

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM (GIMCS) REPORT

**OFFICE OF NUCLEAR REGULATORY RESEARCH
1st Quarter of Fiscal Year 2008**

Generic Issue Management Control System

TABLE OF CONTENTS

Description	iii
Legend	iv
Data Elements	v-vi
1st Qtr FY 2008 GIMCS Report for Active GIs	1-28
❖ GI-156.6.1, "Pipe Break Effects on Systems and Components"	1-3
❖ GI-163, "Multiple Steam Generator Tube Leakage"	4-7
❖ GI-186, "Potential Risk and Consequences of Heavy Load Drops in Nuclear Power"	8-10
❖ GI-189, "Susceptibility of Ice Condenser and Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident"	11-15
❖ GI-191, "Assessment of Debris Accumulation on PWR Sump Performance"	16-21
❖ GI-193, "BWR ECCS Suction Concerns"	22-24
❖ GI-199, "Implications of Updated Probabilistic Seismic Hazard Estimates in Central and Eastern United States for Existing Plants"	25-28

Generic Issue Management Control System

Description

The Generic Issue Management Control System (GIMCS) provides information necessary to manage the resolution of generic issues (GIs). The resolution of any GI has the potential for safety enhancements and the promulgation of new or revised requirements or guidance.

GIMCS is part of an integrated system of reports and procedures that is designed to manage GIs from issue identification through resolution (development of new criteria, management review and approval, public comments, and incorporation into the regulations, as appropriate). The priority evaluation of generic issues (i.e., listed as HIGH- or MEDIUM) is primarily of historical significance only as issue prioritization was discontinued in 1999 with issuance of MD 6.4. Issue priority in this report and in NUREG-0933, "A Prioritization of Generic Safety Issues," is retained, where applicable, for historical purposes.

For reactor issues, the "Procedures for Processing Generic Issues" are outlined in the Office of Nuclear Regulatory Research (RES) Office Instruction TEC-002, dated September 29, 2005. The procedures for processing non-reactor issues are documented in the Office of Nuclear Material Safety and Safeguards (NMSS) Policy and Procedures Letter 1-57, Revision 1, "NMSS Generic Issues Program," dated October 1997. In 1999, Management Directive (MD) 6.4, "Generic Issues Program," was initiated for the processing of all new GIs; MD 6.4 was revised on July 29, 2005; and is currently under revision again with scheduled completion in mid 2008.

The data fields (or elements) documented in GIMCS include 22 items as described below. Some of these data fields (e.g., priority) are not used for new GIs, but have historical value for tracking legacy GIs.

Generic Issue Management Control System

Legend

ACRS	- Advisory Committee on Reactor Safeguards
ASME	- American Society of Mechanical Engineers
BNL	- Brookhaven National Laboratory
BWR	- Boiling Water Reactor
BWROG	- Boiling Water Reactor Owners Group
DCH	- Direct Containment Heat
CEUS	- Central and Eastern United States
CRGR	- Committee to Review Generic Requirements
DE	- Division of Engineering
DSA	- Division of Systems Analysis
ECCS	- Emergency Core Cooling System
EPRI	- Electric Power Research Institute
FIN	- Financial Identification Number
FRN	- Federal Register Notice
GI	- Generic Issue (same meaning as GSI)
GIMCS	- Generic Issue Management Control System
GL	- Generic letter
GSI	- Generic Safety Issue
H	- HIGH-priority GSI
IN	- Information Notice
IPEEE	- Individual Plant Examination of External Events
M	- MEDIUM-priority GSI
MD	- Management Directive
NEI	- Nuclear Energy Institute
NMSS	- Office of Nuclear Material Safety and Safeguards
PRA	- Probabilistic Risk Assessment
PWR	- Pressurized Water Reactor
PWROG	- Boiling Water Reactor Owners Group
RES	- Office of Nuclear Regulatory Research
RG	- Regulatory Guide
RI	- Regulatory Impact
S	- Subsumed in Another Issue (No.)
SOW	- Statement of Work
SRP	- Standard Review Plan
SSE	- Safe Shutdown Earthquake
STS	- Standard Technical Specification
TAC	- Task Action Control
T/A	- Technical Assistance
TAP	- Task Action Plan
TBD	- To be Determined
TI	- Temporary Instruction
TS	- Technical Specification
USI	- Unresolved Safety Issue

Generic Issue Management Control System

Data Elements

Management and control indicators used in GIMCS are defined as follows:

1. Issue No. Generic Issue Number
2. Title Generic Issue Title
3. Identification Date Date the issue was identified
4. Prioritization Date The date that the prioritization evaluation was approved by the RES Director (historical value only for issues identified before 1999)
5. Type Generic Issue (GI)
6. Priority High (H), Medium (M), or Continue (Priority designations of H and M have historical value only for issues identified before 1999)
7. Task Manager Name of assigned individual responsible for resolution
8. Office/
Division/Branch The Office, Division, and Branch of the Task Manager who has lead responsibility for resolving the issue
9. Action Level Active
Technical assistance funds appropriated for resolution and/or Task Manager actively pursuing resolution

Inactive
No technical assistance funds appropriated for resolution, Task Manager assigned to more important work, or no Task Manager assigned

Resolved
All necessary work has been completed and no additional resources will be expended

Regulatory Office Implementation
The GI has exited the formal GIP but actions outside the GIP remain, RES actions of safety / risk assessment or regulatory assessment are complete, and remaining actions reside with program offices
10. Status Coded summary as follows:
3A - (Resolved with requirements)
3B - (Resolved with No requirements)
11. TAC Number Task Action Control (TAC) number assigned to the issue

Generic Issue Management Control System

Data Elements (continued)

- | | | |
|-----|-------------------------------------|--|
| 12. | <u>Resolution Date</u> | Scheduled resolution date for the issue |
| 13. | <u>Work Authorization</u> | Who or what authorized work to be done on the issue |
| 14. | <u>FIN</u> | Financial identification number assigned to contract (if any) for technical assistance |
| 15. | <u>Contractor</u> | Contractor name |
| 16. | <u>Contract Title</u> | Contract Title (if contract issue) |
| 17. | <u>Work Scope</u> | Describes briefly the problem and the work necessary to technically resolve and complete the generic issue |
| 18. | <u>Status</u> | Describes current status of work while also retaining an accurate running narrative discussion of major activities, milestones, and decision points. |
| 19. | <u>Affected Documents</u> | Identifies documents into which the technical resolution will be incorporated |
| 20. | <u>Problem/Resolution</u> | Identifies current problem areas and describes what actions are necessary to resolve them. Note: Discussions of previous problems and resolutions are incorporated into the status narrative, as appropriate. |
| 21. | <u>Reasons for Schedule Changes</u> | Describes reasons for and explain current changes in milestones (additions, deletions, and delays). |
| 22. | <u>Milestones</u> | Selected significant milestones:

<u>Original</u>
Scheduled dates reflected in the original Task Action Plan, plus additional milestone dates added during resolution of the GI

<u>Current</u>
Expected date of completion, or changes in the original scheduled dates

<u>Actual</u>
The date the milestone was completed |

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Issue Number 0156.6.1 **Type:** GSI **Office/Division/Branch:** RES/DRA/OEGIB
Title: PIPE BREAK EFFECTS ON SYSTEMS AND COMPONENTS

Priority H	Action Level ACTIVE	Resolution Status:
Task Manager: H. Vandermolten	TAC Number:	
Identification: 02/1991	Prioritization/Screen: 07/1999	Technical Assessment: 12/2007
Identification Status: Complete	Priority/Screen Status: Complete	Technical Assessment Status: C
Regulation and Guidance Development:		Regulation and Guidance Issuance Status:
Regulation and Guidance Development Status: TBD		Regulation and Guidance Development Status: TBD
Implementation:	Verification:	Closure: 12/2007
Implementation Status: TBD	Verification Status: TBD	Closure Status: C
Work Authorization: Memo from A. Thadani to E. Rossi dated July 16, 1999.		

FIN NUMBER	CONTRACTOR	CONTRACT TITLE	EXPENDED (\$k)
Y6406	ISL		\$195.00
Total Resources Expended (K):			\$195.00

WORK SCOPE:

Description

GDC 4 is the primary regulatory requirement of concern. It requires, in part, that structures, systems and components important to safety be appropriately protected against the environmental and dynamic effects that may result from equipment failures, including the effects of pipe whipping and discharging fluids. Several possible scenarios for plants that do not have adequate protection against pipe whip were identified as a result of research.

Work Scope

The objective of the GI-156.6.1 TAP is to determine through analysis if: (1) a high energy pipe break inside a BWR Mark I containment has the potential to perforate the drywell shell and possibly disable accident mitigation systems; and (2) a high energy pipe break inside a BWR Mark I or Mark II containment can disable the control rod drive (CRD) scram system. The TAP is a follow-on to NUREG/CR-6395, "Enhanced Prioritization of Generic Safety Issue 156.6.1 Pipe Break Effects on Systems and Components Inside Containment," which was performed by the Idaho National Engineering and Environmental Laboratory (INEEL) and issued in November 1999, and the screening evaluation, "A Screening Evaluation of GSI-80 Pipe Break Effects on Control Rod Drive Hydraulic Lines in the Drywell of BWR Mark I and II Containments" attached to the February 14, 2003 memorandum from Thadani to Collins concerning GSI-80. Individual TAP section reports will be issued when analysis information is obtained. All TAP sections are not required to be completed if a bounding analysis finds the associated risk to be inconsequential.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

STATUS:

A letter was sent from F. Eltawila (NRC) to W. Glenn Warren (BWROG) expressing concerns related to the GSI. The BWROG responded on 01-10-2001 that a committee was formed to coordinate the response to the ACRS. There are a total of 16 SEP III BWRs. A Task Action Plan for resolving the issue was approved in May 2001.

Task 4 of Contract Y6406 (NRC-04-01-67) was issued to Information Systems Laboratories (ISL). ISL issued a draft report in September addressing many of the BWROG peer review comments on the prioritization done by INEEL (issued in 1999). The ISL report has been reviewed and comments have been made. In December 2002, ISL completed its review of technical comments made by the BWROG on the INEEL 's "Enhanced Prioritization of Generic Safety Issue 156.6.1 Pipe Break Effects on Systems and Components Inside Containment." ISL concluded that, in general, INEEL's analysis was overly conservative in its risk estimates, and simplistic in accident sequence development. A followup meeting was held on 1/15/03 to discuss potential options for resolution of differences. A meeting to discuss options was held on March 19, 2003. The ongoing reevaluation of 10 CFR 50.46, "Acceptance Criteria for Emergency Core Cooling Systems for Light-Water Nuclear Power Plants," will be considered in the technical assessment of this GSI.

The Task Action Plan for the partial resolution of GSI 156.6.1, "Pipe Break Effects on Systems and Components Inside Containment," and GSI-80, "Pipe Break Effects on Control Rod Drive Hydraulic Lines in the Drywells of BWR Mark I and II Containments," was approved on February 3, 2004 (ML040340549). Prior to his departure from the NRC, Task Manager Ron Lloyd completed a technical evaluation of the effects of postulated pipe breaks inside BWR Mark I and Mark II containments in July 2004. The ANSYS finite element code was used to perform nonlinear transient analysis to determine the impact of impulsive loads due to pipe breaks in feedwater, main steam, and recirculation system piping on drywell steel shell and control rod drive (CRD) bundles. The results of the analysis indicated that the structural integrity and leak-tightness of the drywell steel shell will not be compromised due to pipe impact. The calculations indicate that: (1) the drywell steel shell will yield locally at the point of impact but will not perforate and cause an over-pressure in the annular space between the steel shell and concrete shield wall; (2) the CRD bundles will not be impacted by breaks in recirculation, steam, and feedwater system piping after a postulated break. The next step is to confirm the staff's findings with inspections at a minimum of 3 PWR plants.

In October 2005, DSARE conducted a review of 37 operating plants as part of its selection of certain plants for plant walkdowns. In November 2005, DSARE identified 16 plants (23 reactors) that needed to be reviewed. In April 2006, DRASP requested NRR assistance in gathering detailed plant layout information. A survey of the layout of those plants affected by the issue was completed by the Office of Nuclear Reactor Regulation (NRR) in September 2006. The GIP review of piping configurations to identify the most vulnerable plants is complete. One site for which there was insufficient information to form a conclusion may have a vulnerability. This condition appears to be site specific. Staff from RES and NRR met on 06/01/2007 and decided NRR would interface with the licensee for this site to help identify and assess options for further assessment of this potential vulnerability. Based on information provided by the licensee on June 29, 2007, the issue was resolved for this plant site.

A technical assessment report was prepared and transmitted to the ACRS on July 18, 2007. The ACRS was then briefed at its meeting on September 9, 2007. On September 26, 2007, the ACRS issued a memorandum to the NRC Executive Director for Operations stating the ACRS concurs with the staff recommendation that GI-156.6.1 be closed and that no further actions by the NRC staff of licensees with respect to this issue are necessary (ADAMS ML072530615). The issue was closed in December 21, 2007, (ADAMS ML073170185).

Staff Resources Expended: 1570 hours

AFFECTED DOCUMENTS:

None identified.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Task Action Plan Approved	05/01/2001		05/30/2001
Task Manager Reassigned to Other Duties	07/01/2001		07/01/2001
New Task Manager Assigned	01/01/2002		01/01/2002
Draft Contractor Report	09/01/2002		12/31/2002
Meeting to Discuss Options	03/19/2003		03/19/2003
Complete Draft Task Action Plan	11/01/2002		07/31/2003
Decision to Integrate GSI-80 into Technical Assessment of GSI-156.6.1	10/02/2003		10/02/2003
Approval of Task Action Plan	11/30/2003		02/03/2004
High Energy Piping Interactions with BWR Mark I Drywell Shells	03/31/2004		03/31/2004
Analysis and Documentation of Calculation Results	06/30/2004		07/31/2004
Identify Plants to be Visited	11/30/2005		11/30/2005
Select PWRs for Site Visits	09/30/2005		06/08/2006
Complete NRR PMs Survey of Affected Plants	03/31/2006		09/30/2006
Complete GIP Review of Piping Configurations at PWR Plants	09/30/2005	12/31/2006	10/17/2006
Perform Assessment of Plants Based on Specific Piping Configurations	04/30/2007	04/30/2007	03/30/2007
Draft Recommendations	08/31/2004	06/30/2007	06/27/2007
Meet with ACRS	02/28/2006	09/30/2007	09/06/2007
Close Out Issue with Memo to the EDO	06/30/2006	12/31/2007	12/21/2007

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

ensuring SG tube integrity.)

On December 1, 1997, the industry informed the staff of an industry initiative, NEI 97-06, "Steam Generator Tube Integrity Guidelines," which paralleled the above draft regulatory guide and which all PWR licensees had committed (among themselves) to implement. NEI 97-06 provides a programmatic, performance based approach to ensuring SG tube integrity. With commission approval, the staff put the above generic letter initiative on hold and worked with the industry to identify revised technical specifications which would be aligned with the NEI 97-06 initiative and which would ensure that all PWR licensee's are implementing programs which ensure that SG tube integrity will be maintained. This effort was completed in May 2005 with the staff's approval of the TSTF-449, Rev 4 which includes a new standard technical specification template governing SG tube integrity.

Regarding the DPO, its nature evolved considerably in the years subsequent to 1991, adding additional concerns relating to alternate tube repair criteria, iodine spiking assumptions for radiological analysis, severe accidents, and many other concerns. The staff prepared a DPO consideration document which it provided to the EDO on September 1, 1999. At the EDO's request, the ACRS served as an equivalent ad hoc panel to review the DPO issues. The ACRS met with the DPO author and other members of the NRC staff and reviewed the documentation related to the DPO issues. The ACRS issued NUREG-1740 on February 1, 2001 documenting its conclusions and recommendations. By memorandum dated May 11, 2001, NRR and RES developed a joint action plan to address the conclusions and recommendations in the ACRS report. This action plan and resolution of GSI 163 was later incorporated into the NRC Steam Generator Action Plan, the status of which was presented to the Commission in SECY-03-0080 and discussed at a Commission meeting on May 19, 2003. (A copy of the NRC SG Action Plan, milestones, schedule, and current status can be found on the NRC public web page.)

The scope of the DPO issues and followup SG Action Plan tasks relevant to GSI 163 are those which could potentially impact needed SG tube inspection, maintenance and repair activities. In contrast, any needed actions to address containment bypass scenarios due to tube failure during severe accidents would likely involve changes to accident management procedures and perhaps hardware modifications not involving the steam generators and, therefore, are outside the scope of GSI-163. Similarly, iodine spiking and radiological assessment issues are outside the scope of GSI-163. DPO issues outside the scope of GSI-163 will continue to be managed under the SG Action Plan umbrella.

STATUS:

In response to NRC Generic Letter 2006-01, "Steam Generator Tube Integrity and Associated Technical Specifications," all PWR licensees have submitted license amendment applications to change their technical specifications in accordance with TSTF-449. The staff has approved and issued amendments for 48 PWRs. The staff has targeted December 31, 2007 for issuing amendments for the remaining PWRs.

SG Action Plan tasks relevant to resolution of GSI-163 have been completed with the exception of task 3.1.k. SG Action Plan task 3.1.k involves evaluation of the conditional probabilities of multiple tube failures for risk assessment pertaining to SG alternate repair criteria. To support the needs of the GSI, the staff is actually performing this task from the broad standpoint of the integrity of the overall tube rather than being narrowly focused on tube locations with alternate repair criteria.

The staff is targeting April 30, 2009 for issuing memorandum to the EDO documenting the resolution of GSI-163 and the supporting technical bases.

Staff Resources Expended: 1800 hours

AFFECTED DOCUMENTS:

NUREGs 1430 - 1432 regarding Standard Technical Specifications
NRC Generic Letter 2006-01
plant specific technical specifications for PWRs

PROBLEM/RESOLUTION:

First Quarter, FY08

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Lessons learned from work completed so far have necessitated several modifications and additions to tasks. These are being formalized in the RES Operating Plan and the SG Action Plan.

REASONS FOR SCHEDULE CHANGES:

1. As approved by the Commission in an SRM dated December 21, 1998, development of new technical specifications for ensuring SG tube integrity involved a cooperative effort between the NRC staff and the industry. That it took seven years to reach agreement with the industry is attributable to the complexity of the issues involved and that consensus building within the industry itself proved to be a time consuming process.
2. ACRS findings in NUREG-1740 necessitated followup tasks relating to GSI-163, the last of which is not scheduled for completion until January 2008.

Milestone	Original Date	Current Date	Actual Date
Regulatory Analysis	05/01/1997		05/01/1997
Proposed GL Package	06/01/1997		10/01/1997
ACRS Endorsement	06/01/1997		10/01/1997
GL Package Placed in Concurrence	10/01/1997		10/01/1997
NEI 97-06 Submitted	12/01/1997		12/01/1997
GL Package Sent to CRGR by NRR	07/01/1997		04/01/1998
CRGR Meeting on GL Package	06/01/1998		06/01/1998
CRGR Meeting on Proposed GL	07/01/1998		07/01/1998
NRR Memo to EDO Putting GL on Hold	09/01/1998		09/01/1998
Commission Paper Recommending Hold on Issuance of GL	11/01/1998		10/01/1998
SRM on SECY-98-248	12/01/1998		12/01/1998
DPO Consideration Document to the EDO	09/01/1999		09/01/1999
EDO Establishes an Independent Panel to Review the DPO	02/01/2000		05/01/2000
ACRS to Perform DPO Review Panel Function	10/01/2000		10/01/2000
ACRS to Provide Conclusions and Recommendations	12/01/2000		02/01/2001
NRR & RES Issue Joint Action Plan	05/31/2001		05/31/2001
Approve TSTF-449	05/31/2005		05/31/2005

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Issue Generic Letter 2006-01	01/20/2006		01/20/2006
Issue Revised Technical Specifications - All PWRs	12/31/2007		09/30/2007
Brief ACRS on Proposed GSI Resolution	11/30/2008		
Close Out Issue with Memo to the EDO	02/28/2001	04/30/2009	

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

good practices for crane operations and load movements. Continue to assess implementation of heavy load controls in safety-significant applications through the Reactor Oversight Process. (4) Evaluate the need to establish standardized load drop calculation methodologies for heavy load drops.

STATUS:

The report on the potential risk and consequences of heavy load drops in nuclear power plants was completed in June 2003, after NRR comments were addressed by RES. The publication of the report, NUREG-1774, "A Survey of Crane Operating Experience at U.S. Nuclear Power Plants from 1968 Through 2002," in July 2003 completed the initial screening stage of the issue. The proposed recommendations resulting from the technical assessment of the issue were discussed with the ACRS Full Committee on September 11, 2003. Three of the RES recommendations on regulation and guidance development were sent to NRR on November 12, 2003. By letter dated February 4, 2004, NRR informed RES that these three recommendations would be implemented through issuance of a Regulatory Issue Summary that clarifies and reemphasizes existing regulatory guidance for control of heavy loads. The remaining recommendation was resolved by DET/RES on May 4, 2004, with the conclusion that existing industry standards were adequate for application to load drop analyses.

The staff has been participating with the American Society of Mechanical Engineers (ASME) Cranes for Nuclear Facilities Committee in comparing the provisions of the industry crane standard, ASME NOG-1, "Rules for Construction of Overhead and Gantry Cranes," with the NRC guidelines contained in NUREG-0554, "Single Failure-Proof Cranes for Nuclear Power Plants," in support of future endorsement of the industry standard. In September 2004, NRR reported that the Committee action in support of NRC endorsement was delayed. In April 2005, the staff identified an emergent concern with the adequacy of evaluations of heavy load drops. NRR issued Regulatory Issue Summary (RIS) 2005-25 on October 31, 2005, to clarify and reemphasize existing regulatory guidance for the control of heavy loads.

Though its work with the Committee, the NRR staff has concluded that the industry standard, ASME NOG-1, provides improved guidance for construction of new single-failureproof cranes. Therefore, the staff elected to endorse the ASME NOG-1, 2004, through the Standard Review Plan Update Program in March 2007. The NRC staff understands that the committee will provide the comparison as an appendix to a future revision of ASME NOG-1. The staff also modified the guidelines for slings used with single-failure-proof handling systems in the Standard Review Plan (NUREG-0800), Section 9.1.5, "Overhead Heavy Load Handling Systems," based on a review of operating experience issues. The staff issued Supplement 1 to RIS 2005-25 to notify industry of the changes to SRP Section 9.1.5 and further clarify existing regulatory expectations associated with 10 CFR 50.59 and 50.71(e), as these requirements relate to the safe handling of heavy loads and load drop analyses.

On September 14, 2007, the Nuclear Energy Institute (NEI) notified the NRC that the nuclear industry approved a formal initiative that specifies actions each plant will take to ensure that heavy load lifts continue to be conducted safely and that plant licensing bases accurately reflect plant practices. The initiative is expected to clarify the licensing basis with respect to handling of heavy loads, and the NRC staff is modifying guidance documents to accommodate the initiative. The initiative includes development of guidelines for realistic load drop analyses. On December 13, 2007, the NRC staff participated in a public meeting with NEI to discuss implementation of the initiative and criteria for acceptable reactor vessel head load drop analyses.

Staff Resources Expended: 3,000 hours

AFFECTED DOCUMENTS:

NUREG-1774
Standard Review Plan (NUREG-0800), Section 9.1.5

PROBLEM/RESOLUTION:

None

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

REASONS FOR SCHEDULE CHANGES:

The NRC staff is modifying guidance documents to allow time for implementation of the initiative and to be consistent with the expected end state following implementation. The ACRS brief is rescheduled to conform with the additional time necessary to modify guidance documents.

Milestone	Original Date	Current Date	Actual Date
Publish NUREG-1774	06/30/2003		06/30/2003
Meet with ACRS Full Committee	09/01/2003		09/11/2003
ACRS Memo to the EDO on Staff Recommendations	09/24/2003		09/24/2003
Complete Technical Assessment and Transfer Issue to NRR for Regulation and Guidance Development	10/31/2003		11/12/2003
DSARE/RES Memo to DET/RES Requesting Industry Code Committee Evaluation	11/21/2003		11/21/2003
DET/RES Memo to DSARE/RES Concluding Existing Industry Code Adequate for Load Drop Analysis	05/04/2004		05/04/2004
Issue RIS 2005-25 to Clarify and Reemphasize Existing Regulatory Guidance for Control of Heavy Loads	12/31/2004		10/31/2005
Issue RIS 2005-25, Supplement 1 to Address Endorsement of Industry Standard	02/28/2006	04/30/2007	05/29/2007
Enhance Inspection Procedures for Heavy Loads	09/30/2007	06/30/2008	
Brief ACRS on Implementation of Recommendations	11/30/2004	08/28/2008	
Issue Closeout Memo to the EDO	08/31/2005	08/28/2008	

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

On January 30, 2003, NRR prepared a reply memorandum that outlined the next steps in the resolution of this GI. NRR prepared a Task Action Plan to complete MD 6.4, Stage 4, Regulation and Guidance Development, based on a preliminary decision to issue an Order. The staff reviewed the proposed regulatory actions and associated draft documents with senior management and OGC, and senior management decided to pursue Rulemaking rather than an Order. The staff held a public meeting on June 18, 2003, to receive feedback from licensees and other stakeholders regarding the need to provide a backup power supply to the hydrogen igniters and NRR's consideration of rulemaking for the resolution of GI-189. NRR staff briefed the ACRS on November 6, 2003, and recommended providing a backup power supply to the hydrogen igniters. On November 17, 2003, the ACRS Chairman wrote the NRC Chairman recommending the NRC proceed with rulemaking to require a backup power supply to the hydrogen igniters for PWR ice-condenser and BWR MARK III plants. The ACRS recommended that rulemaking include a small pre-staged generator with installed cables, conduit, panels, and breakers, or an equivalent diverse power supply. The ACRS also recommended that the rulemaking be accompanied by guidance that specifies the design requirements.

NRR developed design criteria for the backup power supply, and administered a contract to merge and enhance the existing technical assessment into a regulatory analysis. NRR held a public meeting with the public and industry on September 21, 2004, to get external stakeholders' input on the draft design criteria. The BWR owners indicated a willingness to make modifications to supply power from the existing HPCS diesel generator, and agreed to provide additional information regarding implementation cost for the prestaged generator and relative risk contribution of SBO events at each of the four Mark III plants. Duke power, representing two PWR ice condenser sites, Catawba 1 & 2, McGuire 1 & 2, indicated a willingness to make modifications to an existing safe shutdown diesel generator that could manually connect to provide backup power source as needed. American Electric Power representatives indicated a willingness to provide backup power source for D. C. Cook 1 & 2 from the large diesel generators intended to support an increased allowed outage time for the emergency diesel generators. TVA, representing two PWR ice condenser sites, Sequoyah 1 & 2, Watts Bar 1, also indicated a willingness to provide a backup power source from a supplemental diesel generator. In November 2004, the staff reached a consensus to evaluate the proposed voluntary initiatives and pursue that path as a preferential solution.

In February and early March 2005, the NRR staff met with representatives of RES, NSIR, and OEDO to develop an understanding of newly identified safety/security interface issues and actions initiated in the security arena that could impact the solution of the issue. On March 30, 2005, the staff met with senior representatives of the six affected utilities to present security-related insights.

On June 14, 2005, the EDO issued a memorandum to the Commissioners to inform the Commission of the regulatory analysis results and recent staff activities on GSI-189. The regulatory analysis indicated that the backup power modification may provide a substantial safety benefit at a justifiable cost for the PWRs with an ice-condenser containment, and the proposed voluntary actions provide the majority of the benefit. The costs exceed the benefits for all BWR regulatory options, and none of the options for the BWRs provides a substantial increase in the overall protection of public health and safety. However, external events and security insights were not fully evaluated in the regulatory analysis, and defense-in-depth considerations in improving the balance among accident prevention and mitigation provide an additional un-quantified benefit for both containment types.

STATUS:

Based on an understanding that many of the voluntary physical modifications had been completed, the staff elected to delay seeking specific commitments while security-related reviews of the facilities were ongoing. On March 1, 2006, the EDO issued a memo informing the Commission of the staffs intent to delay the request for commitments until after the security-related reviews were completed in September 2006. Because this issue was not incorporated in the scope of security-related modifications, the staff has held closed meetings in December 2006 and January 2007 to further explore the proper consideration of security insights in the design of the modifications. The staff received industry proposals for modifications that incorporate security insights in late February and early March 2007. The staff reviewed the industry proposals and concluded that the proposed modifications would resolve GSI-189 and provide benefit for some security scenarios. On April 23, 2007, the EDO issued a memo informing the Commission of the staffs intent to accept the commitments and perform verification inspections at the affected sites. On June 15, 2007, the NRC staff issued letters to affected licensees accepting the commitments. The NRC staff also notified licensees of the intent to perform verification inspections at the affected sites and clarified the scope of the inspection relative to the commitments. Based on industry proposals, the staff expects full implementation of the modifications to be completed by June 2008 at nearly all affected units, with two units delayed as late as early 2010 for more complex modifications.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Staff Resources Expended: 8,000 hours

AFFECTED DOCUMENTS:

10 CFR 50.44
10 CFR 50.34

PROBLEM/RESOLUTION:

The costs exceed the benefits for all BWR regulatory options, and none of the options for the BWRs provides a substantial increase in the overall protection of public health and safety. However, external events and security insights were not fully evaluated in the regulatory analysis, and defense-in-depth considerations in improving the balance among accident prevention and mitigation provide an additional un-quantified benefit for both containment types. With consideration of security insights, all affected licensees have proposed modifications that adequately address the identified safety issue.

REASONS FOR SCHEDULE CHANGES:

The staff received initial industry proposals for modifications that incorporate security insights in late February and early March 2007.. The staff reviewed the industry proposals and concluded that the proposed modifications would resolve GSI-189 and provide benefit for some security scenarios. On April 23, 2007, the EDO issued a memo informing the Commission of the staffs intent to accept the commitments and perform verification inspections at the affected sites. On June 15, 2007, the NRC staff issued letters to affected licensees accepting the commitments. The NRC staff also notified licensees of the intent to perform verification inspections at the affected sites and clarified the scope of the inspection relative to the commitments. Based on industry proposals, the staff expects full implementation of the modifications to be completed by June 2008 at nearly all affected units, with two units delayed as late as early 2010 for more complex modifications.

Milestone	Original Date	Current Date	Actual Date
Draft Technical Assessment	05/01/2002		05/01/2002
Meet with ACRS	06/01/2002		06/06/2002
Second Meeting on Technical Assessment with ACRS Sub-Committee	10/01/2002		11/05/2002
Final Technical Assessment	11/01/2002		11/10/2002
Meet with ACRS Full Committee	11/01/2002		11/13/2002
Transfer GSI to NRR	12/01/2002		12/17/2002
Review RES Technical Assessment	02/28/2003		02/28/2003
Public Meeting with Stakeholders	02/28/2003		02/28/2003
Determine Best Course of Action	02/28/2003		02/28/2003
Finalize CRGR Package	03/26/2003		03/26/2003

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Prepare Guidance and Provide Results to NRR Management	03/26/2003		03/26/2003
Distribute Draft Order and SECY Paper	03/26/2003		03/26/2003
Provide Draft Order to OGC and Draft SECY to EDO	03/28/2003		03/28/2003
Meet with Rulemaking Committee	05/05/2003		05/05/2003
Conduct Public Meeting	06/18/2003		06/18/2003
Meet with OPA to Develop Communication Plan	06/24/2003		06/24/2003
Complete Communication Plan	07/10/2003		07/10/2003
NRR Meeting with ACRS	11/06/2003		11/06/2003
Public Meeting to Address Design Criteria	11/06/2003		11/06/2003
Public Meeting with Stakeholders	02/03/2004		02/03/2004
Brief Commissioner Merrifield	03/04/2004		03/04/2004
Public Meeting with Stakeholders	03/31/2004		03/31/2004
Issue Draft Design Criteria for Comment	08/13/2004		08/13/2004
Public Meeting with Stakeholders	09/21/2004		09/21/2004
Internal Meeting to Discuss Pursuit of Rulemaking	11/02/2004		11/02/2004
Perform Sensitivity Analysis to Determine Whether 2-Hour Startup Time for BWRs is Acceptable	11/30/2004		11/30/2004
Finalize Design Criteria	11/30/2004		11/30/2004
Decision on Voluntary Licensee Initiatives as Alternative to Rulemaking	11/30/2004		11/30/2004
Evaluate Safety/Security Interface	03/31/2005		03/30/2005
Issue Status Paper to Commission	05/31/2005		06/14/2005
Brief Commissioner Jaczko on Regulatory Analysis Results and Safety Significance	07/18/2005		07/18/2005
Meet with Owners to Discuss Safety-Security Interface Issues	08/03/2005		08/03/2005
Update Commission Regarding Licensee Plans for Voluntary Measures	03/01/2006		03/01/2006
Seek Commitment for Implementation of Voluntary Initiatives	08/31/2005		03/09/2007

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Request Information from Owners on Voluntary Actions Implemented	12/31/2005		03/09/2007
Complete Regulation and Guidance Development	06/30/2006	04/30/2007	04/23/2007
Clarify Commitments to Resolve Any Remaining Issues	12/31/2007	12/31/2007	06/15/2007
Complete Implementation	06/30/2008	01/31/2010	
Complete Verification	06/30/2009	06/30/2010	
Close Out Issue with Memo to the EDO	06/30/2010	06/30/2010	

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

ECCS operation to facilitate the review of any changes to plants that may be warranted; and (4) issue Generic Communication and work with the industry plan to evaluate and resolve GSI-191 for all PWRs.

Preliminary parametric calculations were completed in July 2001 indicating the potential for debris accumulation for 69 cases. These 69 cases were representative of, but not identical to, the operating PWR population. The staff's Technical Assessment concluded that GSI-191 was a credible concern for the population of domestic PWRs, and that detailed plant-specific evaluations were needed to determine the susceptibility of each U.S.-licensed PWR to ECCS sump blockage. Following the ACRS agreement with the staff's Technical Assessment of the issue in 09/2001, the issue was forwarded to NRR in a memorandum dated September 28, 2001. Consistent with Management Directive 6.4, NRR has the lead for Stages 4 through 6 of the Generic Issues Process for GSI-191. NRR has evaluated the technical assessment, and prepared a Task Action Plan for developing appropriate regulatory guidance and resolution of GSI-191.

STATUS:

Following meetings with stakeholders on March 5 and April 29, 2003, the NRC issued Bulletin 2003-01 to PWR licensees on June 9, 2003 to: (1) confirm their compliance with 10 CFR 50.46 (b)(5) and other existing applicable regulatory requirements, or (2) describe any compensatory measures that have been implemented to reduce the potential risk due to post-accident debris blockage, as evaluations to determine compliance proceed. All PWR licensees provided a response to the Bulletin, indicating interim compensatory measures and candidate operator actions that would be implemented. The Safety Issues Resolution Branch (SSIB) reviewed and evaluated the information provided and determined that the licensee's actions were responsive, and met the requirements of Bulletin 2003-01. The Division of Reactor Licensing (DORL) issued close-out letters to the PWR licensees as these reviews were completed. Generic close-out of Bulletin 2003-01 was completed in December 2005.

Generic Letter (GL) 2004-02 was issued in September 2004 requesting licensees to perform plant-specific mechanistic evaluations of sump performance following LOCA and HELB events, and to implement corrective actions as required to ensure compliance with regulatory requirements. The Nuclear Energy Institute (NEI) provided a guidance report (GR) to the staff in May 2004 containing the industry's proposed evaluation methodology for performing the plant specific evaluations. The staff reviewed the GR and issued a draft Safety Evaluation (SE), which supplemented the GR. The staff presented the SE to CRGR, and to the ACRS Subcommittee and Full Committee in September and October 2004, respectively. The final SE was issued in December 2004, resulting in an NRC-approved evaluation methodology. In January and April 2005, the staff held public meetings with NEI and owners to discuss the GL and SE, and to address questions as the evaluations were performed using the SE and GR.

Generic Letter 2004-02 required licensees to respond within 90 days to document the actions planned by the licensee to perform the sump evaluation, and the proposed schedule for completion. All PWR licensees responded to the GL on schedule in September 2005. All PWR licensees committed to modify their containment sump strainer, except for three plants who had modified their containment sump strainers within the last five years. The staff evaluated all 90-day responses to Generic Letter 2004-02 and in January 2006 issued comments to licensees to be addressed in their final response submittals.

To address concerns regarding the potential for chemical precipitates and corrosion products to significantly block a fiber bed and increase the head loss across an ECCS sump screen, a joint NRC/Industry Integrated Chemical Effects Testing program was started in 2004 and completed in August 2005. Chemical precipitation products were identified during the test program, and follow-up testing and analyses will be needed to address the effect on head loss. IN 2005-26, "Results of Chemical Effects Head Loss Tests in a Simulated PWR Sump Pool Environment," was issued on September 16, 2005.

The NRC conducted additional research in certain areas to support evaluation efforts and provide confirmatory information. These areas include research on chemical effects to determine if the pressurized-water reactor sump pool environment generates byproducts which contribute to sump clogging, research on pump head losses caused by accumulation of containment materials and chemical byproducts, and research to predict the chemical species that may form in these environments. The staff completed reports on the chemical effects on ice condenser containments on 01/13/2006 (ML053550433), and on PWR containments on 01/20/2006 (ML060190713). Supplement 1 to IN 2005-26 was issued on January 26, 2006 to specifically provide additional information regarding test results related to chemical effects in environments containing dissolved phosphate (e.g., from trisodium phosphate) and dissolved calcium.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

NRR anticipates that recipients will review the information for applicability to their facilities and consider taking actions, as appropriate, to avoid similar issues. Research was also conducted and documented on the transportability of coating chips in containment pool environments, and on the effect of ingested debris on downstream valve performance.

Between July and September 2006, the staff completed research on: (1) the thermodynamic simulation of containment sump pool chemical constituents, to predict the chemical reactions/byproducts in the pools; (2) the pressure loss across containment sump screens due to fiber insulation, chemical precipitates, and coating debris; and (3) a literature survey to summarize the knowledge base to date on the potential contribution of material leached from containment coatings to the chemical products formed in the containment sump pool, after a loss-of-coolant accident. Additional research activities included development of a revised head-loss correlation and completion of a peer review of the NRC's chemical effects research program. All planned NRC-sponsored research activities for GSI-191 are now complete and documented, though information obtained as the staff reviews industry activities to support issue closure may indicate the need for additional NRC-sponsored research.

As part of the plan to confirm adequate implementation and resolution of GSI-191, the NRC is conducting detailed plant audits examining the analyses and design changes used to address the technical issues. Visits to strainer vendor test facilities will also be included as part of this audit process. Two pilot audits were performed in 2005 (Crystal River Unit 3 and Fort Calhoun) to provide opportunities to exercise and improve the NRC evaluation process. Currently, 10 plant audits are planned. Audit reports will be posted on the NRC's ADAMS document control system as they become available. Audit activity in the 1st Quarter of FY-2008 included completion of the Waterford and North Anna audits, and visits to Salem and Indian Point to initiate audits. To support the audits, the NRC staff also made some visits to sump strainer vendor facilities to observe ongoing head loss and chemical effects testing, and the staff is reviewing vendor head loss testing protocols. The NRC staff is also systematically evaluating remaining technical questions related to GSI-191 to support a decision on whether additional confirmatory research is needed and if so, on what time frame.

In addition to the plant audits identified above, the staff will use inputs from review of licensee responses to GL 2004-02 and items identified from Regional inspections using Temporary Instruction TI-2515/166 to support closure of GSI-191. Review of final licensee GL responses will begin after they are submitted, which is expected to occur in early 2008. Inspections by regional staff will verify proper implementation of planned modifications.

Plant-specific issues have led to many licensees identifying a need to request an extension beyond the NRC-identified date of December 31, 2007 for completion of certain corrective actions for GL 2004-02. Because they are plant-specific, these extension requests must be evaluated on a case-by-case basis. As of early December 2007, extension requests for 17 PWR units have been approved based on, for example, strong sets of interim compensatory measures, significant interim or final sump screen area, fibrous insulation removal or lack of fibrous insulation issues, installation of debris interceptors, and short periods of extension. Another 30 requests were under staff review in early December. Most extensions requested have been for relatively short periods (i.e., less than six months). However, several licensees have requested longer extensions. The NRC has indicated to such licensees that they must show the plant can be operated safely for the extension period and that the extension is absolutely necessary (i.e., actions cannot reasonably be performed sooner).

To provide open communication on NRC activities associated with GSI-191 resolution, public meetings with NEI and industry representatives continue to be held regularly, as schedules allow. Briefings of ACRS have been scheduled periodically to provide opportunities for communication on technical issues and additional public involvement.

AFFECTED DOCUMENTS:

- (1) Regulatory Guide 1.82, Rev. 3
- (2) NUREG-0800
- (3) Generic Letter 85-22
- (4) Bulletin 2003-01
- (5) Generic Letter 2004-02

PROBLEM/RESOLUTION:

First Quarter, FY08

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Complexities associated with chemical effects are making it challenging for licensees to test the adequacy of modified sump screens, and for the NRC to evaluate the testing. This is the principal reason many licensees will not have completely addressed GL 2004-02 by the end of 2007. Head loss testing accounting for chemical effects is ongoing but expected to generally be complete in the first half of 2008.

REASONS FOR SCHEDULE CHANGES:

Activities will occur in 2008 that are needed to support issue closure. The final staff audits of licensee corrective actions will be complete in spring 2008. Licensees will submit supplemental responses to GL 2004-02 by February 29, 2008. The staff's review of GL responses is expected to be complete in October 2008. The staff is also verifying, using Temporary Inspection Procedure TI-2515/166, that licensees have accomplished the activities related to GL 2004-02 to which they have committed. Completion reports for the TI will be due in summer 2008. Integrating these activities to support final issue closure, including ACRS and management reviews, results in planned closure of the GSI in December 2008. This date is two months later than the date that was provided in the last quarterly update. The additional time resulted from the NRC's decision to allow licensees an additional two months beyond December 31, 2007, to submit the responses to GL 2004-02. The NRC views the short additional time for licensees to submit requested documentation as of little safety significance.

Complexities associated with the impact of chemical effects on sump strainer performance continue to be challenging. With chemical effects testing ongoing, there is the possibility that additional time may be needed to fully address this aspect of the GSI.

RES changed the status of GI-191 to Regulatory Office Implementation (see ML071630094). This change is part of improvements to the Generic Issues Program (GIP) described in SECY-07-0022, "Status Report on Proposed Improvements to the Generic Issues Program," (ML063460239). This improvement obviates the need for milestones specifically associated with the Generic Issue Program after the implementation phase begins. Issue closure will occur in accordance with applicable NRR Office programs as indicated in the remaining milestones.

Milestone	Original Date	Current Date	Actual Date
NRR User Need Request Sent to RES	12/01/1995		12/01/1995
User Need Request Assigned to GSIB/RES	01/01/1996		01/01/1996
Reassessment Declared a New GSI	09/01/1996		09/01/1996
Issue SOW for Evaluation of GSI A-43	11/01/1996		11/01/1996
Complete Evaluation of GSI A-43	04/01/1997		03/01/1997
Issue SOW for Reassessment of Debris Blockages in PWR Containments Impact on ECCS Performance	09/01/1998		09/01/1998
Complete Collection and Review of PWR Containment and Sump Design and Operation Data	12/01/1999		12/01/1999
Complete All Debris Transport Tests	09/01/2000		08/01/2000
Complete Parametric Evaluation	07/01/2001		07/31/2001

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Proposed Recommendations to the ACRS	08/31/2001		08/31/2001
ACRS Review Completed	09/30/2001		09/14/2001
Issue Transferred from RES to NRR	09/28/2001		09/28/2001
Complete Reassessment of Debris Blockages in PWR Containments Impact on ECCS Performance	09/30/2001		09/28/2001
Prepare Memo Discussing Proposed Recommendations (End of Technical Assessment Stage of Generic Issue Process)	04/01/2002		09/28/2001
Complete Estimate of Average CDF Reduction, Benefits, and Costs	04/01/2002		09/28/2001
Issue Bulletin 2003-01	05/01/2003		06/01/2003
Complete Development of Models and Methods for Analyzing Impact of Debris Blockages in PWR Containments on ECCS Performance	04/01/2001		06/09/2003
Discuss Reg. Guide 1.82, Rev. 3 with ACRS Subcommittee on Thermal-Hydraulic Phenomena	08/20/2003		08/20/2003
Present Final Version of Reg. Guide 1.82, Rev. 3 to ACRS Full Committee	09/11/2003		09/11/2003
ACRS Letter on Final Version of Reg. Guide 1.82, Rev. 3	09/30/2003		09/30/2003
Draft Industry Guidance for Plant-Specific Analyses	10/30/2003		10/31/2003
Issue Reg. Guide 1.82, Rev.3	09/30/2003		11/30/2003
NRC Meeting with Stakeholders	03/23/2004		03/23/2004
NRC Meeting with Stakeholders	05/25/2004		05/25/2004
Receive Industry Guidance for Plant-Specific Analyses	09/30/2003		05/28/2004
NRC Meeting with Stakeholders	06/17/2004		06/17/2004
Brief ACRS Subcommittee on Proposed Generic Letter	06/22/2004		06/22/2004
NRC Meeting with Stakeholders	06/29/2004		06/29/2004
Brief Full ACRS Committee on Proposed Generic Letter	07/07/2004		07/07/2004
Develop Generic Letter for Resolution of GSI	07/07/2004		07/07/2004
Meet with CRGR on Proposed Generic Letter	08/10/2004		08/10/2004

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Issue Generic Letter 2004-02	09/13/2004		09/13/2004
Meet with ACRS on Safety Evaluation of NEI 04-07	10/07/2004		10/07/2004
ACRS Response on Safety Evaluation of NEI 04-07	10/18/2004		10/18/2004
Brief Commissioners Jaczko and Lyons on Status	07/18/2005		07/18/2005
EDO Briefing of ACRS on Status	09/09/2005		09/09/2005
Receive All GL Responses Addressing Plant-Specific Analyses	05/31/2005		09/15/2005
Issue Information Notice 2005-26	09/16/2005		09/16/2005
Complete Review of Licensee Responses to GL 2004-02	01/20/2006		01/20/2006
Issue Supplement 1 to IN 2005-26	01/20/2006		01/20/2006
Complete Research Programs Evaluating Coating Transportability and Surrogate Throttle Valve Debris Ingestion	02/28/2006		02/28/2006
Brief ACRS on Staff Evaluation of Licensee Responses to GL 2004-02 and Results of Chemical Effects Tests	03/09/2006		03/09/2006
Complete Testing and Analysis Associated with Initial Phase of Chemical Effects Research	05/30/2006		05/30/2006
Complete Containment Material Head Loss Testing	06/15/2006		06/15/2006
Complete Thermodynamic Simulation of Containment Sump Pool Chemical Constituents	09/30/2006		09/30/2006
Licensees (Except Those Who Have Obtained Extensions) Complete GL-2004-02 Activities	01/31/2007	12/31/2007	
Complete Last Audit Report	05/23/2008		
Regions Complete TI Inspections	06/30/2008		
Receive Last TI Verifications From Regions	08/11/2008		
Complete Review of TI Verifications	08/25/2008		
Complete Review of Licensee GL 2004-02 Responses for Adequacy	12/31/2007	09/24/2008	
Prepare Closure Memo for GL-04-02 Responses and TI Verifications	11/23/2008		
Complete NRR Review and Approval of GL Closure Memo	12/28/2008		

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Issue Number 0193

Type: GSI

Office/Division/Branch: RES/DSA/NARB

Title: BWR ECCS SUCTION CONCERNS

Priority

Action Level ACTIVE

Resolution Status: Cn

Task Manager: P. Kadambi

TAC Number:

Identification: 05/2002

Prioritization/Screen: 10/2003

Technical Assessment:

Identification Status: Complete

Priority/Screen Status: Complete

Technical Assessment Status: TBD

Regulation and Guidance Development:

Regulation and Guidance Issuance Status:

Regulation and Guidance Development Status: TBD

Regulation and Guidance Development Status: TBD

Implementation:

Verification:

Closure:

Implementation Status: TBD

Verification Status: TBD

Closure Status:

Work Authorization: Memorandum to A. Thadani from F. Eltawila, "Results of Initial Screening of Generic Safety Issue 193, 'BWR ECCS Suction Concerns,'" October 16, 2003

FIN NUMBER	CONTRACTOR	CONTRACT TITLE	EXPENDED (\$k)
	None		
			Total Resources Expended (K):

WORK SCOPE:

Description

The Generic Safety Issue (GSI) - 193, "BWR ECCS Suction Concerns" (initiated in 2002) evaluates possible failure of the emergency core cooling systems (ECCS) pumps (or degraded performance) due to unanticipated, large quantities of entrained gas in the suction piping from suppression pools in BWR Mark I, II, and III containments during LOCA conditions that could cause gas binding, vapor locking, or cavitation.

Work Scope

Screening analysis narrowed the safety concerns to Mark I containments. A Task Action Plan (TAP) for the Technical Assessment of the issue was approved in May 2004. Staff completed a literature search for information on ECCS pump performance during intake conditions at high voiding in March 2005. Staff continued the literature search, in accordance with Phase I of the Task Action Plan, and found experimental evidence that gas could reach the ECCS pumps during a loss-of-coolant accident. Staff also found that the pumps can recover with as much as 20 percent void fraction; however, the impact of voiding on the continued operation of the pumps remained an item of concern. The staff's plans are to pursue additional information from NRR, Region I, and foreign sources.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

The TAP to resolve this GSI involves a three part evaluation of suppression pool designs, dynamics of air entrainment in the suppression pool, and the impact on ECCS pump performance. Part A reviewed wetwell and suppression pool designs to establish bounding parameters and values. Part B reviewed relevant experiments on pool dynamics to identify available data. Part C originally intended to use information from Parts A and B to estimate the void fraction at the suction strainers as a function of the time after accident initiation, to assess the plausibility of air ingress into the ECCS strainers and intake piping, and to predict the potential impact on the ECCS pumps' ability to fulfill their design function. Part C has been modified to involve industry in assessing this issue, as appropriate. This applies the approach to resolving generic safety issues described in SECY-07-022, "Status Report on Proposed Improvements to the Generic Issues Program."

STATUS:

Staff completed parts A and B of the work scope and continues to work on part C of the work scope as described below.

A proposal for performing technical analyses was received from BNL in June 2005, in response to an RFP issued by the NRC in April 2005. In September 2005, information on suppression pool experiments was requested from the Technical Research Center of Finland and experimental results on thermal hydraulic phenomena from one plant were evaluated. A literature search was completed for two specific thermal-hydraulic phenomena: liquid gas jet; and bubble breakup.

During the 4th Quarter of FY-2006, discussions were initiated with NRR regarding commonality of concerns between GI-193 and those being addressed in a proposed Generic Letter addressing gas accumulation in ECCS suction piping covering all reactors. It was decided initially that the resolution of GI-193 would be pursued by RES independently, but with appropriate coordination with the NRR activities on gas management issues. After consideration of a research project to model the central issue in GI-193 (i.e., ability of Mark-1 BWR ECCS pumps to tolerate short periods of high void fraction operation), RES reached a decision in favor of working with NRR to issue an appropriate generic communication to affected licensees and revised milestones accordingly. Discussions with NRR ensued on the specifics of the generic communication and the schedule for its issuance.

By March 2007, the continuing discussions between RES and NRR about including this GI in the scope of the NRR proposed Generic Letter (GL) resulted in agreement not to include this GI in the GL. Inclusion was deemed impractical because the proposed the GI was sufficiently different and the proposed GL was substantially developed such that including the GI would have resulted in substantial rework. Accordingly, in March 2007, RES and NRR (the Generic Communication and Power Uprate Branch) decided to collaborate on obtaining BWR Owners Group cooperation to support the ongoing assessment of this GI. This approach is consistent with the principles described in SECY-07-022, "Status Report on Proposed Improvements to the Generic Issues Program."

NRR made preliminary contact with the BWROG in March 2007 and requested information to support this GI. The plan was to hold a meeting with BWROG by June 2007 to discuss their input and identify alternatives for completing part C of the work scope. Any resulting decisions on regulatory actions, implementation, and verification depend on the outcome of part C.

Instead of a meeting with the BWROG, a conference call was held on June 6, 2007 (ML071640257). The BWROG informed the staff that no plant specific studies have been done relative to GI-193 issues. They did not have any information regarding operability of ECCS pumps when air ingress might lead to void fractions greater than 20 percent. Within the first 30 seconds, no information was available on the period of time over which blow-down gas clears the suppression pool. However, they did provide references to two research reports from a the Lappeenranta University of Technology laboratory in Finland (ML071640273 and ML071640280). Subsequently, the BWROG informed the staff that further contacts with Finland have been initiated. The staff has also independently pursued contact with Finland through the Office of International Programs. The significance of the information provided by Finland will be evaluated.

The information obtained from Finland has been evaluated. The preliminary conclusion is that cavitation of the ECCS pumps is not as great a concern as when the issue was formulated. If more detailed examination of the applicability of the data to the plants subject to the concerns of GI-193 shows that the preliminary assessment are validated, the issue will be closed out without further action.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Staff Resources Expended: Estimated at 2,000 hours.

AFFECTED DOCUMENTS:

To be determined.

PROBLEM/RESOLUTION:

Engage the BWROG through the Generic Communication and Power Uprate Branch of NRR to participate in further evaluation of this GI for completing part C of the proposed work scope. (See ADAMS ML070920154 for e-mail between NRR and BWROG representatives on this collaborative effort.) Any resulting decisions on regulatory actions, implementation, and verification depend on the outcome of part C.

REASONS FOR SCHEDULE CHANGES:

Initial intent to include this issue in the scope of the GL on the topic of gas accumulation in suction piping of ECCS pumps being developed by NRR proved to be impractical based on the differences and development status of this GL. Later developments that placed information from Finland on the resolution path have extended the time needed to review BWROG data.

Milestone	Original Date	Current Date	Actual Date
Complete Task Action Plan for a Technical Assessment	03/31/2004		05/24/2004
ECCS Pump Performance Literature Search	03/31/2005		03/31/2005
Issue RFP to BNL for Technical Assistance	04/26/2005		04/26/2005
Receive Proposal for Technical Assistance from BNL	06/03/2005		06/03/2005
Request Information from Technical Research Center of Finland	09/12/2005		09/12/2005
Complete Literature Search for Two Specific Thermal-Hydraulic Phenomena	09/30/2005		09/30/2005
Evaluate Experimental Results on Thermal-Hydraulic Phenomena	09/30/2005		09/30/2005
Assign New Task Manager	05/15/2006		05/15/2006
RES Decision to Work with NRR on Generic Communication	08/31/2006		08/31/2006
Arrange Meeting With BWROG and Obtain Their Input	06/30/2007		06/06/2007
Review BWROG Data and Determine Regulatory Action	09/30/2007	12/31/2007	12/31/2007
Initiate Appropriate Action Commensurate With Risk-Significance	12/31/2007	03/31/2008	
Complete Safety and Risk Assessment Phase of GIP for this GI	03/31/2007	06/30/2008	

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Issue Number 0199

Type: GSI

Office/Division/Branch: RES/DRA/OEGIB

Title: IMPLICATIONS OF UPDATED PROBABILISTIC SEISMIC HAZARD ESTIMATES IN CENTRAL AND EASTERN UNITED STATES FOR EXISTING

Priority

Action Level ACTIVE

Resolution Status:

Task Manager: T. Mitts

TAC Number: K81095

Identification: 05/2005

Prioritization/Screen: 12/2006

Technical Assessment:

Identification Status: Complete

Priority/Screen Status: Incomplete

Technical Assessment Status: TBD

Regulation and Guidance Development:

Regulation and Guidance Issuance Status:

Regulation and Guidance Development Status: TBD

Regulation and Guidance Development Status: TBD

Implementation:

Verification:

Closure:

Implementation Status: TBD

Verification Status: TBD

Closure Status:

Work Authorization:

FIN NUMBER	CONTRACTOR	CONTRACT TITLE	EXPENDED (\$k)
Y6912	ISL	Assistance for Screening Analysis of Generic Issue 199	\$25.00
Total Resources Expended (K):			\$25.00

WORK SCOPE:

Description

Regulatory Guide 1.165, developed in the early 1990s, specifies a reference probability for exceedance of a safe shutdown earthquake (SSE) ground motion, i.e., seismic hazard, at a median annual value of 10E-5. This reference probability value is based on the annual probability of exceeding the SSEs for 29 Central and Eastern United States (CEUS) nuclear power sites and is used to establish the SSEs for future nuclear facilities. Based on preliminary results from work performed by the United States Geological Survey (USGS) in 2004, it appears the reference probability for the 29 CEUS has increased to about 6 to 7 x 10E-5. The increase in the reference probability value is primarily due to recent developments in the modeling of earthquake ground motion in the CEUS. When the staff first identified this issue, no new plants had applied for a Construction Permit or Early Site Permit (ESP) since 10 CFR Part 100 was revised and Regulatory Guide 1.165 was issued in 1997. When the staff began review of the ESP applications, the staff realized the impact of the revised regulation and the regulatory guide as they relate to future plants and operating reactors.

From the staff's review of the ESP applications with support from the 2004 USGS work report, it appeared that the perception of seismic hazard for operating plants in the CEUS region had increased. Based on the evaluations of the Individual Plant Examination of External Events (IPEEE) Program, the staff had determined that seismic designs of operating plants in the CEUS provided an adequate level of protection. However, in light of the preliminary results from the USGS work of 2004, the staff also recognized that the probability of exceeding the SSE at some of the currently operating sites in the CEUS is higher than previously understood. Therefore, the staff initiated this GI to assess the impact of increased estimates of seismic hazards on selected current nuclear

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

power plants in the CEUS region that might be impacted by the updated seismic research, information, and models.

Work Scope

This GI is in Stage 2 (Screening) of the MD 6.4 process.

STATUS:

In August 2005, the NRC Office of Research (RES) issued a task order for a contractor to develop a probabilistic screening analysis for the increased probabilities of exceedance of the safe-shutdown earthquake ground motion on current nuclear power plants in the CEUS. The contractor was to use information provided by the NRC to perform this task in accordance with guidelines of Section 3.3 and Appendix B.3.2 of NUREG-1489, "A Review of NRC Staff Uses of Probabilistic Risk Assessment." The information to be provided by the NRC included Electric Power Research Institute (EPRI) Report NP-6395-D, "Probabilistic Seismic Hazard Evaluations at Nuclear Power Plant Sites in the Central and Eastern United States: Resolution of the Charleston Earthquake Issue," April 1989. In May 2007, the NRC and the contractor agreed to stop work on this task order because the NRC and EPRI had not resolved issues with releasing the copyrighted EPRI Report NP-6395-D to the NRC contractor for performing this task.

The NRC Office of RES had decided, in April 2007, to complete the USGS update of seismic hazard assessment of CEUS plants and then use this information to perform the screening analysis for this GI. In May 2007, the staff developed a plan to complete the screening analysis for GI-199 by February 2008, and began work on initial tasks described in this plan. In June 2007, the staff decided to focus the screening analysis efforts on using existing USGS seismic hazard information to address the seven criteria for screening GIs described in SECY-07-0022, "Status Report on Proposed Improvements to the Generic Issues Program," dated January 30, 2007 (ML063460239). In July 2007, the staff completed their preliminary screening analysis and in August 2007, provided it to the screening analysis review panel.

In October 2007, the staff determined that the screening analysis should consider seismic hazard data and models besides those available from the USGS. This determination is based on the staff's ongoing interactions with stakeholders to develop a new performance-based approach for assessing seismic hazards for new reactors as described in a memorandum to the Commission, "A Performance-Based Approach to Define the Safe Shutdown Earthquake Ground Motion," dated July 26, 2006 (ADAMS Accession No. ML052360044). The staff's ongoing work on this performance-based approach resulted in issuance of NRC Regulatory Guide 1.208, "A Performance-Based Approach to Define the Site-Specific Earthquake Ground Motion," dated March 2007 that endorses the performance-based approach.

On November 15, 2007, staff from RES involved in the GI-199 initial screening analysis met with seismic hazard and analysis expert representatives from NRR and the Office of New Reactors (NRO) to discuss actions and identify an approach that is suitable for completing the screening analysis and that also supports the more detailed safety/risk analysis, should GI-199 proceed to that GIP stage. The staff is using guidance contained in MD 6.4 and SECY-07-0022 that provides a qualitative justification for screening the issue. The staff has completed the screening analysis in December 2007, and plans to reconvene the screening panel, submit the screening panel recommendation, and obtain RES Office Director approval by February 2008.

AFFECTED DOCUMENTS:

None Identified.

PROBLEM/RESOLUTION:

Progress on performing the screening analysis was delayed due to issues with releasing the copyrighted EPRI Report NP-6395-D to the NRC contractor. To overcome this issue, RES re-assessed alternatives for proceeding with the screening assessment of GI-199 in accordance with MD 6.4 and SECY-07-0022. From April 2007 through September 2007, staff performed the initial screening analysis of GI-199 using currently available seismic hazard information from the USGS. Then, in October 2007, the staff determined that the screening analysis should consider seismic hazard data and models besides those available

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

from the USGS. On November 15, 2007, staff from RES involved in the GI-199 initial screening analysis met with seismic hazard and analysis expert representatives from NRR and the Office of New Reactors (NRO) to discuss actions and identify an approach that is suitable for completing the screening analysis and that also supports the more detailed safety/risk analysis, should GI-199 proceed to that GIP stage. The staff is using guidance contained in MD 6.4 and SECY-07-0022 that provides a qualitative justification for screening the issue. The staff has completed the screening analysis in December 2007, and plans to reconvene the screening panel, submit the screening panel recommendation, and obtain RES Office Director approval by February 2008.

REASONS FOR SCHEDULE CHANGES:

Schedule delays involving the initial screening analysis were caused by not identifying an amenable solution for EPRI release of NP-6395-D to the NRC contractor for performing the screening analysis task. Based on discussions with the USGS, the staff determined the time frame for obtaining current seismic hazard update information for CEUS plant sites would be mid-2008 as opposed to October 2007. Accordingly, the staff changed the date for the milestone: "Receive Seismic Hazard Update Results for Selected CEUS Plants from USGS," from 10/30/2007 to 6/30/2008. In support of completing the screening analysis, consistent with timeliness targets described in SECY-07-0022, the staff decided to base the screening analysis on currently available seismic hazard information from the USGS. Following this approach, the staff completed the milestone: "Generate Screening Analysis," on July 27, 2007, and then completed the milestone: "Screening Panel Meeting," on September 12, 2007.

Then in October 2007, the staff determined that the screening analysis should consider seismic hazard data and models besides those available from the USGS. This determination is based on the staff's ongoing interactions with stakeholders to develop a new performance-based approach for assessing seismic hazards for new reactors as described in a memorandum to the Commission, "A Performance-Based Approach to Define the Safe Shutdown Earthquake Ground Motion," dated July 26, 2006 (ADAMS Accession No. ML052360044). The staff's ongoing work on this performance-based approach resulted in issuance of NRC Regulatory Guide 1.208, "A Performance-Based Approach to Define the Site-Specific Earthquake Ground Motion," dated March 2007 that endorses the performance-based approach. On November 15, 2007, staff from RES involved in the GI-199 initial screening analysis met with seismic hazard and analysis expert representatives from NRR and the Office of New Reactors (NRO) to discuss actions and identify an approach that is suitable for completing the screening analysis and that also supports the more detailed safety/risk analysis, should GI-199 proceed to that GIP stage. The staff is using guidance contained in MD 6.4 and SECY-07-0022 that provides a qualitative justification for screening the issue. The staff plans to complete the screening analysis in December 2007, reconvene the screening panel and submit the screening panel recommendation for approval in January 2008, and obtain RES Office Director approval by February 2008.

Milestone	Original Date	Current Date	Actual Date
Issue RFP to ISL for Technical Assistance	07/07/2005		07/07/2005
Receive Proposal from ISL	08/11/2005		08/11/2005
Generate Screening Analysis	10/31/2006	12/31/2007	07/27/2007
Screening Panel Meeting	11/30/2006	01/31/2008	09/12/2007
Prepare Screening Analysis Applying Criteria from MD 6.4 and SECY-07-0022	12/15/2007	12/31/2007	
Reconvene Screening Panel	12/15/2007	01/13/2008	
Provide Screening Panel Recommendation Memo for RES Director Approval"	01/31/2007	02/15/2008	
Issue RES Director Approved Screening Analysis and Panel Recommendation	12/31/2006	02/28/2008	

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

All Action Levels: Selected Issue(s)

Milestone	Original Date	Current Date	Actual Date
Receive Seismic Hazard Update Results for Selected CEUS Plants From USGS	10/30/2007	06/30/2008	
Close Generic Issue 199	06/30/2009		