



Crystal River Nuclear Plant  
Docket No. 50-302  
Operating License No. DPR-72

Ref: 10 CFR 21.21(a)(2)

July 13, 2011  
3F0711-06

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555-0001

Subject: Crystal River Unit 3 – Second Interim Report of the Evaluation of a Deviation Pursuant to 10 CFR 21.21(a)(2)

- References: (1) AREVA NP, Inc. to CR-3 letter, dated March 3, 2011, "Customer Notification per 10 CFR 21.21(b)"
- (2) CR-3 to NRC letter, dated April 26, 2011, "Crystal River Unit 3 – Interim Report of the Evaluation of a Deviation Pursuant to 10 CFR 21.21(a)(2)"

Dear Sir:

Pursuant to the requirements of 10 CFR 21.21(a)(2), Florida Power Corporation (FPC), doing business as Progress Energy Florida, Inc. (PEF), hereby provides this second interim report as notification that the evaluation of a deviation cannot be completed by August 31, 2011, as stated in the Crystal River Unit 3 (CR-3) letter dated April 26, 2011 (Reference 2). The notification of deviation from AREVA NP, Inc. (Reference 1) regards the Babcock & Wilcox (B&W) plant baffle bolt pattern limitations and safety assessment. A historical review of the technical evaluation developed for the B&W Owners Group reactor vessel internals program revealed assumptions considered to be non-conservative relative to current industry knowledge and understanding. Specifically, some of the analyses and conclusions in the AREVA calculation, which support the conclusions of AREVA Technical Report BAW-2392P, "B&W Designed Reactor Vessel Internals Baffle Bolt Failure Structural Analysis and Safety Assessment," have been found to be in error. The consequence of these errors and omissions in the AREVA calculation challenges the conclusion in BAW-2392P regarding fuel-baffle impact loads and baffle bolt stress conditions.

AREVA stated that the deviation does not create a potential defect unless BAW-2392P has been used in a justification of continued operation (JCO). In November 2005, loose/broken core baffle bolts were identified during CR-3 Refueling Outage 14. A JCO prepared by AREVA for CR-3 made reference to BAW-2392P and was used to support a use-as-is determination in the associated CR-3 nonconforming condition report.

The AREVA JCO is being revised, on a delayed schedule, to remove reference to BAW-2392P and to apply updated calculations. Revision to the AREVA JCO and completion of the deviation evaluation are scheduled to be completed by October 20, 2011. No new regulatory commitments are contained in this submittal.

If you have any questions regarding this submittal, please contact Mr. Dan Westcott, Superintendent, Licensing & Regulatory Programs, at (352) 563-4796.

Sincerely,

Jon A. Franke  
Vice President  
Crystal River Unit 3

JAF/dwh

xc: NRC CR-3 Project Manager  
NRC Regional Administrator, Region II  
Senior Resident Inspector

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IE19  
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