gr. Beta	29.71 ± 0.33	30.90	18.5 - 43.3	Pass	
SPAP-3859	7/21/2015	Gr. Beta	41.59 ± 0.12	43.61	26.17 - 61.05	Pass	
SPAP-3861	7/21/2015	Cs-134	1.69 ± 0.60	1.69	1.0 - 2.4	Pass	

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			Concentration (p	Ci/L) ^a		
Lab Code ^b	Date	Analysis	Laboratory results 2s, n=1 °	Known Activity	Control Limits ^d	Acceptanc
SPAP-3861	7/21/2015	Cs-137	93.71 ± 2.64	96.45	57.87 - 135.03	Pass
SPMI-3863	7/21/2015	Cs-137 Cs-134	33.71 ± 2.04 38.21 ± 5.12	90.45 47.02	28.21 - 65.83	Pass
SPMI-3863	7/21/2015	Cs-134 Cs-137	78.65 ± 7.94	47.02 73.18	43.91 - 102.45	Pass Pass
SPMI-3863	7/21/2015	Sr-90	41.05 ± 1.62	37.78	22.67 - 52.89	Pass
SPW-3871	7/21/2015	Cs-134	41.05 ± 1.62 45.59 ± 6.39	47.02	22.07 - 52.09 28.21 - 65.83	Pass Pass
SPW-3871	7/21/2015	Cs-134 Cs-137	45.59 ± 6.59 78.73 ± 7.03	47.02 73.18	43.91 - 102.45	Pass Pass
SPW-3871	7/21/2015	Sr-90	38.36 ± 1.58	37.78	43.91 - 102.45 22.67 - 52.89	Pass
SPW-3873	7/21/2015	H-3	60,034 ± 671	57,199	34,319 - 80,079	Pass
SPW-3875	7/21/2015	Ni-63	451.3 ± 3.3	403.7		
					242.2 - 565.2	Pass
SPW-3877	7/21/2015	Tc-99	483.0 ± 8.3	539.1	323.5 - 754.7	Pass
SPMI-3879	7/21/2015	C-14	4,921 ± 19	4,736	2,842 - 6,630	Pass
SPSO-4037	7/21/2015	Ni-63	42,458 ± 309	40,370	24,222 - 56,518	Pass
SPW-072515	7/17/2015	Ra-228	35.48 ± 3	31.44	18.86 - 44.02	Pass
SPF-4104	7/29/2015	Cs-134	661.5 ± 115.9	740.0	444.0 - 1036.0	Pass
SPF-4104	7/29/2015	Cs-137	2,469 ± 59	2,340	1,404 - 3,276	Pass
SPW-81015	8/10/2015	Gr. Alpha	21.59 ± 0.46	20.10	12.06 - 28.14	Pass
SPW-81015	8/10/2015	Gr. Beta	27.58 ± 0.32	30.90	18.54 - 43.26	Pass
SPW-81315	8/13/2015	Ra-226	15.05 ± 0.36	16.70	10.02 - 23.38	Pass
SPW-90615	9/6/2015	Gr. Alpha	18.32 ± 0.40	20.10	12.06 - 28.14	Pass
SPW-90615	9/6/2015	Gr. Beta	29.43 ± 0.33	30.90	18.54 - 43.26	Pass
W-091415	9/14/2016	Gr. Alpha	19.35 ± 0.51	20.10	12.06 - 28.14	Pass
W-091415	9/14/2016	Gr. Beta	31.53 ± 0.35	30.90	18.54 - 43.26	Pass
W-100815	10/8/2015	Ra-228	12.27 ± 0.33	16.70	10.02 - 23.38	Pass
W-100615	10/6/2016	Gr. Alpha	20.62 ± 0.43	20.10	12.06 - 28.14	Pass
W-100615	10/6/2016	Gr. Beta	29.35 ± 0.33	30.90	18.54 - 43.26	Pass
W-5277	10/16/2015	H-3	5,224 ± 218	5,466	3,280 - 7,652	Pass
W-5277	10/16/2015	Cs-134	99.40 ± 6.64	99.20	59.52 - 138.88	Pass
W-5277	10/16/2015	Cs-137	89.60 ± 6.64	83.20	49.92 - 116.48	Pass
W-110415	11/4/2015	Ra-226	12.27 ± 0.33	16.70	10.02 - 23.38	Pass
W-111115	11/11/2015	Ra-228	31.78 ± 2.48	31.44	18.86 - 44.02	Pass
W-6086,6087	11/18/2015	H-3	10,882 ± 309	11,231	6,738 - 15,723	Pass
W-6086,6087	11/18/2015	Cs-134	92.98 ± 7.29	96.25	57.75 - 134.75	
W-6086,6087	11/18/2015	Cs-134 Cs-137	92.98 ± 7.29 76.65 ± 7.81	90.25 82.94	49.76 - 116.12	Pass
						Pass
W-112515 W-112515	11/25/2015 11/25/2015	Gr. Alpha Gr. Beta	20.91 ± 0.52 31.59 ± 0.35	20.10 30.90	12.06 - 28.14 18.54 - 43.26	Pass Pass
W-120715	12/7/2015	Fe-55	2 421 + 07	2 240	1 201 2 247	Dees
	12/7/2015		2,431 ± 97	2,319	1,391 - 3,247	Pass
W-120815	12/8/2015	Gr. Alpha	20.72 ± 0.43	20.10	12.06 - 28.14	Pass
W-120815	12/8/2015	Gr. Beta	29.50 ± 0.33	30.90	18.54 - 43.26	Pass
W-121515	12/15/2015	Ra-226	14.77 ± 0.42	16.70	10.02 - 23.38	Pass

TABLE A-3. In-House "Spiked" Samples

^a Liquid sample results are reported in pCi/Liter, air filters(pCi/m3), charcoal (pCi/charcoal canister), and solid samples (pCi/kg).

^b Laboratory codes : W (Water), MI (milk), AP (air filter), SO (soil), VE (vegetation), CH (charcoal canister), F (fish), U (urine).

° Results are based on single determinations.

^d Control limits are established from the precision values listed in Attachment A of this report, adjusted to ± 2s.

NOTE: For fish, Jello is used for the spike matrix. For vegetation, cabbage is used for the spike matrix.

TABLE A-4. In-House "Blank" Samples

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					Concentration (pCi/L) ^a			
Lab Code	Sample	Date	Analysis ^b	Laborato	ry results (4.66σ)	Acceptance		
<u></u>	Туре			LLD	Activity ^c	Criteria (4.66 o		
M 000215	Motor	0/2/2015	Bo 006	0.02	0.02 ± 0.02	4		
W-020315	Water	2/3/2015	Ra-226	0.03	0.03 ± 0.02	1		
W-021215	Water	2/12/2015	Gr. Alpha	0.47	-0.37 ± 0.30	2		
W-021215	Water	2/12/2015	Gr. Beta	0.76	-0.62 ± 0.51	4		
SPW-686	Water	2/27/2015	Ni-63	2.36	-0.74 ± 1.42	20		
SPAP-688	Air Particulate	3/2/2015	Gr. Beta	0.003	-0.001 ± 0.002	0.01		
SPAP-690	Air Particulate	3/2/2015	Cs-134	0.006	0.428 ± 0.927	0.05		
SPAP-690	Air Particulate	3/2/2015	Cs-137	0.006	-0.785 ± 1.146	0.05		
W-030215	Water	3/2/2015	Ra-228	0.76	0.22 ± 0.38	. 2		
SPW-692	Water	3/2Ì2015	Cs-134	6.70	-1.57 ± 3.55	10		
SPW-692	Water	3/2/2015	Cs-137	6.18	-0.15 ± 3.20	10		
SPW-692	Water	3/2/2015	Sr-89	0.61	-0.51 ± 0.51	5		
SPW-692	Water	3/2/2015	Sr-90	0.60	0.38 ± 0.33	1		
SPMI-696	Milk	3/2/2015	Cs-134	3.75	-0.25 ± 2.24	10		
SPMI-696	Milk	3/2/2015	Cs-137	4.36	-0.25 ± 2.24	10		
SPMI-696	Milk	3/2/2015	Sr-89	0.80	-0.40 ± 0.84	5		
SPMI-696	Milk	3/2/2015	Sr-90	0.49	0.98 ± 0.32	1		
SPW-698	Water	3/2/2015	H-3	144.0	28.6 ± 88.9	200		
SPW-1035	Water	3/16/2015	Fe-55	599.7	72.6 ± 368.1	1000		
SPW-1037	Water	3/16/2015	C-14	8.94	2.16 ± 5.47	200		
SPF-1039	Fish	3/16/2015	Cs-134	13.54	-1.00 ± 6.80	100		
SPF-1039	Fish	3/16/2015	Cs-137	9.80	4.87 ± 7.00	100		
		0,10,2010	00.01	0.00				
W-040615	Water	4/6/2015	Ra-226	0.04	0.01 ± 0.03	2		
W-1373	Water	4/6/2015	U-238	0.08	0.01 ± 0.01	1		
W-1375	Water	4/6/2015	Pu-238	0.03	0.00 ± 0.01	1		
W-050715	Water	5/7/2015	Gr. Alpha	0.38	-0.10 ± 0.25	. 2		
Ŵ-050715 Ŵ-050715	Water	5/7/2015	Gr. Beta	0.74	-0.14 ± 0.51	4		
W-061215	Water	6/12/2015	Gr. Alpha	0.42	-0.10 ± 0.29	4		
W-061215 W-061215	Water	6/12/2015	Gr. Beta	0.42	-0.04 ± 0.53	4		
		· .						
SPW-3858	Water	7/21/2015	Gr. Beta	0.003	0.004 ± 0.002	2		
SPAP-3860	Air Particulate	7/21/2015	Cs-134	0.011	0.010 ± 0.005	0.05		
SPAP-3860	Air Particulate	7/21/2015	Cs-137	0.009	0.000 ± 0.005	0.05		
SPMI-3862	Milk	7/21/2015	Cs-134	3.13 🗔	1.56 ± 1.74	10		
SPMI-3862	Milk	7/21/2015	Cs-137	3.20	1.69 ± 1.89	10		
SPMI-3862	Milk	7/21/2015	Sr-89	2.17	-1.30 ± 2.05	5		
SPMI-3862	Milk	7/21/2015	Sr-90	0.90	0.74 ± 0.50	1		
SPW-3870	Water	7/21/2015	Cs-134	3.01	0.71 ± 1.66	10		
SPW-3870	Water	7/21/2015	Cs-137	3.94	0.81 ± 1.86	10		
SPW-3870	Water	7/21/2015	Śr-89	2.28	-0.42 ± 1.80	5		
SPW-3870	Water	7/21/2015	Sr-90	0.84	0.25 ± 0.42	1		

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TABLE A-4. In-House "Blank" Samples

					Concentration (pCi/	L) ^a
Lab Code	Sample	Date	Analysis ^b	Laborator	y results (4.66σ)	Acceptance
	Туре			LLD	Activity ^c	Criteria (4.66 o
SPW-3872	Water	7/21/2015	H-3	142.6	82.7 ± 79.4	200
SPW-3874	Water	7/21/2015	Ni-63	2.98	0.77 ± 1.82	20
SPW-3876	Water	7/21/2015	Tc-99	5.49	-3.81 ± 3.26	10
SPW-3878	Water	7/21/2015	C-14	17.06	8.52 ± 10.54	200
SPSO-4036	Soil	7/21/2015	Ni-63	135.7	51.3 ± 83.0	1000
SPF-4103	Fish	7/29/2015	Cs-134	14.17	-37.70 ± 9.67	100
SPF-4103	Fish	7/29/2015	Cs-137	12.39	1.13 ± 8.06	100
W-081015	Water	8/10/2015	Gr. Alpha	0.48	-0.10 ± 0.33	2
W-081015	Water	8/10/2015	Gr. Beta	0.78	-0.18 ± 0.54	4
W-081815	Water	8/18/2015	Ra-226	0.03	0.03 ± 0.02	2
W-090615	Water	9/6/2015	Gr. Alpha	0.40	0.00 ± 0.28	2
W-090615	Water	9/6/2015	Gr. Beta	0.77	0.22 ± 0.54	4
W-091415	Water	9/14/2015	Gr. Alpha	0.41	0.10 ± 0.30	2.
W-091415	Water	9/14/2015	Gr. Beta	0.77	0.04 ± 0.54	4
W-100615	Water	10/6/2015	Gr. Alpha	0.41	-0.15 ± 0.27	2
W-100615	Water	10/6/2015	Gr. Beta	0.75	-0.12 ± 0.52	4
W-112515	Water	11/25/2015	Gr. Alpha	0.42	0.05 ± 0.30	2
W-112515	Water	11/25/2015	Gr. Beta	0.78	-0.31 ± 0.54	4
W-120815	Water	12/8/2015	Gr. Alpha	0.42	-0.08 ± 0.29	2
W-120815	Water	12/8/2015	Gr. Beta	0.76	0.17 ± 0.54	4
W-121515	Water	12/15/2015	Ra-226	0.01	0.01 ± 0.01	2

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^a Liquid sample results are reported in pCi/Liter, air filters(pCi/m³), charcoal (pCi/charcoal canister), and solid samples (pCi/kg).

^b I-131(G); iodine-131 as analyzed by gamma spectroscopy.

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^c Activity reported is a net activity result.

				Concentration (pCi/L) ^a		
			•		Averaged	
Lab Code	Date	Analysis	First Result	Second Result	Result	Acceptance
CF-62,63	1/7/2015	Gr. Beta	5.72 ± 0.12	5.78 ± 0.12	5.75 ± 0.42	Pass
CF-62,63	1/7/2015	Be-7	0.915 ± 0.135	0.919 ± 0.102	0.917 ± 0.15	Pass
CF-62,63	1/7/2015	K-40	3.97 ± 0.28	3.88 ± 0.23	3.92 ± 0.33	Pass
CF-62,63	1/7/2015	Sr-90	0.017 ± 0.006	0.011 ± 0.006	0.014 ± 0.004	Pass
SG-83,84	1/12/2015	K-40	10.11 ± 1.42	9.69 ± 1.20	9.90 ± 1.16	Pass
SG-83,84	1/12/2015	TI-208	0.57 ± 0.07	0.56 ± 0.06	0.57 ± 0.05	Pass
SG-83,84	1/12/2015	Pb-212	1.73 ± 0.10	1.58 ± 0.09	1.65 ± 0.13	Pass
SG-83,84	1/12/2015	Pb-214	13.33 ± 0.33	13.88 ± 0.28	13.61 ± 0.22	Pass
SG-83,84	1/12/2015	Bi-214	13.48 ± 0.39	13.45 ± 0.29	13.47 ± 0.24	Pass
SG-83,84	1/12/2015	Ra-226	25.68 ± 2.19	26.22 ± 1.53	25.95 ± 1.34	Pass
SG-83,84	1/12/2015	Ac-228	13.33 ± 0.59	12.86 ± 0.43	13.09 ± 0.36	Pass
AP-011215A/B	1/12/2015	Gr. Beta	0.025 ± 0.004	0.023 ± 0.004	0.024 ± 0.003	Pass
NW-315,316	1/27/2015	H-3	1,961 ± 178	1,868 ± 174	1,915 ± 124	Pass
DW-60010,60011	1/28/2015	Ra-226	1.25 ± 0.14	1.40 ± 0.15	1.33 ± 0.10	Pass
DW-60010,60011	1/28/2015	Ra-228	2.00 ± 0.66	1.39 ± 0.60	1.70 ± 0.45	Pass
SG-336,337	1/30/2015	Bi-214	6.63 ± 0.20	6.45 ± 0.45	6.54 ± 0.21	Pass
SG-336,337	1/30/2015	Pb-214	6.45 ± 0.19	6.45 ± 0.37	6.45 ± 0.21	Pass
SG-336,337	1/30/2015	Ac-228	4.43 ± 0.24	4.20 ± 0.58	4.32 ± 0.31	Pass
AP-020415A/B	2/4/2015	Gr. Beta	0.021 ± 0.004	0.019 ± 0.035	0.035 ± 0.020	Pass
AP-021115A/B	2/11/2015	Gr. Beta	0.034 ± 0.004	0.040 ± 0.047	0.037 ± 0.003	Pass
DW-60023,60024	2/26/2015	Ra-226	1.52 ± 0.15	1.51 ± 0.15	1.52 ± 0.11	Pass
DW-60023,60024	2/26/2015	Ra-228	0.97 ± 0.48	1.66 ± 0.58	1.32 ± 0.38	Pass
S-799,800	2/26/2015	K-40	11.96 ± 0.98	11.49 ± 0.82	11.72 ± 0.64	Pass
S-799,800	2/26/2015	TI-208	0.36 ± 0.04	0.31 ± 0.04	0.34 ± 0.03	Pass
S-799,800	2/26/2015	Pb-212	0.92 ± 0.06	0.91 ± 0.06	0.91 🛨 0.05	Pass
S-799,800	2/26/2015	Bi-212	1.26 ± 0.45	1.50 ± 0.40	1.38 ± 0.30	Pass
S-799,800	2/26/2015	Ac-228	1.35 ± 0.22	1.23 ± 0.17	1.29 ± 0.14	Pass
SG-834,835	2/2/2015	Gr. Alpha	113.3 ± 6.3	117.2 ± 2.8	115.2 ± 3.4	Pass
SG-834,835	2/2/2015	Gr. Beta	82.27 ± 2.79	84.33 ± 2.74	83.30 ± 1.96	Pass
DW-60031,60032	3/4/2015	Gr. Alpha	185.4 ± 7.4	177.0 ± 7.2	181.2 ± 5.2	Pass
DW-60036,60037	3/4/2015	Ra-226	6.89 ± 0.34	6.88 ± 0.32	6.89 ± 0.23	Pass
DW-60036,60037	3/4/2015	Ra-228	4.43 ± 0.73	4.41 ± 0.72	4.42 ± 0.51	Pass
DW-60048,60049	3/4/2015	Ra-226	0.84 ± 0.10	0.94 ± 0.11	0.89 ± 0.07	Pass
DW-60048,60049	3/4/2015	Ra-228	0.68 ± 0.41	1.42 ± 0.58	1.05 ± 0.36	Pass
AP-1169,1170	3/19/2015	Be-7	0.20 ± 0.02	0.24 ± 0.10	0.22 ± 0.07	Pass
DW-60069,60070	4/8/2015	Gr. Alpha	3.58 ± 0.88	3.92 ± 0.88	3.75 ± 0.62	Pass
AP-040915	4/9/2015	Gr. Beta	0.027 ± 0.005	0.023 ± 0.005	0.025 ± 0.003	Pass
WW-2394,2395	4/13/2015	H-3	1,628 ± 139	1,695 ± 141	1,662 ± 99	Pass
SG-1847,1848	4/20/2015	K-40	3.24 ± 1.18	1.99 ± 0.76	2.62 ± 0.70	Pass
SG-1847,1848	4/20/2015	Pb-214	5.80 ± 0.22	6.23 ± 0.76	6.02 ± 0.40	Pass
SG-1847,1848	4/20/2015	Ac-228	5.26 ± 0.51	5.00 ± 0.42	5.13 ± 0.33	Pass
XWW-2267,2268	4/23/2015	H-3	6,584 ± 244	6,164 ± 237	6,374 ± 170	Pass
XWW-2078,2079	4/27/2015	H-3	359.0 ± 89.6	418.7 ± 92.3	388.9 ± 64.3	Pass

			(Concentration (pCi/L) ^a		
					Averaged	
Lab Code	Date	Analysis	First Result	Second Result	Result	Acceptance
XWW-2162,2163	4/28/2015	H-3	4,408 ± 201	4,242 ± 198	4,325 ± 141	Pass
SG-1868,1869	4/28/2015	Gr. Alpha	47.57 ± 3.63	43.61 ± 3.58	45.59 ± 2.55	Pass
SG-1868,1869	4/28/2015	Gr. Beta	50.90 ± 1.94	51.90 ± 2.02	51.40 ± 1.40	Pass
SG-1868,1869	4/28/2015	Pb-214	13.80 ± 0.52	13.54 ± 0.62	13.67 ± 0.40	Pass
SG-1868,1869	4/28/2015	Ra-228	20.10 ± 0.92	22.10 ± 1.29	21.10 ± 0.79	Pass
AP-042915	4/29/2015	Gr. Beta	0.014 ± 0.003	0.014 ± 0.003	0.014 ± 0.002	Pass
DW-60076,60077	5/4/2015	Ra-228	2.89 ± 0.61	2.45 ± 0.57	2.67 ± 0.42	Pass
AP-050515	5/5/2015	Gr. Beta	0.026 ± 0.004	0.025 ± 0.004	0.026 ± 0.003	Pass
AP-051115	5/11/2015	Gr. Beta	0.006 ± 0.005	0.010 ± 0.005	0.008 ± 0.004	Pass
DW-60087,60088	5/14/2015	Ra-226	1.58 ± 0.17	1.52 ± 0.17	1.55 ± 0.12	Pass
DW-60087,60088	5/14/2015	Ra-228	0.94 ± 0.50	0.94 ± 0.50	0.94 ± 0.35	Pass
SG-2436,2437	5/15/2015	Pb-214	22.90 ± 2.31	24.10 ± 2.43	23.50 ± 1.68	Pass
SG-2436,2437	5/15/2015	Ra-228	47.95 ± 0.61	47.80 ± 0.71	47.88 ± 0.47	Pass
SG-2436,2437	5/15/2015	Gr. Alpha	267.8 ± 7.9	254.6 ± 7.6	261.2 ± 5.5	Pass
SG-2458,2459	5/19/2015	Pb-214	75.00 ± 1.66	77.70 ± 1.75	76.35 ± 1.21	Pass
SG-2458,2459	5/19/2015	Ra-228	41.10 ± 0.92	40.80 ± 0.83	40.95 ± 0.62	Pass
DW-60095,60096	5/26/2015	Gr. Alpha	1.34 ± 0.69	0.91 ± 0.62	1.13 ± 0.46	Pass
AP-052715	5/27/2015	Gr. Beta	0.010 ± 0.003	0.010 ± 0.003	0.010 ± 0.002	Pass
S-2627,2628	5/29/2015	Pb-214	0.85 ± 0.07	0.85 ± 0.07	0.85 ± 0.05	Pass
S-2627,2628	5/29/2015	Ac-228	0.85 ± 0.14	1.08 ± 0.12	0.97 ± 0.09	Pass
S-2627,2628	5/29/2015	Cs-137	0.07 ± 0.02	0.07 ± 0.02	0.07 ± 0.01	Pass
S-2605,2606	6/1/2015	Ac-228	0.42 ± 0.06	0.38 ± 0.07	0.40 ± 0.05	Pass
S-2605,2606	6/1/2015	Ra-226	0.44 ± 0.03	0.49 ± 0.03	0.47 ± 0.02	Pass
S-2605,2606	6/1/2015	K-40	10.89 ± 0.51	11.40 ± 0.48	11.15 ± 0.35	Pass
S-2605,2606	6/1/2015	Cs-137	0.05 ± 0.01	0.05 ± 0.01	0.05 ± 0.01	Pass
S-2858,2859	6/2/2015	Cs-137	34.30 ± 16.05	40.66 ± 17.79	37.48 ± 11.98	Pass
S-2858,2859	6/2/2015	Be-7	1501 ± 264	1171 ± 214	1336 ± 170	Pass
S-2858,2859	6/2/2015	K-40	22,122 ± 658	20,987 ± 600	21,555 ± 445	Pass
AP-060315	6/3/2015	Gr. Beta	0.022 ± 0.004	0.021 ± 0.004	0.022 ± 0.003	Pass
DW-30107,30108	6/8/2015	Gr. Alpha	1.34 ± 0.82	1.47 ± 0.85	1.41 ± 0.59	Pass
SG-2900,2901	6/9/2015	Ac-228	10.22 ± 1.36	8.32 ± 1.07	9.27 ± 0.87	Pass
SG-2900,2901	6/9/2015	Pb-214	7.55 ± 0.43	7.27 ± 0.41	7.41 ± 0.30	Pass
AP-061515	6/15/2015	Gr. Beta	0.022 ± 0.004	0.021 ± 0.004	0.022 ± 0.003	Pass
XWW-3173,3174	6/18/2015	H-3	841.9 ± 123.6	799.3 ± 122.4	820.6 ± 87.0	Pass
AP-062215	6/22/2015	Gr. Beta	0.023 ± 0.004	0.018 ± 0.004	0.020 ± 0.003	Pass
S-3216,3217	6/24/2015	K-40	10.38 ± 0.51	10.51 ± 0.53	10.45 ± 0.37	Pass
S-3216,3217	6/24/2015	Be-7	3.65 ± 0.24	3.38 ± 0.27	3.52 ± 0.18	Pass
VE-3300,3301	6/24/2015	Be-7	0.78 ± 0.15	0.83 ± 0.23	0.81 ± 0.14	Pass
VE-3300,3301	6/24/2015	K-40	29.12 ± 0.62	29.36 ± 0.64	29.24 ± 0.45	Pass
AP-062915	6/29/2015	Gr. Beta	0.023 ± 0.005	0.023 ± 0.005	0.023 ± 0.003	Pass
WW-3632,3633	6/30/2015	H-3	5,169 ± 225	5,058 ± 223	5,114 ± 158	Pass

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		-	Concentration (pCi/L) ^a					
					Averaged			
Lab Code	Date	Analysis	First Result	Second Result	Result	Acceptanc		
AP-3822, 3823	7/1/2015	Be-7	0.075 ± 0.011	0.068 ± 0.012	0.072 ± 0.008	Pass		
AP-3969, 3970	7/1/2015	Be-7	0.063 ± 0.008	0.064 ± 0.010	0.063 ± 0.006	Pass		
WW-3632, 3633	7/6/2015	H-3	5,169 ± 225	5,058 ± 223	5,114 ± 159	Pass		
N-4368, 4369	7/6/2015	Gr. Alpha	26.70 ± 4.00	24.10 ± 3.90	25.40 ± 2.79	Pass		
N-4368, 4369	7/6/2015	Gr. Beta	34.62 ± 2.10	33.30 ± 2.02	33.96 ± 1.46	Pass		
DW-60138, 60139	7/7/2015	Ra-226	0.07 ± 0.04	0.11 ± 0.05	0.09 ± 0.03	Pass		
DW-60138, 60139	7/7/2015	Ra-228	1.04 ± 0.41	1.15 ± 0.47	1.10 ± 0.31	Pass		
NW-4158, 4159	7/9/2015	H-3	138.8 ± 82.4	174.0 ± 84.1	156.4 ± 58.9	Pass		
MI-2902, 2903	7/10/2015	K-40	1271 ± 118	1308 ± 115	1289 ± 82	Pass		
SG-3533, 3534	7/10/2015	Gr. Alpha	238.0 ± 8.2	249.5 ± 8.5	243.8 ± 5.9	Pass		
DW-60150, 60151	7/10/2015	Ra-226	1.53 ± 0.16	1.49 ± 0.12	1.51 ± 0.10	Pass		
DW-60150, 60151	7/10/2015	Ra-228	2.68 ± 0.68	1.89 ± 0.62	2.29 ± 0.46	Pass		
/E-3716, 3717	7/14/2015	K-40	3.85 ± 0.33	3.71 ± 0.31	3.78 ± 0.23	Pass		
MI-3759, 3760	7/15/2015	K-40	1819 ± 127	1764 ± 140	1791 ± 94	Pass		
MI-3759, 3760	7/15/2015	Sr-90	1.00 ± 0.36	0.61 ± 0.32	0.80 ± 0.24	Pass		
AP-072115	7/21/2015	Gr. Beta	0.022 ± 0.004	0.027 ± 0.004	0.024 ± 0.003	Pass		
VE-4053, 4054	7/21/2015	Be-7	0.52 ± 0.15	0.49 ± 0.11	0.50 ± 0.09	Pass		
VE-4053, 4054	7/21/2015	K-40	8.00 ± 0.42	7.61 ± 0.31	7.81 ± 0.26	Pass		
AP-4200, 4201	7/29/2015	Be-7	1.06 ± 0.12	0.96 ± 0.11	1.01 ± 0.08	Pass		
AP-4200, 4201	7/29/2015	K-40	5.03 ± 0.24	4.96 ± 0.23	4.99 ± 0.16	Pass		
N-4137, 4138	7/31/2015	Ra-226	0.58 ± 0.13	0.45 ± 0.14	0.52 _, ± 0.10	Pass		
XWW-4431, 4432	8/5/2015	H-3	4,773 ± 213	4,915 ± 216	4,844 ± 152	Pass		
SG-4305, 4306	8/6/2015	Ra-228	10.34 ± 0.58	11.46 ± 0.62	10.90 ± 0.42	Pass		
AP-081015	8/10/2015	Gr. Beta	0.038 ± 0.005	0.039 ± 0.005	0.039 0.004	Pass		
AP-081115	8/11/2015	Gr. Beta	0.024 ± 0.004	0.020 ± 0.004	0.022 0.003	Pass		
VE-4452, 4453	8/11/2015	K-40	3.77 ± 0.29	3.78 ± 0.26	3.77 ± 0.20	Pass		
AP-081715	8/17/2015	Gr. Beta	0.030 ± 0.005	0.030 ± 0.005	0.030 ± 0.003	Pass		
DW-60195, 60196	8/17/2015	Ra-226	0.39 ± 0.10	0.37 ± 0.10	0.38 ± 0.07	Pass		
DW-60195, 60196	8/17/2015	Ra-228	1.43 ± 0.51	1.97 ± 0.61	1.70 ± 0.40	Pass		
DW-60198, 60199	8/17/2015	Gr. Alpha	2.93 ± 0.94	2.11 ± 0.96	2.52 ± 0.67	Pass		
vE-4578, 4579	8/18/2015	К-40	4.14 ± 0.25	4.32 ± 0.24	4.23 ± 0.17	Pass		
SW-4662, 4663	8/25/2015	H-3	351.3 ± 89.8	415.6 ± 92.8	383.4 ± 64.6	Pass		
DW-60212, 60213	8/25/2015	Ra-226	0.09 ± 0.07	0.10 ± 0.08	0.10 ± 0.05	Pass		
LW-4788, 4789	8/27/2015	Gr. Beta	0.97 ± 0.51	1.68 ± 0.59	1.32 ± 0.39	Pass		
AP-083115	8/31/2015	Gr. Beta	0.032 ± 0.005	0.031 ± 0.005	0.031 ± 0.003	Pass		
AP-4875, 4876	9/3/2015	Be-7	0.294 ± 0.125	0.202 ± 0.109	0.248 ± 0.083	Pass		
VE-5083, 5084	9/14/2015	Be-7	0.47 ± 0.23	0.56 ± 0.19	0.52 ± 0.15	Pass		
VE-5083, 5084	9/14/2015	K-40	6.20 ± 0.51	6.36 ± 0.50	6.28 ± 0.36	Pass		
VE-5167, 5168	9/16/2015	Be-7	0.40 ± 0.11	0.41 ± 0.10	0.41 ± 0.07	Pass		
√E-5167, 5168	9/16/2015	K-40	3.56 ± 0.27	3.91 ± 0.24	3.74 ± 0.18	Pass		
BS-5188, 5189	9/16/2015	K-40	9.69 ± 0.51	10.51 ± 0.52	10.10 ± 0.36	Pass		
F-5419, 5420	9/17/2015	K-40	3.48 ± 0.47	3.49 ± 0.56	3.49 ± 0.36	Pass		
DW-60238, 60239	9/18/2015	Ra-226	1.93 ± 0.23	2.31 ± 0.26	2.12 ± 0.17	Pass		
DW-60238, 60239	9/18/2015	Ra-228	4.44 ± 0.78	5.61 ± 0.84	5.03 ± 0.57	Pass		
AP-092215A/B	9/22/2015	Gr. Beta	0.021 ± 0.004	0.025 ± 0.004	0.023 ± 0.00	Pass		
WW-5398, 5399	9/22/2015	H-3	1,857 ± 145	$1,846 \pm 144$	$1,852 \pm 102$	Pass		
AP-6007, 6008	9/28/2015	Be-7	0.08 ± 0.01	0.08 ± 0.01	0.08 ± 0.01	Pass		

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			Concentration (pCi/L) ^a					
		Analysis			Averaged			
Lab Code	Date		First Result	Second Result	Result	Acceptance		
XW-7490, 7491	9/29/2015	Ni-63	2,332 ± 233	2,108 ± 211	2,220 ± 157	Pass		
WW-5377, 5378	9/30/2015	H-3	220.0 ± 84.6	197.0 ± 83.5	208.5 ± 59.4	Pass		
AP-6028, 6029	9/30/2015	Be-7	0.073 ± 0.009	0.083 ± 0.012	0.078 ± 0.007	Pass		
G-5461,2	10/1/2015	Be-7	2.02 ± 0.32	1.98 ± 0.25	2.00 ± 0.20	Pass		
G-5461,2	10/1/2015	K-40	8.77 ± 0.66	9.31 ± 0.59	9.04 ± 0.44	Pass		
SO-5482, 5483	10/1/2015	Ac-228	0.76 ± 0.12	0.74 ± 0.30	0.75 ± 0.16	Pass		
SO-5482, 5483	10/1/2015	Bi-214	0.53 ± 0.04	0.52 ± 0.04	0.52 ± 0.03	Pass		
SO-5482, 5483	10/1/2015	Cs-137	0.12 ± 0.03	0.12 ± 0.03	0.12 ± 0.02	Pass		
SO-5482, 5483	10/1/2015	K-40	2.17 ± 0.73	2.10 ± 0.72	2.13 ± 0.51	Pass		
SO-5482, 5483	10/1/2015	Pb-214	0.57 ± 0.04	0.55 ± 0.04	0.56 ± 0.03	Pass		
SO-5482, 5483	10/1/2015	Ra-226	1.45 ± 0.27	1.46 ± 0.30	1.45 ± 0.20	Pass		
SO-5482, 5483	10/1/2015	TI-208	0.24 ± 0.03	0.25 ± 0.03	0.24 ± 0.02	Pass		
WW-5524, 5525	10/5/2015	H-3	1,192 ± 123	1,318 ± 127	$1,255 \pm 89$	Pass		
AP-5881, 5882	10/5/2015	Be-7	0.078 ± 0.008	0.085 ± 0.011	0.082 ± 0.007	Pass		
AP-5881, 5882	10/5/2015	K-40	0.009 ± 0.000	0.010 ± 0.006	0.002 ± 0.007 0.010 ± 0.004	Pass		
SG-6400,1	10/5/2015	Gr. Alpha	19.09 ± 3.14	19.45 ± 3.25	19.27 ± 2.26	Pass		
SG-6400,1	10/5/2015	Gr. Beta	31.36 ± 2.08	29.80 ± 2.13	30.58 ± 1.49	Pass		
•	10/12/2015	K-40	4.29 ± 0.29		4.21 ± 0.22	Pass		
VE-5923, 5924	10/12/2015	Ac-228	4.29 ± 0.29 0.20 ± 0.06	4.13 ± 0.33 0.24 ± 0.06	4.21 ± 0.22 0.22 ± 0.04	Pass		
SS-5818, 5819		AC-220 Cs-137						
SS-5818, 5819	10/14/2015		0.03 ± 0.02	0.02 ± 0.01	0.03 ± 0.01	Pass		
SS-5818, 5819	10/14/2015	Gr. Beta	8.10 ± 0.87	8.08 ± 0.96	8.09 ± 0.65	Pass		
SS-5818, 5819	10/14/2015	Pb-212	0.19 ± 0.03	0.17 ± 0.02	0.18 ± 0.02	Pass		
SS-5818, 5819	10/14/2015	Ra-226	0.47 ± 0.24	0.45 ± 0.19	0.46 ± 0.15	Pass		
SS-5818, 5819	10/14/2015	TI-208	0.06 ± 0.02	0.06 ± 0.02	0.06 ± 0.01	Pass		
DW-60251, 60252	10/15/2015	Ra-226	0.56 ± 0.12	0.50 ± 0.08	0.53 ± 0.07	Pass		
DW-60251, 60252	10/15/2015	Ra-228	0.79 ± 0.48	1.16 ± 0.59	0.98 ± 0.38	Pass		
SO-5944, 5945	10/21/2015	Ac-228	1.08 ± 0.15	1.14 ± 0.15	1.11 ± 0.10	Pass		
SO-5944, 5945	10/21/2015	Bi-214	0.89 ± 0.08	0.82 ± 0.06	0.85 ± 0.05	Pass		
SO-5944, 5945	10/21/2015	Cs-137	0.06 ± 0.02	0.08 ± 0.03	0.07 ± 0.02	Pass		
SO-5944, 5945	10/21/2015	Pb-212	1.06 ± 0.06	0.99 ± 0.05	1.03 ± 0.04	Pass		
SO-5944, 5945	10/21/2015	Pb-214	1.00 ± 0.09	0.89 ± 0.06	0.95 ± 0.05	Pass		
SO-5944, 5945	10/21/2015	Ra-226	2.13 ± 0.43	2.16 ± 0.37	2.14 ± 0.28	Pass		
SO-5944, 5945	10/21/2015	TI-208	0.36 ± 0.04	0.34 ± 0.04	0.35 ± 0.03	Pass		
S-6175, 6176	10/23/2015	K-40	16.86 ± 1.92	14.28 ± 1.66	15.57 ± 1.27	Pass		
XWW-6196, 6197	10/26/2015	H-3	; 2,856 ± 170	$2,815 \pm 169$	$2,836 \pm 120$	Pass		
SO-6259, 6260	10/28/2015	Ac-228	0.60 ± 0.10	0.53 ± 0.08	0.57 ± 0.07	Pass		
SO-6259, 6260	10/28/2015	Bi-214	0.40 ± 0.06	0.50 ± 0.05	0.45 ± 0.04	Pass		
SO-6259, 6260	10/28/2015	Cs-137	0.17 ± 0.03	0.19 ± 0.03	0.18 ± 0.02	Pass		
SO-6259, 6260	10/28/2015	Gr. Beta	21.6 ± 1.1	23.36 ± 1.21	22.48 ± 0.82	Pass		
SO-6259, 6260	10/28/2015	Pb-212	0.53 ± 0.04	0.49 ± 0.04	0.51 ± 0.03 0.18 ± 0.02	Pass		
SO-6259, 6260	10/28/2015	TI-208	0.16 ± 0.03	0.19 ± 0.04	0.10 ± 0.02	Pass		

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	•		Concentration (pCi/L) ^a				
					Averaged		
Lab Code	Date	Analysis	First Result	Second Result	Result	Acceptance	
LW-6280, 6281	10/29/2015	Gr. Beta	2.03 ± 0.91	1.97 ± 0.97	2.00 ± 0.67	Pass	
MI-6484, 6485	11/11/2015	K-40	1,384 ± 82	1,432 ± 89	1,408 ± 60	Pass	
SO-6841, 6842	11/24/2015	Cs-137	0.18 ± 0.03	0.16 ± 0.03	0.17 ± 0.02	Pass	
SO-6841, 6842	11/24/2015	K-40	13.62 ± 0.76	13.67 ± 0.69	13.64 ± 0.51	Pass	
WW-6978, 6979	11/30/2015	H-3	569.0 ± 97.7	480.3 ± 93.9	524.7 ± 67.8	Pass	
SW-6936, 6937	12/10/2015	H-3	151.9 ± 80.0	176.2 ± 81.2	164.0 ± 57.0	Pass	
SW-7017, 7018	12/10/2015	H-3	584.3 ± 98.7	451.6 ± 93.9	518.0 ± 68.1	Pass	
_W-7020, 7021	12/10/2015	H-3	236.9 ± 84.2	285.6 ± 86.5	261.2 ± 60.3	Pass	
AP-7351, 7352	12/29/2015	Be-7	0.099 ± 0.020	0.084 ± 0.018	0.091 ± 0.014	Pass	
AP-7414, 7415	12/30/2015	Be-7	0.049 ± 0.013	0.048 ± 0.011	0.048 ± 0.008	Pass	

Note: Duplicate analyses are performed on every twentieth sample received in-house. Results are not listed for those

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analyses with activities that measure below the LLD.

^a Results are reported in units of pCi/L, except for air filters (pCi/Filter or pCi/m3), food products, vegetation, soil, sediment (pCi/g).

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			Concentration ^a						
				Known	Control				
Lab Code ^b	Date	Analysis	Laboratory result	Activity	Limits ^c	Acceptance			
MASO-975	2/1/2015	Ni-63	341 ± 18	448	314 - 582	Pass			
MASO-975	2/1/2015	Sr-90	523 ± 12	653	457 - 849	Pass			
MASO-975	2/1/2015	Tc-99	614 ± 12	867	607 - 1,127	Pass			
MASO-975	2/1/2015	Cs-134	533 ± 6	678	475 - 881	Pass			
MASO-975	2/1/2015	Cs-137	0.8 ± 2.5	0.0	NA ^c	Pass			
MASO-975	2/1/2015	Co-57	0.5 ± 1.0	0.0	NA °	Pass			
MASO-975	2/1/2015	Co-60	741 ± 8	817	572 - 1,062	Pass			
MASO-975	2/1/2015	Mn-54	1,153 ± 9	1,198	839 - 1,557	Pass			
MASO-975	2/1/2015	Zn-65	892 ± 18	1064	745 - 1,383	Pass			
MAW-969	2/1/2015	Am-241	0.650 ± 0.078	0.654	0.458 - 0.850	Pass			
MAW-969	2/1/2015	Cs-134	21.1 ± 0.3	23.5	16.5 - 30.6	Pass			
MAW-969	2/1/2015	Cs-137	19.6 ± 0.3	19.1	13.4 - 24.8	Pass			
MAW-969 ^d	2/1/2015	Co-57	10.2 ± 0.4	29.9	20.9 - 38.9	Fail			
MAW-969	2/1/2015	Co-60	0.02 ± 0.05	0.00	NA °	Pass			
MAW-969	2/1/2015	H-3	569 ± 13	563	394 - 732	Pass			
MAW-969	2/1/2015	Fe-55	6.00 ± 6.60	6.88	4.82 - 8.94	Pass			
MAW-969	2/1/2015	Mn-54	0.02 ± 0.07	0.00	NA °	Pass			
MAW-969	2/1/2015	Ni-63	2.9 ± 3.0	0.00	NA °	Pass			
MAW-969	2/1/2015	Zn-65	16.5 ± 0.9	18.3	12.8 - 23.8	Pass			
MAW-969	2/1/2015	Tc-99	3.40 ± 0.60	3.18	2.23 - 4.13	Pass			
MAW-969	2/1/2015	Pu-238	0.02 ± 0.03	0.01	NA °	Pass			
MAW-969	2/1/2015	Pu-239/240	0.81 ± 0.10	0.83	0.58 - 1.08	Pass			
MAW-969	2/1/2015	U-233/234	0.150 ± 0.040	0.148	0.104 - 0.192	Pass			
MAW-969	2/1/2015	U-238	0.84 ± 0.09	0.97	0.68 - 1.26	Pass			
MAW-969	2/1/2015	Sr-90	9.40 ± 1.30	9.48	6.64 - 12.32	Pass			
MAW-950	2/1/2015	Gr. Alpha	0.66 ± 0.05	1.07	0.32 - 1.81	Pass			
MAW-950	2/1/2015	Gr. Beta	2.72 ± 0.06	2.79	1.40 - 4.19	Pass			
MAW-947	2/1/2015	I-129	1.26 ± 0.12	1.49	1.04 - 1.94	Pass			
MAAP-978	2/1/2015	Am-241	0.069 ± 0.200	0.068	0.048 - 0.089	Pass			
MAAP-978	2/1/2015	Cs-134	1.00 ± 0.04	1.15	0.81 - 1.50	Pass			
MAAP-978	2/1/2015	Cs-137	0.004 ± 0.023	0.00	NA °	Pass			
MAAP-978 ^f	2/1/2015	Co-57	0.04 ± 0.04	1.51	1.06 - 1.96	Fail			
MAAP-978	2/1/2015	Co-60	0.01 ± 0.02	0.00	NA °	Pass			
MAAP-978	2/1/2015	Mn-54	1.11 ± 0.08	1.02	0.71 - 1.33	Pass			
MAAP-978	2/1/2015	Zn-65	0.83 ± 0.10	0.83	0.58 - 1.08	Pass			
MAAP-978	2/1/2015	Pu-238	-0.003 ± 0.010	0.000	NA ^c	Pass			
MAAP-978	2/1/2015	Pu-239/240	0.090 ± 0.022	0.085	0.059 - 0.110	Pass			
MAAP-978	2/1/2015	U-233/234	0.020 ± 0.010	0.016	0.011 - 0.020	Pass			
MAAP-978	2/1/2015	U-238	0.073 ± 0.018	0.099	0.069 - 0.129	Pass			

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TABLE A-6. Department of Energy's Mixed Analyte Performance Evaluation Program (MAPEP).

		Concentration ^a						
				Known	Contro!			
Lab Code ^b	Date	Analysis	Laboratory result	Activity	Limits ^c	Acceptanc		
MAAP-981	2/1/2015	Sr-89	38.1 ± 1.0	47.5	33.3 - 61.8	Pass		
MAAP-981	2/1/2015	Sr-90	1.22 ± 0.13	1.06	0.74 - 1.38	Pass		
MAAP-984	2/1/2015	Gr. Alpha	0.59 ± 0.06	1.77	0.53 - 3.01	Pass		
MAAP-984	2/1/2015	Gr. Beta	0.95 ± 0.07	0.75	0.38 - 1.13	Pass		
MAVE-972	2/1/2015	Cs-134	6.98 ± 0.13	7.32	5.12 - 9.52	Pass		
MAVE-972	2/1/2015	Cs-137	9.73 ± 0.21	9.18	6.43 - 11.93	Pass		
MAVE-972	2/1/2015	Co-57	0.01 ± 0.04	0.00	NA °	Pass		
MAVE-972	2/1/2015	Co-60	3.89 ± 0.20	5.55	3.89 - 7.22	Pass		
MAVE-972	2/1/2015	Mn-54	0.04 ± 0.07	0.00	NA [°]	Pass		
MAVE-972	2/1/2015	Zn-65	0.09 ± 0.12	0.00	NA °	Pass		
MAAP-978	2/1/2015	Pu-238	-0.003 ± 0.010	0.000	NA °	Pass		
MAAP-978	2/1/2015	Pu-239/240	0.090 ± 0.022	0.085	0.059 - 0.110	Pass		
MAAP-978	2/1/2015	U-233/234	0.020 ± 0.010	0.016	0.011 - 0.020	Pass		
MAAP-978	2/1/2015	U-238	0.073 ± 0.018	0.099	0.069 - 0.129	Pass		
MAAP-981	2/1/2015	Sr-89	38.1 ± 1.0	47.5	33.3 - 61.8	Pass		
MAAP-981	2/1/2015	Sr-90	1.22 ± 0.13	1.06	0.74 - 1.38	Pass		
MAAP-984	2/1/2015	Gr. Alpha	0.59 ± 0.06	1.77	0.53 - 3.01	Pass		
MAAP-984	2/1/2015	Gr. Beta	0.95 ± 0.07	0.75	0.38 - 1.13	Pass		
MAVE-972	2/1/2015	Cs-134	6.98 ± 0.13	7.32	5.12 - 9.52	Pass		
MAVE-972	2/1/2015	Cs-137	9.73 ± 0.21	9.18	6.43 - 11.93	Pass		
MAVE-972	2/1/2015	Co-57	0.01 ± 0.04	0.00	NA °	Pass		
MAVE-972	2/1/2015	Co-60	3.89 ± 0.20	5.55	3.89 - 7.22	Pass		
MAVE-972	2/1/2015	Mn-54	0.04 ± 0.07	0.00	NA °	Pass		
MAVE-972	2/1/2015	Zn-65	0.09 ± 0.12	0.00	NA ^c	Pass		
MASO-4903	8/1/2015	Ni-63	556 ± 18	682	477 - 887	Pass		
MASO-4903 ⁹	8/1/2015	Sr-90	231 ± 7	425	298 - 553	Fail		
MASO-4903 ^g	8/1/2015	Sr-90	352 ± 10	425	298 - 553	Pass		
MASO-4903 ^h	8/1/2015	Tc-99	411 ± 11	631	442 - 820	Fail		
MASO-4903	8/1/2015	Cs-134	833 ± 10	1,010	707 - 1,313	Pass		
MASO-4903	8/1/2015	Cs-137	808 ± 11	809.00	566 - 1,052	Pass		
MASO-4903	8/1/2015	Co-57	1,052 ± 10	1,180	826 - 1,534	Pass		
MASO-4903	8/1/2015	Co-60	2 ± 2	1.3	NA®	Pass		
MASO-4903	8/1/2015	Mn-54	1,331 ± 13	1,340	938 - 1,742	Pass		
MASO-4903	8/1/2015	Zn-65	686 ± 15	662	463 - 861	Pass		

TABLE A-6. Department of Energy's Mixed Analyte Performance Evaluation Program (MAPEP).

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				Concentration	a	
				Known	Control	``
Lab Code ^b	Date	Analysis	Laboratory result	Activity	Limits ^c	Acceptance
MAW-5007	8/1/2015	Cs-134	16.7 ± 0.4	23.1	` 16.2 - 30.0	Pass
MAW-5007	8/1/2015	Cs-137	-0.4 ± 0.1	0.0	NA °	Pass
MAW-5007	8/1/2015	Co-57	21.8 ± 0.4	20.8	14.6 - 27.0	Pass
MAW-5007	8/1/2015	Co-60	17.3 ± 0.3	17.1	12.0 - 22.2	Pass
MAW-5007	8/1/2015	H-3	227.5 ± 8.9	216.0	151.0 - 281.0	Pass
MAW-5007 ⁱ	8/1/2015	Fe-55	4.2 ± 14.1	13.1	9.2 - 17.0	Fail
MAW-5007	8/1/2015	Mn-54	16.6 ± 0.5	15.6	10.9 - 20.3	Pass
MAW-5007	8/1/2015	Ni-63	9.1 ± 2.6	8.6	6.0 - 11.1	Pass
MAW-5007	8/1/2015	Zn-65	15.5 ± 0.9	13.9	9.7 - 18.1	Pass
MAW-5007	8/1/2015	Tc-99	6.80 ± 0.60	7.19	5.03 - 9.35	Pass
MAW-5007	8/1/2015	Sr-90	4.80 ± 0.50	4.80	3.36 - 6.24	Pass
MAW-5007	8/1/2015	Gr. Alpha	0.41 ± 0.04	0.43	0.13 - 0.73	Pass
MAW-5007	8/1/2015	Gr. Beta	3.45 ± 0.07	3.52	1.76 - 5.28	Pass
MAW-5007	8/1/2015	I-129	1.42 ± 0.13	1.49	1.04 - 1.94	Pass
MAAP-4911	8/1/2015	Sr-89	3.55 ± 0.67	3.98	2.79 - 5.17	Pass
MAAP-4911	8/1/2015	Sr-90	0.94 ± 0.16	1.05	0.74 - 1.37	Pass
MAAP-4907	8/1/2015	Gr. Alpha	0.30 ± 0.04	0.90	0.27 - 1.53	Pass
MAAP-4907	8/1/2015	Gr. Beta	1.85 ± 0.09	1.56	0.78 - 2.34	Pass
MAVE-4901	8/1/2015	Cs-134	5.56 ± 0.16	5.80	4.06 - 7.54	Pass
MAVE-4901	8/1/2015	Cs-137	-0.02 ± 0.06	0.00	NA °	Pass
MAVE-4901	8/1/2015	Co-57	7.74 ± 0.18	6.62	4.63 - 8.61	Pass
MAVE-4901	8/1/2015	Co-60	4.84 ± 0.15	4.56	3.19 - 5.93	Pass
MAVE-4901	8/1/2015	Mn-54	8.25 ± 0.25	7.68	5.38 - 9.98	Pass
MAVE-4901	8/1/2015	Zn-65	5.78 ± 0.29	5.46	3.82 - 7.10	Pass

TABLE A-6. Department of Energy's Mixed Analyte Performance Evaluation Program (MAPEP).

^a Results are reported in units of Bq/kg (soil), Bq/L (water) or Bq/total sample (filters, vegetation).

^b Laboratory codes as follows: MAW (water), MAAP (air filter), MASO (soil), MAVE (vegetation).

^c MAPEP results are presented as the known values and expected laboratory precision (1 sigma, 1 determination) and control limits as defined by the MAPEP. A known value of "zero" indicates an analysis was included in the testing series as a "false positive". MAPEP does not provide control limits.

^d Lab result was 27.84. Data entry error resulted in a non-acceptable result.

^e Provided in the series for "sensitivity evaluation". MAPEP does not provide control limits.

^f Lab result was 1.58. Data entry error resulted in a non-acceptable result.

⁹ The incomplete separation of calcium from strontium caused a failed low result. The result of reanalysis acceptable.

^h The complex sample matrix is interfering with yield calculations causing a failed low result. An investigation is in process to determine a more reliable yield determination.

¹ The known activity was below the routine laboratory detection limits for the available aliquot fraction.

. . h	Concentration (pCi/L) ^b								
Lab Code ^b	Date	Analysis	Laboratory Result ^c	ERA Result ^d	Control Limits	Acceptance			
			Nesull	ncoun	LIIIII(S	,			
ERAP-1091	3/16/2015	Am-241	46.8 ± 2.2	49.8	30.7 - 67.4	Pass			
ERAP-1091	3/16/2015	Co-60	85.1 ± 2.9	79.1	61.2 - 98.8	Pass			
ERAP-1091	3/16/2015	Cs-134	825.6 ± 34.7	909.0	578.0 - 1,130.0	Pass			
ERAP-1091	3/16/2015	Cs-137	1,312 ± 12	1,170	879 - 1,540	Pass			
ERAP-1091	3/16/2015	Fe-55	760.6 ± 48.2	836.0	259.0 - 1630.0	Pass			
ERAP-1091	3/16/2015	Mn-54	<2.7	<50	. 0.0 - 50.0	Pass			
ERAP-1091	3/16/2015	Pu-238	51.0 ± 3.9	52.1	35.7 - 68.5	Pass			
ERAP-1091	3/16/2015	Pu-239/240	38.3 ± 1.3	40.3	29.20 - 52.70	Pass			
ERAP-1091	3/16/2015	Sr-90	95.3 ± 11.4	96.6	47.2 - 145.0	Pass			
ERAP-1091	3/16/2015	U-233/234	29.0 ± 1.2	34.3	21.3 <i>-</i> 51.7 [′]	Pass			
ERAP-1091	3/16/2015	U-238	31.0 ± 1.1	34.0	22.0 - 47.0	Pass			
ERAP-1091	3/16/2015	Zn-65	1099.3 ± 146.5	986.0	706.0 - 1360.0	Pass			
ERAP-1094	3/16/2015	Gr. Alpha	73.7 ± 0.7	62.2	20.8 - 96.6	Pass			
ERAP-1094	3/16/2015	Gr. Beta	69.6 ± 0.8	58.4	36.9 - 85.1	Pass			
ERSO-1098	3/16/2015	Am-241	1571.8 ± 209.6	1,500	878 - 1,950	Pass			
ERSO-1098	3/16/2015	Ac-228	1198.8 ± 140.4	1,250	802 - 1,730	Pass			
ERSO-1098	3/16/2015	Bi-212	1420.1 ± 455.7	1,780	474 - 2,620	Pass			
ERSO-1098	3/16/2015	Bi-214	3466.9 ± 86.9	4,430	2,670 - 6,380	Pass			
ERSO-1098	3/16/2015	Co-60	1779.8 ± 41.0	1,880	1,270 - 2,590	Pass			
ERSO-1098	3/16/2015	Cs-134	5204.6 ± 64.5	6,390	4,180 - 7,680	Pass			
ERSO-1098	3/16/2015	Cs-137	1417.1 ± 41.9	1,490	1,140 - 1,920	Pass			
ERSO-1098	3/16/2015	K-40	10,597 ± 380	10,700	7,810 - 14,400	Pass			
ERSO-1098	3/16/2015	Mn-54	<62.2	< 1000	0.0 - 1,000	Pass			
ERSO-1098	3/16/2015	Pb-212	1,032 ± 41	1,230	806 - 1,710	Pass			
ERSO-1098	3/16/2015	Pb-214	3,629 ± 93	4,530	2,640 - 6,760	Pass			
ERSO-1098	3/16/2015	Pu-238	942.9 ± 128.8	998.0	600.0 - 1,380.0	Pass			
ERSO-1098	3/16/2015	Pu-239/240	1,185 ± 140	1,210	791 - 1,670	Pass			
ERSO-1098	3/16/2015	Sr-90	1,724 ± 125	1,940	740 - 3,060	Pass			
ERSO-1098	3/16/2015	Th-234	3,666 ± 948	3,890	1,230 - 7,320	Pass			
ERSO-1098	3/16/2015	U-233/234	3,474 ± 226	3,920	2,400 - 5,020	Pass			
ERSO-1098	3/16/2015	U-238	3,620 ± 232	3,890	2,410 - 4,930	Pass			
ERSO-1098	3/16/2015	Zn-65	7,362 ± 145	7,130	5,680 - 9,470	Pass			
ERW-1095	3/16/2015	Gr. Alpha	93.4 ± 11.5	119.0	42.2 - 184.0	Pass			
ERW-1095	3/16/2015	Gr. Beta	145.2 ± 4.8	158.0	90.5 - 234.0	Pass			
ERW-1110	3/16/2015	H-3	10,573 ± 78	10,300	6,900 - 14,700	Pass			
ERVE-1100	3/16/2015	Am-241	4,537 ± 266	4,340	2,650 - 5,770	Pass			
ERVE-1100	3/16/2015	Cm-244	1,338 ± 146	1,360	666 - 2,120	Pass			

TABLE A-7. Interlaboratory Comparison Crosscheck program, Environmental Resource Associates (ERA)^a.

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Concentration (pCi/L) ^b									
Lab Code ^b	Date	Analysis	Laboratory	ERA	Control				
			Result °	Result ^d	Limits	Acceptance			
ERVE-1100 °	3/16/2015	Co-60	1,030 ± 29	1,540	1,060 - 2,150	Fail			
ERVE-1100 ^f	3/16/2015	Co-60	1,684 ±48	1,540	1,060 - 2,150	Pass			
ERVE-1100 °	3/16/2015	Cs-134	1,615 ±27	2,650	1,700 - 3,440	Fail			
ERVE-1100 ^f	3/16/2015	Cs-134	2,554 ± 49	2,650	1,700 - 3,440	Pass			
ERVE-1100 ^e	3/16/2015	Cs-137	1,248 ± 29	1,810	1,310 - 2,520	Fail			
ERVE-1100 ^f	3/16/2015	Cs-137	2,078 ± 68	1,810	1,310 - 2,520	Pass			
ERVE-1100 ^e	3/16/2015	K-40	22,037 ± 463	30,900	22,300 - 43,400	Fail			
ERVE-1100 ^f	3/16/2015	K-40	34,895 ± 764	30,900	22,300 - 43,400	Pass			
ERVE-1100 ^e	3/16/2015	Mn-54	<13.8	<300	0.0 - 300.0	Pass			
ERVE-1100 ^f	3/16/2015	Mn-54	<24.4	<300	0.0 - 300.0	Pass			
ERVE-1100	3/16/2015	Pu-238	3,232 ± 232	3,680	2,190 - 5,040	Pass			
ERVE-1100	3/16/2015	Pu-239/240	3,606 ± 240	4,180	2,570 - 5,760	Pass			
ERVE-1100	3/16/2015	Sr-90	6,023 ± 326	6,590	3,760 - 8,740	Pass			
ERVE-1100	3/16/2015	U-233/234	2,653 ± 153	3,150	2,070 - 4,050	Pass			
ERVE-1100	3/16/2015	U-238	2,717 ± 163	3,130	2,090 - 3,980	Pass			
ERVE-1100 ^e	3/16/2015	Zn-65	<94.6	1,090	786 - 1,530	Fail			
ERVE-1100 ^f	3/16/2015	Zn-65	1,306 ± 75	1,090	786 - 1,530	Pass			
ERW-1103	3/16/2015	Am-241	47.1 ± 4.0	46.0	31.0 - 61.7	Pass			
ERW-1103	3/16/2015	Co-60	1,217 ± 17	1,250	1,090 - 1,460	Pass			
ERW-1103	3/16/2015	Cs-134	1,121 ± 18	1,260	925 - 1,450	Pass			
ERW-1103	3/16/2015	Cs-137	1,332 ± 31	1,360	1,150 - 1,630	Pass			
ERW-1103	3/16/2015	Mn-54	<3.7	<100	0.00 - 100.00	Pass			
ERW-1103	3/16/2015	Pu-238	54.5 ± 1.6	72.4	53.6 - 90.1	Pass			
ERW-1103 ⁹	3/16/2015	Pu-239/240	140.2 ± 7.8	184.0	143.0 - 232.0	Fail			
ERW-3742 ^h	9/27/2012	Pu-239/240	89.3 ± 4.9	97.7	66.6 - 108.0	Pass			
ERW-1103	3/16/2015	U-233/234	56.5 ± 6.4	61.8	46.4 - 79.7	Pass			
ERW-1103	3/16/2015	U-238	58.4 ± 5.8	61.3	46.7 - 75.2	Pass			
ERW-1103	3/16/2015	Zn-65	1,191 ± 136	1,180	984 - 1,490	Pass			
ERW-1103	3/16/2015	Fe-55	1,149 ± 144	1,070	638 - 1,450	Pass			
ERW-1103	3/16/2015	Sr-90	860.0 ± 37.0	912.0	594.0 - 1,210.0	Pass			

TABLE A-7. Interlaboratory Comparison Crosscheck program, Environmental Resource Associates (ERA)^a.

^a Results obtained by Environmental, Inc., Midwest Laboratory as a participant in the crosscheck program for proficiency testing administered by Environmental Resources Associates, serving as a replacement for studies conducted previously by the Environmental Measurements Laboratory Quality Assessment Program (EML).

^b Laboratory codes as follows: ERW (water), ERAP (air filter), ERSO (soil), ERVE (vegetation). Results are reported in units

of pCi/L, except for air filters (pCi/Filter), vegetation and soil (pCi/kg).

^c Unless otherwise indicated, the laboratory result is given as the mean ± standard deviation for three determinations.

^d Results are presented as the known values, expected laboratory precision (1 sigma, 1 determination) and control limits as provided by ERA. A known value of "zero" indicates an analysis was included in the testing series as a "false positive". Control limits are not provided.

^e Technician error weighing sample caused submitted gamma results to be understated and outside the control limits.(low)

^f The result of reanalysis with the correct sample volume (Compare to original result, footnoted "e" above).

^g The results of reanalysis were outside the control limits (low).

^h Sample ERW-3742 was ordered from ERA to determine why ERW-1103 results for Pu-239 were outside the acceptable range. The results for ERW-3742 were acceptable. No reason for the unacceptable results for ERW-3742 was determined.

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Appendix B 2015 REMP Data Summary Reports

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	Pathway Sampled Units	Type and Total Number of Anaiyses Performed	Lower Limit of Detection (LLD) *	Mean for All Locations Detected/Collected Range	Mean for Indicator Locations Detected/Collected Range	Location with Hic Location # Distance & Direction	hest Annual Mean Mean Detected/Collected Range	Mean for Control Locations Detected/Collected Range
	Air pCi/m3	Be-7 28	N/A	0.063 28/28 0.046 - 0.088	0.062 24/24 0.046 - 0.087	6 11.1 SSW	0.072 4/4 0.057 - 0.088	0.072 4/4 0.057 - 0.088
	Air pCi/m3	Co-58 28	N/A	< LLD 0 / 28 	< LLD 0 / 24 —	_	_	< LLD 0/4
~	Air pCi/m3	Co-60 28	N/A	< LLD 0 / 28 —	< LLD 0 / 24 —	_	_	< LLD 0 / 4 —
	Air pCi/m3	Cs-134 28	0.037	< LLD 0 / 28 —	< LLD 0 / 24 	_	_	< LLD 0/4
	Air pCi/m3	Cs-137 28	0.045	< LLD 0 / 28 —	< LLD 0 / 24 —	·	_	< LLD 0/4 —
	Air pCi/m3	Gross Beta 361	0.0075	0.027 361/361 0.006 - 0.065	0.027 309/309 0.006 - 0.065	6 11.1 SSW	0.028 52/52 0.008-0.064	0.028 52/52 0.008-0.064
1	Air pCi/m3	I-131 361	0.05	<lld 0/361 —</lld 	<lld 0/309 —</lld 	_	_	<lld 0/52</lld

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	Turnerand		Locations I Detected/Collected I	Mean for Indicator	Location with High	hest Annual Mean	Mean for Control Locations Detected/Collected Range
Pathway Sampled Units				Locations Detected/Collected Range	Location # Distance & Direction	Mean Detected/Collected Range	
Fish pCi/kg wet	K-40 14	N/A	1148.7 12/14 585 - 1889	1007.5 6/6 585 - 1202	32 15.8 WSW	1289.8 6/8 786 - 1889	1289.8 6/8 786 - 1889
Fish pCi/kg wet	Mn-54 14	94	<lld 0/14 —</lld 	<lld 0/6 —</lld 			<lld 0/8</lld
Fish pCi/kg wet	Fe-59 14	195	<lld 0/14</lld 	<lld 0/6</lld 	_		<lld 0/8 —</lld
Fish pCi/kg wet	Co-58 14	97	<lld 0/14 —</lld 	<lld 0/6 —</lld 	_	_	<lld 0/8</lld
Fish pCi/kg wet	Co-60 14	97	<lld 0/14 —</lld 	<lld 0/6 —</lld 	_	_	<lld 0/8</lld
Fish pCi/kg wet	Zn-65 14	195	<lld 0/14</lld 	<lld 0/6 —</lld 	_	_	<lld 0/8</lld
Fish pCi/kg wet	Cs-134 14	97	<lld 0/14</lld 	<lld 0/6</lld 	_		<lld 0/8</lld

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		Lower Limit of Detection (LLD) *	Mean for All	Mean for Indicator	Location with Highest Annual Mean		Mean for Control
Pathway Sampled Units	Sampled Total Number of Analyses		Locations Locations Detected/Collected Detected/Collected Range Range	Detected/Collected	Location # Distance & Direction	Mean Detected/Collected Range	Locations Detected/Collected Range
Fish pCi/kg wet	Cs-137 14	112	<lld 0/14 </lld 	<lld 0/6</lld 	_	_	<lld 0/8</lld
Broadleaf Vegetation pCi/kg wet	Be-7 69	N/A	469.0 45/69 150 - 1169	469.0 45/57 150 - 1169	70 17.1 SSW	637.4 10/12 237 - 1169	637.4 10/12 237 - 1169
Broadleaf Vegetation pCi/kg wet	K-40 69	N/A	5210.0 69/69 2793 - 12725	4954.4 57/57 2793 - 9638	70 17.1 SSW	6424.5 12/12 4498 - 12725	6424.5 12/12 4498 - 12725
Broadleaf Vegetation pCi/kg wet	Co-58 69	N/A	<lld 0/69</lld 	<lld 0/57 —</lld 		-	<lld 0/12</lld
Broadleaf Vegetation pCi/kg wet	Co-60 69	N/A	<lld 0/69 —</lld 	<lld 0/57 </lld 	_	_	<lld 0/12 —</lld
Broadleaf Vegetation pCi/kg wet	I-131 69	45	<lld 0/69 —</lld 	<lld 0/57</lld 		_	<lld 0/12</lld
Broadleaf Vegetation pCi/kg wet	Cs-134 69	45	<lld 0/69</lld 	<lld 0/57</lld 	_		<lld 0/12</lld

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Environmental Radiological Monitoring Program Annual Summary Perry Nuclear Power Plant Docket Number 50-440/50-441 Lake County, Ohio Reporting Period: 2015

		Lower Limit of Detection (LLD) *	Mean for All	Mean for Indicator Locations Detected/Collected Range	Location with Hig	Mean for Control	
Sampled To	Type and Total Number of Analyses Performed		Locations Detected/Collected Range		Location # Distance & Direction	Mean Detected/Collected Range	Locations Detected/Collected Range
Broadleaf Vegetation pCi/kg wet	Cs-137 69	60	<lld 0/69</lld 	<lld 0/57</lld 	_	_	<lld 0/12 —</lld
Milk pCi/L	K-40 46	N/A	1393.3 46/46 717 - 1923	1764.3 27/27 1196 - 1923	18 2.6 E	1764.3 8/8 1350 - 1923	1320.7 19/19 717 - 1445
Milk pCi/L	I-131 46	0.8	<lld 0/46</lld 	<lld 0/27</lld 	_	_	<lld 0/19</lld
Milk pCi/L	Cs-134 46	11	<lld 0/46 </lld 	<lld 0/27</lld 	_	_	<lld 0/19</lld
Milk pCi/L	Cs-137 46	13	<lld 0/46 —</lld 	<lld 0/27 —</lld 	_	_	<lld 0/19</lld
Milk pCi/L	Ba-140 46	45	<lld 0/46</lld 	<lld 0/27</lld 	_	_	<lld 0/19</lld
Milk pCi/L	La-140 46	11	<lld 0/46 —</lld 	<lld 0/27</lld 	_	_	<lld 0/19</lld

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	There and		Mean for All	Mean for Indicator	Location with Hig	hest Annual Mean	Mean for Control Locations Detected/Collected Range
Pathway Sampled Units	npled Total Number of Analyses	Lower Limit of Detection (LLD) *	Locations Detected/Collected Range	Locations Detected/Collected Range	Location # Distance & Direction	Mean Detected/Collected Range	
Sediment pCi/kg wet	K-40 5	N/A	6502.0 5/5 2268 - 9693	6502.0 5/5 2268 - 9693	66 1.4 NE	6849.7 3/3 6423 - 7272	NA NA NA
Sediment pCi/kg wet	Co-58 5	50	<lld 0/5</lld 	<lld 0/5 —</lld 	_	_	NA NA NA
Sediment pCi/kg wet	Co-60 5	40	<lld 0/5 —</lld 	<lld 0/5</lld 	—	_	NA NA NA
Sediment pCi/kg wet	Cs-134 5	112	<lld 0/5 —</lld 	<lld 0/5 —</lld 		_	NA NA NA
Sediment pCi/kg wet	Cs-137 5	135	<lld 0/5 —</lld 	<lld _ 0/5 </lld 	_	_	NA NA NA
TLD (E) mR/91 days	Direct 116	1.0	12.0 116/116 5.9 - 19.0	_ 12.0 108/108 5.9 - 19.0	33 4.7 S	16.3 4/4 12.1 - 19.0	11.4 8/8 8.0 - 13.0
TLD (Q) mR/91 days	Direct 116	1.0	12.9 116/116 8.0 - 19.8	12.9 108/108 8.0 - 19.8	29 4.5 SSE	17.0 4/4 15.6 - 18.5	13.1 8/8 12.0 - 14.1

			Mean for All	Mean for Indicator	Location with Hi	ghest Annual Mean	Mean for Control
Pathway Type and Sampled Total Number of Analyses Performed	Lower Limit of Detection (LLD) *	Locations Detected/Collected Range	Locations Detected/Collected Range	Location # Distance & Direction	Mean Detected/Collected Range	Locations Detected/Collected Range	
TLD mR/365 days	Direct 29	1.0	57.2 29/29 48.4 - 71.2	57.1 27/27 48.4 - 71.2	33 4.7 S	71.2 1/1 71.2 - 71.2	59.1 2/2 54.8 - 63.4
Water pCi/L	Gross Beta 54	3.0	1.5 47/54 0.9 - 2.7	1.5 32/42 0.9 - 2.7	28 20.6 ENE	1.9 6/6 1.0 - 2.6	1.5 11/12 0.9 - 2.6
Water pCi/L	H-3 18	1500	<lld 0/18</lld 	<lld 0/14 —</lld 	_		<lld 0/4 —</lld
Water pCi/L	Mn-54 54	. 11	<lld 0/54 —</lld 	<lld 0/42</lld 		_	<lld 0/12</lld
Water pCi/L	Fe-59 ∙54	22	<lld ' 0/54 —</lld 	<lld 0/42 —</lld 			<lld 0/12</lld
Water pCi/L	Co-58 54	11 -	<lld 0/54</lld 	<lld 0/42</lld 		_	<lld 0/12 —</lld
Water pCi/L	Co-60 54	11	<lld 0/54</lld 	<lld 0/42 —</lld 	_	_	<lld 0/12</lld

Pathway Sampled Units	Type and Total Number of Analyses Performed	Lower Limit of Detection (LLD) *	Mean for All Locations Detected/Collected Range	Mean for Indicator Locations Detected/Collected Range	Location with Hig Location # Distance & Direction	hest Annual Mean Mean Detected/Collected Range	Mean for Control Locations Detected/Collected Range
Water pCi/L	Zn-65 54	22	<lld 0/54</lld 	<lld - 0/42</lld 	_	_	<lld 0/12 </lld
Water pCi/L	Zr-95 54	22	<lld 0/54 —</lld 	<lld 0/42 —</lld 	_	—	<lld 0/12 —</lld
Water pCi/L	Nb-95 . 54	11	<lld 0/54</lld 	<lld 0/42 </lld 		_	<ĽLD 0/12 —
Water pCi/L	Cs-134 54	11	<lld 0/54</lld 	<lld 0/42</lld 		_	<lld 0/12 —</lld
Water pCi/L	Cs-137 54	13	<lld 0/54</lld 	<lld 0/42</lld 	_	_	<lld 0/12</lld
Water pCi/L	Ba-140 54	45	<lld 0/54</lld 	<lld 0/42 —</lld 	_		<lld 0/12 —</lld
Water pCi/L	La-140 54	11	<lld 0/54 —</lld 	<lld 0/42</lld 	.—	_	<lld 0/12 —</lld

*This value is the LLD that is met by the vendor and is lower than required by the ODCM

Appendix C 2015 REMP Detailed Data Report