<u>06/29/2011</u>

Page 1

Power Reacto	r				Event #	ŧ 46997
U	ite: SUMME nit: 1 pe: [1] W-3-I pe: DRY AM	Region P	: 2 State : SC		me: 06/29/2011 me: 06/27/2011 ion: 06/29/2011	09:49 (EDT) 16:11 (EDT)
NRC Notified I HQ Ops Offic Emergency Clas 10 CFR Section 21.21	er: STEVE S ss: NON EM on:	SANDIN		Notifications: MA PA	RK FRANKE RT 21 GP (email)	R2DO NRR
Unit Scram Co	ode RX Crit	Init Power	Initial RX Mode	Curr Power	Current RX Mod	e
1 N	Yes	100	Power Operation	100	Power Operation	ו

APPENDIX R ANALYSES FAILS TO RECOGNIZE HOT-SHORT FAILURE RESULTING IN THE LOSS OF AN ESSENTIAL ELECTRICAL BUS

The following Part 21 report was received via fax:

"10 CFR 21: Appendix R analyses conducted for Virgil C. Summer Nuclear Station (VCSNS) failed to identify that a fire-induced hot-short failure in an ammeter circuit would result in a loss of the B-train 7.2KV essential electrical bus (XSW1DB).

"Appendix R analyses performed by Gilbert/Commonwealth (now WorleyParsons) in the early 1980s failed to recognize the possibility of a fire-induced hot-short condition in a circuit that was identified as being required for safe shutdown. This circuit connects a set of sensing current transformers (CTs) to an ammeter on the Main Control Board, and provides over-current sensing for an over-current relay. Gilbert/Commonwealth recognized that a fire-induced open circuit in this ammeter circuit would result in damage to, or a fire in, the B-train 7.2kV essential switchgear. Thyrite protectors were added to the circuit to protect the CTs from this open circuit condition as part of the Appendix R analysis.

"However, this analysis and resolution failed to consider the hot-short-to-ground failure mode. Current from a hotshort could flow through the ammeters, or neutral conductor, and then through the bus neutral over-current relay to ground. This could actuate the over-current relay, which in turn would actuate a lock-out relay and trip all incoming breakers to bus XSW1DB. This bus provides credited B-train power to safe-shutdown components credited for this scenario. The Appendix R analyses conducted for VCSNS by Gilbert/Commonwealth did not address the hot-short scenario and is considered to be a defect, or omission. reportable under 10 CFR 21.

"This condition was identified during the circuit analysis review for transitioning the Appendix R Fire Protection

JE14 NRR

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U.S. Nuclear Regulatory Commission Operations Center Event Report

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Program	r Reactor to NFPA 805 and was reported to the NRC as an unanalyzed condition on 05/03/2011 ion No. 46811). Corrective actions have been taken to address this issue."	Event # I (see Event	46997
The licer	nsee informed the NRC Resident Inspector.		

NRC	FORM	361	

(12-2000)

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U.S. NUCLEAR REGULATORY COMMISSION OPERATIONS CENTER

REACTOR PLANT

EN# 46997

									(-))	1
NRC OPERATION TE [2nd] 301-415-0550 ar	LEPHONE NUMBER: PRIM nd [3rd] 301-415-0553	ARY ·	- 301-81	6-5100 or 800		•	-		00-449-3694*, ovided these telephon	e numbers.
NOTIFICATION TIME			UNIT	UNIT NAME OF CALLER			·	CALL BACK #	ALL BACK #	
0949 VC Summer Nuclear			ion	1	Bruce T	hompson			(803) 931-5042	
EVENT TIME & ZONE	POW	POWERMODE BEFORE			POWERMODE AFTER					
1611	6 27 / 1 100% Mode 1					100% Mode 1				
EVENT CLASSIFICATIONS			-Hr. No	n-Emergen	cy 10 CFR 5	50.72(b)(1)	(v)(A) Safe S/D	Capability	AINA
GENERAL EMERGENCY GENAAEC			TS Deviation ADEV			(v)(B	(v)(B) RHR Capability All			
SITE AREA EMERGEN	ICY SIT/AAEC	4	-Hr. No	n-Emergen	cy 10 CFR 5	50.72(b)(2)	(v)(C) Control (of Rad Release	AINC
ALERT	ALE/AAEC		(i) .	TS Required S/D	1	ASHU	(v)(C) Accident	Mitigation	AIND
UNUSUAL EVENT	UNU/AAEC		(iv)(A) I	ECCS Discharge	to RCS	ACCS	(xii)	Offsite N	ledical	AMED
50.72 NON-EMERGENCY (see next columns)			(iv)(B) 1	RPS Actuation (s	icram)	ARPS	(xiä)	Loss Co	mm/Asmt/Resp	ACOM
PHYSICAL SECURITY (73.71) CDDD			(xi) (Offsite Notificatio	חנ	APRE	6	0-Day Opt	ional 10 CFR 50.73	3(a)(1)
MATERIAL/EXPOSURE B???			8-Hr. Non-Emergency 10 CFR 50.72(b)(3)				Invalid S	pecified System Actuation	AINV	
FITNESS FOR DUTY HETT			(ii) (A) I	Degraded Condit	ion	ADEG	Othe	r Unspeci	fied Requirement	(Identify)
OTHER UNSPECIFIED REGMT. (see last column)			(ii)(B) (Unanalyzed Con	dition	AUNA	\checkmark			NONR
INFORMATION ONLY NOF			(iv)(A) 3	Spacified System	n Actuation	AESF				NONR

DESCRIPTION

Include: Systems affected, actuations and their initiating signals, causes, effect of event on plant, actions taken or planned, etc. (Continue on back)

10 CFR 21: Appendix R analyses conducted for Virgil C. Summer Nuclear Station (VCSNS) failed to identify that a fire-induced hot-short failure in an ammeter circuit would result in a loss of the B-train 7.2KV essential electrical bus (XSW1DB).

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This condition was identified during the circuit analysis review for trainsitioning the Appendix R Fire Protection Program to NFPA 805 and was reported to the NRC as an unanalyzed condition on 05/03/2011 (see Event Notification No. 46811). Corrective actions have been taken to address this issue.

NOTIFICATIONS	YES	NO	WILL BE	ANYTHING UNUSUAL OR		NO	
NRC RESIDENT	✓			NOT UNDERSTOOD?	YES (Explain above)		
STATE(s)		1					
LOCAL		\checkmark		FUNCTION AS REQUIRED?	YES	NO (Explain above)	
OTHER GOV AGENCIES		\checkmark		MODE OF OPERATION	ESTIMATED	ADDITIONAL INFO ON BACK	
MEDIA/PRESS RELEASE		\checkmark		UNTIL CORRECTED:	RESTART DATE:	TES NO	