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Pow	er Reactor					Event	# 46554
	Site:	OCONEE	-		Notification Date / 7	Time: 01/19/2011	17:44 (EST)
	Unit:	3	B Region	: 2 State : SC	Event Date / 7	Time: 01/06/2011	17:02 (EST)
R	leactor Type:	[1] B&W-	L-LP,[2] B&'	W-L-LP,[3] B&W-L-L	P Last Modifica	ation: 01/19/2011	
Contai	inment Type:	DRY AM	B DRY AN	IB DRY AMB			
NRC	Notified by:	RANDY <sup>-</sup>	TODD		Notifications: M	ARVIN SYKES	R2DO
HQ	Ops Officer:	JOE O'H	ARA		P/	ART 21 GRP EMA	L
Emerg	gency Class:	NON EM	ERGENCY				
10 0	CFR Section:						
21.21		UNSPEC	IFIED PAR	AGRAPH			
Unit	Scram Code	RX Crit	Init Power	Initial RX Mode	Curr Powe	er   Current RX Mo	de
3	N	Yes	100	Power Operation	100	Power Operation	n
	1	I	1	I	i	I	

#### FOREIGN MATERIAL FOUND IN SPARE HEAT EXCHANGER

"Part 21 reportable defect foreign material (FM) was discovered in a spare heat exchanger (HX) when visually inspected on 10/19/10 as part of contingency planning for potential installation of the HX during the 3EOC 25 refueling outage.

Visual inspections with a video probe (fiberscope) were being performed on the inlet/outlet channels and the tubes of a spare HX to verify that no FM was present. However, when performing these inspections, a problem was encountered in that maintenance was unable to disassemble and remove the bonnet from the lower (inlet) pressure seal connector on the HX.

"Subsequently, on 10/19/10 a fiberscope inspection was performed of the "back side" of the pressure seal bonnet by inserting a fiberscope thru the inlet nozzle access. A foreign object that appeared as a u-shaped piece of thin wire was seen just behind the pressure seal bonnet, laying in the bottom of the body. Due to the inability to disassemble the pressure seal connector, the affected HX was not installed,

"The FM was retrieved from the spare HX on December 27,2010 and its size and material type confirmed. The "U-shaped" material is type 304 stainless steel and appears to be a metal shaving. It is about 1-1/2" in length if the "u" was stretched out, 3/32" wide and less than 1/64" thick.

"The HX was intended for use as a Reactor Coolant System Letdown Cooler. It was ordered in 2006 and was supplied by Energy Steel Supply Company (ESSC), Catalog 10 350952, SIN N32389-1, Purchase Order No. DP 19279.

"Initial Safety Significance: This event has no actual significance since the issue was discovered prior to installation

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#### Power Reactor

Event # 46554

of the component and the component was not installed. However, Part 21 requires evaluation based on the assumption that the component was installed in the as-found condition. On 1/6/2011, Oconee Engineering concluded that, in some low probability scenarios, this FM could constitute a Substantial safety hazard. Per the postulated scenarios, the FM would be flushed out of the cooler when placed in service. A low potential exists that the FM could be trapped in a downstream valve and damage the seat, resulting in leakage. The cooler outlet valve and the next valve in series are containment isolation valves. If the FM caused leakage in one of those valves and the other was affected by a postulated single failure, any resultant leakage would constitute containment leakage. Also, in some event scenarios, unacceptable leakage past the cooler outlet valve seat could render the Oconee Standby Shutdown Facility (SSF) Reactor Coolant Make-up System inoperable. Therefore, Oconee concludes this issue meets the Part 21 definition as a reportable defect.

"Corrective Action(s): The affected spare cooler was not installed. Subsequently, the FM was removed. A second spare cooler was inspected, no similar FM was found, and the second cooler was installed."

The NRC Resident Inspector will be notified.

cooler was inspected, no similar r w was round, and the secon

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NRC Event Notification Worksheet

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	NRC Event Notification Worksheet									
Notification Time	Notification Time Facility or Organization			Caller's Name	Call Back #					
Oconee Nuclea Station		•	3	Randy Toda	ENS 256-993 (864) 873-32					
NRC Oper	ation	s Officer Contacted:		NRC Even	t Number:					
				Den in Maria A A						
Event Time/Zon	e	Event Date		ower/Mode Before	Power/Mode After					
17:02 / EST		1/6/2011	1009	%/Mode 1	100%/Mode 1					
Event	Class	ifications	3	-Hour Non-Emergend	y 10 CFR 50.72 (b) (	3)				
General Emerg	gency			(ii) (A) Degraded Con	dition					
🔲 Site Area Erne	rgeno	y .		(ii) (B) Unanalyzed Co	ondition					
🔲 Alert				(xiii) Loss of emerger						
			capability/offsite communications							
Unusual Event		4	(iv) (A) System Actuation							
columns)	erger	icy (see other	RPS							
72.75 Spent Fi	uel (IS	SFSI)	Containment isolation							
73.71 Physical	•	•								
Transportation										
20.2202 Mater		rosure	Containment spray/coolers							
26.73 Fitness 1		•	Emergency AC (Keowee Hydro)							
		) Initial Notification	Г	(v) (A) Safe Shutdown Capability						
		,,	Π	(v) (B) Residual Heat Removal Capability						
1-Hour Non-emerg	gency	10 CFR 50.72 (b)(1)		□ (v) (C) Control of radiological material						
Deviation from	TSC	per 50.54(x)		(v) (D) Accident Mitigation						
	'			(xii) Transport contaminated person to offsite						
		1		médical facility						
4-Hour Non-Emerg	gency	10 CFR 50.72 (b)(2)								
(i) TS Required Shutdown										
(iv) (A) ECCS	Disch	arge into RCS								
🗌 (iv) (B) RPS A	ctuati	on (while critical)								
(xi) News release/notification to other										

government agencies

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### NRC Event Notification Worksheet

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Event Description							
(Include systems affected, actuations and their initiating signals, causes, effected, actuations and their initiations are actuated at the signal sign	ect of event on plant, actions taken or						
Event							
Part 21 reportable Defect: Foreign material (FM) was discovered in a s visually inspected on 10/19/10 as part of contingency planning for potenthe 3EOC-25 refueling outage.	pare heat exchanger (HX) when ntial installation of the HX during						
Visual inspections with a videoprobe (fiberscope) were being performed on the inlet/outlet channels and the tubes of a spare HX to verify that no FM was present. However, when performing these inspections, problem was encountered in that Maintenance was unable to disassemble and remove the bonnet from the lower (inlet) pressure seal connector on the HX.							
Subsequently, on 10/19/10 a fiberscope inspection was performed of the "back side" of the pressure seal bonnet by inserting a fiberscope thru the inlet nozzle access. A foreign object that appeared as a u-shaped piece of thin wire was seen just behind the pressure seal bonnet, laying in the bottom of the body. Due to the inability to disassemble the pressure seal connector, the affected HX was not installed.							
The FM was retrieved from the spare HX on December 27, 2010 and its size and material type confirmed. The "U-shaped" material is type 304 stainless steel and appears to be a metal shaving. It is about 1-1/2" in length if the "U" was stretched out, 3/32" wide and less than 1/64" thick.							
The HX was intended for use as a Reactor Coolant System Letdown C was supplied by Energy Steel Supply Company (ESSC), Catalog ID 35 Order No. DP 19279.							
Initial Safety Significance: This event has no actual significance since the issue was discovered p and the component was not installed. However, Part 21 requires evalu the component was installed in the as-found condition. On 1/6/2011, O in some low probability scenarios, this FM could constitute a substantia scenarios, the FM would be flushed out of the cooler when placed in se the FM could be trapped in a downstream valve and damage the seat, outlet valve and the next valve in series are containment isolation valve one of those valves and the other was affected by a postulated single f constitute containment leakage. Also, in some event scenarios, unacco outlet valve seat could render the Oconee Standby Shutdown Facility ( System inoperable. Therefore, Oconee concludes this issue meets the defect.	vation based on the assumption that become Engineering concluded that, al safety hazard. Per the postulated ervice. A low potential exists that resulting in leakage. The cooler es. If the FM caused leakage in failure, any resultant leakage would eptable leakage past the cooler SSF) Reactor Coolant Make-up						
Corrective Action(s): The affected spare cooler was not installed. Subsequently, the FM wa was inspected, no similar FM was found, and the second cooler was in							
Anything unusual or not understood? Yes (Explain above							
Did all systems function as required? Yes	No (Explain above)						
Mode of operations until corrected: Estimated restart da	are.						
Does event result in a radiological release, RCS leak, or steam generator tube leak?	Yes (complete page X No						

Does the event result in any of the units	Yes (complete Oconee Plant Status	🛛 No
experiencing a transient?	sheet)	

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### NRC Event Notification Worksheet

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Notifications							
NRC Resident: Y/N will be Plant Manager: Y/N will be							
Notified By:	Time:	Notified By:	Time:				
State(s):	Y will be	Operations Superintendent	Y/N Will be				
Notified By:	Time:	Notified By:	Time:				
Local:	Y (Wwill be	Other Government Agencies:	Y will be				
Notified By:	Time:	Notified By:	Time:				
Media/Press Release:	Y Will be	Other.	YNWill be				
Notified By:	Time	Notified By	Time				

Operations Shift Manager/Emergency Coordinator Approval:	Date/Time:			
ALL DA ESTI Diale	1-19-2011 1724			
NRC Notification Complete by Caller/NRC Communicator.	Date/Time:			

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### NRC Event Notification Worksheet

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Additional Information for Radiological Releases									
Radiological Release (check as applicable with specific details in event description including release path)									
Liquid Release Gaseous Release Unplanned Release Planned R Monitored Unmonitored Off-Site Release TS Exceed									
				elease			TS Exce		
	Rad Mon Alarms	:  [			rotected	d Actic	ons [	] Termina	ted
Exposed or			Rec	omme	ended				
Contaminated							r		/
		Ц			acuated			] Ongoing	
[	Release	% T	Slímit	НОС	Guide	Total	Activ	ity % TS Li	mit HOO Guide
	Rate	,, ,,	O Linit				Ci) /		
	(Ci/sec)					``	_/_		
Noble gas:				0.1	Ci/sec	7	<i>[</i>		1000 Ci
lodine;					Ci/sec/				0.01 Ci
Particulate:			1/	1μ(	Ci/sec				1 mCi
Liquid (excluding tritium and dissolved noble gases):	·		XX	10 /	Ci/min				0.1 Ci
Liquid (tritium):		IX	$/\gamma$	r	Ci/min				5 Ci -
Total Activity:			$\_$						
F		1	/						
	Plant-Stack	P	ondense Ejecto		Main I Li	Steam ne	n   S(	G Blowdowr	1 Other
Rad Monitor Readings:	/								
Alarm Setpoints:						4			
% TS Limit (if applicable):									
	_/								
Additional Information for Reactor Coolant Leaks and Steam Generator Tube Leaks									
Location of the leak (e.g. SG valve, pipe, etc.)									
Leak Rate:	Units (gpm/gpd):		TS L	imit:		ļ	Sudd	en or Long	Term
(at the spectrum sp				Development:					
Leak Start Date:	Time:		Cool	ant Ad	ctivity &	Units:			
Primary -									
				Secondary -					
List of Safety Related Equipment Not Operational:									