

Tennessee Valley Authority, Post Office Box 2000, Decatur, Alabama 35609-2000

March 21, 2012

10 CFR 21 10 CFR 50.73

ATTN: Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555-0001

> Browns Ferry Nuclear Plant, Units 1, 2, and 3 Facility Operating License Nos. DPR-33, DPR-52, and DPR-68 NRC Docket Nos. 50-259, 50-260, and 50-296

Subject: Licensee Event Report 50-259/2011-002-01

Reference: Letter from TVA to NRC, "Licensee Event Report 50-259/2011-002-00," dated June 27, 2011

On June 27, 2011, the Tennessee Valley Authority (TVA) submitted Revision 0 to Licensee Event Report (LER) 50-259/2011-002. In Revision 0, TVA indicated that it was evaluating the Unit 1/2 C Emergency Diesel Generator (EDG) for past inoperability, and if there was a past inoperability issue identified, TVA would revise the LER for the loss of safety function that resulted from the loss of power from the Unit 1/2 C EDG due to an oil leak.

TVA is submitting this supplemental LER in accordance with 10 CFR 21.2(c), reporting of defects; 10 CFR 50.73(a)(2)(i)(B), as an operation or condition that is prohibited by the plant's Technical Specifications; 10 CFR 50.73(a)(2)(iv)(A), as any event or condition that resulted in manual or automatic actuation of any of the systems listed in 10 CFR 50.73(a)(2)(iv)(B); and 10 CFR 50.73(a)(2)(v)(B), as an event that could have prevented the fulfillment of a safety function for systems needed to remove decay heat.

There are no new regulatory commitments contained in this letter. Should you have any questions concerning this submittal, please contact J. E. Emens, Jr., Nuclear Site Licensing Manager, at (256) 729-2636.

Respectfully,

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K. J. Polson Vice President

cc: See Page 2

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U.S. Nuclear Regulatory Commission Page 2 March 21, 2012

Enclosure: Licensee Event Report 259/2011-002-01 - Loss of Safety Function (SDC) Resulting from Loss of Power from Unit 1/2 C EDG Due to Oil Leak

cc (w/ Enclosure):

NRC Regional Administrator - Region II NRC Senior Resident Inspector - Browns Ferry Nuclear Plant

ENCLOSURE

Browns Ferry Nuclear Plant Units 1, 2, and 3

Licensee Event Report 259/2011-002-01

Loss of Safety Function (SDC) Resulting from Loss of Power from Unit 1/2 C EDG Due to Oil Leak

See Attached

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NARRATIVE

I. PLANT CONDITION(S)

On April 27, 2011, severe weather in the Tennessee Valley Service Area caused grid instability and a loss of all 500-kV offsite power sources that resulted in a scram of all three Browns Ferry Nuclear Plant (BFN) units. At the time of the event being reported [April 28, 2011, at 2338 hours Central Daylight Time (CDT)], BFN Units 1, 2, and 3 were in Mode 4 (Cold Shutdown) with shutdown 4-kV buses supplied by seven of eight onsite emergency diesel generators (EDG) [EK].

II. DESCRIPTION OF EVENT

A. Event

On April 28, 2011, at 2338 hours CDT, seven of eight EDGs were in service to provide AC power for emergency core cooling loads. Operations personnel observed a small leak on the governor hydraulic oil system piping for the Unit 1/2 C EDG. When the piping was found leaking, Operations personnel began to switch Residual Heat Removal (RHR) [BO] pumps to unload the Unit 1/2 C EDG. The leak rapidly progressed from 1 drop per minute to a steady stream/spray and upon witnessing worsening voltage and frequency fluctuations; Operations personnel initiated an emergency shutdown of the Unit 1/2 C EDG. Upon loss of the Unit 1/2 C EDG, the C 4-kV Shutdown Board [EB] de-energized. This resulted in a loss of Shutdown Cooling (SDC) on Units 1 and 2. Unit 1 SDC was lost due to a Group 2 Primary Containment Isolation System (PCIS) [JE] actuation from the loss of the C 4-kV shutdown board. SDC was restored to Unit 1 in 47 minutes after the PCIS actuation and SDC was restored to Unit 2 four minutes after the failure. While the Unit 1/2 C EDG was out of service, the failed fitting was replaced with a like-for-like fitting. The failed fitting was sent to an offsite laboratory for failure analysis.

As a result of performing a past operability evaluation on the Unit 1/2 C EDG, it was determined the Unit 1/2 C EDG would have been unable to perform its specified safety function from April 1, 2011, at 1526 hours CDT until April 30, 2011, at 1047 hours CDT, when the failed fitting was replaced and post maintenance testing was successfully completed. In addition, it has been determined, as a result of performing a separate past operability evaluation on the Unit 1/2 A EDG, that the Unit 1/2 A EDG would have also been unable to perform its specified safety function during this time period. The Unit 1/2 A EDG inoperability is addressed in LER 50-259/2011-003-01.

B. Inoperable Structures, Components, or Systems that Contributed to the Event

Loss of offsite power was a contributor to this event.

C. Dates and Approximate Times of Major Occurrences

April 5, 2011, at 2208 hours CDT

Unit 2 enters Mode 2 from refueling outage that started on February 26, 2011, at 0001 hours Central Standard Time.

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April 24, 2011, at 2003 hours C	DT		declared ino ed maintenar						
April 27, 2011, at 1636 hours C	DT	sources. and 7 of (3B EDC	8 EDGs star 6 was inopera ble due to pla	I 3 BFN units ted. able and					
April 28, 2011, at 0243 hours C	DT	Unit 3 ei	ntered Mode	4.					
April 28, 2011, at 0545 hours C	DT	Unit 2 ei	ntered Mode	4.					
April 28, 2011, at 1337 hours C	DT	Unit 1 ei	ntered Mode	4.					
April 28, 2011, at 2330 hours C	DT	hydraulie by Oper Mainten prepare personn	to add oil. O	ak observed nnel. nel notified to perations o switch from					
April 28, 2011, at 2338 hours C	DT	emerger C EDG (leak on (ons personne ncy shutdowr due to hydrau governor. SE nd Unit 2.	of Unit 1/2 lic oil piping					
April 28, 2011, at 2342 hours 0	DT	Unit 2 S RHR pu		by starting 2D)				
April 29, 2011, at 0025 hours 0	DT		DC restored gnal and esta	by resetting blishing SDC					
April 30, 2011, at 1047 hours C	TDT	replacer	C EDG Oper ment of failed tting and suc ance testing.	hydraulic oil					
May 2, 2011, at 2010 hours CE	т	from qua sources	alified 161-k\ , and all EDG /n and in star		r				

There were no other systems or secondary functions affected.

NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION (10-2010)LICENSEE EVENT REPORT (LER) CONTINUATION SHEET FACILITY NAME (1) DOCKET (2) LER NUMBER (6) PAGE (3) YEAR SEQUENTIAL REVISION NUMBER NUMBER Browns Ferry Nuclear Plant Unit 1 05000259 2011 -- 002 -- 01 4 of 12 NARRATIVE E. Method of Discovery Operations personnel were continuously monitoring the EDGs for leaks, abnormal noises, and vibration. Operations personnel observed a small leak (1 drop per minute) on the governor hydraulic oil system piping for the Unit 1/2 C EDG. F. Operator Actions The Unit 1/2 C EDG oil leak rapidly progressed from 1 drop per minute to a steady stream/spray. Upon witnessing worsening voltage and frequency fluctuations, Operations personnel initiated an emergency shutdown of the Unit 1/2 C EDG. G. Safety System Responses During the loss of SDC event, the Technical Specifications (TS) required Emergency Core Cooling System (ECCS) subsystems were Operable to ensure adequate core coolina. **III. CAUSE OF THE EVENT** This LER addresses the cause of the failure of the Unit 1/2 C EDG. The cause of the failure of the Unit 1/2 A EDG is addressed in LER 50-259/2011-003-01. A. Immediate Cause The immediate cause of this event was a leaking 1/8-inch threaded brass fitting on the governor-to-governor booster pump hydraulic oil piping of the Unit 1/2 C EDG. **B.** Root Cause The root cause was determined to be less than adequate design of the Unit 1/2 C EDG governor hydraulic oil piping to compensate for vibration loading. As a result, the 1/8-inch threaded brass fitting failed due to fatigue from cyclic vibration loads transmitted from the EDG governor. **IV. ANALYSIS OF THE EVENT** The Tennessee Valley Authority (TVA) is reporting this event in accordance with 10 CFR 50.73(a)(2)(iv)(A), as any event or condition that resulted in manual or automatic actuation of any of the systems listed in 10 CFR 50.73(a)(2)(iv)(B). This event is also reportable in accordance with 10 CFR 50.73(a)(2)(v)(B), as an event that could have prevented the fulfillment of a safety function for systems needed to remove decay heat. The Unit 1/2 C EDG is a common EDG which feeds Unit 1 and Unit 2 equipment. Loss of the Unit 1/2 C EDG also caused a loss of the 1B Reactor Protection System (RPS) [JC] because the 1B RPS is fed from the 480V Reactor Motor Operated Valve Board [ED] 1B, which is fed from the 480V Shutdown Board [ED] 1B, which is fed from the C 4-kV Shutdown Board. Loss of power to the 1B RPS caused PCIS Group 2, 3, 6, and 8 isolations. Unit 1 SDC was lost due to the Group 2 PCIS isolation. SDC was restored 47 minutes later for Unit 1 in accordance with Abnormal Operating Instruction (AOI) 1-AOI-74, Loss of Shutdown Cooling. In addition to the loss of SDC, automatic initiation of the three trains of Standby Gas Treatment (SGT) System [BH], automatic initiation of one train of Control Room Emergency Ventilation (CREV) System [VI], a trip of the

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Reactor Water Cleanup System [C of the A Control Air System [LE] co			ion on PCIS	isolation),	and loss
The 2B RHR pump is fed by the C resulting in a momentary suspension restore SDC to Unit 2 four minutes	ion of SDC to	Unit 2. 1	The 2D RHR	pump was	
Based on the following, TVA is also 10 CFR 50.73(a)(2)(i)(B), as an op TS.					plant's
As a result of performing a pas determined the Unit 1/2 C EDG safety function from April 1, 20 1047 hours CDT, when the fail was successfully completed. T evaluation of the Unit 1/2 C ED addition, it has been determine evaluation on the Unit 1/2 A ED unable to perform its specified EDG inoperability is addressed these past operability evaluation	S would have 11, at 1526 h led fitting was The determina OG operating l ed, as a result DG, that the L safety function d in LER 50-25	been una ours CDT replaced ation was history ar of perfor Jnit 1/2 A on during 59/2011-	able to perfor F until April 3 and post m based on th based on th the cause ming a sepa EDG would this time per 003-01. Bas	rm its spec 30, 2011, a aintenance te results o of the fail arate past o have also riod. The sed on the	cified at e testing of an ure. In operability been Unit 1/2 A results of
During this time period, the sta	itus of the thre	e BFN u	i <mark>nits was as</mark> f	follows.	
 Unit 1 was in Mode 1 until t 1636 hours CDT (entering power sources. Unit 1 subs 1337 hours CDT. Unit 2 was in a refueling ou transitioned from Mode 4 to subsequently transitioned t automatically scrammed or as a result of the loss of all entered Mode 4 on April 28 Unit 3 was in Mode 1 until t 1636 hours CDT (entering power sources. Unit 3 subs 0243 hours CDT. 	Mode 3) as a sequently ent to Mode 2 on A to Mode 2 on A to Mode 1 and n April 27, 20 500-kV offsit 3, 2011, at 05 the unit auton Mode 3) as a	result of tered Mod April 5, 20 d remained 11, at 163 e power s 45 hours natically s result of	the loss of a de 4 on April om this refue 011, at 2208 ed there until 36 hours CD sources. Un CDT. scrammed of the loss of a	II 500-kV 28, 2011, eling outag hours CD I the unit T (entering it 2 subse n April 27, II 500-kV	offsite at ge and T. Unit 2 g Mode 3) quently 2011, at offsite
BFN Units 1 and 2 TS Limiting Operating," requires, in part, th The Unit 1/2 EDGs consist of t EDGs are inoperable, TS 3.8.1 restored to Operable status wit met within 2 hours, then both L hours and in Mode 4 within 36 Unit 1/2 EDGs were inoperable	hat Unit 1/2 El the A, B, C, ar 1 Required Ac thin 2 hours. Jnits 1 and 2 hours. Since	DGs be C nd D EDC tion H.1 If this TS are requi e it was no	Operable in N Gs. If two or requires all to 3.8.1 Requi red to be in I ot recognized	Nodes 1, 2 more Unit out one EE ired Action Mode 3 wi d that the	2, and 3. t 1/2 DG to be this not thin 12 two

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operability evaluations, BF than allowed by the TS.	N Units 1 and 2 o	perated	with inopera	able EDGs I	onger
In addition, BFN Unit 2 star Mode 4 to Mode 2 on April and Unit 1/2 C EDG were i LCO is not met except und Therefore, BFN Unit 2 ente EDGs Operable contrary to	5, 2011, at 2208 noperable. LCO er certain conditioner d the applicabi	hours C 3.0.4 pr ons that lity of TS	DT while the phibits Mode were not ap S LCO 3.8.1	e Unit 1/2 A changes w plicable to t	EDG /hen a his event.
BFN Units 1 and 2 TS LCC between the offsite transm Power Distribution System required offsite power sour TS 3.8.1 Required Action J requires action be initiated Mode 3 within 13 hours, ar satisfied as a result of the 1636 hours CDT, placing e the units to Mode 4 within f	ission network an be Operable in M ces concurrent w I.1 requires imme within 1 hour to p nd Mode 4 within automatic scrams ach unit in Mode	d the or lodes 1, ith the to diate en lace the 37 hours that oc 3, and s	site Class 1 2 and 3. U wo inoperab try into LCC unit in Mod . These re- curred on Ap subsequent	E AC Electr pon loss of le Unit 1/2 E 3.0.3. LCC e 2 within 1 quirements pril 27, 2011	rical all EDGs, D 3.0.3 0 hours, were , at
BFN Units 1 and 2 TS LCC the four Unit 1/2 EDGs eac onsite Class 1E AC electric "Distribution Systems - Shu necessary portions of the A Operable to support equip movement of irradiated fue one or more required Unit B.1.1, B.1.2, B.1.3, and B. suspension of movement of initiation of action to suspe vessel (OPDRVs), and initi Operable status.	ch capable of sup cal power distribu utdown," to be Op AC and DC electri ment required to b assemblies in th 1/2 EDGs are ino 1.4 require immed of irradiated fuel a nd operations wit	plying of tion sub perable. cal pow pe Opera le secor perable, liate sus ssembli h the po	he 4-kV shu systems req LCO 3.8.8 i er distributio able in Mode idary contain TS 3.8.2 Re spension of 6 es in second tential for di	tdown board uired by LC requires the s 4 and 5 a nment. In the quired Acti Core Alterat dary contain raining the re	d of the O 3.8.8, ms to be and during ne event, ons ions, ment, eactor
During the time period that (April 1, 2011, to April 5, 20 1/2 C EDG were credited v no Core Alterations, no mo containment, and no OPDF Required Actions were sat and Unit 1/2 C EDG was n immediately initiate action was not satisfied.	D11), it is likely the vith satisfying the ovement of irradia RVs were in progr isfied. Because t ot recognized, the	at either require ted fuel ress. Th he inope a TS 3.8	the Unit 1/2 ments of LC assemblies perefore, the prability of th .2 Required	A EDG and O 3.8.2. Ho in secondar associated le Unit 1/2 A Action B.1.4	d/or Unit owever, y TS 3.8.2 EDG 4 to
After BFN Units 1 and 2 au entered for each of the unit assemblies in secondary c	ts, no Core Altera	tions, n	o movement	of irradiate	d fuel

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	associated TS 3.8.2 Required the Unit 1/2 A EDG and Unit 1/2 Action B.1.4 to immediately init Operable status was not satisfi	/2 C EDG was tiate action to	s not reco	gnized, the	TS 3.8.2 Re	quired				
	BFN Unit 3 TS LCO 3.8.1 required Unit 1/2 EDGs capable of supp by LCO 3.8.7, "Distribution Sys Unit 3 TS LCO 3.8.7.g required 3.6.4.3, "Standby Gas Treatme Emergency Ventilation (CREV) and Unit 1/2 D EDG are required 3A, 3B, 3C, and 3D EDGs. If of Action B.4, applicable at the tir restored to Operable status with a planned maintenance outage TS 3.8.1 Required Action is no 12 hours and in Mode 4 within Actions were satisfied as a res April 27, 2011, at 1636 hours of cooldown of the unit to Mode 4 or more required Unit 1/2 EDG TS 3.8.1 Required Action K.2 r be declared inoperable within 3 A EDG was inoperable until the Unit 3 operated with an inopera longer than allowed by the TS.	blying the Unit stems - Opera s the Unit 1 ar ent (SGT) Sys) System," to ed to be Oper one Unit 3 ED ne of this eve thin 7 days. T e on April 24, t met, then Uf 36 hours. Fo ult of the auto CDT, placing t within the rea is inoperable requires the a 30 days. Since able required	1 and 2 ting," be d 2 AC a tem," and be Operation able). The G is inop nt, require The 3B El 2011, at nit 3 is react the 3B El 2011, at somatic sc he unit ir quired tin (in this ca ffected S ar it was of the pas Unit 1/2	4-kV shutdo Operable in and DC board d LCO 3.7.3, able (i.e., the he Unit 3 ED erable, TS 3 ed the Unit 3 DG was deci 2003 hours (quired to be EDG, the TS ram that occ mode 3, an he periods. I ase, the Unit GT and CRE not recognized st operability EDG (i.e., Unit	wn boards r Modes 1, 2 ds required "Control Ro Unit 1/2 A B Gs consist 3.8.1 Requires EDG to be lared inoper CDT. If this in Mode 3 v 3.8.1 Requires 3.8.1 Requires 3.8.1 Requires and subseque However, wi 1/2 A EDG) V subsyste ed that the U evaluation, nit 1/2 A ED	required , and 3. by LCO com EDG of the ed able for within uired ant ith one), ms to Unit 1/2 BFN G)				
	BFN Unit 3 TS LCO 3.8.1 also the offsite transmission networ Distribution System be Operate power sources concurrent with J.1 requires immediate entry in within 1 hour to place the unit i and Mode 4 within 37 hours. The automatic scram that occurred in Mode 3, and subsequent co- periods.	k and the ons ole in Modes 1 one Unit 3 E nto LCO 3.0.3 in Mode 2 with These require on April 27, 2	ite Class , 2 and 3 DG inope . LCO 3. nin 10 ho ments we 2011, at 1	1E AC Elec Depon loss arable, TS 3. 0.3 requires urs, Mode 3 are satisfied 1636 hours C	trical Power of required 8.1 Require action be in within 13 ho as a result o CDT, placing	offsite d Action hitiated burs, of the g the unit				
	BFN Unit 3 TS LCO 3.8.2 required of supplying one 4-kV shutdown distribution subsystems required the necessary portions of the A be Operable to support equipment during movement of irradiated event, one or more required UB B.1.1, B.1.2, B.1.3, and B.1.4 r	n board of the ed by LCO 3.8 C and DC ele nent required fuel assembli nit 3 EDGs ar	e onsite (3.8 to be ectrical p to be Op es in the e inopera	Class 1E AC Operable. L ower distribu erable in Mo secondary d able, TS 3.8.1	electrical po CO 3.8.8 re ition subsys des 4 and 5 containment. 2 Required	ower quires tems to and . In the Actions				

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suspension of movement of irradiated fuel assemblies in secondary containment, initiation of action to suspend OPDRVs, and initiation of action to restore the required Unit 3 EDGs to Operable status. After BFN Unit 3 automatically scrammed on April 27, 2011, and Mode 4 was entered, no Core Alterations, no movement of irradiated fuel assemblies in secondary containment, and no OPDRVs occurred. In addition, actions continued to restore the 3B EDG to Operable status. Therefore, these associated TS 3.8.2 Required Actions were satisfied.

TVA is also submitting this report in accordance with 10 CFR 21 due to the root cause determination of inadequate design of the Unit 1/2 C EDG governor-to-governor booster pump hydraulic oil line. See Part VII.F of this report for details.

V. ASSESSMENT OF SAFETY CONSEQUENCES

This event could have been significant from a nuclear safety standpoint. The EDGs are one of the most important BFN systems as determined by the plant's Probabilistic Risk Assessment. Loss of an EDG limits the capability for the unit to respond to an accident or transient when accompanied by a loss of offsite power. The Unit 1/2 C EDG operated for approximately 31 hours prior to Operations personnel performing an emergency shutdown. The 31 hours of operation did not meet the 7-day mission time of the Unit 1/2 C EDG. In addition, the Unit 1/2 A EDG operated for approximately 4 days and 13 hours prior to the output breaker trip. This period of operation did not meet the 7-day mission time of the Unit 1/2 A EDG. If multiple EDGs had experienced these failures concurrently, BFN could have lost its ability to provide power to components essential for core cooling, heat removal, and accident mitigation. However, in actuality the Unit 1/2 C EDG was returned to Operable status prior to the Unit 1/2 A EDG output trip breaker event. Based on this operating experience, it is reasonable to assume that both the Unit 1/2 A EDG and Unit 1/2 C EDG would not have been concurrently unavailable.

During the loss of SDC event, there was sufficient redundancy to support the core cooling requirements for Units 1 and 2 since the remaining two Unit 1/2 EDGs and the remaining three Unit 3 EDGs were Operable and the required ECCSs were Operable. In addition during the remainder of the time of concurrent EDG inoperabilities, the Unit 1/2 EDGs and the Unit 3 EDGs could have been paralleled, as shown in the Updated Final Safety Analysis Report Figure 8.5-24 (e.g., 4-kV shutdown board A to 4-kV shutdown board 3EA), to provide power to the necessary components to maintain adequate core cooling, heat removal capability, and SGT and CREV subsystem capabilities for each of the units. Therefore, this event was of minimal safety significance.

The oil leak caused a loss of the Unit 1/2 C EDG and subsequent loss of SDC on Unit 1 and Unit 2. Unit 1 restored SDC in 47 minutes after the failure and SDC was restored on Unit 2 four minutes after the failure. This information is relevant because the time to boil was approximately 2 hours for each unit. In addition, TS 3.4.8, "Residual Heat Removal (RHR) Shutdown Cooling System - Cold Shutdown," contains provisions that allow both RHR Shutdown Cooling subsystems to be out of service for up to one hour. Therefore, while this event could have been significant from a radiological safety standpoint, in actuality it had minimal safety significance.

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 In addition, the equivalent 1/8-inch EDGs were removed and submitteevidence of cracking as identified. (PT) nondestructive examination woof cracking was identified. The Unfollowing the loss of offsite power of approximate 5-day run time of their cracking of the fittings, it can be convould most likely have not been at Evaluations of the risk associated the unavailability of the Unit 1/2 C Unit 1/2 A EDG considered that thi 0626 CDT until May 7, 2011, 0439 considered that this EDG was una 30, 2011, at 1047 CDT. For the Unit unavailability, the risk impacts were TVA has concluded that there was public for this event. VI. CORRECTIVE ACTIONS - The concorrective action program. The concause of the failure of the Unit 1/2 cause of the failure 0 the Unit 1/2 cause of	ed for laborato on the Unit 1/2 vas performed nit 1/2 A, B, an event that beg se EDGs and oncluded that ffected by this with the sepa EDG were als is EDG was u 0 CDT. The even vailable from nit 1/2 A EDG re determined orrective action C EDG. The A EDG are act on c EDG. The A EDG are act on c EDG. The A EDG are act on sective actions l performed an were transferr nch brass thre raulic oil system see, for failur esting on all for e values on the ne results. e (<1/2-inch) for available vibra e (<1/2-inch) for available vibra	ory analys ory analys 2 C EDG d on each ad D EDG gan on Ap the labor the 7-day failure m rate unav so perform navailable valuation April 28, 1 unavailable valuation April 28, 1 unavailable valuation address correctiv ddressed a sent to re analysi our Unit 1 the Unit 1/2 tubing/pip [BJ] and g weight, ation data tubing/pip	sis to determ A Liquid Per of the fitting s ran for apport or 27, 2011. atory analys y mission tim hode. vailability of U med. The eve for the Unit 7 2011, at 233 bility and Un all. In to the heat ing manager sed in this L e actions that in LER 50-2 ken: ency shutdow operating El ng (like-for-ling Alstom TTTI s. /2 EDG gov bing connect determine it dimensions, a. bing connect	ine if there enetrant ⁻ is and no proximate Based of is that for the for thes Julit 1/2 A valuation 2, 2011, a 1/2 C EDO 8 CDT ur it 1/2 C EDO 8 CDT ur	re was any Testing Indication ely 5 days on the und no se EDGs EDG and for the at G ntil April EDG afety of the 's ess the s the 003-01. Unit 1/2 e Unit 1/2 tory, piping. ofting. he High ability for g ne Reactor

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NARRATIVE

vibration loading considering tubing weight, dimensions, operating experience, and any available vibration data.

• Review the small bore (<1/2-inch) tubing/piping connections on all eight EDGs and determine acceptability for vibration loading considering tubing weight, dimensions, operating experience, and any available vibration data.

B. <u>Corrective Actions to Prevent Recurrence</u>

The governor-to-governor booster pump hydraulic oil lines were redesigned and replaced with flexible hose rather than hard tubing. The hard tubing was replaced on all four Unit 1/2 EDGs. In addition, the sister fittings on the Unit 1/2 A, B, and D EDGs were removed for lab analysis to determine if there was any evidence of cracking. A PT nondestructive examination was performed on each of the fittings and no indication of cracking was identified.

The four Unit 3 EDG governor-to-governor booster pump hydraulic oil lines were originally supplied with flexible hose. Therefore, no modifications were necessary for those components.

VII. ADDITIONAL INFORMATION

A. Failed Components

Unit 1/2 C EDG

B. <u>Previous Similar Events</u>

A search of BFN LERs for Units 1, 2, and 3 for approximately the past five years did not identify any similar issues involving cracked or failed tubing threaded fittings for EDGs.

Review of BFN Work Orders (WO) concluded that numerous oil leaks have occurred on the EDGs. However, none of them appeared to be major leaks that caused the EDG to be declared inoperable. There were also no WOs that indicated the failed fitting for this event had ever been replaced. This indicates that the fitting is originally installed equipment (Unit 1/2 EDGs delivered in 1970).

A search of the BFN corrective action program was performed. There were several Problem Evaluation Reports (PERs) that documented oil leaks and oil level issues. None of the PERs reviewed were associated with fitting failures similar to the current event, in that there were no low stress high cycle fatigue cracks on the EDGs previously identified.

C. Additional Information

The corrective action document for this report is PER 362395.

D. Safety System Functional Failure Consideration

In accordance with NEI 99-02, this event is considered a safety system functional failure due to the loss of shutdown cooling on Unit 1 and Unit 2.

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Ε.	Scram With Complication	ns Considerati	on				
	This event did not include a	a reactor scram	•				
F.	<u>10 CFR Part 21 Reporting</u>	<u>Requirement</u>	<u>s</u>				
	The following information is 10 CFR 21.21(d)(4)(i) thru	•	eet the re	equirements	of		
	(i) Name and address of t	he individual or	individu	als informing	g the Comr	mission.	
	K. J. Polson Site Vice President Tennessee Valley Autho Browns Ferry Nuclear F Post Office Box 2000 Decatur, Alabama 3560	Plant					
	 (ii) Identification of the faci such facility or such act contains a defect. 						
	Facility: Browns Ferry	Nuclear Plant					
	Basic component which	n contains a de	fect:				
	Unit 1/2 C EDG, Model number 9464).	999-20, (Engir	e Model	Number 20	-645E4, or	der	
	(iii) Identification of the firm component which fails				g the basic	5	
	Basic component supp	lier:					
	General Motors, Electro	o-Motive Divisio	on, La Gi	range, Illinoi	S		
	(iv) Nature of the defect or or could be created by				ard which	is created	
	Nature of the defect:						
	The root cause was de C EDG governor hydra result, the 1/8-inch bras vibration loads transmit	ulic oil piping to ss threaded fitti	o comper ng failed	nsate for vib due to fatig	ration load	ing. As a	
	Safety hazard which co	ould be created	by such	defect:			
	Loss of an EDG limits t transient when accomp	• •	r the uni	t to respond	to an acci	dent or	

	SEE EVENT R CONTINUATION DOCKET (2)		(LER)			
	CONTINUATION					
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(v) The date on which the obtained.	information of s	uch defec	t or failure to	o comply w	as	
BFN Site Engineering evaluation on June 21		ngineerin	g Group) cor	mpleted the	Part 21	
(vi) In the case of a basic on number and location o supplied for one or mo part.	f all such compo	nents in i	use at, suppl	ied for, or b	being	
Other 1/8-inch brass to supplied by the vendo					I.	
(vii) The corrective action w the individual or organ that has been or will b	ization responsil	ble for the	e action; and			
As stated in Section V hydraulic oil lines were hard tubing. The hard	redesigned and	replaced	d with flexible	e hose rath	er than	
The four Unit 3 EDG g were originally supplie does not exist on the f	d with flexible ho	ose. Thei				
The individual respons	ible is K. J. Pols	on, Site \	/ice Presider	nt of BFN.		
The action was comple	eted on June 27,	2011.				