AmerGen Energy Company, LLC
Three Mile Island Unit 1
Route 441 South, P.O. Box 480
Middletown, PA 17057

Telephone: 717-948-8000

AmerGen SM
An Exelon Company

April 30, 2007
5928-07-20096

U. S. Nuclear Regulatory Commis
sion
Washington, DC 20555

Attn: Document Control Desk

SUBJECT: THREE MILE ISLAND NUCLEAR STATION UNIT 1 AND UNIT 2
OPERATING LICENSE NO. DPR-50 AND POSSESSION ONLY LICENSE NO. DPR 73
DOCKET NOS. 50-289 AND 50-320
COMBINED 2006 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

The 2006 Annual Radioactive Effluent Release Report required by TMI-1 Technical Specification 6.9.4.1, TMI-2 Technical Specifications 6.8.1.2, and 6.12, and the Off-Site Dose Calculation Manual Part 4, Section 2.1 is enclosed. Also, an errata report for the 2005 Annual Radioactive Effluent Release Report is submitted in Attachment 11.

Attachment 1 contains a summary of the quantities of radioactive liquid and gaseous effluents released from the site as outlined in Reg. Guide 1.21, Rev. 1, with data summarized on a quarterly basis following the format of Appendix B thereof.

Attachment 2 contains information for each type of solid waste shipped offsite during the report period including the container volume, total curie quantity (specified as determined by measurement or estimate), principal radionuclides (specified as determined by measurement or estimate), type of waste, type of shipment and solidification agent(s).

Attachment 3 includes a summary of unplanned releases from the site to unrestricted areas of radioactive materials in gaseous and liquid effluents made during the reporting period.

Attachment 4 describes any changes made during 2006 to the Process Control Program (PCP) documents or to the Offsite Dose Calculation Manual (ODCM) and a listing of new locations for dose calculations and/or environmental monitoring identified by the land use census pursuant to Part 3, Section 8.2 of the ODCM.

Attachment 5 reports all instrumentation not returned to operable status within 30 days per the TMI ODCM Part 1, Sections 2.1.1.b and 2.1.2.b and Part 2, Section 2.1.2.b.

Attachment 6 is quarterly summaries of hourly meteorological data collected for 2006 in the form of joint frequency distribution of wind speed, wind direction and atmospheric stability.

IE48
A009

Attachment 7 is an assessment of the radiation doses due to the radioactive liquid and gaseous effluents released from the respective unit during 2006.

Attachment 8 is an assessment of the radiation doses from the radioactive liquid and gaseous effluents to members of the public due to their activities inside the site boundary during 2006.

Attachment 9 is an assessment of the radiation doses to the most likely exposed real individual from reactor releases and other nearby uranium fuel cycle sources including doses from primary effluent pathways and direct radiation for 2006.

Attachment 10 is a summation of deviations from the sampling and analysis regime specified in the ODCM for TMI-1 and TMI-2.

Attachment 11 is an errata report for the 2005 Annual Radioactive Effluent Release Report. Only the affected pages are attached. Changes are annotated in bold font.

Enclosure 1 is a copy of the ODCM change for TMI's Offsite Dose Calculation Manual (ODCM), revision 25, which was issued on August 9, 2006 and current as of December 31, 2006. There was one revision made to the ODCM during 2006.

Please contact Laura Weber at extension x8947 if you have any questions concerning this report.

Sincerely,

Thomas J. Dougherty
Plant Manager

TJD/lkw

Attachments/Enclosures

cc: Region 1 Administrator
TMI Senior Resident Inspector
TMI-1 Senior Project Manager
TMI-2 Project Manager
GPU Nuclear Cognizant Officer
File 07028

Attachment 1
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Summary of Radioactive Liquid and Gaseous Effluents Released from TMI during 2006

TABLE 1A
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
GASEOUS EFFLUENTS-SUMMATION OF ALL RELEASES
TMI-1

	UNITS	2006 1ST QUARTER	2006 2ND QUARTER	2006 3RD QUARTER	2006 4TH QUARTER	EST. TOTAL ERROR %
A. FISSION AND ACTIVATION GASES						
1. TOTAL RELEASE	Ci	3.30E-01	2.37E-02	2.85E-02	1.12E+00	25%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	4.24E-02	3.01E-03	3.58E-03	1.40E-01	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	
B. IODINES						
1. TOTAL IODINE I-131	Ci	1.71E-07	1.88E-07	5.83E-07	1.30E-06	25%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	2.19E-08	2.38E-08	7.34E-08	1.63E-07	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	
C. PARTICULATES						
1. PARTICULATES WITH HALF-LIVES > 8 DAYS	Ci	<1.00E-11	<1.00E-11	<1.00E-11	2.10E-06	25%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	NA	NA	NA	2.65E-07	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	
4. GROSS ALPHA RADIOACTIVITY	Ci	<1.00E-11	<1.00E-11	<1.00E-11	<1.00E-11	
D. TRITIUM						
1. TOTAL RELEASE	Ci	2.81E+01	2.59E+01	2.37E+01	1.17E+01	15%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	3.61E+00	3.30E+00	2.98E+00	1.47E+00	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	

* % ODCM LIMITS: LISTED ON DOSE SUMMARY TABLE

TABLE 1C
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT (2006)
GASEOUS EFFLUENTS - GROUND LEVEL RELEASES
TMI-1

		CONTINUOUS		BATCH		CONTINUOUS		BATCH	
NUCLIDES RELEASED	UNIT	QUARTER 1	QUARTER 2	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4	QUARTER 3	QUARTER 4
1. FISSION GASES									
AR 41	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
KR 85M	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
KR 85	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	9.66E-01
KR 87	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
KR 88	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
XE131M	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	2.41E-04	4.59E-03
XE 133	Ci	2.90E-01	<1.00E-04	3.92E-02	2.36E-02	<1.00E-04	<1.00E-04	2.81E-02	1.46E-01
XE133M	Ci	<1.00E-04	<1.00E-04	<1.00E-04	2.70E-05	<1.00E-04	<1.00E-04	<1.00E-04	1.08E-04
XE 135M	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
XE 135	Ci	1.32E-04	<1.00E-04	9.85E-06	<1.00E-04	1.20E-04	1.63E-04	<1.00E-04	<1.00E-04
XE 138	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
TOTAL FOR PERIOD	Ci	2.90E-01	NA	3.92E-02	2.36E-02	1.20E-04	1.63E-04	2.83E-02	1.12E+00
2. IODINES									
I 131	Ci	1.71E-07	1.88E-07	<1.00E-12	<1.00E-12	5.83E-07	1.30E-06	<1.00E-12	<1.00E-12
I 133	Ci	1.34E-06	1.75E-06	<1.00E-12	<1.00E-12	7.63E-06	6.53E-06	<1.00E-12	<1.00E-12
TOTAL FOR PERIOD	Ci	1.51E-06	1.94E-06	NA	NA	8.21E-06	7.83E-06	NA	NA
3. PARTICULATES									
CS 137	Ci	<1.00E-11	<1.00E-11	<1.00E-11	<1.00E-11	<1.00E-11	2.10E-06	<1.00E-11	<1.00E-11
TOTAL FOR PERIOD	Ci	NA	NA	NA	NA	NA	2.10E-06	NA	NA

TABLE 2A
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
LIQUID EFFLUENTS-SUMMATION OF ALL RELEASES
TMI-1

	UNITS	2006 1ST QUARTER	2006 2ND QUARTER	2006 3RD QUARTER	2006 4TH QUARTER	EST. TOTAL ERROR %
A. FISSION AND ACTIVATION PRODUCTS						
1. TOTAL RELEASES (NOT INCLUDING TRITIUM, GASES, ALPHA)	Ci	2.71E-05	3.25E-06	8.63E-05	1.79E-05	25%
2. AVERAGE DILUTED CONCENTRATION DURING PERIOD	uCi/ml	4.02E-12	5.19E-13	1.53E-11	3.14E-12	
3. PERCENT OF APPLICABLE LIMIT	%	*	*	*	*	
B. TRITIUM						
1. TOTAL RELEASE	Ci	3.24E+01	2.01E+01	8.97E+01	4.10E+02	25%
2. AVERAGE DILUTED CONCENTRATION DURING PERIOD	uCi/ml	4.81E-06	3.22E-06	1.59E-05	7.17E-05	
3. PERCENT OF APPLICABLE LIMIT	%	*	*	*	*	
C. DISSOLVED AND ENTRAINED GASES						
1. TOTAL RELEASE	Ci	<1.00E-5	<1.00E-5	1.49E-05	1.28E-04	25%
2. AVERAGE DILUTED CONCENTRATION DURING PERIOD	uCi/ml	NA	NA	2.65E-12	2.25E-11	
3. PERCENT OF APPLICABLE LIMIT	%	*	*	*	*	
D. GROSS ALPHA ACTIVITY						
1. TOTAL RELEASE	Ci	<1.00E-7	<1.00E-7	<1.00E-7	<1.00E-7	25%
E. VOLUME OF WASTE RELEASED (PRIOR TO DILUTION)	liters	9.75E+06	9.59E+06	1.07E+07	4.31E+08	10%
F. VOLUME OF DILUTION WATER USED	liters	6.73E+09	6.25E+09	5.63E+09	5.71E+09	10%

* % ODCM LIMITS: LISTED ON DOSE SUMMARY TABLE

TABLE 2B
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT (2006)
LIQUID EFFLUENTS
TMI-1

		CONTINUOUS		BATCH		CONTINUOUS		BATCH	
NUCLIDES RELEASED	UNIT	QUARTER 1	QUARTER 2	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4	QUARTER 3	QUARTER 4
CR 51	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
MN 54	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
FE 59	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
CO 58	Ci	<5.00E-07	<5.00E-07	1.17E-05	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
CO 60	Ci	<5.00E-07	<5.00E-07	5.29E-06	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	1.65E-06
ZN 65	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
SR 89	Ci	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08
SR 90	Ci	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08	<5.00E-08
ZR 95	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
NB 95	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
MO 99	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
TC 99M	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
I 131	Ci	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06
CS 134	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
CS 137	Ci	<5.00E-07	<5.00E-07	4.00E-05	3.25E-06	7.43E-05	3.74E-07	1.21E-05	1.59E-05
BA 140	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
LA 140	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
CE 141	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07
FE 55	Ci	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06	<1.00E-06
H3	Ci	8.25E-02	1.46E-01	3.23E+01	2.00E+01	1.77E-01	3.09E-01	8.95E+01	4.09E+02
TOTAL FOR PERIOD	Ci	0.00E+00	0.00E+00	5.70E-05	3.25E-06	7.43E-05	3.74E-07	1.21E-05	1.76E-05
XE 133	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	1.49E-05	1.28E-04
XE 135	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04

SUPPLEMENTAL INFORMATION

FACILITY: TMI UNIT 1 LICENSE: DPR 50-289

1. REGULATORY LIMITS - - - REFER TO TMI OFFSITE DOSE CALCULATION MANUAL

A. FISSION AND ACTIVATION GASES:
B. IODINES:
C. PARTICULATES, HALF-LIVES > 8 DAYS:
D. LIQUID EFFLUENTS:

2. MAXIMUM EFFLUENT CONCENTRATIONS - - - TEN TIMES 10 CFR 20, APPENDIX B TABLE 2

PROVIDE THE MAXIMUM EFFLUENT CONCENTRATIONS USED IN DETERMINING ALLOWABLE RELEASE RATES OR CONCENTRATIONS.

A. FISSION AND ACTIVATION GASES:
B. IODINES:
C. PARTICULATES, HALF-LIVES > 8 DAYS:
D. LIQUID EFFLUENTS:

3. AVERAGE ENERGY

PROVIDE THE AVERAGE ENERGY (E-BAR) OF THE RADIONUCLIDE MIXTURE IN RELEASES OF FISSION AND ACTIVATION GASES, IF APPLICABLE

E-BAR BETA = 1.91E-01 MeV
E-BAR GAMMA = 2.14E-02 MeV
E-BAR BETA AND GAMMA = 2.12E-01 MeV

4. MEASUREMENTS AND APPROXIMATIONS OF TOTAL RADIOACTIVITY

PROVIDE THE METHODS USED TO MEASURE OR APPROXIMATE THE TOTAL RADIOACTIVITY IN EFFLUENTS AND THE METHODS USED TO DETERMINE RADIONUCLIDE COMPOSITION:

A. FISSION AND ACTIVATION GASES: HPGE SPECTROMETRY, LIQUID SCINTILLATION
B. IODINES: HPGE SPECTROMETRY
C. PARTICULATES HPGE SPECTROMETRY, GAS FLOW PROPORTIONAL, BETA SPECTROMETRY
D. LIQUID EFFLUENTS: HPGE SPECTROMETRY, LIQUID SCINTILLATION

5. BATCH RELEASES

PROVIDE THE FOLLOWING INFORMATION RELATING TO BATCH RELEASES OF RADIOACTIVITY MATERIALS IN LIQUID AND GASEOUS EFFLUENTS.

A. LIQUID (ALL TIMES IN MINUTES)	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4
1. NUMBER OF BATCH RELEASES:	27	6	11	25
2. TOTAL TIME PERIOD FOR BATCH RELEASES:	6215	1290	2995	10757
3. MAXIMUM TIME PERIOD FOR A BATCH RELEASE:	260	250	385	1300
4. AVERAGE TIME PERIOD FOR BATCH RELEASES:	230	215	272	430
5. MINIMUM TIME PERIOD FOR A BATCH RELEASE:	210	120	235	212
6. AVERAGE STREAM FLOW DURING PERIODS OF RELEASE OF EFFLUENT INTO A FLOWING STREAM: (CFS)	3.31E+06	2.16E+06	1.89E+06	2.67E+06

B. GASEOUS (ALL TIMES IN MINUTES)				
1. NUMBER OF BATCH RELEASES:	8	6	6	9
2. TOTAL TIME PERIOD FOR BATCH RELEASES:	5405	4443	4024	6143
3. MAXIMUM TIME PERIOD FOR A BATCH RELEASE:	830	829	780	855
4. AVERAGE TIME PERIOD FOR BATCH RELEASES:	675	740	670	682
5. MINIMUM TIME PERIOD FOR A BATCH RELEASE:	222	540	225	160

6. ABNORMAL RELEASES

A. LIQUID				
1. NUMBER OF RELEASES:	-0-	-0-	-0-	1
2. TOTAL ACTIVITY RELEASED: (CURIES)	N/A	N/A	N/A	1.18E-01

B. GASEOUS				
1. NUMBER OF RELEASES:	1	1	1	7
2. TOTAL ACTIVITY RELEASED: (CURIES)	2.92E-01	9.72E-03	0.00E+00	2.13E-01

TABLE 1A
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
GASEOUS EFFLUENTS-SUMMATION OF ALL RELEASES
TMI-2

	UNITS	2006 1ST QUARTER	2006 2ND QUARTER	2006 3RD QUARTER	2006 4TH QUARTER	EST. TOTAL ERROR %
A. FISSION AND ACTIVATION GASES						
1. TOTAL RELEASE	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	25%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	N/A	N/A	N/A	N/A	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	
B. IODINES		NOT APPLICABLE FOR TMI-2				
C. PARTICULATES						
1. PARTICULATES WITH HALF-LIVES > 8 DAYS	Ci	<1.00E-10	<1.00E-10	<1.00E-10	<1.00E-10	25%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	N/A	N/A	N/A	N/A	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	
4. GROSS ALPHA RADIOACTIVITY	Ci	<1.00E-11	<1.00E-11	<1.00E-11	<1.00E-11	
D. TRITIUM						
1. TOTAL RELEASE	Ci	2.20E-01	2.16E-01	2.21E-01	2.89E-01	25%
2. AVERAGE RELEASE RATE FOR PERIOD	uCi/sec	2.83E-02	2.74E-02	2.78E-02	3.64E-02	
3. PERCENT OF TECH SPEC LIMIT	%	*	*	*	*	
# BATCH RELEASES		0	0	0	0	

* % ODCM LIMITS: LISTED ON DOSE SUMMARY TABLE

TABLE 1C
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT (2006)
GASEOUS EFFLUENTS - GROUND LEVEL RELEASES
TMI-2

		CONTINUOUS		BATCH		CONTINUOUS		BATCH	
NUCLIDES RELEASED	UNIT	QUARTER 1	QUARTER 2	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4	QUARTER 3	QUARTER 4
1. FISSION GASES									
KR 85M	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
KR 85	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
KR 87	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
KR 88	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
XE 133	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
XE 135M	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
XE 135	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
XE 138	Ci	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04	<1.00E-04
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A
2. IODINES		NOT APPLICABLE TO TMI-2							
3. PARTICULATES									
CS 137	Ci	<1.00E-10	<1.00E-10	N/A	N/A	<1.00E-10	<1.00E-10	N/A	N/A
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A
4. TRITIUM									
TRITIUM	Ci	2.20E-01	2.16E-01	<1.00E-06	<1.00E-06	2.21E-01	2.89E-01	<1.00E-06	<1.00E-06

TABLE 2A
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
LIQUID EFFLUENTS-SUMMATION OF ALL RELEASES
TMI-2

	UNITS	2006 1ST QUARTER	2006 2ND QUARTER	2006 3RD QUARTER	2006 4TH QUARTER	EST. TOTAL ERROR %
A. FISSION AND ACTIVATION PRODUCTS						
1. TOTAL RELEASES (NOT INCLUDING TRITIUM, GASES, ALPHA)	Ci	5.00E-07	6.02E-06	8.71E-06	2.47E-06	25%
2. AVERAGE DILUTED CONCENTRATION DURING PERIOD	uCi/ml	N/A	9.62E-13	1.55E-12	4.33E-13	
3. PERCENT OF APPLICABLE LIMIT	%	*	*	*	*	
B. TRITIUM						
1. TOTAL RELEASE	Ci	1.88E-05	1.32E-04	3.59E-06	<1.00E-05	25%
2. AVERAGE DILUTED CONCENTRATION DURING PERIOD	uCi/ml	2.80E-12	2.10E-11	6.37E-13	N/A	
3. PERCENT OF APPLICABLE LIMIT	%	*	*	*	*	
C. DISSOLVED AND ENTRAINED GASES						
1. TOTAL RELEASE	Ci	<5.00E-07	<5.00E-07	<5.00E-07	<5.00E-07	25%
2. AVERAGE DILUTED CONCENTRATION DURING PERIOD	uCi/ml	NA	NA	NA	NA	
3. PERCENT OF APPLICABLE LIMIT	%	*	*	*	*	
D. GROSS ALPHA ACTIVITY						
1. TOTAL RELEASE	Ci	<1.00E-07	<1.00E-07	<1.00E-07	<1.00E-07	25%
E. VOLUME OF WASTE RELEASED (PRIOR TO DILUTION)	liters	1.53E+03	7.59E+03	1.00E+04	1.91E+03	10%
F. VOLUME OF DILUTION WATER USED	liters	6.73E+09	6.25E+09	5.63E+09	5.71E+09	10%
# BATCH RELEASES		1	4	6	2	

* % ODCM LIMITS: LISTED ON DOSE SUMMARY TABLE

TABLE 2B
EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT (2006)
LIQUID EFFLUENTS
TMI-2

		CONTINUOUS		BATCH		CONTINUOUS		BATCH	
NUCLIDES RELEASED	UNIT	QUARTER 1	QUARTER 2	QUARTER 1	QUARTER 2	QUARTER 3	QUARTER 4	QUARTER 3	QUARTER 4
CO 60	Ci	NA	NA	<5.00E-07	<5.00E-07	NA	NA	<5.00E-07	<5.00E-07
ZN 65	Ci	NA	NA	<5.00E-07	<5.00E-07	NA	NA	<5.00E-07	<5.00E-07
SR 90	Ci	NA	NA	<5.00E-08	5.18E-07	NA	NA	<5.00E-08	<5.00E-08
SB 125	Ci	NA	NA	<5.00E-07	<5.00E-07	NA	NA	<5.00E-07	<5.00E-07
NB 95	Ci	NA	NA	<5.00E-07	<5.00E-07	NA	NA	<5.00E-07	<5.00E-07
CS 134	Ci	NA	NA	<5.00E-07	<5.00E-07	NA	NA	<5.00E-07	<5.00E-07
CS 137	Ci	NA	NA	<5.00E-07	5.50E-06	NA	NA	8.71E-06	2.47E-06
H3	Ci	NA	NA	1.88E-05	1.32E-04	NA	NA	3.59E-06	<1.00E-05
TOTAL FOR PERIOD	Ci	NA	NA	1.88E-05	1.38E-04	NA	NA	1.23E-05	2.47E-06

Attachment 2
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Solid Waste Shipped Offsite During 2006

2006 TMI-1 TABLE 3 EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT SOLID WASTE AND IRRADIATED FUEL SHIPMENTS			
A. Solid waste shipped off-site for burial or disposal (not irradiated fuel)			
1. Type of waste	UNIT	12 month period	EST. Total Error %
a. Spent resins, filter sludges, Evaporator bottoms, etc.	m^3 Ci	38.7 m3 30.9 Ci	25%
b. Dry compressible waste, contaminated equipment, etc.	m^3 Ci	313.0 m3 .079 Ci	25%
c. Irradiated components, control rods, etc.	m^3 Ci	N/A	N/A
d. Other (describe) :	m^3 Ci	N/A	N/A
2. Estimate of major nuclide composition (by type of waste)			
a. H3	90.5 %		
Ni63	7.3 %		
Cs137	1.7 %		
Co60	0.3 %		
b. Cs137	45.7 %		
Ni63	23.2 %		
Fe55	14.8 %		
Co58	4.95 %		
c. N/A			
d. N/A			
3. Solid Waste Disposition Number of Shipments	Mode of Transportation		Destination
See attached for this information			
B. Irradiated Fuel Shipments (Disposition)			
	None		
Number of Shipments			
N/A	Mode Transport	Destination	

WASTE SHIPPED AS FOLLOWS

A.1.a

Eight (8) –Poly HICs @ 170.8 ft3 each – Evaporator Bottoms

A.1.b

Seven (7) – Steel Cargo Containers @ 1280 ft3 each- DAW-Metal

One (1)-Intermodal Container @ 1280 ft3 each – Metal

Eight (8) Steel Boxes @ 96 ft3 each –DAW/Metal

One(1)- Steel Box @ 44 ft3 each- Soil

A.3.a

Eight Shipments	Hittman Transport/Cask	Duratek, Oak Ridge TN

A.3.b

Four Shipments	Hittman Transport/Flatbed	Duratek-Oak Ridge , TN.
Two Shipment	Hittman Transport/Flatbed	Duratek-Kingston, TN.

NOTE- All Shipments were TYPE-A LSA-II

There were no changes to the Process Control Program (PCP) for TMI-1 during 2006.

2006 TMI-2 TABLE 3 EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT SOLID WASTE AND IRRADIATED FUEL SHIPMENTS			
A. Solid waste shipped off-site for burial or disposal (not irradiated fuel)			
1. Type of waste	UNIT	12 month period	EST. Total Error %
a. Spent resins, filter sludges, Evaporator bottoms, etc.	m^3 Ci	N/A	N/A
b. Dry compressible waste, contaminated equipment, etc.	m^3 Ci	N/A	N/A
c. Irradiated components, control rods, etc.	m^3 Ci	N/A	N/A
d. d. Other (describe): Mixed Waste	m^3 Ci	N/A	N/A
2. Estimate of major nuclide composition (by type of waste)			
a. Cs137	N/A		
Cs134			
Ni63			
Fe55			
b. Co58	N/A		
Cs137			
Ni63			
Sr90			
c. Ni63	N/A		
Co58			
Fe55			
Co60			
d. N/A	N/A		
3. Solid Waste Disposition Number of Shipments	Mode of Transportation		Destination
No Shipment during this period			
B. Irradiated Fuel Shipments (Disposition)			
	None		
Number of Shipments			
N/A	Mode Transport	Destination	

Attachment 3
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Summary of Unplanned Releases from the TMI Site During 2006

There were no unplanned releases from TMI-2 in 2006. The unplanned releases for TMI-1 are summarized in the supplemental information in Attachment 1. The information is reported separately for liquid and gaseous releases, and the number of releases is reported for each quarter with a total curies released. The activity for these releases is also included in Tables 1A, 1C, 2A and 2B.

Attachment 4
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Changes to the Process Control Program and the Offsite Dose Calculation Manual during 2006
And a listing of new locations for dose calculations and/or environmental monitoring Identified by the Land Use Census

1. **Changes to the Process Control Program**

   There were no changes to the Process Control Program.

2. **Changes to the Offsite Dose Calculation Manual**

   There was one change to the Offsite Dose Calculation Manual. The procedure change is attached as an attachment.

3. **A listing of new locations for dose calculations and/or environmental monitoring identified by the Land Use Census.**

   Based on the results of the 2006 land use census, no changes to the Radiological Environmental Monitoring Program are required. The residential census identified a change in the nearest residence in the north-northwest sector. A summer residence was abandoned on Henry Island. The distance of the nearest receptor went from 1067 meters to 1832 meters.

Attachment 5
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Instrumentation not Returned to Operable Status within 30 Days During 2006

There was no instrumentation not returned to operable status within 30 days per the TMI ODCM Part 1, Sections 2.1.1.b and 2.1.2.b and Part 2, Section 2.1.2.b during 2006.

Attachment 6
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Annual Summary of Hourly Meteorological Data for 2006

The osprey did return and nest on the TMI meteorological tower. However, the station was able to calibrate the sensors and instrumentation before and after the osprey nested. The percent data recovery for meteorological information for 2006 was 98.4 percent. The data are presented by quarter.

Three Mile Island Nuclear Station
Period of Record: January - March 2006
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	>24	Total
N	0	3	1	1	0	0	5
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	1	0	0	0	0	1
S	0	1	1	0	0	0	2
SSW	0	1	8	1	1	0	11
SW	1	1	6	0	0	0	8
WSW	0	5	0	0	0	0	5
W	0	2	3	0	0	0	5
WNW	2	3	6	0	0	0	11
NW	2	5	4	2	2	0	15
NNW	0	10	6	4	4	0	24
Variable	0	0	0	0	0	0	0
Total	5	32	35	8	7	0	87

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 8

Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	1	1	0	0	0	2
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	3	0	0	0	0	3
E	0	0	0	0	0	0	0
ESE	0	2	2	0	0	0	4
SE	0	2	1	0	0	0	3
SSE	0	1	0	0	0	0	1
S	0	1	0	0	0	0	1
SSW	0	1	3	1	0	0	5
SW	0	3	6	0	0	0	9
WSW	1	2	2	0	0	0	5
W	2	1	5	4	5	0	17
WNW	2	0	3	8	9	0	22
NW	0	3	18	3	9	6	66
NNW	1	6	5	7	1	7	27
Variable	0	0	0	0	0	0	0
Total	6	26	46	50	24	13	165

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 3
Hours of missing stability measurements in all stability classes: 8

Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	3	1	0	0	0	4
NNE	0	1	0	0	0	0	1
NE	0	0	0	0	0	0	0
ENE	0	0	0	0	0	0	0
E	0	0	1	0	0	0	1
ESE	0	0	2	0	0	0	2
SE	0	0	0	0	0	0	0
SSE	1	0	0	0	0	0	1
S	1	0	0	0	0	0	1
SSW	0	2	1	1	0	0	4
SW	0	0	3	0	1	0	4
WSW	1	0	1	1	0	0	3
W	0	1	4	0	1	0	6
WNW	0	3	8	2	2	0	15
NW	1	4	13	24	11	1	54
NNW	1	2	10	5	1	3	22
Variable	0	0	0	0	0	0	0
Total	5	16	44	33	16	4	118

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 8

Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	5	12	18	3	0	0	38
NNE	3	19	4	0	0	0	26
NE	3	18	1	0	0	0	22
ENE	2	8	1	0	0	0	11
E	5	13	12	1	0	0	31
ESE	6	2	17	2	0	0	52
SE	2	18	8	0	0	0	28
SSE	6	11	3	0	0	0	20
S	7	23	6	0	0	0	36
SSW	5	19	13	5	0	0	42
SW	7	22	11	3	0	0	43
WSW	4	24	23	8	0	0	59
W	6	38	43	31	4	0	122
WNW	4	30	65	60	18	0	177
NW	5	22	87	108	35	5	262
NNW	3	23	43	18	1	2	90
Variable	0	0	0	0	0	0	0
Total	73	327	355	239	58	7	1059

Hours of calm in this stability class: 6
Hours of missing wind measurements in this stability class: 9
Hours of missing stability measurements in all stability classes: 8

Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	6	15	1	0	0	0	22
NNE	4	7	2	0	0	0	13
NE	2	2	3	0	0	0	7
ENE	1	4	0	0	0	0	5
E	4	5	6	0	0	0	15
ESE	6	6	16	5	0	0	33
SE	3	10	9	1	0	0	23
SSE	10	6	3	2	0	0	21
S	6	7	3	4	0	0	20
SSW	5	10	13	10	0	0	38
SW	9	12	6	0	0	0	27
WSW	9	9	4	2	0	0	24
W	10	21	4	7	0	1	43
WNW	10	9	14	2	0	0	35
NW	7	13	5	8	5	1	39
NNW	10	12	8	6	0	0	36
Variable	0	0	0	0	0	0	0
Total	102	148	97	47	5	2	401

Hours of calm in this stability class: 23
Hours of missing wind measurements in this stability class: 11
Hours of missing stability measurements in all stability classes: 8

Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	5	4	0	0	0	0	9
NNE	1	0	0	0	0	0	1
NE	0	0	0	0	0	0	0
ENE	0	1	1	0	0	0	2
E	1	1	0	0	0	0	2
ESE	1	1	0	0	0	0	2
SE	4	2	0	0	0	0	6
SSE	1	2	0	0	0	0	3
S	2	5	0	0	0	0	7
SSW	12	6	1	0	0	0	19
SW	8	7	1	0	0	0	16
WSW	4	3	0	0	0	0	7
W	9	7	0	0	0	0	16
WNW	6	3	0	0	0	0	9
NW	12	4	0	0	0	0	16
NNW	9	4	2	0	0	0	15
Variable	0	0	0	0	0	0	0
Total	75	50	5	0	0	0	130

Hours of calm in this stability class: 7
Hours of missing wind measurements in this stability class: 6
Hours of missing stability measurements in all stability classes: 8

Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	3	1	0	0	0	0	4
NNE	3	0	0	0	0	0	3
NE	0	0	0	0	0	0	0
ENE	5	2	0	0	0	0	7
E	2	0	0	0	0	0	2
ESE	5	0	0	0	0	0	5
SE	2	1	0	0	0	0	3
SSE	7	3	0	0	0	0	10
S	8	4	0	0	0	0	12
SSW	15	8	0	0	0	0	23
SW	14	8	0	0	0	0	22
WSW	6	6	0	0	0	0	12
W	1	2	1	0	0	0	4
WNW	4	1	0	0	0	0	5
NW	2	0	0	0	0	0	2
NNW	0	1	0	0	0	0	1
Variable	0	0	0	0	0	0	0
Total	77	37	1	0	0	0	115

Hours of calm in this stability class: 9
Hours of missing wind measurements in this stability class: 3
Hours of missing stability measurements in all stability classes: 8

Period of Record: April - June 2006
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	1	14	0	1	0	0	16
NNE	0	3	0	0	0	0	3
NE	1	3	0	0	0	0	4
ENE	0	2	3	0	0	0	5
E	0	2	3	3	0	0	8
ESE	0	0	1	4	0	0	5
SE	0	1	1	0	0	0	2
SSE	0	2	0	0	0	0	2
S	0	0	1	0	0	0	1
SSW	0	6	6	0	0	0	12
SW	2	12	1	0	0	0	15
WSW	2	4	2	0	0	0	8
W	9	8	3	0	0	0	20
WNW	5	13	9	1	0	0	28
NW	18	35	21	12	2	0	88
NNW	11	37	10	2	0	0	60
Variable	0	0	0	0	0	0	0
Total	49	142	61	23	2	0	277

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 1

Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	4	4	5	0	0	0	13
NNE	0	1	1	0	0	0	2
NE	0	3	0	0	0	0	3
ENE	0	4	0	0	0	0	4
E	0	1	1	0	0	0	2
ESE	0	0	1	1	0	0	2
SE	0	0	2	2	0	0	4
SSE	0	2	1	0	0	0	3
S	0	2	0	0	0	0	2
SSW	2	3	3	0	0	0	8
SW	1	9	3	0	0	0	13
WSW	0	6	1	0	0	0	7
W	3	3	5	2	0	0	13
WNW	3	6	5	5	0	0	19
NW	5	16	20	11	1	0	53
NNW	2	8	10	4	0	0	24
Variable	0	0	0	0	0	0	0
Total	20	68	58	25	1	0	172

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 1

Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	1	5	3	0	0	0	9
NNE	0	1	0	0	0	0	1
NE	0	1	0	0	0	0	1
ENE	0	0	0	0	0	0	0
E	0	3	2	0	0	0	5
ESE	0	0	3	1	1	0	5
SE	1	1	2	2	0	0	6
SSE	0	2	0	0	0	0	2
S	1	2	1	0	0	0	4
SSW	1	2	0	0	0	0	3
SW	1	0	3	0	0	0	4
WSW	2	0	0	0	0	0	2
W	1	1	2	2	0	0	6
WNW	2	5	6	5	0	0	18
NW	2	5	6	11	1	0	25
NNW	2	7	7	4	0	0	20
Variable	0	0	0	0	0	0	0
Total	14	35	35	25	2	0	111

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 1

Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	4	20	20	3	0	0	47
NNE	4	7	1	0	0	0	12
NE	3	6	1	0	0	0	10
ENE	1	6	2	0	0	0	9
E	2	12	29	1	0	0	44
ESE	2	13	21	4	0	0	40
SE	1	9	6	5	0	0	21
SSE	1	9	5	0	0	0	15
S	5	14	9	1	0	0	29
SSW	6	14	6	0	0	0	26
SW	6	2	5	1	0	0	32
WSW	6	30	7	0	0	0	43
W	13	17	18	4	1	0	53
WNW	6	30	42	23	1	0	102
NW	15	27	63	28	3	1	137
NNW	9	31	30	12	0	1	83
Variable	0	0	0	0	0	0	0
Total	84	265	265	82	5	2	703

Hours of calm in this stability class: 6
Hours of missing wind measurements in this stability class: 5
Hours of missing stability measurements in all stability classes: 1

Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	8	54	15	1	0	0	78
NNE	5	12	3	0	0	0	20
NE	8	9	0	0	0	0	17
ENE	2	10	2	0	0	0	14
E	3	20	11	0	0	0	34
ESE	4	13	6	1	0	0	24
SE	5	12	3	2	1	0	23
SSE	7	10	1	0	0	0	18
S	4	27	5	0	0	0	36
SSW	8	17	9	0	0	0	34
SW	7	14	2	0	0	0	23
WSW	15	15	0	1	0	0	31
W	16	22	2	0	0	0	40
WNW	23	20	10	5	0	0	58
NW	21	25	7	2	1	0	56
NNW	20	25	8	3	1	0	57
Variable	0	0	0	0	0	0	0
Total	156	305	84	15	3	0	563

Hours of calm in this stability class: 17
Hours of missing wind measurements in this stability class: 9
Hours of missing stability measurements in all stability classes: 1

Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	4	8	1	0	0	0	13
NNE	4	0	0	0	0	0	4
NE	6	0	0	0	0	0	6
ENE	0	3	0	0	0	0	3
E	4	2	0	0	0	0	6
ESE	5	1	0	0	0	0	6
SE	3	3	0	0	0	0	6
SSE	7	2	0	0	0	0	9
S	10	4	0	0	0	0	14
SSW	10	8	0	0	0	0	18
SW	11	9	1	0	0	0	21
WSW	21	5	0	0	0	0	26
W	16	8	0	0	0	0	24
WNW	12	1	0	0	0	0	13
NW	13	7	1	0	0	0	21
NNW	7	18	1	0	0	0	26
Variable	0	0	0	0	0	0	0
Total	133	79	4	0	0	0	216

Hours of calm in this stability class: 16
Hours of missing wind measurements in this stability class: 3
Hours of missing stability measurements in all stability classes: 1

Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	1	1	0	0	0	0	2
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	1	0	0	0	0	1
E	2	0	0	0	0	0	2
ESE	4	1	0	0	0	0	5
SE	3	1	0	0	0	0	4
SSE	3	1	0	0	0	0	4
S	3	1	0	0	0	0	4
SSW	5	7	0	0	0	0	12
SW	9	3	0	0	0	0	12
WSW	3	1	0	0	0	0	4
W	11	1	0	0	0	0	12
WNW	2	0	0	0	0	0	2
NW	3	1	0	0	0	0	4
NNW	6	3	0	0	0	0	9
Variable	0	0	0	0	0	0	0
Total	55	22	0	0	0	0	77

Hours of calm in this stability class: 8
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 1

Period of Record: July - September 2006

Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	3	19	6	1	0	0	29
NNE	1	2	1	0	0	0	4
NE	0	4	0	0	0	0	4
ENE	0	1	0	0	0	0	1
E	0	4	1	0	0	0	5
ESE	0	3	4	0	0	0	7
SE	0	3	2	0	0	0	5
SSE	0	4	0	0	0	0	4
S	0	3	3	0	0	0	6
SSW	5	15	15	0	0	0	35
SW	2	29	10	0	0	0	41
WSW	10	11	2	0	0	0	23
W	9	14	3	0	0	0	26
WNW	5	21	6	0	0	0	32
NW	17	49	13	5	0	0	84
NNW	9	41	17	0	0	0	67
Variable	0	0	0	0	0	0	0
Total	61	223	83	6	0	0	373

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 3
Hours of missing stability measurements in all stability classes: 30

Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	3	1	0	0	0	4
NNE	0	3	0	0	0	0	3
NE	0	0	0	0	0	0	0
ENE	0	4	2	0	0	0	6
E	1	5	1	0	0	0	7
ESE	0	0	3	0	0	0	3
SE	0	3	0	0	0	0	3
SSE	0	0	1	0	0	0	1
S	1	1	0	0	0	0	2
SSW	1	5	3	0	0	0	9
SW	2	8	1	0	0	0	11
WSW	4	6	2	0	0	0	12
W	4	2	4	0	0	0	10
WNW	3	4	4	1	0	0	12
NW	8	6	9	1	0	0	24
NNW	3	11	7	0	0	0	21
Variable	0	0	0	0	0	0	0
Total	27	61	38	2	0	0	128

Hours of calm in this stability class: 2
Hours of missing wind measurements in this stability class: 1
Hours of missing stability measurements in all stability classes: 30

Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	0	1	0	0	0	1
NNE	0	0	0	0	0	0	0
NE	0	0	0	0	0	0	0
ENE	0	0	1	0	0	0	1
E	0	4	2	0	0	0	6
ESE	0	1	3	0	0	0	4
SE	0	0	1	0	0	0	1
SSE	0	2	1	0	0	0	3
S	1	0	1	0	0	0	2
SSW	0	1	1	0	0	0	2
SW	0	6	0	0	0	0	6
WSW	3	5	0	0	0	0	8
W	4	1	1	0	0	0	6
WNW	3	4	0	0	0	0	7
NW	2	6	1	0	0	0	9
NNW	0	5	3	0	0	0	8
Variable	0	0	0	0	0	0	0
Total	13	35	16	0	0	0	64

Hours of calm in this stability class: 1
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 30

Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	5	27	0	0	0	0	32
NNE	5	11	0	0	0	0	16
NE	1	17	0	0	0	0	18
ENE	3	20	1	0	0	0	24
E	4	16	13	0	0	0	33
ESE	1	15	10	0	0	0	26
SE	3	8	3	0	0	0	14
SSE	6	19	1	0	0	0	26
S	3	20	8	0	0	0	31
SSW	7	19	10	0	0	0	36
SW	10	24	8	0	0	0	42
WSW	10	21	3	0	0	0	34
W	10	25	6	2	0	0	43
WNW	7	35	14	0	0	0	56
NW	9	30	18	4	0	0	61
NNW	13	18	9	0	0	0	40
Variable	0	0	0	0	0	0	0
Total	97	325	104	6	0	0	532

Hours of calm in this stability class: 9
Hours of missing wind measurements in this stability class: 7
Hours of missing stability measurements in all stability classes: 30

Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	9	34	1	0	0	0	44
NNE	6	33	3	0	0	0	42
NE	4	18	10	0	0	0	32
ENE	6	19	2	0	0	0	27
E	6	26	5	0	0	0	37
ESE	11	15	0	0	0	0	26
SE	13	5	0	0	0	0	18
SSE	8	13	0	0	0	0	21
S	17	19	3	0	0	0	39
SSW	10	34	6	0	0	0	50
SW	24	27	2	0	0	0	53
WSW	22	33	6	0	0	0	61
W	36	43	1	0	0	0	80
WNW	12	30	1	0	0	0	43
NW	16	12	7	0	0	0	35
NNW	21	34	4	0	0	0	59
Variable	0	0	0	0	0	0	0
Total	221	395	51	0	0	0	667

Hours of calm in this stability class: 27
Hours of missing wind measurements in this stability class: 10
Hours of missing stability measurements in all stability classes: 30

Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	8	12	0	0	0	0	20
NNE	1	3	0	0	0	0	4
NE	3	2	0	0	0	0	5
ENE	6	3	1	0	0	0	10
E	10	4	0	0	0	0	14
ESE	11	3	0	0	0	0	14
SE	12	2	0	0	0	0	14
SSE	11	3	0	0	0	0	14
S	7	2	0	0	0	0	9
SSW	8	6	0	0	0	0	14
SW	18	4	0	0	0	0	22
WSW	22	3	0	0	0	0	25
W	31	7	0	0	0	0	38
WNW	19	3	1	0	0	0	23
NW	17	3	0	0	0	0	20
NNW	12	18	0	0	0	0	30
Variable	0	0	0	0	0	0	0
Total	196	78	2	0	0	0	276

Hours of calm in this stability class: 47
Hours of missing wind measurements in this stability class: 3
Hours of missing stability measurements in all stability classes: 30

Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	1	0	0	0	0	0	1
NNE	0	0	0	0	0	0	0
NE	1	0	0	0	0	0	1
ENE	0	1	0	0	0	0	1
E	0	2	0	0	0	0	2
ESE	0	0	0	0	0	0	0
SE	1	0	0	0	0	0	1
SSE	1	0	0	0	0	0	1
S	1	0	0	0	0	0	1
SSW	0	1	0	0	0	0	1
SW	2	0	0	0	0	0	2
WSW	2	0	0	0	0	0	2
W	2	1	0	0	0	0	3
WNW	5	0	0	0	0	0	5
NW	1	1	0	0	0	0	2
NNW	1	1	0	0	0	0	2
Variable	0	0	0	0	0	0	0
Total	18	7	0	0	0	0	25

Hours of calm in this stability class: 3
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 30

Period of Record: October - December2006
Stability Class - Extremely Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	3	2	0	0	0	5
NNE	0	1	0	0	0	0	1
NE	0	2	0	0	0	0	2
ENE	0	0	0	0	0	0	0
E	0	1	0	0	0	0	1
ESE	0	0	0	0	0	0	0
SE	0	0	0	0	0	0	0
SSE	0	0	0	0	0	0	0
S	1	3	0	2	0	0	6
SSW	0	8	8	3	0	0	19
SW	1	6	5	1	0	0	13
WSW	2	0	1	0	0	0	3
W	3	0	2	4	0	0	9
WNW	3	3	11	1	0	0	18
NW	2	5	11	3	0	0	21
NNW	1	6	0	0	0	0	7
Variable	0	0	0	0	0	0	0
Total	13	38	40	14	0	0	105

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 7

Stability Class - Moderately Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	0	0	0	0	0	0
NNE	0	1	0	0	0	0	1
NE	0	0	1	0	0	0	1
ENE	1	0	0	0	0	0	1
E	0	2	0	0	0	0	2
ESE	0	1	2	0	0	0	3
SE	0	0	0	0	0	0	0
SSE	0	0	0	0	0	0	0
S	1	0	0	0	0	0	1
SSW	0	2	0	0	0	0	2
SW	1	3	4	0	0	0	8
WSW	1	1	1	0	0	0	3
W	0	0	1	4	1	0	6
WNW	2	1	6	9	0	0	18
NW	4	3	3	11	2	0	23
NNW	1	1	3	2	0	0	7
Variable	0	0	0	0	0	0	0
Total	11	15	21	26	3	0	76

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 0
Hours of missing stability measurements in all stability classes: 7

Stability Class - Slightly Unstable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	0	0	0	0	0	0	0
NNE	0	1	0	0	0	0	1
NE	0	1	2	0	0	0	3
ENE	0	0	0	0	0	0	0
E	0	0	0	0	0	0	0
ESE	0	1	3	0	0	0	4
SE	0	1	4	0	0	0	5
SSE	0	0	0	0	0	0	0
S	0	0	1	2	0	0	3
SSW	0	3	2	1	0	0	6
SW	2	1	1	0	0	0	4
WSW	1	0	0	0	0	0	1
W	0	0	0	2	1	0	3
WNW	0	1	4	13	1	0	19
NW	0	3	7	2	4	0	16
NNW	0	2	6	1	0	0	9
Variable	0	0	0	0	0	0	0
Total	3	14	30	21	6	0	74

Hours of calm in this stability class: 0
Hours of missing wind measurements in this stability class: 1
Hours of missing stability measurements in all stability classes: 7

Stability Class - Neutral - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	6	19	14	0	0	0	39
NNE	2	11	2	0	0	0	15
NE	8	15	6	0	0	0	29
ENE	2	1	1	0	0	0	4
E	0	14	5	0	0	0	19
ESE	5	7	53	0	0	0	65
SE	2	10	2	1	0	0	15
SSE	3	10	0	1	0	0	14
S	3	10	15	6	3	0	37
SSW	7	13	10	0	0	0	30
SW	8	16	15	1	0	0	40
WSW	3	16	6	9	0	0	34
W	5	27	36	26	4	0	98
WNW	5	25	83	62	2	0	177
NW	11	14	60	36	2	0	123
NNW	6	17	23	12	0	0	58
Variable	0	0	0	0	0	0	0
Total	76	225	331	154	11	0	797

Hours of calm in this stability class: 15
Hours of missing wind measurements in this stability class: 2
Hours of missing stability measurements in all stability classes: 7

Stability Class - Slightly Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	13	12	1	0	0	0	26
NNE	2	16	0	0	0	0	18
NE	2	11	2	0	0	0	15
ENE	3	27	4	0	0	0	34
E	6	18	13	0	0	0	37
ESE	5	7	13	2	0	0	27
SE	2	5	3	3	0	0	13
SSE	9	9	3	1	0	0	22
S	6	16	19	9	1	0	51
SSW	6	17	15	0	0	1	39
SW	18	27	4	0	0	0	49
WSW	16	23	4	0	1	0	44
W	12	28	13	1	0	0	54
WNW	17	18	4	2	2	0	43
NW	19	20	37	15	1	0	92
NNW	19	26	5	3	0	0	53
Variable	0	0	0	0	0	0	0
Total	155	280	140	36	5	1	617

Hours of calm in this stability class: 37
Hours of missing wind measurements in this stability class: 12
Hours of missing stability measurements in all stability classes: 7

Stability Class - Moderately Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	4	5	0	0	0	0	9
NNE	5	3	0	0	0	0	8
NE	3	3	0	0	0	0	6
ENE	3	1	0	0	0	0	4
E	6	5	1	0	0	0	12
ESE	4	3	0	0	0	0	7
SE	10	1	0	0	0	0	11
SSE	7	2	0	0	0	0	9
S	6	3	1	0	0	0	10
SSW	15	7	0	0	0	0	22
SW	25	4	1	0	0	0	30
WSW	17	7	0	0	1	0	25
W	11	9	1	0	1	0	22
WNW	14	7	1	0	0	0	22
NW	12	3	1	0	0	0	16
NNW	11	11	2	0	0	0	24
Variable	0	0	0	0	0	0	0
Total	153	74	8	0	2	0	237

Hours of calm in this stability class: 62
Hours of missing wind measurements in this stability class: 8
Hours of missing stability measurements in all stability classes: 7

Stability Class - Extremely Stable - 150Ft-33Ft Delta-T (F)
Winds Measured at 100 Feet
Wind Speed (in mph)

Wind Direction	1-3	4-7	8-12	13-18	19-24	> 24	Total
N	2	0	0	0	0	0	2
NNE	4	0	0	0	0	0	4
NE	0	0	0	0	0	0	0
ENE	2	0	0	0	0	0	2
E	0	1	0	0	0	0	1
ESE	8	1	0	0	0	0	9
SE	1	1	0	0	0	0	2
SSE	3	1	0	0	0	0	4
S	7	5	0	0	0	0	12
SSW	12	8	1	0	0	0	21
SW	10	1	0	0	0	0	11
WSW	9	11	0	0	0	0	20
W	13	6	0	0	0	0	19
WNW	4	2	0	0	0	0	6
NW	4	4	0	0	0	0	8
NNW	5	0	0	0	0	0	5
Variable	0	0	0	0	0	0	0
Total	84	41	1	0	0	0	126

Hours of calm in this stability class: 30
Hours of missing wind measurements in this stability class: 2
Hours of missing stability measurements in all stability classes: 7

Attachment 7
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Assessment of Radiation Doses Due to Radioactive Liquid and Gaseous Effluents Released from TMI during 2006

TMI-1

The attached table presents the maximum hypothetical doses to an individual and the general population resulting from 2006 TMI-1 releases of gaseous and liquid effluents. Provided below is a brief explanation of the table.

A. Liquid (Individual)

Calculations were performed on the four age groups and seven organs recommended in Regulatory Guide 1.109. The pathways considered for TMI-1 were the consumption of drinking water and fish and standing on the shoreline influenced by TMI-1 effluents. The latter two pathways are considered to be the primary recreational activities associated with the Susquehanna River in the vicinity of TMI. The "critical receptor" or Receptor 1 was that individual who 1) consumed Susquehanna River water from the nearest downstream drinking water supplier (Wrightsville Water Supply), 2) consumed fish residing in the vicinity of the TMI-1 liquid discharge outfall and 3) occupied an area of shoreline influenced by the TMI-1 liquid discharge.

For 2006 the calculated maximum whole body (or total body) dose from TMI-1 liquid effluents was 1.35E-2 mrem to an adult (line 1). The maximum organ dose was 1.51E-2 mrem to the liver of an adult (line 2).

B. Gaseous (Individual)

There were six major pathways considered in the dose calculations for TMI-1 gaseous effluents. These were: (1) plume exposure (2) inhalation, consumption of; (3) cow milk, (4) vegetables and fruits, (5) meat, and (6) standing on contaminated ground. Real-time meteorology was used in all dose calculations for gaseous effluents.

Lines 3 and 4 present the maximum plume exposure at or beyond the site boundary. The notation of "air dose" is interpreted to mean that these doses are not to an individual, but is considered to be the maximum doses that

would have occurred at or beyond the site boundary. The table presents the distance in meters to the location in the affected sector (compass point) where the theoretical maximum plume exposures occurred. The calculated maximum plume exposures were 2.07E-5 mrad and 3.16E-4 mrad for gamma and beta, respectively.

The maximum organ dose due to the release of iodines, particulates and tritium from TMI-1 in 2006 was 1.07E-2 mrem to the thyroid of a child residing 2000 meters from the site in the SE sector (line 5). This dose again reflects the maximum exposed organ for the appropriate age group.

For 2006, TMI-1 liquid and gaseous effluents resulted in maximum hypothetical doses that were a small fraction of the quarterly and yearly ODCM dose limits.

TMI-1
SUMMARY OF MAXIMUM INDIVIDUAL DOSES FOR TMI-1 FROM
January 1, 2006 through December 31, 2006

Effluent	Applicable Organ	Estimated Dose (mrem)	Age Group	Location Dist (m)	Location Dir (to)	% of ODCM Dose Limit		ODCM Dose Limit (mrem)	
						Quarter	Annual	Quarter	Annual
(1) Liquid	Total Body	1.35E-2	Adult	Receptor 1		9.00E-1	4.50E-1	1.5	3
(2) Liquid	Liver	1.51E-2	Adult	Receptor 1		3.02E-1	1.51E-1	5	10
(3) Noble Gas	Air Dose (gamma-mrad)	2.07E-5	---	1000	E	4,14E-4	2.07E-4	5	10
(4) Noble Gas	Air Dose (beta-mrad)	3.16E-4	---	3000	SE	3.16E-3	1.58E-3	10	20
(5) Iodine, Tritium & Particulates	Thyroid	1.07E-2	Child	2000	SE	1.43E-1	7.13E-2	7.5	15

The attached table presents the maximum hypothetical doses to an individual and the general population resulting from 2006 TMI-2 releases of gaseous and liquid effluents. Provided below is a brief explanation of the table.

A. Liquid (Individual)

Calculations were performed on the four age groups and seven organs recommended in Regulatory Guide 1.109. The pathways considered for TMI-2 were the consumption of drinking water and fish and standing on the shoreline influenced by TMI-2 effluents. The latter two pathways are considered to be the primary recreational activities associated with the Susquehanna River in the vicinity of TMI. The "critical receptor" or Receptor 1 was that individual who 1) consumed Susquehanna River water from the nearest downstream drinking water supplier (Wrightsville Water Supply), 2) consumed fish residing in the vicinity of the TMI-2 liquid discharge outfall and 3) occupied an area of shoreline influenced by the TMI-2 liquid discharge.

For 2006 the calculated maximum whole body (or total body) dose from TMI-2 liquid effluents was 4.39E-4 mrem to an adult (line 1). The maximum organ dose was 6.90E-4 mrem to the liver of a teen (line 2).

B. Gaseous (Individual)

There were six major pathways considered in the dose calculations for TMI-2 gaseous effluents. These were: (1) plume exposure (2) inhalation, consumption of; (3) cow milk, (4) vegetables and fruits, (5) meat, and (6) standing on contaminated ground. Real-time meteorology was used in all dose calculations for gaseous effluents.

Since there were no noble gases released from TMI-2 during 2006, the gamma and beta air doses (lines 3 and 4, respectively) were zero.

The maximum organ dose due to the release of particulates and tritium from TMI-2 in 2006 was 3.97E-5 mrem to the liver, total body, thyroid, kidney, lung and GI tract of a child residing 2000 meters from the site in the SE sector (line 5).

For 2006, TMI-2 liquid and gaseous effluents resulted in maximum hypothetical doses that were a small fraction of the quarterly and yearly ODCM dose limits.

TMI-2
SUMMARY OF MAXIMUM INDIVIDUAL DOSES FOR TMI-2 FROM
January 1, 2006 through December 31, 2006

Effluent	Applicable Organ	Estimated Dose (mrem)	Age Group	Location Dist (m)	Location Dir (to)	% of ODCM Dose Limit Quarter	% of ODCM Dose Limit Annual	ODCM Dose Limit (mrem) Quarter	ODCM Dose Limit (mrem) Annual
(1) Liquid	Total Body	4.39E-4	Adult	Receptor 1		2.93E-2	1.46E-2	1.5	3
(2) Liquid	Liver	6.90E-4	Teen	Receptor 1		1.38E-2	6.90E-3	5	10
(3) Noble Gas	Air Dose (gamma-mrad)	0	---	---	---	0	0	5	10
(4) Noble Gas	Air Dose (beta-mrad)	0	---	---	---	0	0	10	20
(5) Tritium & Particulate	Liver, Total Body, Thyroid, Kidney, Lung & GI Tract	3.97E-5	Child	2000	SE	5.29E-4	2.65E-4	7.5	15

Attachment 8
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Assessment of Radiation Doses from Liquid and Gaseous Effluents Releases to Members of the Public within the TMI Site Boundaries during 2006

The Offsite Dose Calculation Manual requires an assessment of the radiation doses from radioactive liquid and gaseous effluents to members of the public due to their activities inside the site boundary during the reporting period. The estimated dose to a member of the public at or within the TMI Site Boundary was 0.65 mrem for 2006.

The following are the assumptions made in this assessment:

Access to the TMI Owner Controlled Area is limited to only those persons who have business related activities that support the operation of the facility. Therefore, based on the definition of a 'member of the public' in NUREG-1301, there is no credible scenario for this individual to receive non-occupational dose inside the TMI Owner Controlled Area. The scenario selected will be recreational use of the Susquehanna River and shoreline next to the Owner Controlled Area fence. Based on the two definitions of Site Boundary in the ODCM, this scenario is AT the Site Boundary for liquid releases but INSIDE the Site Boundary for gaseous releases.

A member of the public stays next to the owner controlled area for 67 hours. The 67 hours is based upon Reg. Guide 1.109 shoreline recreation period given in Table E-5. This is a table of recommended values to be used for the maximum exposed individual in lieu of site-specific data. Three Mile Island is co-located with other islands in the Lake Frederick area of the Susquehanna River. This area is used recreationally for boating and fishing over the summer months. The application of the 67 hours of recreational use from Reg. Guide 1.109 is appropriate.

The dose from liquid effluents was included in this scenario. The highest activity contained in releases from TMI is from batch releases from the Waste Evaporator Condensate Storage Tanks. The maximum time period for a single release was 22 hours. Since the time of a single release is less than the 67 hours of recreational use of the river, the highest quarterly cumulative dose from liquid effluents will be used in this calculation. The highest quarterly cumulative dose was 3.56E-2 mrem, total body. This cumulative dose included both batch and continuous liquid releases. Assuming that the total dose from a quarter was received in the 67 hours is conservative.

The highest dose from a single airborne release is characterized by release G200605549. This release contained the highest concentration of tritium of any gaseous release. In 2006, tritium released in gaseous effluents comprised 98 percent of the total curies released to the environment. No other releases would yield a higher dose than the release with the highest tritium concentration. This release occurred over 503 hours. The entire dose from this release will be applied to the 67 hour recreational use period. The application of the total dose from this release to 67 hours is

conservative. The total dose from release G200605549 was 8.24E-4 mrem to the critical receptor.

The highest fenceline TLD result (assumed to be equal to dose) will be added to the dose from the highest liquid and gaseous releases to yield the hypothetical maximum dose to a member of the public within the site boundaries.

The highest fenceline TLD result for 2006 was from Station L1-1 and was 6.6 mrem per standard month. The net TLD dose, obtained by subtracting the results from a control station TLD from the indicator results, was not used. This again is conservative.

Calculations:

6.6 mrem/std mo. * 1/30.44 d/std mo. * 1/24 hr/day * 67 hr = 0.61 mrem

The dose from gas release G200605549 was 0.000823 mrem.

The quarterly cumulative dose from liquid effluents was 0.0356 mrem.

Total Dose Calculation

0.61 mrem + 0.000823 mrem + 0.0356 = 0.65 mrem

Attachment 9
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Assessment of Radiation Dose to Most Likely Exposed Real Individual per 40 CFR 190

Dose calculations were performed to demonstrate compliance with 40 CFR 190 (ODCM Part IV Section 2.10). Gaseous and liquid effluents released from TMI-1 and TMI-2 in 2006 resulted in maximum individual doses (regardless of age group) of 0.0211 mrem to the thyroid and 0.0264 mrem to any other organ including the whole (total) body. The direct radiation component was determined using the highest quarterly fence-line exposure rate as measured by an environmental TLD, and subtracting from it, the lowest quarterly environmental TLD exposure rate.

Based on the maximum exposure rate of 6.6 mR/standard month, a person residing at the fence-line for 67 hours (shoreline exposure from Reg. Guide 1.109) received an exposure of 0.61 mR. Based on the lowest exposure rate of 4.1 mR/standard month and converting it by the same method yielded a background exposure of 0.38 mR. Therefore, the net exposure from direct radiation from TMINS was 0.23 mR. Combining the direct radiation exposure (assumed to be equal to dose) with the maximum organ doses from liquid and gaseous releases, the maximum potential (total) doses were 0.25 mrem to the thyroid and 0.26 mrem to any other organ. Both doses were well below the limits specified in 40 CFR 190.

Attachment 10
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Deviations from the ODCM Sampling and Analysis Regime during 2006

There were two deviations from the ODCM sampling program in 2006.

The Unit 1 Turbine Building Sump Compositor was found empty on October 23, 2006. The compositor was repaired and returned to service immediately. This is a flow proportional composite sampler for continuous releases from the Turbine Building Sump to the Industrial Waste Treatment System. This is a requirement of the ODCM, Part 1, Table 3.2-1-A.2. Radioactivity concentrations for this sample location are very consistent. Sample results from the week after were used with the volume of water released from the Industrial Waste Treatment System to calculate release doses for the time period the compositor was Out of Service.

On December 29, 2006, the ESF ventilation system was run for a surveillance without the effluent radiation monitor RM-A-14 and its associated sampling panel in service. The system was shutdown immediately upon recognizing this fact. A release permit was created to calculate the dose for the approximately 4 hours the system ran. This system is emergency ventilation for the fuel handling building and the normal radiation monitor, RM-A-4, for this building saw no change in reading over this time frame. The only isotope occasionally seen when this system runs is tritium. The normal weekly grab sample tritium for the spent fuel pool area was used for the dose calculation since no tritium sample was obtained via RM-A-14.

Attachment 11
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

Errata report for the 2005 Annual Radioactive Effluent Releases Report

An errata report was required due to the same values as 2004 being reported in 2005 in the Summary of Maximum Individual Doses for TMI-1 Table. The calculations were performed for 2005, but the table was not updated with the new values before submitting the annual report. Only the affected pages are attached. The changes are annotated in bold font.

Assessment of Radiation Doses Due to Radioactive Liquid and Gaseous Effluents Released from TMI during 2005

TMI-1

The attached table presents the maximum hypothetical doses to an individual and the general population resulting from 2005 TMI-1 releases of gaseous and liquid effluents. Provided below is a brief explanation of the table.

A. Liquid (Individual)

Calculations were performed on the four age groups and seven organs recommended in Regulatory Guide 1.109. The pathways considered for TMI-1 were the consumption of drinking water and fish and standing on the shoreline influenced by TMI-1 effluents. The latter two pathways are considered to be the primary recreational activities associated with the Susquehanna River in the vicinity of TMI. The "critical receptor" or Receptor 1 was that individual who 1) consumed Susquehanna River water from the nearest downstream drinking water supplier (Wrightsville Water Supply), 2) consumed fish residing in the vicinity of the TMI-1 liquid discharge outfall and 3) occupied an area of shoreline influenced by the TMI-1 liquid discharge.

For 2005, the calculated maximum whole body (or total body) dose from TMI-1 liquid effluents was **4.35E-2** mrem to an adult (line 1). The maximum organ dose was **5.76E-2** mrem to the liver of an adult (line 2).

B. Gaseous (Individual)

There were six major pathways considered in the dose calculations for TMI-1 gaseous effluents. These were: (1) plume exposure (2) inhalation, consumption of; (3) cow milk, (4) vegetables and fruits, (5) meat, and (6) standing on contaminated ground. Real-time meteorology was used in all dose calculations for gaseous effluents.

Lines 3 and 4 present the maximum plume exposure at or beyond the site boundary. The notation of "air dose" is interpreted to mean that these doses are not to an individual, but is considered to be the maximum doses that would have occurred at or beyond the site boundary. The table presents the distance in meters to the location in the affected sector (compass point) where the theoretical maximum plume exposures occurred. The calculated maximum plume exposures were **9.93E-5** mrad and **1.38E-4** mrad for gamma and beta, respectively.

The maximum organ dose due to the release of iodines, particulates and tritium from TMI-1 in 2005 was **1.85E-2** mrem to the liver, total body, thyroid, kidney, lung and GI-LLI of an child residing **2000** meters from the site in the **SE** sector (line 5). This dose again reflects the maximum exposed organs for the appropriate age group.

For 2005, TMI-1 liquid and gaseous effluents resulted in maximum hypothetical doses that were a small fraction of the quarterly and yearly ODCM dose limits.

TMI-1
SUMMARY OF MAXIMUM INDIVIDUAL DOSES FOR TMI-1 FROM
January 1, 2005 through December 31, 2005

Effluent	Applicable Organ	Estimated Dose (mrem)	Age Group	Location Dist (m)	Location Dir (to)	% of ODCM Dose Limit		ODCM Dose Limit (mrem)	
						Quarter	Annual	Quarter	Annual
(1) Liquid	Total Body	**4.35E-2**	Adult	Receptor 1		**2.90**	**1.45**	1.5	3
(2) Liquid	Liver	**5.76E-2**	Adult	Receptor 1		**1.15**	**5.76E-1**	5	10
(3) Noble Gas	Air Dose (gamma-mrad)	**9.93E-5**	---	**2000**	**WSW**	**1.99E-3**	**9.93E-4**	5	10
(4) Noble Gas	Air Dose (beta-mrad)	**1.38E-4**	---	**4000**	**WNW**	**1.38E-3**	**6.9E-4**	10	20
(5) Iodine, Tritium & Particulates	Liver, Total Body, Thyroid, Kidney, Lung & GI-LLI	**1.85E-2**	Child	**2000**	**SE**	**2.47E-1**	**1.23E-1**	7.5	15

Enclosure 1
2006 Annual Radioactive Effluent Releases Report for TMI
5928-07-20096

ODCM change for
TMI Offsite Dose Calculation Manual, Revision 25
6610-PLN-4200.01

(Revision 25 was issued on
August 9, 2006)

ATTACHMENT 1
Procedure Approval Form
Page 1 of 1

AD-AA-101-1002
Revision 9

Document Number: 6610 - PLN - 4200.01 **Revision:** 24

Title: Offsite Dose Calculation Manual (ODCM)

☐ New ☐ Cancel Document ☐ Cancel Revision ☒ Revision ☐ Editorial ☐ Batch EC#: ____ PCR#: PC-20986 PPIS#: ____ ER#: ____ AR#: ____ #: ____

☐ Supersede corporate document(s) List: 11716169

Revision Summary:
Attach add'l descript, if req'd see attached change summary matrix

CONFIRM that **no** commitments (i.e., those steps annotated with CM-X) have been changed or deleted unless evaluated via completion of LS-AA-110 commitment change/deletion form and **INITIAL** [Originator]: LKW

Originator: L. K. Weber (Print) 7/21/06 (Date) TMI/8947 (Location/Ext)

Applicable Site Contacts (Check box and provide name)

BR ☐		DR ☐		QC ☐	
BY ☐		LA ☐		CL ☐	
PB ☐		OC ☐		LG ☐	
TMI ☐		ZN ☐		Other ☐	
SA ☐		HC ☐		Other ☐	

Validation Req'd: ☒ No ☐ Yes (attach) (Validation requirement see AD-AA-101) Print/Signature

Common Training Req'd: ☒ No ☐ Yes

Site Specific Training Req'd: ☐ No ☒ Yes

Change Management: ☐ HU-AA-1101 Change Checklist Attached ☐ Document Traveler ☒ None Required

Level of Use: ☐ Level 1 - Continuous Use ☐ Level 2 - Reference Use ☒ Level 3 - Information Use

Approval

CFAM (Standard Procedures)	Print/Sign	Date	Location/Ex:

Approval Location: TMI

Site Document(s) to be superseded:
Use additional sheets as necessary. Assure that all pending changes are dispositioned. ✓

☐ Temp. Change ☐ Interim Change Temp or Interim Change #: Interim Change expiration:

10CFR50.59 Applicable: ☐ No ☒ Yes (Or applicable regulatory process reviews) ☐ Exempt per

10CFR72.48 Applicable: ☒ No ☐ Yes **Tracking Number**

PORC Required: ☐ No ☒ Yes **PORC Number** (after PORC Approval): 2006-020

☐ If superceding a document containing commitments, notify the Commitment Tracking Coordinator per LS-AA-110 so the CTD can be updated as appropriate.

SQR/ITR/RTR ☐ N/A Req'd Reviews/Approval: (list) **Surveillance Coordinator Review Req'd** ☒ No ☐ Yes

Cross Discipline Reviews

Print	Signature	Date	Discipline or Org.
R. Freeman		7/21/2006	ENG.
Adam Miller	Adam Miller	7/24/2006	Ops Assurance
John A. Wilson per telecon	John A. Wilson	8/2/06	Chem CFAM

Attach additional if req'd

Temp Change Authorization Only	SRO Print/Sign/Date	SQR Print/Sign/Date	Impl. Date	Exp. Date

SQR Approval indicates that all required Cross-Disciplinary reviews have been performed and the reviewers have signed this form. This procedure is technically and functionally accurate for all functional areas.

SQR Approval: F.W. LINSENBACH (Print/Sign) 8/2/2006 (Date) RP/RD/NPPT (Discipline or Org.)

Site Authorization: S. CHRIS BAKER (SFAM Print/Sign) 08/02/2006 (Date) 8/9/06 (Impl. Date)

GLEN EARL CHICK (Plant Manager Print/Sign (when required by procedure)) 8/7/2006 (Date)

ATTACHMENT 1
Procedure Approval Form
Page 1 of 1

AD-AA-101-1002
Revision 9

Document Number: 6610 - PLN - 4200.01 Revision: 24

Title: Offsite Dose Calculation Manual (ODCM)

☐ New ☐ Cancel Document ☐ Cancel Revision ☒ Revision ☐ Editorial ☐ Batch EC#: ____ PCR#: PC-20186 PPIS#: ____ ER#: ____ AR#: ____ #: ____

☐ Supersede corporate document(s) List: ____

Revision Summary:
Attach add'l descript. if req'd: see attached change summary matrix

CONFIRM that no commitments (i.e., those steps annotated with CM-X) have been changed or deleted unless evaluated via completion of LS-AA-110 commitment change/deletion form and **INITIAL** [Originator]: LKW

Originator:	L. K. Weber (Print)	7/21/06 (Date)	4M1/8947 (Location/Ext)		
Applicable Site Contacts Check box and provide name	BR ☐	DR ☐	QC ☐		
	BY ☐	LA ☐	CL ☐		
	PB ☐	OC ☐	LG ☐		
	TMI ☐	ZN ☐	Other ☐		
	RA ☐	HC ☐	Other ☐		

Validation Req'd: ☒ No ☐ Yes (attach) Common Training Req'd: ☒ No ☐ Yes

Site Specific Training Req'd: ☐ No ☒ Yes

(Validation requirement see AD-AA-101) Print/Signature

Change Management: ☐ HU-AA-1101 Change Checklist Attached ☐ Document Traveler ☒ None Required

Level of Use: ☐ Level 1 - Continuous Use ☐ Level 2 - Reference Use ☒ Level 3 - Information Use

Approval				
	CFAM (Standard Procedures)	Print/Sign	Date	Location/Ex

Approval Location: TMI

Site Document(s) to be superseded:
Use additional sheets as necessary. Assure that all pending changes are dispositioned.

☐ Temp. Change ☐ Interim Change Temp or Interim Change #: Interim Change expiration:

10CFR50.59 Applicable: ☐ No ☒ Yes (Or applicable regulatory process reviews) ☐ Exempt per ____

10CFR72.48 Applicable: ☒ No ☐ Yes Tracking Number

PORC Required: ☐ No ☒ Yes PORC Number (after PORC Approved):

☐ If superseding a document containing commitments, notify the Commitment Tracking Coordinator per LS-AA-110 so the CTD can be updated as appropriate.

SQR/ITR/RTR ☐ N/A Req'd Reviews/Approval: (list) **Surveillance Coordinator Review Req'd** ☒ No ☐ Yes

Cross Discipline Reviews

Print	Signature	Date	Discipline or Org.
R. Freeman	[signature]	7/21/2006	ENG
Adam Miller	Adam Miller	7/24/2006	Ops Asst Shift Supv
CARY R. LEIDICH	Gary R Leidich	8/1/2006	GPU Licensing Officer

Attach additional if req'd

Temp Change Authorization Only	SRO Print/Sign/Date	SQR Print/Sign/Date	Impl. Date	Exp. Date

SQR Approval indicates that all required Cross-Disciplinary reviews have been performed and the reviewers have signed this form. This procedure is technically and functionally accurate for all functional areas.

SQR Approval: Print/Sign ____ Date ____ Discipline or Org. ____

Site Authorization:

SFAM Print/Sign	Date	Impl. Date
Plant Manager Print/Sign (when required by procedure)	Date	

FirstEnergy

FAX Transmittal *FAX # 330-384-5669*

AUGUST 1, 2006

Please Deliver
To: Jim Byrne

Location: TMI

Subject: Procedure Approval Form – GPU Cognizant Officer

Fax: 330-315-8470

Phone: 717-948-8461

From: Ted Burgner

Number of pages:
(including cover) 2

If you do not receive all pages, please call Ted Burgner, **330-761-7890.**

Sir:

Attached is the signed Procedure Approval Form you requested. If you would like me to send the original, please let me know.

Thank you,

Ted

330-761-7890

Exelon Nuclear

Training Needs/Cost Analysis Worksheet Job Aid

Part One - Needs Analysis

TRAINING TOPIC: OFFSITE DOSE CALCULATION MANUAL (ODCM) CHANGE

1. What are the reasons for performing Needs Analysis?

	Performance deficiency		Management request for training
X	Regulation change		OPEX (LER, SOER, SER, etc.)
X	Procedure change		System/equipment hardware change
	System/equipment operating characteristic change (flows, pressure, temperatures, etc.)		Feedback (student, instructor, management, assessment, or inspection results)
	New or changed job position description		Other:

Describe reasons: THE ODCM IS BEING UPDATED TO REVISE REQUIREMENT OF THE CONDENSER OFF GAS MONITORING REQUIREMENTS AS WELL AS CLARITY NOTES ON RM-A-7 OPERABILITY AND SURVEILLANCE RM-A-4 & 6 IODINE SAMPLERS

2. Will this need for training be evaluated by another process?

	YES: Need IS being addressed via other processes such as Job and Task Analysis maintenance. (Stop Need Analysis)
X	**NO:** Need IS NOT being addressed via other processes (continue at question 3)

3. What prevents or may prevent personnel from performing this task satisfactorily?

Lack of cognitive knowledge or physical skills. These situations would most likely be resolved through training.

	New/revised procedure or equipment		Plant modification
X	New/revised regulatory requirement		Industry event
	Inadequate training on tasks		Change in job scope
	Change in task skill or knowledge requirements		Unsatisfactory work control

Describe what is not known or cannot be done:

Improper attitude. Affective behavior can be resolved by management initiatives which might include training. Factors impacting affective behavior include:

	Confusing work assignments
	Poor supervision observation/follow-up on work in progress
	Work assignments that are not challenging
	Poor interpersonal relationships on job
	Lack of acknowledgment of the worth of the work
	Misconception of one's own abilities
	"Rewarding" satisfactory performance by increasing the work

Describe factors below:

	Environmental obstacles: Environmental obstacles can usually be resolved by management initiatives other than training:		
	Distractions on the job		Poor lighting
	Lack of resources		Poor equipment
	Inadequate procedures		Poor work planning by the organization
	Poorly defined work goals		Unsatisfactory work control

Describe obstacles: (If ONLY Environmental obstacles are marked, continue at question 9)

4. Will training raise level of personnel performance to the desired level; or enhance the safety and reliability of plant operations; or help prevent errors based on lessons learned from operating experience?

	YES / MAYBE	Explain and continue at question 5:
	NO	Explain and continue at question 9:

5. Are the consequences of performance deficiencies important?

	YES: Threat to health and safety of plant personnel or general public. Personnel injury, fuel damage or radioactive release may result. (continue at question 6)
	YES: Plant availability affected by loss of system operability or equipment down time. Regulatory violations or major equipment damage may result. (continue at question 6)
	YES: Additional compensating actions are required to complete a task, prevent minor regulatory infractions, or non-essential equipment failures. Inefficient use of materials or manpower may result. (continue at question 6)
X	**NO:** No significant impact on plant operations. Performance does not impact plant, personnel or public. (continue at question 9)

Describe Consequences:

6. What is the estimated cost of actual/potential performance deficiency.

The cost of continued performance deficiencies is the value the company places on the mistake if it has already happened or if it were to happen. These costs may be hard to quantify, but a realistic approximation is needed to justify spending company resources. For example, if the incident did or could shut the plant down for 1 day, the lost production and power replacement cost of approximately $1M per day plus repair costs, overtime for crews, and other costs would need to be considered as the cost of the performance deficiency. If the training is regulatory-driven, consider what the costs would be if the training was not done. For example, there might be regulatory fines, plant shutdowns, stop work actions, union walkouts, public relations problems to correct. Describe the impact of not correcting the problem.

Estimated cost of performance deficiency:
Describe:

7. Which personnel and how many are impacted by this training need? (check all applicable boxes)

☐	Licensed Operator (RO, SRO):____	☐	Non-Licensed Operator: _____
☐	Shift Technical Advisor:____	☐	Fuel Handler/SRO Limited: _____
☐	Engineer: _____	☐	Chemistry technician: _____
☐	Radiation technician: _____	☐	New employee: _____
☐	Electrical Maint. Technician: _____	☐	Instrument Maint. technician: _____
☐	Mechanical Maint. Technician:_____	☐	Other: _____

8. Perform Cost Analysis, using Excel method (page 4), or manual method (page 5)

☐	Return on Investment (ROI) is less than 1	Do not train.
☐	Return on Investment (ROI) is between 1 and 2	Training may be part of solution.
☐	Return on Investment (ROI) is greater than 2	Training is the solution.

9. TRAINING RECOMMENDATION: (check all applicable boxes)

☐	**Train immediately** (immediately contact training supervision)	X	Tailgate /Just-In-Time (JIT) training
		☐	Classroom
		☐	Simulator / Dynamic Learning Activity
		☐	Lab
		☐	Other:
☐	**Include in existing training program** (proceed with job and task analysis, if needed)	☐	Next Continuing Training session
		X	Future Continuing Training session
		X	Next Initial Training class
☐	Do NOT Train		

Describe: PERFORM JIT TRAINING HANDOUT/SUPERVISORY BRIEF ON ODCM CHANGE. INCORPORATE INTO LESSON PLAN FOR FUTURE CONTINUING TRAINING AS WELL AS INITIAL LICENSE TRAINING.

10. Non-Training Actions to consider (check the applicable boxes):

☐	Change the procedure	☐	Change work design or conditions
☐	Provide a job aid	☐	Improve incentives / consequences
☐	Modify the equipment	☐	Improve tools
☐	Change job assignments	☐	Conduct crew briefings
☐	Improve feedback, coaching, or supervisory oversight	☐	Other: _____
		☐	None

Describe:

R.S. Campbell
8-2-06

Part Two - Training Cost Analysis Worksheet – Excel® Method

Design and Development (D&D) Costs:

Enter projected training time per student:	
Enter D&D - to - Training Ratio:	
Total D&D time:	0.00
Enter D&D personnel labor rate:	

Total D&D Costs: $ -

External Costs (entire training sequence): (Complete items below or attach vendor proposal)

Visuals:	
Materials:	
Consultants/Vendor Trainers:	
Hardware:	
Travel:	
Refreshments:	
Other: $ (Describe):	

Total External Costs: $ -

Implementation Costs:

Enter number of Classes:	
Enter number of instructors per class:	
Enter instructor administration time/class:	
Enter instructor labor rate:	
Other: $ (Describe):	

Total Instructor Implementation Costs: $ -

Enter number of students per class:	
Enter average student travel/lodging cost:	
Enter average student labor rate:	
Other: $ (Describe):	

Total Student Implemetation Costs: $ -

Total Implementation Costs: $ -

Evaluation Costs:

Enter number of evaluations planned:	
Enter time to conduct each evaluation:	
Enter number of evaluators/evaluation:	
Enter evaluator labor rate:	

Total Evaluation Costs: $ -

TOTAL COST OF TRAINING: $ -

COST OF CONTINUED PERFORMANCE DEFICIENCY:

RETURN ON INVESTMENT (ROI): #DIV/0!

50.59 REVIEW COVERSHEET FORM

Station/Unit(s): TMI-1

Activity/Document Number: PC-20986 for 6610-PLN-4200.01 **Revision Number:** 0/ 24

Title: Offsite Dose Calculation Manual

NOTE: For 50.59 Evaluations, information on this form will provide the basis for preparing the biennial summary report submitted to the NRC in accordance with the requirements of 10 CFR 50.59(d)(2).

Description of Activity:
(Provide a brief, concise description of what the proposed activity involves.)

The ODCM is being updated to reflect revisions to the requirements for operability of the offgas ventilation radiation monitors. Based on the NRC letter to Chris Crane dated 7-19-06 (see attached) confirming that revisions to the condenser vent system low range noble gas monitors operability requirements are now governed by the TS requirements for ODCM revisions, AmerGen does not consider these operability requirements as regulatory commitments in accordance with ASLB Order LBP-84-47. The ODCM is also being changed to clarify a note for operability of RM-A-7 and references to the iodine sampler and radiation monitors for RMA-4 and 6. REMP sample locations were also updated to reflect current locations and to use more accurate distance and azimuth information acquired through GPS readings.

Reason for Activity:
(Discuss why the proposed activity is being performed.)

The revised requirements were requested to bring TMI-1 more in line with current industry guidelines for primary to secondary leakage. The note clarification was identified in IR 44097 during ILT training. The REMP location updates were a result of Global Positioning System (GPS) readings taken during normal sample rounds, corrections from REMP reviews and the addition of an alternate milk sampling farm and food products location in case one of the regular locations is not available. The clarifications for RM-A-4 and RM-A-6 monitors came about as a result of reviews for ECR 06-00061.

Effect of Activity:
(Discuss how the activity impacts plant operations, design bases, or safety analyses described in the UFSAR.)

The revised requirements for the condenser offgas monitoring are consistent with industry guidance provided in EPRI documents for primary to secondary leakage. The note clarification and REMP sample location updates will have no impact on radiological effluent control nor will they change any of the calculations for doses from pathways in the environment.

Summary of Conclusion for the Activity's 50.59 Review:
(Provide justification for the conclusion, including sufficient detail to recognize and understand the essential arguments leading to the conclusion. Provide more than a simple statement that a 50.59 Screening, 50.59 Evaluation, or a License Amendment Request, as applicable, is not required.)

The NRC letter to Chris Crane dated 7-19-06 states that once the operability requirements were relocated to the ODCM they were bounded by ODCM change requirements. Per Technical Specification 6.14, ODCM changes do not require pre-NRC approval, but they do require submittal with the next annual Radiological Effluents Report to the NRC, proper justification and appropriate cross-disciplinary, RTR/SQR and PORC review.

Attachments:
Attach all 50.59 Review forms completed, as appropriate.
(NOTE: if both a Screening and Evaluation are completed, no Screening No. is required.)
Forms Attached: (Check all that apply.)

X	**Applicability Review**				
	50.59 Screening	**50.59 Screening No.**	_____	**Rev.**	_____
	50.59 Evaluation	**50.59 Evaluation No.**	_____	**Rev.**	_____

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

JULY 19, 2006

Mr. Christopher M. Crane
President and Chief Executive Officer
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT THREE MILE ISLAND NUCLEAR STATION, UNIT 1 - REQUEST TO REVISE CONDENSER VENT SYSTEM LOW RANGE NOBLE GAS MONITOR OPERABILITY REQUIREMENT (TAC NO. MD2058)

Dear Mr. Crane:

By letter dated May 25, 2006 (Agencywide Documents Access and Management System Accession Number ML061510356), AmerGen Energy Company (AmerGen) submitted a request for Nuclear Regulatory Commission (NRC) approval of a proposed commitment revision regarding the operability requirement for the condenser vent system low range noble gas monitors specified in the Three Mile Island (TMI) Unit 1 offsite dose calculation manual (ODCM).

AmerGen, in its request, stated that the current ODCM operability requirements applicable to the condenser vent system low range noble gas (condenser vent radiation) monitors fulfill a regulatory commitment in accordance with NRC Atomic Safety and Licensing Board (ASLB) Order LBP-84-47, dated October 31, 1984. Further, AmerGen stated that TMI Unit 1 Technical Specification (TS) Amendment No. 103, dated December 21, 1984, specifically states that the revised TS included in the amendment conformed with the condition imposed by the ASLB. TS 3.21.2 specified the allowed condenser vent radiation monitors outage time. TMI Unit 1 TS Amendment No. 197, dated October 2, 1995, relocated TS 3.21.2 requirements to the ODCM. In accordance with the AmerGen commitment management program, the proposed commitment change modifies the method of compliance specifically defined in the ASLB Order and, therefore, according to AmerGen's program, requires NRC's prior approval.

The NRC staff has reviewed AmerGen's submittal, its referenced documents, and TMI TS Section 6.14, "Offsite Dose Calculation Manual." Based on its review, the NRC staff determined that once Amendment No. 197 was issued, the sections that were relocated from the TS to the ODCM are bounded by the requirements of TS Section 6, "Administrative Controls," which govern technical review and controls of procedures and programs. TS Section 6.5.1.12 requires that AmerGen perform a 10 CFR 50.59 analysis to determine whether or not NRC approval is required. If it is determined that NRC approval is required, then AmerGen should submit its request in the form of a license amendment request, pursuant to 10 CFR 50.90.

This completes the NRC staff's activities with respect to TAC No. MD2058.

Sincerely,

Brooke D. Poole

Brooke D. Poole, Acting Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-289

cc: See next page

Three Mile Island Nuclear Station, Unit 1

cc:

Site Vice President - Three Mile Island Nuclear Station, Unit 1
AmerGen Energy Company, LLC
P. O. Box 480
Middletown, PA 17057

Senior Vice President - Nuclear Services
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Vice President - Operations, Mid-Atlantic
AmerGen Energy Company, LLC
200 Exelon Way, KSA 3-N
Kennett Square, PA 19348

Vice President - Licensing and Regulatory Affairs
AmerGen Energy Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Chairman
Board of County Commissioners
of Dauphin County
Dauphin County Courthouse
Harrisburg, PA 17120

Chairman
Board of Supervisors
of Londonderry Township
R.D. #1, Geyers Church Road
Middletown, PA 17057

Senior Resident Inspector (TMI-1)
U.S. Nuclear Regulatory Commission
P.O. Box 219
Middletown, PA 17057

Director - Licensing and Regulatory Affairs
AmerGen Energy Company, LLC
200 Exelon Way, KSA 3-E
Kennett Square, PA 19348

Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Protection
Rachel Carson State Office Building
P.O. Box 8469
Harrisburg, PA 17105-8469

Plant Manager - Three Mile Island Nuclear Station, Unit 1
AmerGen Energy Company, LLC
P. O. Box 480
Middletown, PA 17057

Regulatory Assurance Manager - Three Mile Island Nuclear Station, Unit 1
AmerGen Energy Company, LLC
P.O. Box 480
Middletown, PA 17057

Ronald Bellamy, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Michael A. Schoppman
Framatome ANP
Suite 705
1911 North Ft. Myer Drive
Rosslyn, VA 22209

Vice President, General Counsel and Secretary
AmerGen Energy Company, LLC
2301 Market Street, S23-1
Philadelphia, PA 19101

Three Mile Island Nuclear Station, Unit 1

cc:

Dr. Judith Johnsrud
National Energy Committee
Sierra Club
433 Orlando Avenue
State College, PA 16803

Eric Epstein
TMI Alert
4100 Hillsdale Road
Harrisburg, PA 17112

Correspondence Control Desk
AmerGen Energy Company, LLC
P.O. Box 160
Kennett Square, PA 19348

Manager Licensing - Three Mile Island Nuclear Station, Unit 1
Exelon Generation Company, LLC
200 Exelon Way, KSA 3-E
Kennett Square, PA 19348

Assistant General Counsel
AmerGen Energy Company, LLC
200 Exelon Way
Kennett Square, PA 19348

50.59 APPLICABILITY REVIEW FORM

Activity/Document Number: PC-20986 for 6610-PLN-4200.01 **Revision Number:** 0/ 24

Address the questions below for all aspects of the Activity. If the answer is yes for any portion of the Activity, apply the identified process(es) to that portion of the Activity. Note that it is not unusual to have more than one process apply to a given Activity. See Section 4 of the Resource Manual (RM) for additional guidance.

I.	Does the proposed Activity involve a change:		
	1. Technical Specifications or Operating License (10CFR50.90)?	X NO __ YES	See Section 4.2.1.1 of the RM
	2. Conditions of License Quality Assurance program (10CFR50.54(a))? Security Plan (10CFR50.54(p))? Emergency Plan (10CFR50.54(q))?	 X NO __ YES X NO __ YES X NO __ YES	See Section 4.2.1.2 of the RM
	3. Codes and Standards IST Program Plan (10CFR50.55a(f))? ISI Program Plan (10CFR50.55a(g))?	 X NO __ YES X NO __ YES	See Section 4.2.1.3 of the RM
	4. ECCS Acceptance Criteria (10CFR50.46)?	X NO __ YES	See Section 4.2.1.4 of the RM
	5. Specific Exemptions (10CFR50.12)?	X NO __ YES	See Section 4.2.1.5 of the RM
	6. Radiation Protection Program (10CFR20)?	X NO __ YES	See Section 4.2.1.6 of the RM
	7. Fire Protection Program (applicable UFSAR or operating license condition)?	X NO __ YES	See Section 4.2.1.7 of the RM
	8. Programs controlled by the Operating License or the Technical Specifications (such as the ODCM).	__ NO X YES	See Section 4.2.1.7 of the RM
	9. Environmental Protection Program	X NO __ YES	See Section 4.2.1.7 of the RM
	10. Other programs controlled by other regulations.	X NO __ YES	See Section 4.2.1 of the RM
II.	Does the proposed Activity involve maintenance which restores SSCs to their original condition or involve a temporary alteration supporting maintenance that will be in effect during at-power operations for 90 days or less?	X NO __ YES	See Section 4.2.2 of the RM
III.	Does the proposed Activity involve a change to the:		
	1. UFSAR (including documents incorporated by reference) that is excluded from the requirement to perform a 50.59 Review by NEI 96-07 or NEI 98-03?	X NO __ YES	See Section 4.2.3 of the RM
	2. Managerial or administrative procedures governing the conduct of facility operations (subject to the control of 10CFR50, Appendix B)	X NO __ YES	See Section 4.2.4 of the RM
	3. Procedures for performing maintenance activities (subject to 10 CFR 50, Appendix B)?	X NO __ YES	See Section 4.2.4 of the RM
	4. Regulatory commitment not covered by another regulation based change process (see NEI 99-04)?	X NO __ YES	See Section 4.2.3/4.2.4 of the RM
IV.	Does the proposed Activity involve a change to the Independent Spent Fuel Storage Installation (ISFSI) (subject to control by 10 CFR 72.48)	X NO __ YES	See Section 4.2.6 of the RM

Check one of the following:

☒ **If** all aspects of the Activity are controlled by one or more of the above processes, **then** a 50.59 Screening is not required and the Activity may be implemented in accordance with its governing procedure.

☐ **If** any portion of the Activity is **not** controlled by one or more of the above processes, then process a 50.59 Screening for the portion not covered by any of the above processes. The remaining portion of the activity should be implemented in accordance with its governing procedure.

Signoff:

50.59 Screener/50.59 Evaluator: (Circle One) LKWeber (Print name) Sign: LKWeber (Signature) Date: 7/24/06

ATTACHMENT 1
ODCM Change Summary Matrix
Page 1 of 1

Item No.	(old) Rev. 24 page No.	(new) Rev. 25 page No.	Description of Change
1	23	23	Added reference to AmerGen letter to NRC dated May 26, 2006.
2	25	25	Changed the minimum operable channels for RMA-5 to 1.
3	26	26	Correct 5.b from samples to sampler to match NUREG 1301 wording.
4	28	28	Clarified wording for note on operability of RM-A-7 per IR 464097.
5	29	29	Rewrite action statement for new compensatory actions.
6	112	112	Remove detail adjectives to describe radiation monitors. ECR 06-00061 removed iodine monitors, but not samplers that are required for ODCM.
7	171-175	37	Update survey distances and azimuths based on GPS sitings peformed in 2006.
8	171	171	Delete air sampling stations B1-4 and Q4-1 that no longer exist and are not required.
9	174	174	Added milk farm F4-1 as an alternate sampling location.
10	175	175	Added Red Hill farm as an alternate food products sampling location. Deleted second control station which is not needed and no longer used.
11	176	176	Removed number for deleted station and reused for new station.
12	177	177	Removed number for deleted station and reused for new station. Corrected duplicate number to correct number per Table 8.5.
13	178	178	Removed number for deleted station.
14			
15			
16			
17			
18			

ATTACHMENT 2
ODCM Change Determination

Station: Three Mile Island Page 1 of 4

ODCM Revision No. 25 Determination No. _____

I. Determination Questions (Check correct response)

1. Does the ODCM change maintain the level of radioactive effluent control required by 10CFR20.1301? X YES ___ NO

   Explain: (provide sufficient information including appropriate analyses justifying the ODCM change)

   10 CFR 20.1301(a)(1) requires that each licensee conduct operations so that the total effective dose equivalent to individual members of the public from TMI operation does not exceed 100 mrem in one year. The revisions to the condenser offgas monitor operability requirements and changes to clarify notes/ descriptions and update REMP locations will not increase the dose to the public or reduce TMI's ability to control or quantify condenser offgas, Waste Gas Decay Tank or Aux/ Fuel Handling building ventilation releases. The additional compensatory sampling for offgas ventilation releases will enable quantification of noble gases via this release path. Continuous particulate and iodine sampling requirements are not impacted by this change. This change also brings TMI more in alignment with the NRC's NUREG-1301, "Offsite Dose Calculation Manual Guidance: Standard Radiological Effluent Controls for Pressurized Water Reactors". Section 3.3.3.11 for Radioactive Gaseous Effluent monitoring of this guide for Condenser Evacuation system requires 1 minimum operable noble gas activity monitor with grab samples every 12 hours if inoperable and restore to operable within 30 days otherwise explain in next Radioactive Release Report. Section 3.3.3.11 of NUREG 1301 also states that the shutdown provisions of controls 3.0.3 and 3.0.4 are not applicable.

   The more stringent grab sample and operability requirements are not required explicitly by Radiological Effluent Controls regulations, but are a result of industry guidance from EPRI for primary to secondary leakage. TMI chooses to implement this guidance via the ODCM.

2. Does the ODCM change maintain the level of radioactive effluent control required by 10CFR20.1302? X YES ___ NO

   Explain: (provide sufficient information including appropriate analyses justifying the ODCM change)

   10 CFR 20.1302 requires licensees to perform appropriate surveys in unrestricted areas and controlled areas to demonstrate compliance with dose limits for individual members of the public. The ability to perform surveys is not impacted by the changes due to the revised requirements or by the note/ description clarifications and updates to REMP sample

ATTACHMENT 2
ODCM Change Determination

locations. Effluent concentration limits will not be impacted by these changes. TMI will still control its effluent path to ensure dose to the public is ALARA.

3. Does the ODCM change maintain the level of radioactive effluent control required by 40CFR190? _X_ YES ___ NO

   Explain: (provide sufficient information including appropriate analyses justifying the ODCM change)

   40CFR190 requires that the annual dose equivalent does not exceed 25 mrem to the whole body, 75 mrem to the thyroid and 25 mrem to any other organ of any member of the public as a result of exposure to planned discharges of radioactive materials to the environment from uranium fuel cycle operations. The changes to the ODCM for the revised requirements or for the note/ description clarifications and REMP location updates will not increase discharges of radioactive materials nor will they negatively impact TMI's ability to control or quantify radioactive releases to the environment.

4. Does the ODCM change maintain the level of radioactive effluent control required by 10CFR50.36a? _X_ YES ___ NO

   Explain: (provide sufficient information including appropriate analyses justifying the ODCM change)

   10CFR 50.36a requires that operating procedures be developed and followed for the control of effluents and that the radioactive waste system be maintained and used. It also specifies that each licensee shall submit a report annually that specifies the quantity of each of the principal radionuclides released to unrestricted areas in liquid and in gaseous effluents during the previous 12 months. These requirements will keep average annual releases of radioactive material in effluents and their resultant committed effective dose equivalents (CEDE) at small percentages of the dose limits. Operation of the rad waste system is not affected by these changes. The operation of the plant releasing and monitoring the effluents from the condenser offgas system will not be negatively impacted by the revised operability requirements. Two radiation monitors will remain installed and maintained on the offgas effluent. These changes will only affect the actions taken if these monitors are out of service. The revised requirements, the note/ description clarifications and REMP location updates wil not impact the annual reporting of radiological effluents to the NRC.

5. Does the ODCM change maintain the level of radioactive effluent control required by Appendix I to 10CFR50? _X_ YES ___ NO

ATTACHMENT 2
ODCM Change Determination

Explain: (provide sufficient information including appropriate analyses justifying the ODCM change)

10CFR50 Appendix I,, **Section II.**A establishes the design objectives and limiting conditions for operation to meet ALARA criteria such that the calculated annual total quantity of all radioactive material above background released to unrestricted areas will not result in an estimated annual dose or dose commitment from liquid effluents in excess of 3 mrems to the total body or 10 mrems to any organ. For gaseous effluents the estimated annual air dose can not exceed 10 mrems for gamma radiation and 20 mrems for beta radiation. Section III establishes that implementation of Section II be demonstrated by calculational procedures based upon models and data such that the actual exposure of an individual through appropriate pathways is unlikely to be substantially underestimated and that the cumulative effect of all sources and pathways being influenced by the plant be considered. Section IV provides guides on limiting conditions for operation. The licensee shall establish appropriate surveillance and monitoring programs for reporting of rad effluents data, environmental monitoring data and resulting radiation doses and changes in the use of unrestricted areas surrounding the plant. If releases of effluents result in exceeding one-half the annual exposure, there are investigative and reporting requirements.

The minor corrections to the azimuth and distance data for REMP locations will not impact ODCM calculations or methodologies for evaluating doses to individuals from principal pathways of exposure. The removal of two air sample stations does not impact TMI's ability to meet ODCM requirements from Table 8.1 to sample from 5 air sampling locations in Table 8.4. Table 8.4 still has two additional sample locations beyond the minimum of 5. The changes to actions for offgas radiation monitoring operability will also not change any methods of sampling or calculation of radiological effluents from this pathway. The note clarifying when releases from the waste gas system may occur without the primary radiation monitor operable also will not impact dose calculations from this release path. Changing the word for the iodine sampler in Table 2.1-2 to match NUREG 1301 and deleting the adjectives for the Aux/ Fuel Handling ventilation radiation monitors does not impact their ability to do iodine and particulate sampling for ODCM calculation requirements.

ATTACHMENT 2
ODCM Change Determination

Page 4 of 4

6. Does the ODCM change maintain the accuracy or reliability of effluent, dose, or setpoint calculations? X YES ___ NO

   Explain: (*provide sufficient information including appropriate analyses* justifying the ODCM change)

   The changes to the ODCM to change actions associated with the offgas radiation monitor operability, the note/ description clarifications and the REMP location updates will not impact dose calculations. Setpoint calculations for the radiation monitors are unchanged by the change to the required actions for the offgas radiation monitor. The accuracy and reliability of the effluent monitoring is also not negatively impacted. Both existing radiation monitors will be maintained. The alarm setpoints for the RM-A-4 and RM-A-6 gas monitors will not be impacted by the removal of their iodine channels via ECR 06-00061.

7. Does the ODCM change maintain the accuracy of radioactive effluent control required by the SAR? X YES ___ NO

   Explain: (provide sufficient information including appropriate analyses justifying the ODCM change)

   Chapter 11 of the UFSAR, specifically sections 11.2.4 and 11.4.3, were reviewed to determine that these ODCM changes will not impact the accuracy of radioactive effluent control required by the SAR. As stated before the revision to actions required for the offgas radiation monitor operability, the note/ description clarifications and REMP location changes are not changing the release paths of effluents to the environment or any ability to control or calculate doses via these pathways.

II. If all questions are answered YES, **then** complete the ODCM Change Determination and implement the Change per this procedure.

III. If any question is answered NO, **then** a change to the ODCM is not permitted

IV. Signoffs:

Determination Preparer: L.K. Weber (Printed Name) _____ (Signature) Date: 7/21/06

Reviewer: F. W. LINSENBACH (Printed Name) _____ (Signature) Date: 8/2/2006

AmerGen Energy Company, LLC
200 Exelon Way
Kennett Square, PA 19348

www.exeloncorp.com

AmerGen SM
An Exelon Company

May 25, 2006
5928-06-20449

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Three Mile Island Nuclear Station, Unit 1
Facility Operating License No. DPR-50
NRC Docket No. 50-289

Subject: Request to Revise Condenser Vent System Low Range Noble Gas Monitor Operability Requirement

References:

1. USNRC Atomic Safety and Licensing Board Order LBP-84-47, NRC Docket No. 50-289-OLA (ASLBP Docket No. 83-491-04-OLA), "Steam Generator Repair," dated October 31, 1984 (cited as 20 NRC 1405 (1984))
2. TMI Unit 1 letter to NRC (H. D. Hukill to J. F. Stolz), Condenser Off-Gas Monitor, dated November 19, 1984
3. TMI Unit 1 Technical Specification Amendment No. 103, Steam Generator Tube Repairs and Return To Operation, dated December 21, 1984
4. PWR Primary-to-Secondary Leak Guidelines – Revision 3, EPRI, Palo Alto, CA: 2004, EPRI Document No. 1008219
5. NRC "Notice of Opportunity to Comment on Model Safety Evaluation on Technical Specification Improvement to Modify Requirements Regarding The Addition of LCO 3.4.[17] on Steam Generator Tube Integrity Using the Consolidated Line Item Improvement Process," (Federal Register notice 70 FR 10298, dated March 2, 2005)
6. U.S.N.R.C. Letter, "Three Mile Island Nuclear Station, Unit 1 – Steam Generator Tube Kinetic Expansion Inspection and Repair Criteria (TAC MC7001)," P. S. Tam to C. M. Crane, November 8, 2005.
7. TMI Unit 1 Technical Specification Amendment No. 197, dated October 2, 1995

This letter is an AmerGen Energy Company, LLC (AmerGen) request for NRC approval of a proposed commitment revision regarding the operability requirement for the Condenser Vent System Low Range Noble Gas Monitors specified in the existing TMI Unit 1 Offsite Dose Calculation Manual (ODCM). The current ODCM operability requirement applicable to the Condenser Vent System Low Range Noble Gas Monitors fulfills a regulatory commitment in accordance with the NRC Atomic Safety and Licensing Board Order LBP-84-47, dated

October 31, 1984 (Reference 1), which authorized the return to service of the TMI Unit 1 steam generators with the kinetic expansion repair technique. In accordance with the AmerGen commitment management program, which implements the commitment change guidance specified in NEI 99-04, Rev. 0, "Guidelines for Managing NRC Commitment Changes," dated July 1999, the proposed commitment change modifies the method of compliance specifically defined in the Order (Reference 1), and therefore requires NRC prior approval.

The Condenser Vent System Low Range Noble Gas Activity Monitors provide data for determination of steam generator primary-to-secondary leakage rate. The existing TMI Unit 1 ODCM requires a minimum of two (2) Low Range Noble Gas Activity Monitors (RM-A5Lo and a suitable equivalent) to be operable when condenser vacuum is established, and allows continued operation with one (1) monitor for up to 28 days. TMI Unit 1 currently utilizes monitors RM-A5Lo and RM-A15. After 28 days, or if one operable monitor does not remain in service or is not placed in service within 1 hour, plant shutdown is required in accordance with TMI Unit 1 Technical Specification 3.0.1. The proposed commitment revision modifies the minimum channels required to be operable from two (2) channels to one (1) channel, and will allow continued plant operation for up to 14 days with both Condenser Vent System Low Range Noble Gas Activity Monitors inoperable, provided sampling and analysis actions are implemented. This proposed commitment change will eliminate outdated requirements, provide a primary-to-secondary leakrate monitoring regimen consistent with PWR industry standards and EPRI Guidelines as implemented by NEI 97-06, as well as prevent unnecessary plant transients if both monitors are temporarily out of service. Additionally, the proposed compensatory actions are consistent with other operating plant requirements for steam generator primary-to-secondary leakage monitoring.

Background and Requirements of NRC ASLB Order LBP-84-47

The current TMI Unit 1 Operating License Condition 2.c.(8), "Repaired Steam Generators," requires a plant shutdown if primary-to-secondary leak rate exceeds 0.1 gpm above baseline leakage. This license condition supported TMI Unit 1 restart with the kinetic expansion repairs on the OTSG tubes. The NRC Atomic Safety and Licensing Board (ASLB) Initial Decision, dated October 31, 1984, Section II.A, regarding proceedings related to the TMI-1 kinetic expansion repairs to the OTSG tubes and the ability to detect a primary-to-secondary leak rate of 0.1 gpm, as specified in the TMI-1 License Condition 2.c.(8), directed that redundancy be supplied in the form of a duplicate RM-A5 system or suitable equivalent of comparable sensitivity and response time. The ASLB further directed that the Technical Specifications (TS) be modified to permit plant operation for a maximum of 28 days with one of these duplicate systems inoperable, and to require plant shutdown if both of these systems are inoperable. As an alternative, the ASLB directed that the RM-A5 system must be operable at all times during plant operation.

The resulting NRC ASLB Order LBP-84-47 (Reference 1) authorized the USNRC Director of Nuclear Reactor Regulation to issue to TMI Unit 1 an operating license amendment that revised the Technical Specifications to recognize steam generator tube repair techniques other than plugging, specifically the kinetic expansion tube repair technique. This authorization was subject to conditions imposed by the Board in the Order, which specified: "A duplicate RM-A5 system or suitable equivalent of comparable sensitivity and response time for monitoring radioactive gas in the secondary system shall be installed. The Technical Specifications shall

be modified to permit plant operation for a maximum of 28 days with one of these duplicate systems inoperable, and to require plant shutdown if both of these systems are inoperable. As an alternative to the installation of a duplicate system, we direct that the RM-A5 system must be operable at all times during plant operation."

TMI Unit 1 letter to the NRC (Reference 2), dated November 19, 1984, clarified that TMI Unit 1 planned to provide a backup to RM-A5Lo using an existing portable monitor, RM-A13 having comparable sensitivity and response time to RM-A-5, and thus meeting the Order requirement of a suitable equivalent system which may be used for up to 28 days in the event that RM-A5Lo is inoperable. This letter further identified that TMI Unit 1 TS 3.21.2 would be changed to address the Board Order to implement the TS 28-day allowed outage time for RM-A5 and the plant shutdown action statement if both systems are inoperable.

The NRC issued TMI Unit 1 TS Amendment No. 103, dated December 21, 1984 (Reference 3), which permitted the return of the steam generators to operation in response to TMI Unit 1 amendment request of May 9, 1983, and in accordance with the ASLB Order LBP-84-47, dated October 31, 1984 (Reference 1). The NRC amendment specifically states that the revised TS included in this amendment incorporate the conditions imposed by the Board (identified above), and that the revised TS included in the amendment conformed with the condition imposed by the Board. The amendment also accepted the proposed alternative suitable equivalent system (portable monitor RM-A13) to RM-A5Lo, described above. The TS 3.21.2 requirement specified an allowed radiation monitor outage time of 28 days and specified a plant shutdown action statement in accordance with TS 3.0.1. TS 3.21.2 Bases specifically identified that the RM-A5 and suitable equivalent monitors provide data for determination of steam generator primary-to-secondary leakage rate, and that the channel operability requirements were based on ASLB Order dated October 31, 1984. The use of portable monitor RM-A13 was subsequently changed to permanently installed monitor RM-A-15, which continues to meet the suitable equivalent criteria to this day.

These TS 3.21.2 requirements were subsequently relocated to the ODCM in TMI Unit 1 TS Amendment No. 197, dated October 2, 1995 (Reference 7), which administratively relocated the TS Radiological Effluent Monitoring requirements to the ODCM in accordance with the guidance contained in NRC Generic Letter 89-01 and NUREG-1430. The operability requirements and action statements applicable to RM-A5 and suitable equivalent monitors, as currently specified in the TMI-1 ODCM, Table 2.1-2 and associated Bases, remained unchanged from the requirements added to the TMI Unit 1 TS 3.21.2 in Amendment No. 103 in accordance with Reference 1, described above.

Proposed Alternative

Markups of the revised TMI Unit 1 ODCM text are provided in Attachment 1 to illustrate implementation of the proposed commitment revision regarding the operability requirement for the Condenser Vent System Low Range Noble Gas Monitors. The following is a summary of the proposed ODCM text.

- ODCM Page 23, Bases – Revised to add reference to the NRC SER approving the proposed revisions to the channel operability requirements originally based on ASLB Order No. LBP-84-47.

- ODCM Page 25, Table 2.1-2 – Revises Minimum Channels Operable requirement from two (2) channels to one (1) channel.

- ODCM Page 29, Table 2.1-2, Action 32 – Revises the Action Statement to allow continued plant operation for up to 14 days with both Condenser Vent System Low Range Noble Gas Activity Monitors inoperable, provided grab samples are taken and analyzed in accordance with the proposed specified minimum frequency requirements of Table 1 of Action 32 in order to quantify primary-to-secondary leakrate. After 14 days, if one operable channel is not returned to service, within 1 hour the provisions of Technical Specification 3.0.1 apply, which would require that the plant be in hot standby within the next 6 hours, hot shutdown within the following 6 hours, and cold shutdown within the subsequent 24 hours. Action 32 is also revised to state that any inoperable channel(s) should be restored to operability as rapidly as practical.

Reason for Change Request

This proposed commitment change will eliminate outdated requirements, provide a primary-to-secondary leakrate monitoring regimen consistent with PWR industry standards and EPRI Guidelines as implemented by NEI 97-06, as well as prevent unnecessary plant transients if both monitors are temporarily out of service. Additionally, the proposed compensatory actions are consistent with other operating plant requirements for steam generator primary-to-secondary leakage monitoring.

Justification of Proposed Change

In accordance with ASLB Order LBP-84-47 (Reference 1), the TMI Unit 1 Offsite Dose Calculation Manual (ODCM), Table 2.1-2, "Radioactive Gaseous Process and Effluent Monitoring Instrumentation," Item 4.a, currently requires a minimum of two operable channels on the Condenser Vent System Low Range Noble Gas Activity Monitor. If one channel becomes inoperable, the ODCM allows 28 days for the inoperable channel to be restored. However, if no operable channels are in service, after one hour the provisions of Technical Specification 3.0.1 apply. TMI Unit 1 Technical Specification 3.0.1 states that in the event a Limiting Condition for Operation (LCO) is not met, within one hour action shall be initiated to place the unit in at least HOT STANDBY within the next 6 hours, and at least HOT SHUTDOWN within the following 6 hours, and at least COLD SHUTDOWN within the subsequent 24 hours. In summary, if both Condenser Vent System Low Range Noble Gas Activity Monitors are inoperable, the plant must begin to shut down within 1 hour.

The primary purpose of the TMI Unit 1 Condenser Vent System Low Range Noble Gas Activity Monitors (RM-A-5 and RM-A-15) is to detect radioactive gases in the secondary system. The amount of gas detected by the monitors is used to evaluate the plant's steam generator tube leakage (i.e., to detect primary-to-secondary leakage and to quantify the primary-to-secondary leakrate).

TMI Unit 1 plans to continue to use the two low range channels (RM-A5 and RM-A15) in accordance with the ODCM, and to revise the minimum channels operable requirement from two (2) channels to one (1) channel. The proposed action statement also specifies that inoperable Condenser Vent System Low Range Noble Gas Activity Monitor channels should be restored to operability as soon as practical. TMI Unit 1 also proposes to modify the commitment

implementation to require that if no operable Condenser Vent System Low Range Noble Gas Activity Monitor channel is in service, the plant may remain in service for up to 14 days if the specified sampling and analysis actions are implemented. The intent of this commitment change is to eliminate outdated requirements, provide a primary-to-secondary leakrate monitoring regimen consistent with PWR industry standards and EPRI Guidelines as implemented by NEI 97-06, as well as prevent unnecessary plant transients if both radiation monitors RM-A5 and RM-A15 are temporarily out of service. Additionally, the proposed compensatory actions are consistent with other operating plant requirements for steam generator primary-to-secondary leakage monitoring. It is expected that the revised commitment requirements will be infrequently used, as the operating history of the RM-A-5 and -15 monitors since the 1980's has shown that the probability of their simultaneous failure is low. Additionally, the sample lines to RM-A5 and RM-A15 have been modified to eliminate single point vulnerabilities that resulted in both detectors being declared inoperable on March 10, 2006.

TMI Unit 1 proposes to implement the proposed commitment change by modifying the ODCM so that a sampling regimen consistent with the EPRI *PWR Primary to Secondary Leak Guidelines* (Reference 4) is implemented if no Condenser Vent System Low Range Noble Gas Activity monitor is available. The current revision of the EPRI Guidelines provides the suggested sampling regimen in Section 3.2.2, entitled "No Available Continuous Radiation Monitor." The following is a summary of the proposed TMI Unit 1 ODCM text consistent with the above EPRI Guidelines:

Entry Requirement: There are no operable channels on the Condenser Vent System Low Range Noble Gas Activity Monitor.

Required Actions:

1. If there is no operable channel of the Condenser Vent System Low Range Noble Gas Activity Monitor for a period of 14 days, within one hour action shall be initiated to place the unit in at least HOT STANDBY within the next 6 hours, and at least HOT SHUTDOWN within the following 6 hours, and at least COLD SHUTDOWN within the subsequent 24 hours.

2. If the primary-to-secondary leakrate was unstable* or was indicating an increasing trend at the initial time when there was no operable channel of the Condenser Vent System Low Range Noble Gas Activity Monitor, analyze grab samples of the reactor coolant system and Condenser OffGas once every 4 hours to provide an indication of primary-to-secondary leakage, and subsequent sample frequency shall be in accordance with Table 1 based on the last sample result. Otherwise, analyze grab samples of the reactor coolant system and Condenser OffGas to provide an indication of primary-to-secondary leakage at the minimum frequency indicated in Table 1, below:

Table 1
Minimum Frequency of Grab Samples When
No Condenser Vent System Low Range Noble Gas Activity Monitor is Operable

Existing Total Primary-to-Secondary Leak Rate (based on last monitor reading or sample result)	**Frequency of Grab Samples**
0 to < 5 GPD	Once per 24 hours
5 to < 30 GPD	Once per 12 hours
30 to < 75 GPD	Once per 4 hours
75 GPD or greater	Place the unit in at least HOT STANDBY within the next 6 hours, and at least HOT SHUTDOWN within the following 6 hours, and at least COLD SHUTDOWN within the subsequent 24 hours.

*unstable is defined as > 10% increase during a 1 hour period, as stated in the EPRI Guidelines.

3. Return a channel of the Condenser Vent System Low Range Noble Gas Activity Monitor to operability as rapidly as practical.

As described above, the proposed TMI Unit 1 sampling regimen is as prescribed in the most recent revision of the EPRI *PWR Primary to Secondary Leak Guidelines*. These guidelines are industry standards and are based on considerable operating experience at Pressurized Water Reactors (PWRs) to ensure that the likelihood of propagation of steam generator tube flaws to tube rupture is minimized. The Guidelines are used by numerous plants and have been cited in the NRC's recent model safety evaluation for the Technical Specification improvements under TSTF-449, Rev. 4 for steam generator tube integrity (Reference 5). The EPRI Guidelines' discussion of the use of grab sampling when no radiation monitors are available for on-line quantification of primary-to-secondary leakage is also discussed in the *NRC Inspection Manual* for "Steam Generator Tube Primary-to-Secondary Leakage." The proposed sampling and analysis actions will be performed using existing plant equipment and procedures. No plant modifications are needed to implement the specified actions.

The TMI Unit 1 steam generator upper tubesheet expansions were repaired by a kinetic expansion process in the 1980's. At the time of these ASLB hearings, the PWR industry had limited operating experience with steam generator tube expansion repairs. At the time the repairs were "a new, large-scale application of the kinetic expansion process" and there was "no directly relevant experience" in the PWR industry (Reference 1, Page 1416). The additional radiation monitor channel, and the requirement for plant shutdown after one hour in the event that no radiation monitor is available to monitor the plant's Condenser Vent, were additional conservative measures with which to assess the success of the kinetic expansion repairs.

Since the early 1980's the PWR industry has obtained considerable experience with tube expansion repairs, including several different types of expansion repair methods. In addition, the TMI Unit 1 kinetic expansions have proven to be successful, leak-tight, and flaw-tolerant repairs. The plant has operated approximately 17 Effective Full Power Years (EFPY) since the kinetic expansions were installed. Since 1997, the maximum primary-to-secondary leakage experienced through the TMI Unit 1 steam generator tubes, including their kinetic expansion joints, has typically been less than 1 gallon per day (GPD).

In addition to the fact that the kinetic expansions were an effective repair, a significant number of kinetic expansion inspections have been performed to monitor their condition. Each (i.e., 100%) of the plant's inservice kinetic expansions was examined with rotating coil eddy current probes during the plant's last outage in the fall of 2005. These examinations were conducted in accordance with the requirements of AmerGen ECR #02-01121, which was approved by the NRC in Reference 6. More than 60,000 examinations of the plant's kinetic expansions have been performed over the last four refueling outages. These examinations and the analyses of the examination results have determined that significant active degradation in the kinetic expansions' required lengths is not occurring. In addition, the kinetic expansions are relatively flaw-tolerant since: (1) the expansion area is captured within the steam generator upper tubesheets, and (2) the expansions are not subjected to bending loads or potential loose part impact.

If the RM-A-5 and –15 radiation monitors are out of service, the proposed sampling regimen will allow evaluation of low levels of primary-to-secondary leakage. Other methods are available to monitor the plant for sizeable primary-to-secondary leaks, including MakeUp Tank level changes, Main Steam Line Radiation Monitors RM-G-26 and –27, Reactor Coolant System Pressurizer level changes, and Condenser Exhaust Hi-Range Radiation Monitor RM-A-5Hi.

There is no estimated risk increase under the plant's risk model associated with the loss of RM-A-5 and RM-A-15 operability, as these monitors are not included as part of any assumptions made in the PRA model regarding Steam Generator Tube Rupture (SGTR) or RCS Leak Rate detection. Steam generator tube ruptures are considered to be the full offset rupture of one tube. For leaks of this size it is assumed that RM-G26 and RM-G27 as well as RCS Pressure and OTSG Level and Feed Rates provide the necessary SGTR detection needed to satisfy the assumptions made by the PRA model. It should be noted that RM-G26 and RM-G27 are capable of some detection of OTSG Tube leakage smaller than what is assumed during a full offset rupture. In addition, particularly small break sizes or leaks that are within the makeup capacity of the normal charging system and would therefore not result in an automatic reactor trip, are not included in the initiating event categories of the plant's risk models. For such small leaks, the plant would be manually shutdown in a controlled fashion, and cooled down and depressurized for repairs, regardless of the leak/break location in the steam generator tubes.

The proposed commitment change provides a limit on the maximum length of time (i.e., 14 days) during which the plant can remain at power without an operable Condenser Vent System Low Range Noble Gas Activity monitor channel. The probability of a significant primary-to-secondary leak event occurring during this short duration is low. TMI Unit 1 has not had a significant primary-to-secondary leak event due to leakage from a kinetic expansion.

Conclusion

The proposed commitment change makes no physical changes to the TMI-1 plant and does not alter the TMI Unit 1 Technical Specification or License Condition maximum allowed primary-to-secondary leak rates at which the plant is required to be shutdown. The intent of the proposed commitment change is to eliminate outdated requirements, provide a primary-to-secondary leakrate monitoring regimen consistent with PWR industry standards and EPRI Guidelines as implemented by NEI 97-06, as well as prevent unnecessary plant transients if both monitors RM-A5 and RM-A15 are temporarily out of service. Additionally, the proposed compensatory actions are consistent with other operating plant requirements for steam generator primary-to-secondary leakage monitoring, and following the methodology of the EPRI PWR Guidelines is an acceptable, alternate method of monitoring the plant's primary-to-secondary leakage. The probability of both RM-A-5 and –15 channels becoming inoperable is low. The probability of a primary-to-secondary leak event occurring while both RM-A-5 and –15 channels are inoperable is also low. Thus, nuclear safety and plant operations are not adversely affected by the proposed commitment change.

We request approval of the proposed commitment change by August 31, 2006, to allow timely update of the affected ODCM requirements.

This submittal requests approval of a revision to an existing regulatory commitment, and no new regulatory commitments are established by this submittal. If you have any questions or require additional information, please contact David J. Distel at (610) 765-5517.

Respectfully,

9 6 K

Pamela B. Cowan
Director – Licensing and Regulatory Affairs
AmerGen Energy Company, LLC

Attachment: 1) TMI Unit 1 ODCM Proposed Markup Revisions

cc: S. J. Collins, Administrator, USNRC Region I
D. M. Kern, USNRC Senior Resident Inspector, TMI Unit 1
F. E. Saba, USNRC Project Manager, TMI Unit 1
D. Allard, Director, Bureau of Radiation Protection – PA Department of Environmental Resources
Chairman, Board of County Commissioners of Dauphin County
Chairman, Board of Supervisors of Londonderry Township
File No. 06025

Attachment 1

TMI Unit 1 ODCM
Proposed Markup Revisions

Revised ODCM Pages

23
25
29

Table 2.1-2

2. If the inoperable gas channel(s) is not restored to service within 14 days, a special report shall be submitted to the Regional Administrator of the NRC Region I Office and a copy to the Director, Office of Inspection and Enforcement within 30 days of declaring the channel(s) inoperable. The report shall describe (a) the cause of the monitor inoperability, (b) action being taken to restore the instrument to service, and (c) action to be taken to prevent recurrence.

ACTION 31 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that within four hours after the channel has been declared inoperable, samples are continuously collected with auxiliary sampling equipment.

ACTION 32 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 14 days, provided that ~~one OPERABLE channel remains in service or is placed in service within 1 hour.~~ **After 14 days, if one OPERABLE channel ~~does not remain in service or~~ is not ~~placed in~~ service within 1 hour, the provisions of Technical Specification 3.0.1 apply, as if this Control were a Tech Spec Limiting Condition for Operation.**

ACTION 33 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel to OPERABLE status within 7 days, or prepare and submit a special report within 30 days outlining the action(s) taken, the cause of the inoperability, and plans and schedule for restoring the system to OPERABLE status.

Condenser Vent System Low Range Noble Gas Activity Monitor inoperable channels should be restored to operability as rapidly as practical.

grab samples are taken and analyzed.
INSERT FROM NEXT PAGE

INSERT TO TMI Unit 1 ODCM PAGE 29

If the primary-to-secondary leakrate was unstable* or was indicating an increasing trend at the initial time when there was no operable channel of the Condenser Vent System Low Range Noble Gas Activity Monitor, analyze grab samples of the reactor coolant system and Condenser OffGas once every 4 hours to provide an indication of primary-to-secondary leakage, and subsequent sample frequency shall be in accordance with Table 1 based on the last sample result. Otherwise, analyze grab samples of the reactor coolant system and Condenser OffGas to provide an indication of primary-to-secondary leakage at the minimum frequency indicated in Table 1, below:

Table 1

Minimum Frequency of Grab Samples When
No Condenser Vent System Low Range Noble Gas Activity Monitor is Operable

Existing Total Primary-to-Secondary Leak Rate (based on last monitor reading or sample result)	**Frequency of Grab Samples**
0 to < 5 GPD	Once per 24 hours
5 to < 30 GPD	Once per 12 hours
30 to < 75 GPD	Once per 4 hours
75 GPD or greater	Place the unit in at least HOT STANDBY within the next 6 hours, and at least HOT SHUTDOWN within the following 6 hours, and at least COLD SHUTDOWN within the subsequent 24 hours.

*unstable is defined as > 10% increase during a 1 hour period, as stated in the EPRI Guidelines.

2.1.2 Radioactive Gaseous Process and Effluent Monitoring Instrumentation

CONTROL:

The radioactive gaseous process and effluent monitoring instrumentation channels shown in Table 2.1-2 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Control 2.2.2.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: As shown in Table 2.1-2.

ACTION:

a. With a radioactive gaseous process or effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above control, immediately suspend the release of radioactive effluent monitored by the affected channel or declare the channel inoperable.

b. With less than the minimum number of radioactive gaseous process or effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 2.1-2. Exert best efforts to return the instrumentation to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Release Report why the inoperability was not corrected in a timely manner.

BASES

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluent during actual or potential releases. The alarm/trip setpoints for these instruments shall be calculated in accordance with NRC approved methods in the ODCM to provide reasonable assurance that the annual releases are within the limits specified in 10 CFR 20.1301.

The low range condenser offgas noble gas activity monitors also provide data for determination of steam generator primary to secondary leakage rate. Channel operability requirements are based on an ASLB Order No. LBP-84-47 dated October 31, 1984, and as cited in 20 NRC 1405 (1984), as revised by NRC SER dated ____.

Table 2.1-2 (Cont'd)

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
4.	Condenser Vent System			
	a. Low Range Noble Gas Activity Monitor (RM-A5Lo and Suitable Equivalent)	1 (1)	##	32

NOTE (1): For one of the channels, an operable channel may be defined for purposes of this control and 3.1.2.1 only as a suitable equivalent monitoring system capable of being placed in service within one hour. A suitable equivalent system shall include instrumentation with comparable sensitivity and response time to the RM-A5Lo monitoring channel. When the equivalent monitoring system is in service, indication will be continuously available to the operator, either through indication and alarm in the Control Room or through communication with a designated individual continuously observing local indication.

AmerGen	TMI - Unit 1 Radiological Controls Procedure	Number 6610-PLN-4200.01
Title **Offsite Dose Calculation Manual (ODCM)**		Revision No. ~~24~~ 25
Applicability/Scope TMI Division	**USAGE LEVEL** 3	Effective Date 07/01/03

This document is within QA plan scope	X	Yes		No
50.59 Applicable	X	Yes		No

List of Effective Pages

Page	Revision	Page	Revision	Page	Revision	Page	Revision
1	24	41	24	81	24	121	24
2	24	42	24	82	24	122	24
3	24	43	24	83	24	123	24
4	24	44	24	84	24	124	24
5	24	45	24	85	24	125	24
6	24	46	24	86	24	126	24
7	24	47	24	87	24	127	24
8	24	48	24	88	24	128	24
9	24	49	24	89	24	129	24
10	24	50	24	90	24	130	24
11	24	51	24	91	24	131	24
12	24	52	24	92	24	132	24
13	24	53	24	93	24	133	24
14	24	54	24	94	24	134	24
15	24	55	24	95	24	135	24
16	24	56	24	96	24	136	24
17	24	57	24	97	24	137	24
18	24	58	24	98	24	138	24
19	24	59	24	99	24	139	24
20	24	60	24	100	24	140	24
21	24	61	24	101	24	141	24
22	24	62	24	102	24	142	24
23	~~24~~ 25	63	24	103	24	143	24
24	24	64	24	104	24	144	24
25	~~24~~ 25	65	24	105	24	145	24
26	~~24~~ 25	66	24	106	24	146	24
27	24	67	24	107	24	147	24
~~28~~	~~24~~ 25	68	24	108	24	148	24
29	~~24~~ 25	69	24	109	24	149	24
30	24	70	24	110	24	150	24
31	24	71	24	111	24	151	24
32	24	72	24	112	~~24~~ 25	152	24
33	24	73	24	113	24	153	24
34	24	74	24	114	24	154	24
35	24	75	24	115	24	155	24
36	24	76	24	116	24	156	24
37	24	77	24	117	24	157	24
38	24	78	24	118	24	158	24
39	24	79	24	119	24	159	24
40	24	80	24	120	24	160	24

List of Effective Pages

Page	Revision	Page	Revision	Page	Revision	Page	Revision
161	24						
162	24						
163	24						
164	24						
165	24						
166	24						
167	24						
168	24						
169	24						
170	24						
171	~~24~~ 25						
172	~~24~~ 25						
173	~~24~~ 25						
174	~~24~~ 25						
175	~~24~~ 25						
176	~~24~~ 25						
177	~~24~~ 25						
178	~~24~~ 25						
179	24						
180	24						
181	24						
182	24						
183	24						
184	24						
185	24						
186	24						
187	24						
188	24						
189	24						
190	24						
191	24						
192	24						
193	24						
194	24						
195	24						
196	24						

2.1.2 Radioactive Gaseous Process and Effluent Monitoring Instrumentation

CONTROL:

The radioactive gaseous process and effluent monitoring instrumentation channels shown in Table 2.1-2 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Control 2.2.2.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: As shown in Table 2.1-2.

ACTION:

a. With a radioactive gaseous process or effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above control, immediately suspend the release of radioactive effluent monitored by the affected channel or declare the channel inoperable.

b. With less than the minimum number of radioactive gaseous process or effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 2.1-2. Exert best efforts to return the instrumentation to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Release Report why the inoperability was not corrected in a timely manner.

BASES

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluent during actual or potential releases. The alarm/trip setpoints for these instruments shall be calculated in accordance with NRC approved methods in the ODCM to provide reasonable assurance that the annual releases are within the limits specified in 10 CFR 20.1301.

The low range condenser offgas noble gas activity monitors also provide data for determination of steam generator primary to secondary leakage rate. Channel operability requirements are based ~~on an ASLB Order No. LBP-84-47 dated October 31, 1984, and as cited in 20 NRC 1405 (1984).~~ on AmerGen Letter #5928-06-20449 "Request to Revise Condenser Vent System Low Range Noble Gas Monitor Operability Requirement", Pamela B. Cowan to U.S.N.R.C., May 5, 2006."

Table 2.1-2 (Cont'd)

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
4.	Condenser Vent System			
	a. Low Range Noble Gas Activity Monitor (RM-A5Lo ~~and Suitable Equivalent~~ or RM-A-15)	1	##	32

NOTE (1): For one of the channels, an operable channel may be defined for purposes of this control and 3.1.2.1 only as a suitable equivalent monitoring system capable of being placed in service within one hour. A suitable equivalent system shall include instrumentation with comparable sensitivity and response time to the RM-A5Lo monitoring channel. When the equivalent monitoring system is in service, indication will be continuously available to the operator, either through indication and alarm in the Control Room or through communication with a designated individual continuously observing local indication.

Table 2.1-2 (Cont'd)

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
5.	Auxiliary and Fuel Handling Building Ventilation System			
a.	Noble Gas Activity Monitor (RM-A8) or (RM-A4 and RM-A6)	1	*	27
b.	Iodine Samples (RM-A8) or (RM-A4 and RM-A6)	1	*	31
c.	Particulate Sampler (RM-A8) or (RM-A4 and RM-A6)	1	*	31
d.	Effluent System Flow Rate Measuring Devices (FR-149 and FR-150)	1	*	26
e.	Sampler Flow Rate Monitor	1	*	26
6.	Fuel Handling Building ESF Air Treatment System			
a.	Noble Gas Activity Monitor (RM-A14 or Suitable Equivalent)	1	****	27, 33
b.	Iodine Cartridge	N/A(2)	****	31, 33
c.	Particulate Filter	N/A(2)	****	31, 33
d.	Effluent System Flow (UR-1104A/B)	1	****	26, 33
e.	Sampler Flow Rate Monitor	1	****	26, 33

NOTE 2: No instrumentation channel is provided. However, for determining operability, the equipment named must be installed and functional or the ACTION applies.

Table 2.1-2

Table Notation

* At all times.
** During waste gas holdup system operation.
*** Operability is not required when discharges are positively controlled through the closure of WDG-V47 ~~and where RM-A8 (or RM-A4 and RM-A6), FT-149, and FT-150 are operable~~. or where RM-A8, FT-149 and FT-150 are operable and RM-A8 is capable of automatic closure of WDG-V47.
**** During Fuel Handling Building ESF Air Treatment System Operation.
\# At all times during containment purging.
At all times when condenser vacuum is established.
During operation of the ventilation system.

ACTION 25 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank may be released to the environment provided that prior to initiating the release:

1. At least two independent samples of the tank's contents are analyzed in accordance with Table 3.2-2, Item A, and

2. At least two technically qualified members of the Unit staff independently verify the release rate calculations and verify the discharge valve lineup.

3. The TMI Plant Manager shall approve each release.

Otherwise, suspend release of radioactive effluent via this pathway.

ACTION 26 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided the flow rate is estimated at least once per 4 hours.

ACTION 27 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided grab samples are taken at least once per 12 hours and the initial samples are analyzed for gross activity (gamma scan) within 24 hours after the channel has been declared inoperable. If RM-A9 is declared inoperable, see also Technical Specification 3.5.1, Table 3-5.1, Item C.3.f.

ACTION 30 1. With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, a grab sample shall be collected and analyzed for the inoperable gas channel(s) at least once per 24 hours. With both channels inoperable, a grab sample shall be collected and analyzed for the inoperable gas channel(s):

(a) at least once per 4 hours during degassing operations.

(b) at least once per 24 hours during other operations (e.g. Feed and Bleed).

Table 2.1-2

2. If the inoperable gas channel(s) is not restored to service within 14 days, a special report shall be submitted to the Regional Administrator of the NRC Region I Office and a copy to the Director, Office of Inspection and Enforcement within 30 days of declaring the channel(s) inoperable. The report shall describe (a) the cause of the monitor inoperability, (b) action being taken to restore the instrument to service, and (c) action to be taken to prevent recurrence.

ACTION 31 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that within four hours after the channel has been declared inoperable, samples are continuously collected with auxiliary sampling equipment.

ACTION 32 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 14 days, provided that ~~one OPERABLE channel remains in service or is placed in service within 1 hour.~~ **After 14 days, if one OPERABLE channel ~~does not remain in service or~~ is not ~~placed in~~ service within 1 hour the provisions of Technical Specification 3.0.1 apply, as if this Control were a Tech Spec Limiting Condition for Operation.**

ACTION 33 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel to OPERABLE status within 7 days, or prepare and submit a special report within 30 days outlining the action(s) taken, the cause of the inoperability, and plans and schedule for restoring the system to OPERABLE status.

Condenser Vent System Low Range Noble Gas Activity Monitor inoperable channels should be restored to operability as rapidly as practical.

grab samples are taken and analyzed.
INSERT FROM NEXT PAGE

6610-PLN-4200.01
PC - 2098

INSERT TO TMI Unit 1 ODCM PAGE 29

If the primary-to-secondary leakrate was unstable* or was indicating an increasing trend at the initial time when there was no operable channel of the Condenser Vent System Low Range Noble Gas Activity Monitor, analyze grab samples of the reactor coolant system and Condenser OffGas once every 4 hours to provide an indication of primary-to-secondary leakage, and subsequent sample frequency shall be in accordance with Table 1 based on the last sample result. Otherwise, analyze grab samples of the reactor coolant system and Condenser OffGas to provide an indication of primary-to-secondary leakage at the minimum frequency indicated in Table 1, below:

Table 1
Minimum Frequency of Grab Samples When
No Condenser Vent System Low Range Noble Gas Activity Monitor is Operable

Existing Total Primary-to-Secondary Leak Rate (based on last monitor reading or sample result)	**Frequency of Grab Samples**
0 to < 5 GPD	Once per 24 hours
5 to < 30 GPD	Once per 12 hours
30 to < 75 GPD	Once per 4 hours
75 GPD or greater	Place the unit in at least HOT STANDBY within the next 6 hours, and at least HOT SHUTDOWN within the following 6 hours, and at least COLD SHUTDOWN within the subsequent 24 hours.

*unstable is defined as > 10% increase during a 1 hour period, as stated in the EPRI Guidelines.

4.4 TMI-1 Gaseous Effluent Release Points and Gaseous Radiation Monitor Data

TMI-1 has eleven (11) required effluent gaseous radiation monitors. These are RM-A4, RM-A5, RM-A15, RM-A6, RM-A7, RM-A8, RM-A9, RM-A14, ALC-RMI-18, WHP-RIT-1, and RLM-RM-1. These gaseous release points, radiation monitors, and sample points are shown in Table 4.1.

4.4.1 RM-A4/RM-A6 Fuel Handling and Auxiliary Building Exhaust

RM-A4 is the ~~particulate, radioiodine and gaseous~~ radiation monitor for the TMI-1 Fuel Handling Building Ventilation (see Figures 4.1 and 4.2). RM-A6 is the ~~particulate, radioiodine, and gaseous~~ radiation monitor for the TMI-1 Auxiliary Building Ventilation (see Figures 4.1 and 4.2). High alarms on RM-A4 or RM-A6 noble gas channels will initiate shutdown of the related building ventilation air supply system. These two radiation monitors concurrently will satisfy requirements for the Station Vent release point in place of RM-A8.

4.4.2 RM-A8 Station Ventilation Exhaust

RM-A8 is the particulate, radioiodine and gaseous radiation monitor for the TMI-1 Station Ventilation (see Figures 4.1 and 4.2). This in plant effluent radiation monitor also has an associated sampling panel with sampling lines located before the sample filters. High alarm on RM-A8 noble gas low channel will initiate shutdown of the Station Ventilation air supply systems. (The Fuel Handling and Auxiliary Building Ventilation). This radiation monitor satisfies requirements for the Station Vent release point in place of RM-A4 and RM-A6.

4.4.3 RM-A5/RM-A15 Condenser Off Gas Exhaust

RM-A5 is the gaseous radiation monitor for the TMI-1 Condenser Off Gas exhaust (see Figures 4.1 and 4.4). RM-A15 is the back up gaseous radiation monitor for the TMI-1 Condenser Off Gas exhaust (see Figures 4.1 and 4.4). High alarms on RM-A5 low channel or RM-A15 noble gas channels will initiate the MAP-5 Radioiodine Processor Station. These two radiation monitors together satisfy requirements for the Condenser Off Gas release point.

4.4.4 RM-A7 Waste Gas Decay Tank Exhaust

RM-A7 is the gaseous radiation monitor for the TMI-1 Waste Gas Decay tanks (see Figures 4.1 and 4.2). This in plant effluent radiation monitor also has an associated sampling panel. High alarm on RM-A7 noble gas channel will initiate shutdown of the Waste Gas Decay Tank release in progress. This radiation monitor satisfies requirements for batch gaseous releases to the Station Vent release point.

4.4.5 RM-A9 Reactor Building Purge Exhaust

RM-A9 is the particulate, radioiodine and gaseous radiation monitor for the TMI-1 Reactor Building Purge system (see Figures 4.1 and 4.3). This in plant effluent radiation monitor also has an associated sampling panel with sampling lines located before the sample filters. High alarm on RM-A9 noble gas low channel will initiate shutdown of the Reactor Building Purge System. This radiation monitor satisfies requirements for the Reactor Building Purge System release point.

TABLE 8.4

TMINS REMP Station Locations-Air Particulate and Air Iodine

Station Code	Distance (miles)	Azimuth (°)	Map No.
~~B1-4~~	~~0.8~~	~~28~~	~~60~~
E1-2	0.4	~~95~~ 97	2
F1-3	0.6	~~105~~ 112	70
G2-1	1.4	~~125~~ 126	74
M2-1	1.3	~~253~~ 256	3
A3-1	~~2.8~~ 2.7	~~358~~ 357	4
H3-1	~~2.3~~ 2.2	~~159~~ 160	5
~~Q4-1~~	~~3.5~~	~~325~~	~~61~~
Q15-1	~~13.5~~ 13.4	~~305~~ 309	8

TABLE 8.5

TMINS REMP Station Locations-Direct Radiation (TLD)

Station Code	Distance (miles)	Azimuth (°)	Map No.
A1-4	0.3	~~5~~ 6	9
B1-1	0.6	25	10
B1-2	0.4	~~20~~ 23	11
C1-2	0.3	~~54~~ 50	13
D1-1	0.2	~~74~~ 76	14
E1-2	0.4	~~95~~ 97	2
E1-4	0.2	~~98~~ 97	16
F1-2	0.2	~~109~~ 112	17
G1-3	~~0.3~~ 0.2	~~129~~ 130	18
H1-1	0.5	167	19
J1-1	0.8	~~184~~ 176	21
J1-3	0.3	189	22
K1-4	0.2	~~208~~ 209	24
L1-1	0.1	~~235~~ 236	26
M1-1	0.1	~~249~~ 250	27
N1-3	0.1	~~270~~ 274	28
P1-1	0.4	~~293~~ 303	29
P1-2	~~0.2~~ 0.1	~~290~~ 292	30

TABLE 8.5

TMINS REMP Station Locations-Direct Radiation (TLD)

Station Code	Distance (miles)	Azimuth (°)	Map No.
Q1-2	0.2	~~318~~ 321	31
R1-1	0.2	335	32
C2-1	~~1.6~~ 1.5	~~48~~ 41	33
K2-1	~~1.1~~ 1.2	200	34
M2-1	1.3	~~253~~ 256	3
A3-1	~~2.6~~ 2.7	~~358~~ 357	4
H3-1	~~2.3~~ 2.2	~~159~~ 160	5
R3-1	2.6	~~338~~ 341	35
B5-1	~~4.8~~ 4.9	~~18~~ 19	36
C5-1	~~4.5~~ 4.7	~~42~~ 43	37
E5-1	~~4.6~~ 4.7	~~81~~ 82	38
F5-1	4.7	~~107~~ 109	39
G5-1	4.8	131	40
H5-1	4.1	~~157~~ 158	41
J5-1	4.9	~~182~~ 181	42
K5-1	~~5.0~~ 4.9	~~200~~ 202	43
L5-1	4.1	228	44
M5-1	4.3	249	45
N5-1	~~4.9~~ 5.0	268	46
P5-1	~~4.9~~ 5.0	~~285~~ 284	47
Q5-1	5.0	~~318~~ 317	48
R5-1	4.9	339	49
D6-1	5.2	~~65~~ 66	50
E7-1	~~6.8~~ 6.7	~~86~~ 88	51
Q9-1	8.5	~~308~~ 310	52
B10-1	~~9.4~~ 9.2	21	53
G10-1	~~9.8~~ 9.7	~~127~~ 128	6
G15-1	14.4	~~124~~ 126	54
J15-1	12.6	~~180~~ 183	7
Q15-1	~~13.5~~ 13.4	~~305~~ 309	8

TABLE 8.6

TMINS REMP Station Locations-Surface Water

Station Code	Distance (miles)	Azimuth (°)	Map No.
J1-2 (R)	0.5	188	57
A3-2 (R)	~~2.5~~ 2.7	~~355~~ 356	59
Q9-1 (F)	8.5	~~308~~ 310	52
Q9-1 (R)	8.5	~~308~~ 310	52
G15-2 (F)	~~13.6~~ 13.3	~~128~~ 129	62
G15-3 (F)	~~14.8~~ 15.7	124	63
F15-1 (R)	12.6	122	65

(R) = Raw Water
(F) = Finished Water

TABLE 8.7

TMINS REMP Station Locations-Aquatic Sediment

Station Code	Distance (miles)	Azimuth (°)	Map No.
A1-3	0.5	~~0~~ 359	67
G1-1	0.3	137	68
K1-3	~~0.3~~ 0.2	~~202~~ 212	69
J2-1	~~1.5~~ 1.4	~~182~~ 179	58
J1-2	0.5	188	57

TABLE 8.8

TMINS REMP Station Locations-Milk

Station Code	Distance (miles)	Azimuth (°)	Map No.
D2-1	1.1	~~65~~ 62	72
E2-2	1.1	~~93~~ 96	73
G2-1	1.4	~~125~~ 126	74
P7-1	6.7	293	77
K15-3	~~14.5~~ 14.4	205	78
F4-1	3.2	104	61

TABLE 8.9

TMINS REMP Station Locations-Fish

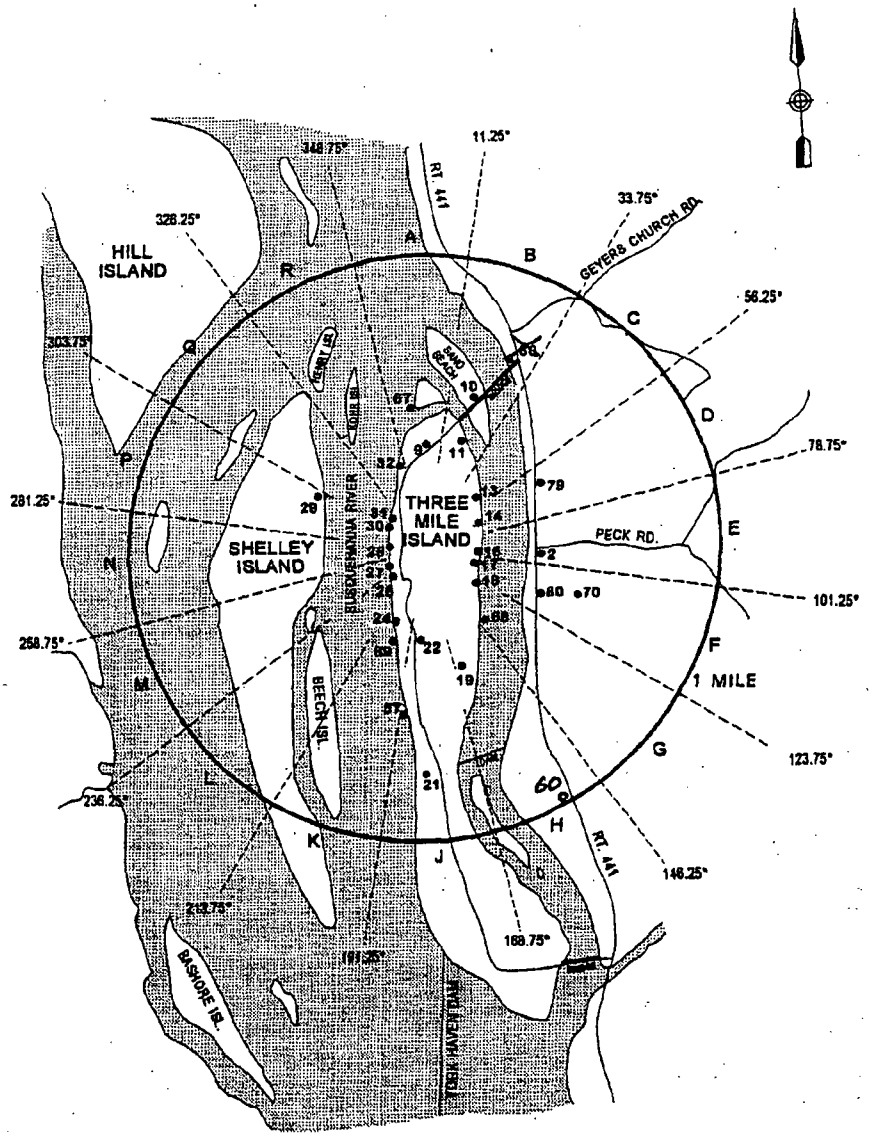
Station Code	Station Location
IND	Downstream of Station Discharge
BKG	Upstream of Station Discharge

TABLE 8.10

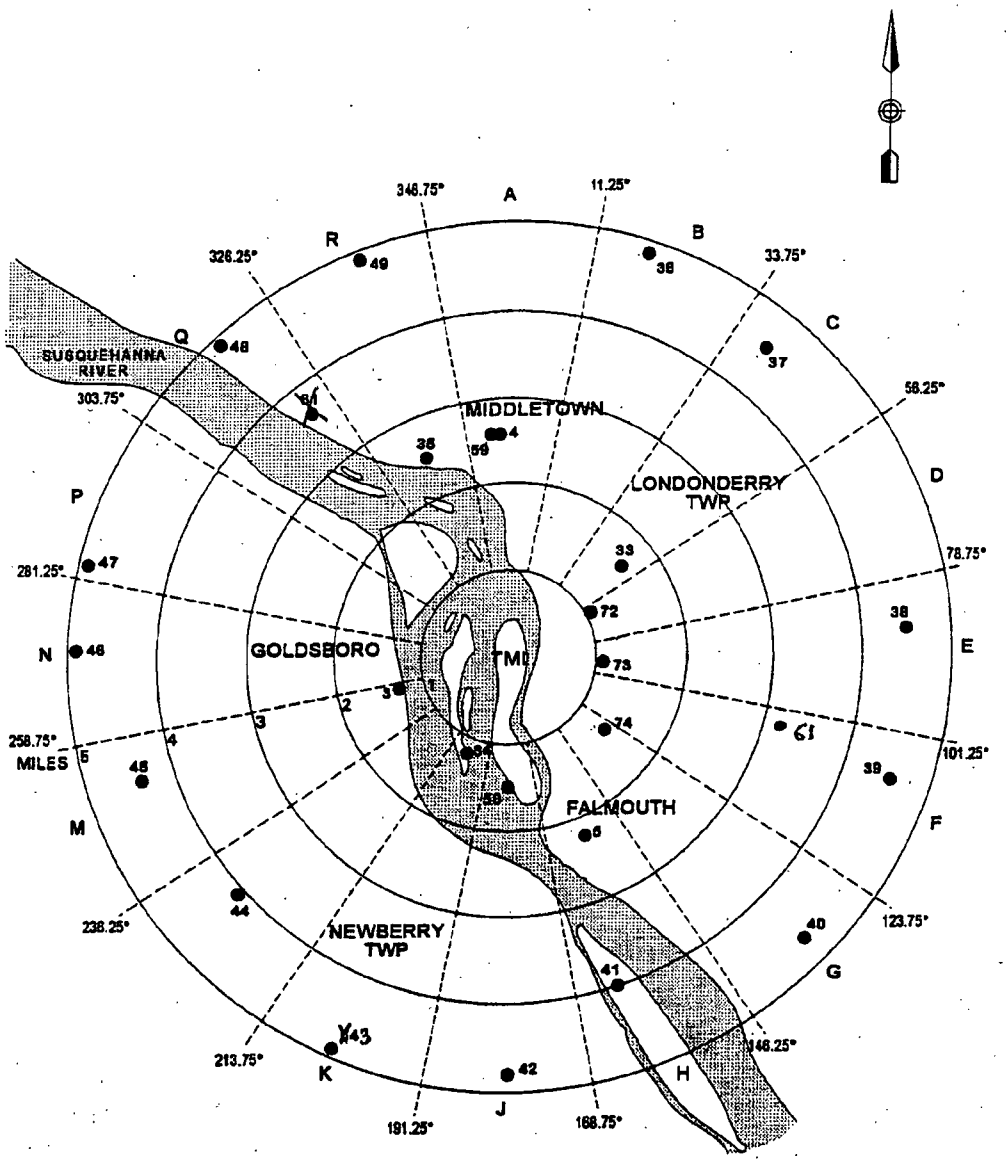
TMINS REMP Station Locations-Food Products

Station Code	Distance (miles)	Azimuth (°)	Map No.
D1-3	0.5	65	79
E1-2	0.4	~~85~~ 97	2
F1-1	0.5	117	80
~~A15-1~~	~~10.5~~	~~10~~	~~85~~
B10-2	~~10.1~~ 10.0	~~28~~ 31	55
H1-2	1.0	151	60

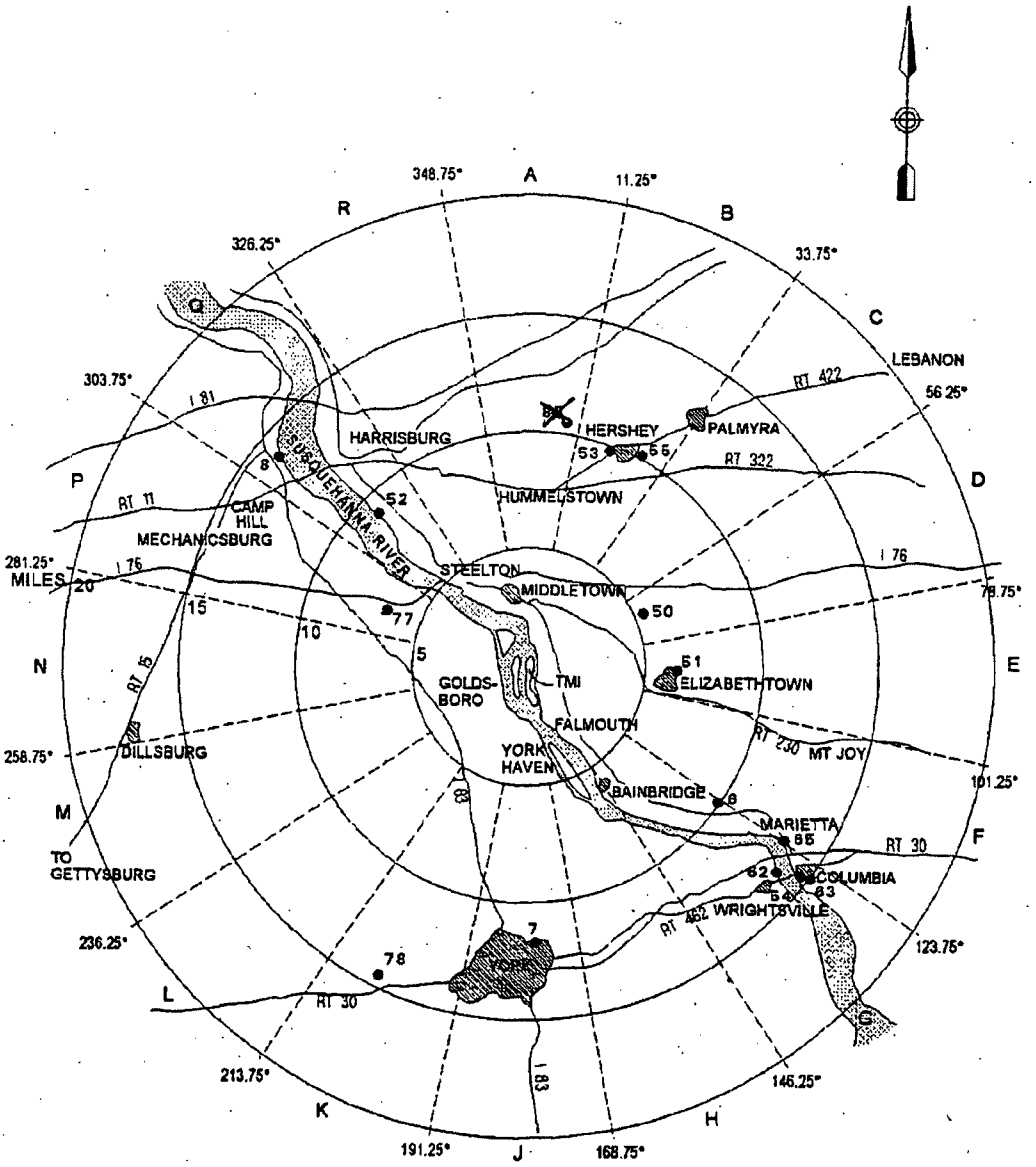
MAP 8.1
THREE MILE ISLAND NUCLEAR STATION
LOCATIONS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
STATIONS WITHIN 1 MILE OF THE SITE



MAP 8.2
THREE MILE ISLAND NUCLEAR STATION
LOCATIONS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
STATIONS WITHIN 5 MILES OF THE SITE



MAP 8.3
THREE MILE ISLAND NUCLEAR STATION
LOCATIONS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
STATIONS GREATER THAN 5 MILES FROM THE SITE



AmerGen	TMI - Unit 1 Radiological Controls Procedure	Number 6610-PLN-4200.01
Title **Offsite Dose Calculation Manual (ODCM)**		Revision No. 25
Applicability/Scope TMI Division	USAGE LEVEL 3	Effective Date 08/09/06

This document is within QA plan scope [X] Yes [ ] No
50.59 Applicable [X] Yes [ ] No

List of Effective Pages

Page	Revision	Page	Revision	Page	Revision	Page	Revision
1	25	41	25	81	25	121	25
2	25	42	25	82	25	122	25
3	25	43	25	83	25	123	25
4	25	44	25	84	25	124	25
5	25	45	25	85	25	125	25
6	25	46	25	86	25	126	25
7	25	47	25	87	25	127	25
8	25	48	25	88	25	128	25
9	25	49	25	89	25	129	25
10	25	50	25	90	25	130	25
11	25	51	25	91	25	131	25
12	25	52	25	92	25	132	25
13	25	53	25	93	25	133	25
14	25	54	25	94	25	134	25
15	25	55	25	95	25	135	25
16	25	56	25	96	25	136	25
17	25	57	25	97	25	137	25
18	25	58	25	98	25	138	25
19	25	59	25	99	25	139	25
20	25	60	25	100	25	140	25
21	25	61	25	101	25	141	25
22	25	62	25	102	25	142	25
23	25	63	25	103	25	143	25
24	25	64	25	104	25	144	25
25	25	65	25	105	25	145	25
26	25	66	25	106	25	146	25
27	25	67	25	107	25	147	25
28	25	68	25	108	25	148	25
29	25	69	25	109	25	149	25
30	25	70	25	110	25	150	25
31	25	71	25	111	25	151	25
32	25	72	25	112	25	152	25
33	25	73	25	113	25	153	25
34	25	74	25	114	25	154	25
35	25	75	25	115	25	155	25
36	25	76	25	116	25	156	25
37	25	77	25	117	25	157	25
38	25	78	25	118	25	158	25
39	25	79	25	119	25	159	25
40	25	80	25	120	25	160	25

List of Effective Pages

Page	Revision	Page	Revision	Page	Revision	Page	Revision
161	25						
162	25						
163	25						
164	25						
165	25						
166	25						
167	25						
168	25						
169	25						
170	25						
171	25						
172	25						
173	25						
174	25						
175	25						
176	25						
177	25						
178	25						
179	25						
180	25						
181	25						
182	25						
183	25						
184	25						
185	25						
186	25						
187	25						
188	25						
189	25						
190	25						
191	25						
192	25						
193	25						
194	25						
195	25						
196	25						

INTRODUCTION

The OFFSITE DOSE CALCULATION MANUAL (ODCM) is a supporting document of the Three Mile Island Nuclear Station (TMI) Unit 1 and Unit 2 PDMS Technical Specifications and implements TMI radiological effluent controls. The ODCM contains the controls, bases, and surveillance requirements for liquid and gaseous radiological effluents. In addition, the ODCM describes the methodology and parameters to be used in the calculation of off-site doses due to radioactive liquid and gaseous effluents. This document also describes the methodology used for calculation of the liquid and gaseous effluent monitoring instrumentation alarm/trip set points. Liquid and Gaseous Radwaste Treatment System configurations are also included.

The ODCM also is used to define the requirements for the TMI radiological environmental monitoring program (REMP) and contains a list and graphical description of the specific sample locations used in the REMP.

The ODCM is maintained at the Three Mile Island (TMI) site for use as a reference guide and training document of accepted methodologies and calculations. Changes in the calculation methods or parameters will be incorporated into the ODCM to ensure the ODCM represents the present methodology in all applicable areas. Changes to the ODCM will be implemented in accordance with the TMI-1 and TMI-2 PDMS Technical Specifications.

The ODCM follows the methodology and models suggested by NUREG-0133, and Regulatory Guide 1.109, Revision 1 for calculation of off-site doses due to plant effluent releases. Simplifying assumptions have been applied in this manual where applicable to provide a more workable document for implementation of the Radiological Effluent Controls requirements.

TMI implements the TMI Radiological Effluent Controls Program and Regulatory Guide 1.21, Revision 1 (Annual Radioactive Effluent Release Report) requirements by use of a computerized system used to determine TMI effluent releases and to update cumulative effluent doses.

TABLE OF CONTENTS

PART I TMI-1 RADIOLOGICAL EFFLUENT CONTROLS

TABLE OF CONTENTS (Cont'd)

PART I TMI-1 RADIOLOGICAL EFFLUENT CONTROLS

TABLE OF CONTENTS (Cont'd)

PART II TMI-2 RADIOLOGICAL EFFLUENT CONTROLS

TABLE OF CONTENTS (Cont'd)

PART II TMI-2 RADIOLOGICAL EFFLUENT CONTROLS

TABLE OF CONTENTS (Cont'd)

PART III EFFLUENT DATA AND CALCULATIONAL METHODOLOGIES

TABLE OF CONTENTS (Cont'd)

PART III ***EFFLUENT DATA AND CALCULATIONAL METHODOLOGIES***

TABLE OF CONTENTS (Cont'd)

PART III EFFLUENT DATA AND CALCULATIONAL METHODOLOGIES

TABLE OF CONTENTS (Cont'd)

PART III EFFLUENT DATA AND CALCULATIONAL METHODOLOGIES

TABLE OF CONTENTS (Cont'd)

PART III **EFFLUENT DATA AND CALCULATIONAL METHODOLOGIES**

TABLE OF CONTENTS *(Cont'd)*

PART IV REPORTING REQUIREMENTS

PART I

TMI-1 RADIOLOGICAL EFFLUENT CONTROLS

1.0 DEFINITIONS

The following terms are defined for uniform interpretation of these controls and surveillances.

1.1 Reactor Operating Conditions

1.1.1 Cold Shutdown

The reactor is in the cold shutdown condition when it is subcritical by at least one percent delta k/k and Tavg is no more than 200°F. Pressure is defined by Technical Specification 3.1.2.

1.1.2 Hot Shutdown

The reactor is in the hot shutdown condition when it is subcritical by at least one percent delta k/k and Tavg is at or greater than 525°F.

1.1.3 Reactor Critical

The reactor is critical when the neutron chain reaction is self-sustaining and Keff = 1.0.

1.1.4 Hot Standby

The reactor is in the hot standby condition when all of the following conditions exist:

a. Tavg is greater than 525°F

b. The reactor is critical

c. Indicated neutron power on the power range channels is less than two percent of rated power. Rated power is defined in Technical Specification Definition 1.1.

1.1.5 Power Operation

The reactor is in a power operating condition when the indicated neutron power is above two percent of rated power as indicated on the power range channels. Rated power is defined in Technical Specification Definition 1.1.

1.1.6 Refueling Shutdown

The reactor is in the refueling shutdown condition when, even with all rods removed, the reactor would be subcritical by at least one percent delta k/k and the coolant temperature at the decay heat removal pump suction is no more than 140°F. Pressure is defined by Technical Specification 3.1.2. A refueling shutdown refers to a shutdown to replace or rearrange all or a portion of the fuel assemblies and/or control rods.

1.1.7 Refueling Operation

An operation involving a change in core geometry by manipulation of fuel or control rods when the reactor vessel head is removed.

1.1.8 *Refueling Interval*

Time between normal refuelings of the reactor. This is defined as once per 24 months.

1.1.9 *Startup*

The reactor shall be considered in the startup mode when the shutdown margin is reduced with the intent of going critical.

1.1.10 Tave

Tave is defined as the arithmetic average of the coolant temperatures in the hot and cold legs of the loop with the greater number of reactor coolant pumps operating, if such a distinction of loops can be made.

1.1.11 Heatup - Cooldown Mode

The heatup-cooldown mode is the range of reactor coolant temperature greater than 200°F and less than 525°F.

1.2 Operable

A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s) and when all necessary attendant instrumentation, controls, electrical power, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its function(s) are also capable of performing their related support function(s).

1.3 Instrument Channel

An instrument channel is the combination of sensor, wires, amplifiers, and output devices which are connected for the purpose of measuring the value of a process variable for the purpose of observation, control, and/or protection. An instrument channel may be either analog or digital.

1.4 Instrumentation Surveillance

1.4.1 Channel Test

A CHANNEL TEST shall be the injection of a simulated signal into the channel as close to the sensor as practical to verify OPERABILITY, including alarm and/or trip functions.

1.4.2 Channel Check

A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrumentation channels measuring the same parameter.

1.4.3 Source Check

A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

1.4.4 Channel Calibration

An instrument CHANNEL CALIBRATION is a test, and adjustment (if necessary), to establish that the channel output responds with acceptable range and accuracy to known values of the parameter which the channel measures or an accurate simulation of these values. Calibration shall encompass the entire channel, including equipment actuation, alarm, or trip and shall be deemed to include the channel test.

1.5 Dose Equivalent I-131

The DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcurie/gram) which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in Table III of TID 14844, "Calculation of Distance Factors for Power and Test Reactor Sites". [Or in Table E-7 of NRC Regulatory Guide 1.109, Revision 1, October 1977.]

1.6 Offsite Dose Calculation Manual (ODCM)

The OFFSITE DOSE CALCULATION MANUAL (ODCM) contains the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluent, in the calculation of gaseous and liquid effluent monitoring Alarm/Trip Setpoints, and in the conduct of the Radiological Environmental Monitoring Program. The ODCM also contains (1) the Radiological Effluent Controls, (2) the Radiological Environmental Monitoring Program and (3) descriptions of the information that should be included in the Annual Radiological Environmental Operating and Annual Radioactive Effluent Release Reports.

1.7 Gaseous Radwaste Treatment

The GASEOUS RADWASTE TREATMENT SYSTEM is the system designed and installed to reduce radioactive gaseous effluent by collecting primary coolant system off gases from the primary system and providing for delay or holdup for the purpose of reducing the total radioactivity prior to release to the environment.

1.8 Ventilation Exhaust Treatment System

A VENTILATION EXHAUST TREATMENT SYSTEM is any system designed and installed to reduce gaseous radioiodine or radioactive material in particulate form in effluent by passing ventilation or vent exhaust gases through charcoal absorbers and/or HEPA filters for the purpose of removing iodine or particulates from the gaseous exhaust system prior to the release to the environment. Engineered Safety Feature (ESF) atmospheric cleanup systems are not considered to be VENTILATION EXHAUST TREATMENT SYSTEMS.

1.9 Purge - Purging

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating conditions in such a manner that replacement air or gas is required to purify the confinement.

1.10 Venting

VENTING is the controlled process of discharging air as gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating conditions in such a manner that replacement air or gas is not provided. Vent used in system name does not imply a VENTING process.

1.11 Member(s) of the Public

MEMBER OF THE PUBLIC means any individual except when that individual is receiving an occupational dose.

1.12 Site Boundary

The SITE BOUNDARY used as the basis for the limits on the release of gaseous effluents is as defined in Section 2.1.2.2 and shown on Figure 2.1-3 of the TMI-1 FSAR. This boundary line includes portions of the Susquehanna River surface between the east bank of the river and Three Mile Island and between Three Mile Island and Shelley Island.

The SITE BOUNDARY used as the basis for the limits on the release of liquid effluents is as shown in Figure 1.1 in Part I of this ODCM.

1.13 Frequency Notation

The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1-1. All Surveillance Requirements shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval. The 25% extension applies to all frequency intervals with the exception of "F." No extension is allowed for intervals designated "F."

1.14 Occupational Dose

OCCUPATIONAL DOSE means the dose received by an individual in the course of employment in which the individual's assigned duties involve exposure to radiation or to radioactive material from licensed and unlicensed sources of radiation, whether in the possession of the licensee or other person. Occupational dose does not include doses received from background radiation, from any medical administration the individual has received, from exposure to individuals administered radioactive material and released under 10CFR35.75, from voluntary participation in medical research programs, or as a member of the public.

Table 1-1

Frequency Notation

Notation	Frequency
S	Shiftly (once per 12 hours)
D	Daily (once per 24 hours)
W	Weekly (once per 7 days)
M	Monthly (once per 31 days)
Q	Quarterly (once per 92 days)
S/A	Semi-Annually (once per 184 days)
R	Refueling Interval (once per 24 months)
P S/U	Prior to each reactor startup, if not done during the previous 7 days
P	Completed prior to each release
N/A (NA)	Not applicable
E	Once per 18 months
F	Not to exceed 24 months

Bases

Section 1.13 establishes the limit for which the specified time interval for Surveillance Requirements may be extended. It permits an allowable extension of the normal surveillance interval to facilitate surveillance scheduling and consideration of plant operating conditions that may not be suitable for conducting the surveillance; e.g., transient conditions or other ongoing surveillance or maintenance activities. It also provides flexibility to accommodate the length of a fuel cycle for surveillances that are specified to be performed at least once each REFUELING INTERVAL. It is not intended that this provision be used repeatedly as a convenience to extend surveillance intervals beyond that specified for surveillances that are not performed once each REFUELING INTERVAL. Likewise, it is not the intent that REFUELING INTERVAL surveillances be performed during power operation unless it is consistent with safe plant operation. The limitation of Section 1.13 is based on engineering judgment and the recognition that the most probable result of any particular surveillance being performed is the verification of conformance with the Surveillance Requirements. This provision is sufficient to ensure that the reliability ensured through surveillance activities is not significantly degraded beyond that obtained from the specified surveillance interval.

FIGURE 1.1

Gaseous Effluent Release Points and Liquid Effluent Outfall Locations

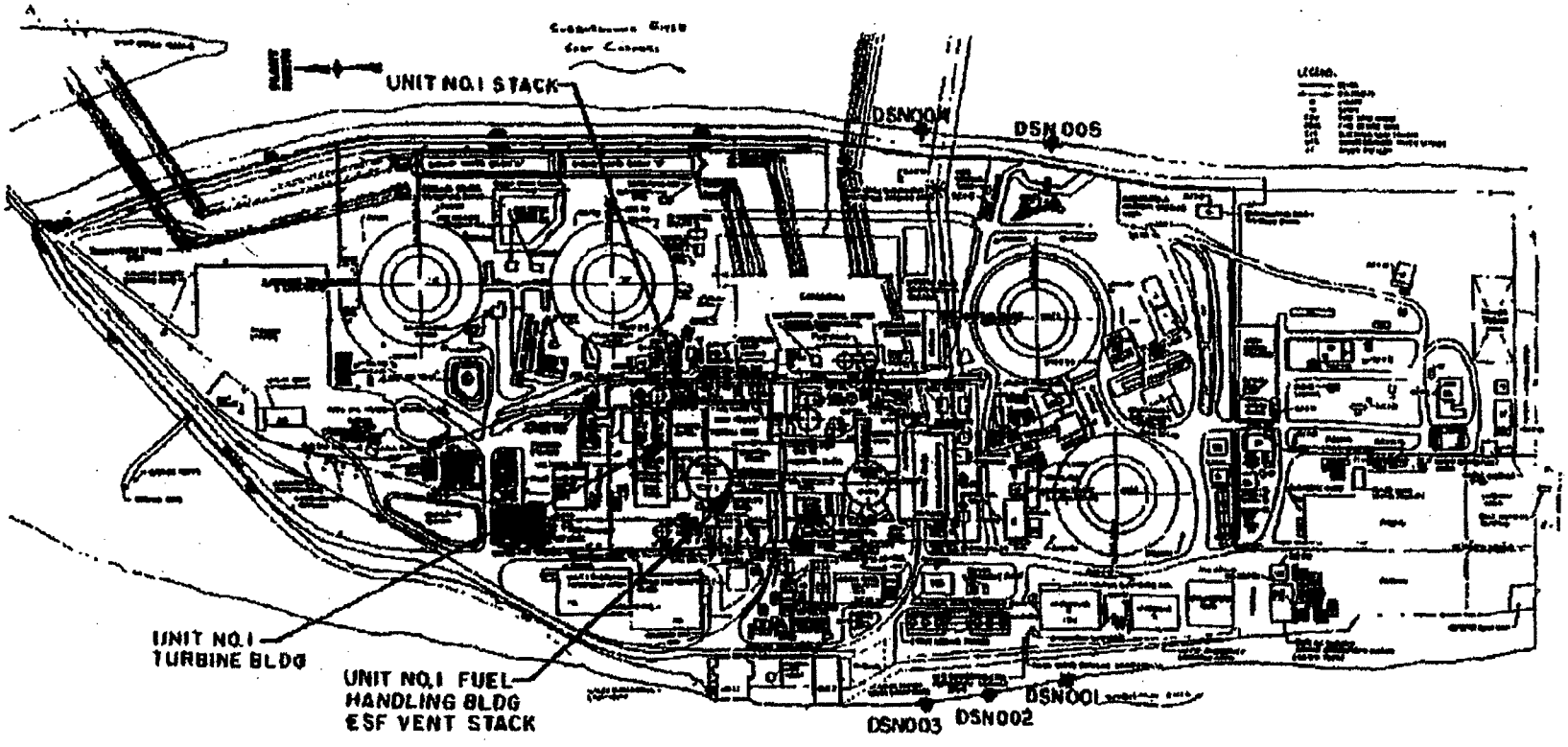


Table 2.1-1

Radioactive Liquid Effluent Instrumentation

	Instrument	Minimum Channels Operable	ACTION
1.	Gross Radioactivity Monitors Providing Automatic Termination of Release		
a.	Unit 1 Liquid Radwaste Effluent Line (RM-L6)	1	18
b.	IWTS/IWFS Discharge Line (RM-L12)	1	20
2.	Flow Rate Measurement Devices		
a.	Unit 1 Liquid Radwaste Effluent Line (FT-84)	1	21
b.	Station Effluent Discharge (FT-146)	1	21

Table Notation

ACTION 18 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases may continue, provided that prior to initiating a release:

1. At least two independent samples are analyzed in accordance with Surveillances 3.2.1.1.1 and 3.2.1.1.2 and;
2. At least two technically qualified members of the Unit staff independently verify the release rate calculations and verify the discharge valve lineup.
3. The TMI Plant Manager shall approve each release.

Otherwise, suspend release of radioactive effluents via this pathway.

ACTION 20 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may commence or continue provided that grab samples are collected and analyzed for gross radioactivity (beta or gamma) at a limit of detection of at least $1x10^{-7}$ microcuries/ml, prior to initiating a release and at least once per 12 hours during release.

ACTION 21 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, radioactive effluent releases via this pathway may continue, provided the flow rate is estimated at least once per 4 hours during actual releases. Pump curves may be used to estimate flow.

2.0 RADIOLOGICAL EFFLUENT CONTROLS AND BASES

2.1 Radioactive Effluent Instrumentation

2.1.1 Radioactive Liquid Effluent Instrumentation

CONTROL:

The radioactive liquid effluent monitoring instrumentation channels shown in Table 2.1-1 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Control 2.2.1.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: At all times *

ACTION:

a. With a radioactive liquid effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above control, immediately suspend the release of radioactive liquid effluent monitored by the affected channel or declare the channel inoperable.

b. With less than the minimum number of radioactive liquid effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 2.1-1. Exert best efforts to return the instrumentation to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Release Report why the inoperability was not corrected in a timely manner.

- For FT-84, and RM-L6, operability is not required when discharges are positively controlled through the closure of WDL-V257.
- For RM-L12 and associated IWTS/IWFS flow interlocks, operability is not required when discharges are positively controlled through the closure of IW-V72, 75 and IW-V280, 281.
- For FT-146, operability is not required when discharges are positively controlled through the closure of WDL-V257, IW-V72, 75 and IW-V280, 281.

BASES

The radioactive liquid effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in liquid effluent during actual or potential releases. The alarm/trip setpoints for these instruments shall be calculated in accordance with NRC approved methods in the ODCM to ensure that the alarm/trip will occur prior to exceeding ten times the effluent concentrations of 10 CFR Part 20.

2.1.2 Radioactive Gaseous Process and Effluent Monitoring Instrumentation

CONTROL:

The radioactive gaseous process and effluent monitoring instrumentation channels shown in Table 2.1-2 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Control 2.2.2.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: As shown in Table 2.1-2.

ACTION:

a. With a radioactive gaseous process or effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above control, immediately suspend the release of radioactive effluent monitored by the affected channel or declare the channel inoperable.

b. With less than the minimum number of radioactive gaseous process or effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 2.1-2. Exert best efforts to return the instrumentation to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Release Report why the inoperability was not corrected in a timely manner.

BASES

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluent during actual or potential releases. The alarm/trip setpoints for these instruments shall be calculated in accordance with NRC approved methods in the ODCM to provide reasonable assurance that the annual releases are within the limits specified in 10 CFR 20.1301.

The low range condenser offgas noble gas activity monitors also provide data for determination of steam generator primary to secondary leakage rate. Channel operability requirements are based on an AmerGen letter #5928-06-20449, "Request to Revise Condenser Vent System Low Range Noble Gas Monitor Operability Requirements", Pamela B. Cowan to U.S.N.R.C., May 25, 2006.

Table 2.1-2

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
1.	Waste Gas Holdup System			
	a. Noble Gas Activity Monitor (RM-A7)	1	***	25
	b. Effluent System Flow Rate Measuring Device (FT-123)	1	***	26
2.	Waste Gas Holdup System Explosive Gas Monitoring System			
	a. Hydrogen Monitor	2	**	30
	b. Oxygen Monitor	2	**	30
3.	Containment Purge Monitoring System			
	a. Noble Gas Activity Monitor (RM-A9)	1	#	27
	b. Iodine Sampler (RM-A9)	1	#	31
	c. Particulate Sampler (RM-A9)	1	#	31
	d. Effluent System Flow Rate Measuring Device (FR-148A, FR-148B)	1	#	26
	e. Sampler Flow Rate Monitor	1	#	26

Table 2.1-2 (Cont'd)

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
4.	Condenser Vent System			
	a. Low Range Noble Gas Activity Monitor (RM-A5Lo or RM-A-15)	1	##	32

Table 2.1-2 (Cont'd)

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
5.	Auxiliary and Fuel Handling Building Ventilation System			
a.	Noble Gas Activity Monitor (RM-A8) or (RM-A4 and RM-A6)	1	*	27
b.	Iodine Sampler (RM-A8) or (RM-A4 and RM-A6)	1	*	31
c.	Particulate Sampler (RM-A8) or (RM-A4 and RM-A6)	1	*	31
d.	Effluent System Flow Rate Measuring Devices (FR-149 and FR-150)	1	*	26
e.	Sampler Flow Rate Monitor	1	*	26
6.	Fuel Handling Building ESF Air Treatment System			
a.	Noble Gas Activity Monitor (RM-A14 or Suitable Equivalent)	1	****	27, 33
b.	Iodine Cartridge	N/A(2)	****	31, 33
c.	Particulate Filter	N/A(2)	****	31, 33
d.	Effluent System Flow (UR-1104A/B)	1	****	26, 33
e.	Sampler Flow Rate Monitor	1	****	26, 33

NOTE 2: No instrumentation channel is provided. However, for determining operability, the equipment named must be installed and functional or the ACTION applies.

Table 2.1-2 (Cont'd)

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
7.	Chemical Cleaning Building Ventilation System			
	a. Noble Gas Activity Monitor (ALC RM-I-18)	1[3]	###	27
	b. Iodine Sampler (ALC RM-I-18)	1[3]	###	31
	c. Particulate Sampler (ALC RM-I-18)	1	###	31
8.	Waste Handling and Packaging Facility Ventilation System			
	a. Particulate Sampler (WHP-RIT-1)	1	###	31
9.	Respirator and Laundry Maintenance Facility Ventilation System			
	a. Particulate Sampler (RLM-RM-1)	1	###	31

NOTE 3: Channel only required when liquid radwaste is moved or processed within the facility.

Table 2.1-2

Table Notation

* At all times.
** *During waste gas holdup system operation.*
*** Operability is not required when discharges are positively controlled through the closure of WDG-V47 or where RM-A8, FT-149 and FT-150 are operable and RM-A8 is capable of automatic closure of WDG-V47.
**** *During Fuel Handling Building ESF Air Treatment System Operation.*
# At all times during containment purging.
At all times when condenser vacuum is established.
During operation of the ventilation system.

ACTION 25 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank may be released to the environment provided that prior to initiating the release:

1. At least two independent samples of the tank's contents are analyzed in accordance with Table 3.2-2, Item A, and

2. At least two technically qualified members of the Unit staff independently verify the release rate calculations and verify the discharge valve lineup.

3. The TMI Plant Manager shall approve each release.

   Otherwise, suspend release of radioactive effluent via this pathway.

ACTION 26 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided the flow rate is estimated at least once per 4 hours.

ACTION 27 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided grab samples are taken at least once per 12 hours and the initial samples are analyzed for gross activity (gamma scan) within 24 hours after the channel has been declared inoperable. If RM-A9 is declared inoperable, see also Technical Specification 3.5.1, Table 3-5.1, Item C.3.f.

ACTION 30 1. With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, a grab sample shall be collected and analyzed for the inoperable gas channel(s) at least once per 24 hours. With both channels inoperable, a grab sample shall be collected and analyzed for the inoperable gas channel(s):

   (a) at least once per 4 hours during degassing operations.

   (b) at least once per 24 hours during other operations (e.g. Feed and Bleed).

2. If the inoperable gas channel(s) is not restored to service within 14 days, a special report shall be submitted to the Regional Administrator of the NRC Region I Office and a copy to the Director, Office of Inspection and Enforcement within 30 days of declaring the channel(s) inoperable. The report shall describe (a) the cause of the monitor inoperability, (b) action being taken to restore the instrument to service, and (c) action to be taken to prevent recurrence.

Table 2.1-2

ACTION 31 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that within four hours after the channel has been declared inoperable, samples are continuously collected with auxiliary sampling equipment.

ACTION 32 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 14 days, provided that grab samples are taken and analyzed.

If the primary-to-secondary leakrate was unstable* or was indicating an increasing trend at the initial time when there was no operable channel of the Condenser Vent System Low Range Noble Gas Activity Monitor, analyze grab samples of the reactor coolant system and Condenser OffGas once every 4 hours to provide an indication of primary-to-secondary leakage, and subsequent sample frequency shall be in accordance with Table 1 based on the last sample result. Otherwise, analyze grab samples of the reactor coolant system and Condenser OffGas to provide an indication of primary-to-secondary leakage at the minimum frequency indicated in Table 1, below:

Table 1
Minimum Frequency of Grab Samples When
No Condenser Vent System Low Range Noble Gas Activity Monitor is Operable

Existing Total Primary-to-Secondary Leak Rate (based on last monitor reading or sample result)	Frequency of Grab Samples
0 to < 5 GPD	Once per 24 hours
5 to < 30 GPD	Once per 12 hours
30 to < 75 GPD	Once per 4 hours
75 GPD or greater	Place the unit in at least HOT STANDBY within the next 6 hours, and at least HOT SHUTDOWN within the following 6 hours, and at least COLD SHUTDOWN within the subsequent 24 hours.

*Unstable is defined as > 10% increase during a 1 hour period, as stated in the EPRI Guidelines.

Condenser Vent System Low Range Noble Gas Activity Monitor inoperable channels should be restored to operability as rapidly as practical.
After 14 days, if one OPERABLE channel is not returned to service, within 1 hour the provisions of Technical Specification 3.0.1 apply, as if this Control were a Tech Spec Limiting Condition for Operation.

ACTION 33 With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel to OPERABLE status within 7 days, or prepare and submit a special report within 30 days outlining the action(s) taken, the cause of the inoperability, and plans and schedule for restoring the system to OPERABLE status.

2.2 Radioactive Effluent Controls

2.2.1 Liquid Effluent Controls

2.2.1.1 *Liquid Effluent Concentration*

CONTROL:

The concentration of radioactive material released at anytime from the unit to unrestricted areas shall be limited to ten times the concentrations specified in 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to 3×10^{-3} uCi/cc total activity.

APPLICABILITY: At all times

ACTION:

With the concentration of radioactive material released from the unit to unrestricted areas exceeding the above limits, immediately restore concentrations within the above limits.

BASES

This control is provided to ensure that the concentration of radioactive materials released in liquid waste effluent from the unit to unrestricted areas will be less than ten times the concentration levels specified in 10 CFR Part 20.1001-20.2401, Appendix B, Table 2. This limitation provides additional assurance that the levels of radioactive materials in bodies of water outside the site will not result in exposures with (1) the Section II.A design objectives of Appendix I, 10 CFR Part 50, to a MEMBER OF THE PUBLIC and (2) the limits of 10 CFR Part 20.1301 to the population. The concentration limit for noble gases is based upon the assumption the Xe-135 is the controlling radioisotope and its MPC in air (submersion) was converted to an equivalent concentration in water using the methods described in International Commission on Radiological Protection (ICRP) Publication 2.

2.2.1.2 Liquid Effluent Dose

CONTROL

The dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from the unit to the SITE BOUNDARY shall be limited:

a. During any calendar quarter to less than or equal to 1.5 mrem to the total body and to less than or equal to 5 mrem to any organ.

b. During any calendar year to less than or equal to 3 mrem to the total body and to less than or equal to 10 mrem to any organ.

APPLICABILITY: At all times

ACTION:

a. With the calculated dose from the release of radioactive materials in liquid effluents exceeding any of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions to be taken to reduce the releases of radioactive materials in liquid effluents during the remainder of the current calendar quarter and during the subsequent 3 calendar quarters so that the cumulative dose or dose commitment to any individual from such releases during these four calendar quarters is within 3 mrem to the total body and 10 mrem to any organ. This Special Report shall also include (1) the result of radiological analyses of the drinking water source, and (2) the radiological impact on finished drinking water supplies with regard to the requirements of 40 CFR 141, Safe Drinking Water Act.

BASES

This control and associated action is provided to implement the requirements of Sections II.A, III.A, and IV.A of Appendix I, 10 CFR Part 50. The Control implements the guides set forth in Section II.A of Appendix I. The ACTION statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in liquid effluents will be kept "as low as is reasonably achievable". Also, for fresh water sites with drinking water supplies which can be potentially affected by plant operations, there is reasonable assurance that the operation of the facility will not result in radionuclide concentrations in the finished drinking water that are in excess of the requirements of 10 CFR 20. The dose calculations in the ODCM implement The requirements in Section III.A. of Appendix I that conformance with the guides of Appendix I is to be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The equations specified in the ODCM for calculating the doses due to the actual release rates of radioactive materials in liquid effluents are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October, 1977, and Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I," April, 1977. NUREG-0133 provides methods for dose calculations consistent with Regulatory Guides 1.109 and 1.113.

2.2.1.3 Liquid Radwaste Treatment System

CONTROL:

The appropriate portions of the liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent from the unit to unrestricted areas would exceed 0.06 mrem to the total body or 0.2 mrem to any organ in any calendar month.

APPLICABILITY: At all times

ACTION:

a. With radioactive liquid waste being discharged without treatment and in excess of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which includes the following information:

1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystems, and the reason for inoperability,

2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and,

3. A summary description of action(s) taken to prevent a recurrence.

BASES

The requirement that the appropriate portions of this system be used, when specified, provides assurance that the releases of radioactive materials in liquid effluents will be kept as low as is reasonably achievable. This control implements the requirements of 10 CFR Part 50.36a, General Design Criterion 60 of Appendix A to 10 CFR Part 50 and the design objective given in Section II.D of Appendix I to 10 CFR Part 50. The intent of Section II.D. is to reduce effluents to as low as is reasonably achievable in a cost effective manner. This control satisfies this intent by establishing a dose limit which is a small fraction (25%) of Section II.A of Appendix I, 10 CFR Part 50 dose requirements. This margin, a factor of 4, constitutes a reasonable reduction.

2.2.1.4 *Liquid Holdup Tanks*

CONTROL

The quantity of radioactive material contained in each of the following tanks shall be limited to less than or equal to 10 curies, excluding tritium and dissolved or entrained noble gases.

a. *Outside temporary tank*

APPLICABILITY: At all times.

ACTION:

a. With the quantity of radioactive material in any of the above listed tanks exceeding the above limit, immediately suspend all *additions of radioactive material to the tank and within 48 hours* reduce the tank contents to within the limit.

BASES

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR *Part 20.1001-20-20.2401, Appendix B, Table 2, Column 2, at the nearest* potable water supply and the nearest surface water supply in an unrestricted area.

2.2.2 Gaseous Effluent Controls

2.2.2.1 Gaseous Effluent Dose Rate

CONTROL:

The dose rate due to radioactive materials released in gaseous effluent from the site shall be limited to the following:

a. For noble gases: less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin, and

b. For I-131, I-133, tritium and *all radionuclides in particulate form* with half lives greater than 8 days: less than or equal to 1500 mrem/yr to any organ.

APPLICABILITY: At *all* times.

ACTION:

With the release rate(s) exceeding the above limits, *immediately decrease the* release rate to comply with the above limit(s).

BASES

The control implements the requirement in Technical Specification (6.8.4.b(7). This specification is provided to ensure that the dose from radioactive materials in gaseous effluents at and beyond the SITE BOUNDARY will be within the annual dose limits of 10 CFR Part 20. The annual dose limits are the doses associated with 10 times the concentrations of 10 CFR Part 20, Appendix B, Table 2, Column 1. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of a MEMBER OF THE PUBLIC, either within or outside the SITE BOUNDARY, to annual average concentrations exceeding the limits specified in Appendix B, Table 2 of 10 CFR Part 20.1302. For MEMBERS OF THE PUBLIC who may at times be within the SITE BOUNDARY, the occupancy of the MEMBER OF THE PUBLIC will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the exclusion area boundary. The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to a MEMBER OF THE PUBLIC at or beyond the SITE BOUNDARY to less than or equal to 500 mrem/year to the total body or to less than or equal to 3000 mrem/year to the skin. These release rate limits also restrict, at all times, the corresponding thyroid dose rate above background to a child via the inhalation pathway to less than or equal to 1500 mrem/year (NUREG 1301).

2.2.2.2 Gaseous Effluents Dose-Noble Gases

CONTROL:

The air dose due to noble gases released in gaseous effluents from the unit to areas at and beyond the SITE BOUNDARY shall be limited to the following:

a. During any calendar quarter: less than or equal to 5 mrad for gamma radiation and less than or equal to 10 mrad for beta radiation and,

b. During any calendar year: less than or equal to 10 mrad for gamma radiation and less than or equal to 20 mrad for beta radiation.

APPLICABILITY: At all times.

ACTION:

a. With the calculated air dose from radioactive noble gases in gaseous effluents exceeding any of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

BASES

This control applies to the release of radioactive materials in gaseous effluents from TMI-1.

This control and associated action is provided to implement the requirements of Section II.B, III.A and IV.A of Appendix I, 10 CFR Part 50. The Control implements the guides set forth in Section II.B of Appendix I. The ACTION statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents will be kept "as low as is reasonably achievable." The Surveillance Requirements implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through the appropriate pathways is unlikely to be substantially underestimated. The dose calculation methodology and parameters established in the ODCM for calculating the doses due to the actual release rates of radioactive noble gases in gaseous effluents are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Release of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water Cooled Reactors," Revision 1, July 1977. The ODCM equations provided for determining the air doses at and beyond the SITE BOUNDARY are based upon the historical average atmospheric conditions. NUREG-0133 provides methods for dose calculations consistent with Regulatory Guides 1.109 and 1.111.

2.2.2.3 Dose - Iodine-131, Iodine-133, Tritium, and Radionuclides In Particulate Form

CONTROL:

The dose to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, Tritium, and all radionuclides in particulate form with half lives greater than 8 days, in gaseous effluents released from the unit to areas at and beyond the SITE BOUNDARY shall be limited to the following:

a. During any calendar quarter: less than or equal to 7.5 mrem to any organ, and

b. During any calendar year: less than or equal to 15 mrem to any organ.

APPLICABILITY: At all times.

ACTION:

With the calculated dose from the release of Iodine-131, Iodine-133, Tritium, and radionuclides in particulate form with half lives greater than 8 days, in gaseous effluents exceeding any of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which identifies the cause(s) for exceeding the limit and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

BASES

This control applies to the release of radioactive materials in gaseous effluents from TMI-1.

This control and associated action is provided to implement the requirements of Section II.C, III.A and IV.A of Appendix I, 10 CFR Part 50. The Controls are the guides set forth in Section II.C of Appendix I. The ACTION statement provides the required operating flexibility and at the same time implements the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive materials in gaseous effluents will be kept "as low as is reasonably achievable." The ODCM calculational methods specified in the surveillance requirements implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The ODCM calculational methodology and parameters for calculating the doses due to the actual release rates of the subject materials are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October, 1977 and *Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and* Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors," Revision 1, July, 1977. These equations also provide for determining the actual doses based upon the historical average atmospheric conditions. The release rate controls for iodine-131, iodine-133, tritium and radionuclides in particulate form with half lives greater than 8 days are dependent upon the existing radionuclide pathways to man, in areas at and beyond the SITE BOUNDARY. The pathways that were examined in the development of these calculations were: 1) individual inhalation of airborne radionuclides, 2) deposition of radionuclides onto green leafy vegetation with subsequent consumption by man, 3) deposition onto grassy areas where milk animals and meat producing animals graze with consumption of the milk and meat by man, and 4) deposition on the ground with subsequent exposure of man.

2.2.2.4 Gaseous Radwaste Treatment System

CONTROL

The GASEOUS RADWASTE TREATMENT SYSTEM and the VENTILATION EXHAUST TREATMENT SYSTEM shall be OPERABLE. The appropriate portions of the GASEOUS RADWASTE TREATMENT SYSTEM shall be used to reduce radioactive materials in the gaseous waste prior to their discharge when the monthly projected gaseous effluent air doses due to untreated gaseous effluent releases from the unit would exceed 0.2 mrad for gamma radiation and 0.4 mrad for beta radiation. The appropriate portions of the VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the monthly projected doses due to gaseous effluent releases from the site would exceed 0.3 mrem to any organ.

APPLICABILITY: At all times.

ACTION:

a. With the GASEOUS RADWASTE TREATMENT SYSTEM and/or the VENTILATION EXHAUST TREATMENT SYSTEM inoperable for more than a month or with gaseous waste being discharged without treatment and in excess of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which includes the following information:

   1. Identification of the inoperable equipment or subsystems and the reason for inoperability,

   2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and

   3. A summary description of action(s) taken to prevent a recurrence.

BASES

The use of the GASEOUS RADWASTE TREATMENT SYSTEM and the VENTILATION EXHAUST TREATMENT SYSTEM ensures that gaseous effluents are treated as appropriate prior to release to the environment. The appropriate portions of this system provide reasonable assurance that the releases of radioactive materials in gaseous effluents will be kept "as low as is reasonably achievable." This control implements the requirements of 10 CFR Part 50.36a, General Design Criterion 60 of Appendix A to 10 CFR Part 50, and the design objectives given in Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the systems were specified as a suitable fraction of the guide set forth in Sections II.B and II.C of Appendix I, 10 CFR Part 50, for gaseous effluents.

2.2.2.5 Explosive Gas Mixture

CONTROL

The concentration of oxygen in the Waste Gas Holdup System shall be limited to less than or equal to 2% by volume whenever the concentration of hydrogen in the Waste Gas Holdup System is greater than or equal to 4% by volume.

AVAILABILITY: At all times.

ACTION:

Whenever the concentration of hydrogen in the Waste Gas Holdup System is greater than or equal to 4% by volume, and:

a. The concentration of oxygen in the Waste Gas Holdup System is greater than 2% by volume, but less than 4% by volume, without delay begin to reduce the oxygen concentration to within its limit.

b. The concentration of oxygen in the Waste Gas Holdup System is greater than or equal to 4% by volume, immediately suspend additions of waste gas to the Waste Gas Holdup System and without delay begin to reduce the oxygen concentration to within its limit.

BASES:

Based on experimental data (Reference 1), lower limits of flammability for hydrogen is 5% and for oxygen is 5% by volume. Therefore, if the concentration of either gas is kept below it lower limit, the other gas may be present in higher amounts without the danger of an explosive mixture. Maintaining the concentrations of hydrogen and oxygen such that an explosive mixture does not occur in the waste gas holdup system provides assurance that the release of radioactive materials will be controlled in conformance with the requirements of General Design Criterion 60 of Appendix A to 10 CFR 50.

REFERENCES

(1) Bulletin 503, Bureau of Mines; Limits of Flammability of Gases and Vapors.

2.2.2.6 Waste Gas Decay Tanks

CONTROL:

The quantity of radioactivity contained in each waste gas decay tank shall be limited to less than or equal to 8800 curies noble gases (considered as Xe-133).

APPLICABILITY: At all times.

ACTION:

a. With the quantity of radioactive material in any waste gas decay tank exceeding the above limit, immediately suspend all additions of radioactive material to the tank and within 48 hours reduce the tank contents to within the limit.

BASES

Restricting the quantity of radioactivity contained in each waste gas decay tank provides assurance that in the event of an uncontrolled release of the tanks contents, the resulting total body exposure to a MEMBER OF THE PUBLIC at the nearest exclusion area boundary will not exceed 0.5 rem. This is consistent with Standard Review Plan 15.7.1, "Waste Gas System Failure."

2.2.3 Total Radioactive Effluent Controls

2.2.3.1 Total Dose

CONTROL:

The annual (calendar year) dose or dose commitment to any MEMBER OF THE PUBLIC, due to releases of radioactivity and to radiation from uranium fuel cycle sources shall be limited to less than or equal to 25 mrem to the total body or any organ except the thyroid, which shall be limited to less than or equal to 75 mrem.

APPLICABILITY: At all times.

ACTION:

With the calculated dose from the release of radioactive materials in liquid or gaseous effluents exceeding twice the limits of Controls 2.2.1.2.a, 2.2.1.2.b, 2.2.2.2.a, 2.2.2.2.b, 2.2.2.3.a, or, 2.2.2.3.b, calculations should be made including direct radiation contributions from the unit and from outside storage tanks to determine whether the above limits of Control 2.2.3.1 have been exceeded. If such is the case, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which defines the corrective action to be taken to reduce subsequent releases to prevent recurrence of exceeding the above limits and includes the schedule for achieving conformance with the above limits. This Special Report, as defined in

10 CFR Part 20.2203(b), shall include an analysis which estimates the radiation exposure (dose) to a MEMBER OF THE PUBLIC from uranium fuel cycle sources, including all effluent pathways and direct radiation, for the calendar year that includes the release(s) covered by this report. It shall also describe levels of radiation and concentrations of radioactive material involved, and the cause of the exposure levels or concentrations. If the estimated dose(s) exceed the above limits, and if the release condition resulting in violation of 40 CFR 190 has not already been corrected, the Special Report shall include a request for a variance in accordance with the provisions of 40 CFR 190. Submittal of the report is considered a timely request, and a variance is granted until staff action on the request is complete.

BASES

This control is provided to meet the dose limitations of 40 CFR Part 190 that have been incorporated into 10 CFR Part 20.1301(d). This control requires the preparation and submittal of a Special Report whenever the calculated doses from plant generated radioactive effluents and direct radiation exceed 25 mrem to the total body or any organ, except the thyroid, which shall be limited to less than or equal to 75 mrem. For sites containing up to 4 reactors, it is highly unlikely that the resultant dose to a MEMBER OF THE PUBLIC will exceed the dose limits of 40 CFR Part 190 if the individual reactors remain within twice the dose design objectives of Appendix I, and if direct radiation doses from the reactor units and outside storage tanks are kept small. The Special Report will describe a course of action that should result in the limitation of the annual dose to a MEMBER OF THE PUBLIC to within the 40 CFR Part 190 limits. For the purposes of the Special Report, it may be assumed that the dose commitment to the member of the public from other uranium fuel cycle sources is negligible, with the exception that dose contributions from other nuclear fuel cycle facilities at the same site or within a radius of 8 km must be considered. If the dose to any member of the public is estimated to exceed the requirements of 40 CFR Part 190, the Special Report with a request for a variance (provided the release conditions resulting in violation of 40 CFR Part 190 have not already been corrected), in accordance with the provisions of 40 CFR Part 190.11 and 10 CFR Part 20.2203(b), is considered to be a timely request and fulfills the requirements of 40 CFR Part 190 until NRC staff action is completed. The variance only relates to the limits of 40 CFR Part 190, and does not apply in any way to the other requirements for dose limitation of 10 CFR Part 20, as addressed in Controls 2.2.1.1 and 2.2.2.1. An individual is not considered a MEMBER OF THE PUBLIC during any period in which he/she is engaged in carrying out any operation that is part of the nuclear fuel cycle.

3.0 **SURVEILLANCES**

3.1 Radioactive Effluent Instrumentation

3.1.1 Radioactive Liquid Effluent Instrumentation

Surveillance Requirements

3.1.1.1 Each radioactive liquid effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, AND CHANNEL TEST operations during the MODES and at the frequencies shown in Table 3.1-1.

Table 3.1-1

Radioactive Liquid Effluent Monitoring Instrumentation Surveillance Requirements

	INSTRUMENT	CHANNEL CHECK	SOURCE CHECK	CHANNEL CALIBRATION	CHANNEL TEST
1.	Radioactivity Monitors Providing Alarm and Automatic Isolation				
	a. Unit 1 Liquid Radwaste Effluents Line (RM-L-6)	D	P	R(2)	Q(1)
	b. IWTS/IWFS Discharge Line (RM-L-12)	D	P	R(2)	Q(1)
2.	Flow Rate Monitors				
	a. Unit 1 Liquid Radwaste Effluent Line (FT-84)	D(3)	N/A	R	Q
	b. Station Effluent Discharge (FT-146)	D(3)	N/A	R	Q

Table 3.1-1

Table Notation

(1) The CHANNEL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occurs if the following condition exists:

1. Instrument indicates measured levels above the high alarm/trip setpoint. (Includes - circuit failure)
2. Instrument indicates a down scale failure. (Alarm function only.) (Includes - circuit failure)
3. Instrument controls moved from the operate mode (Alarm function only).

(2) The initial CHANNEL CALIBRATION for radioactivity measurement instrumentation shall be performed using one or more of the reference standards certified by the National Institute of Standards and Technology or using standards that have been obtained from suppliers that participated in measurement assurance activities with NIST. These standards should permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration should be used. (Operating plants may substitute previously established calibration procedures for this requirement)

(3) CHANNEL CHECK shall consist of verifying indication of flow during periods of release. CHANNEL CHECK shall be made at least once daily on any day on which continuous, periodic, or batch releases are made.

3.1.2 Radioactive Gaseous Process and Effluent Monitoring Instrumentation

Surveillance Requirements

3.1.2.1 Each radioactive gaseous process or effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and CHANNEL TEST operations at the frequencies shown in Table 3.1-2.

Table 3.1-2

Radioactive Gaseous Process and Effluent Monitoring Instrumentation Surveillance Requirements

	INSTRUMENT	CHANNEL CHECK	SOURCE CHECK	CHANNEL CALIBRATION	CHANNEL TEST	APPLICABILITY
1.	Waste Gas Holdup System					
	a. Noble Gas Activity Monitor (RM-A7)	P	P	E(3)	Q(1)	***
	b. Effluent System Flow Rate Measuring Device (FT-123)	P	N/A	E	Q	***
2.	Waste Gas Holdup System Explosive Gas Monitoring System					
	a. Hydrogen Monitor	D	N/A	Q(4)	M	**
	b. Oxygen Monitor	D	N/A	Q(5)	M	**
3.	Containment Purge Vent System					
	a. Noble Gas Activity Monitor (RM-A9)	D	P	E(3)	M(1)	#
	b. Iodine Sampler (RM-A9)	W	N/A	N/A	N/A	#
	c. Particulate Sampler (RM-A9)	W	N/A	N/A	N/A	#
	d. Effluent System Flow Rate Measuring Device (FR-148)	D	N/A	E	Q	#
	e. Sampler Flow Rate Monitor	D	N/A	E	N/A	#
4.	Condenser Vent System					
	a. Noble Gas Activity Monitor (RM-A5 and Suitable Equivalent - See Table 2.1-2, Item 4.a)	D	M	E(3)	Q(2)	##

Table 3.1-2

Radioactive Gaseous Process and Effluent Monitoring Instrumentation Surveillance Requirements

	INSTRUMENT	CHANNEL CHECK	SOURCE CHECK	CHANNEL CALIBRATION	CHANNEL TEST	APPLICABILITY
5.	Auxiliary and Fuel Handling Building Ventilation System					
	a. Noble Gas Activity Monitor (RM-A8) or (RM-A4 and RM-A6)	D	M	E(3)	Q(1)	*
	b. Iodine Sampler (RM-A8) or (RM-A4 and RM-A6)	W	N/A	N/A	N/A	*
	c. Particulate Sampler (RM-A8) or (RM-A4 and RM-A6)	W	N/A	N/A	N/A	*
	d. System Effluent Flow Rate Measurement Devices (FR-149 and FR-150)	D	N/A	E	Q	*
	e. Sampler Flow Rate Monitor	D	N/A	E	N/A	*
6.	Fuel Handling Building ESF Air Treatment System					
	a. Noble Gas Activity Monitor (RM-A14)	D	M	R(3)	Q(2)	****
	b. System Effluent Flow Rate (UR-1104 A/B)	D	N/A	R	Q	****
	c. Sampler Flow Rate Measurement Device	D	N/A	R	Q	****

Table 3.1-2

Radioactive Gaseous Process and Effluent Monitoring Instrumentation Surveillance Requirements

	INSTRUMENT	CHANNEL CHECK	SOURCE CHECK	CHANNEL CALIBRATION	CHANNEL TEST	APPLICABILITY
7.	Chemical Cleaning Building Ventilation System					
	a. Noble Gas Activity Monitor (ALC RM-I-18)	D	M	E(3)	Q(2)	###
	b. Iodine Sampler (ALC RM-I-18)	W	N/A	N/A	N/A	###
	c. Particulate Sampler (ALC RM-I-18)	W	N/A	N/A	N/A	###
8.	Waste Handling and Packaging Facility Ventilation System					
	a. Particulate Sampler (WHP-RIT-1)	D	W	SA	W	###
9.	Respirator and Laundry Maintenance Ventilation System					
	a. Particulate Sampler (RLM-RM-1)	D	W	SA	W	###

Table 3.1-2

Table Notation

* At all times.
** During waste gas holdup system operation.
*** Operability is not required when discharges are positively controlled through the closure of WDG-V47, and where RM-A8 (or RM-A4 and RM-A6), FT-149, and FT-150 are operable.
**** During Fuel Handling Building ESF Air Treatment System Operation.
# At all times during containment purging.
At all times when condenser vacuum is established.
During operation of the ventilation system.

(1) The CHANNEL TEST shall also demonstrate that automatic isolation of this pathway for the Auxiliary and Fuel Handling Building Ventilation System, the supply ventilation is isolated and control room alarm annunciation occurs if the following condition exists:

1. Instrument indicates measured levels above the high alarm/trip setpoint (Includes circuit failure).

2. Instrument indicates a down scale failure (Alarm function only) (Includes circuit failure).

3. Instrument controls moved from the operate mode (Alarm function only).

(2) The CHANNEL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1 Instrument indicates measured levels above the alarm setpoint. (includes circuit failure)

2. Instrument indicates a down scale failure (includes circuit failure).

3. Instrument controls moved from the operate mode.

(3) The initial CHANNEL CALIBRATION for radioactivity measurement instrumentation shall be performed using one or more of the reference standards certified by the National Institute of Standards and Technology or using standards that have been obtained from suppliers that participate in measurement assurance activities with NIST. These standards should permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration should be used. (Operating plants may substitute previously established calibration procedures for this requirement.)

(4) The CHANNEL CALIBRATION shall include the use of standard gas samples containing a nominal:

1. One volume percent hydrogen, balance nitrogen, and

2. Four volume percent hydrogen, balance nitrogen.

(5) The CHANNEL CALIBRATION shall include the use of standard gas samples containing a nominal:

1. One volume percent oxygen, balance nitrogen, and

2. Four volume percent oxygen, balance nitrogen.

3.2 Radiological Effluents

3.2.1 Liquid Effluents

SURVEILLANCE REQUIREMENTS

3.2.1.1 Concentration

3.2.1.1.1 The radioactivity content of each batch of radioactive liquid waste shall be determined prior to release by sampling and analysis in accordance with Table 3.2-1. The results of pre-release analyses shall be used with the calculational methods in the ODCM to assure that the concentration at the point of release is maintained within the limits of Control 2.2.1.1.

3.2.1.1.2 Post-release analysis of samples composited from batch releases shall be performed in accordance with Table 3.2-1. The results of the previous post-release analysis shall be used with the calculational methods in the ODCM to assure that the concentrations at the point of release were maintained within the limits of Control 2.2.1.1.

3.2.1.1.3 The radioactivity concentration of liquids discharged from continuous release points shall be determined by collection and analysis of samples in accordance with Table 3.2-1. The results of the analysis shall be used with the calculational methods of the ODCM to assure that the concentration at the point of release is maintained within the limits of Control 2.2.1.1.

3.2.1.2 Dose Calculations

3.2.1.2.1 Cumulative dose contributions from liquid effluents shall be determined in accordance with the Offsite Dose Calculation Manual (ODCM) at least once a month.

3.2.1.3 Liquid Waste Treatment

3.2.1.3.1 Doses due to liquid releases shall be projected at least once a month, in accordance with the ODCM.

3.2.1.4 Liquid Holdup Tanks

3.2.1.4.1 The quantity of radioactive material contained in each of the tanks specified in Control 2.2.1.4 shall be determined to be within the limit by analyzing a representative sample of the tank's content weekly when radioactive materials are being added to the tank.

Table 3.2-1

Radioactive Liquid Waste Sampling and Analysis Program

	Liquid Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) (µCi/ml) (Note a)
A.1	Batch Waste Release Tanks (Note d)	P Each Batch	P Each Batch	H-3 *Principal Gamma* Emitters (Note f) I-131 Dissolved and Entrained Gases (Gamma Emitters) (Note g)	1×10^{-5} 5×10^{-7} 1×10^{-6} 1×10^{-4}
		P Each Batch	Q Composite (Note b)	*Gross alpha* Sr-89, Sr-90 Fe-55	1×10^{-7} 5×10^{-8} 1×10^{-6}
A.2	Continuous Releases (Note e)	Continuous (Note c)	W Composite (Note c)	Principal Gamma Emitters (Note f) I-131	5×10^{-7} 1×10^{-6}
		Grab Sample M	M	Dissolved and *Entrained* Gases (Gamma Emitters) (Note g)	1×10^{-5}
		Continuous (Note c)	M Composite (Note c)	H-3 Gross alpha	1×10^{-5} 1×10^{-7}
		Continuous (Note c)	Q Composite (Note c)	Sr-89, Sr-90 Fe-55	5×10^{-8} 1×10^{-6}

Table 3.2-1

Table Notation

a. The LLD is defined, for purposes of this surveillance, as the smallest concentration of radioactive material in a sample that will yield a net count above system background that will be detected with 95% probability with 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66\,S_b}{E \times V \times 2.22 \times 10^6 \times Y \times \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above (as microcurie per unit mass or volume),

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute),

E is the counting efficiency (as counts per disintegration),

V is the sample size (in units of mass or volume),

2.22×10^6 is the number of disintegrations per minute per microcurie,

Y is the fractional radiochemical yield (when applicable),

λ is the radioactive decay constant for the particular radionuclide, and

Δt is the elapsed time between midpoint of sample collection and time of counting.

Typical values of E, V, Y and Δt shall be used in the calculation.

It should be recognized that the LLD is defined as an "a priori" (before the fact) limit representing the capability of a measurement system and not as an "a posteriori" (after the fact) limit for a particular measurement.

b. A composite sample is one in which the quantity of liquid sampled is proportional to the quantity of liquid waste discharged and in which the method of sampling employed results in a specimen which is representative of the liquids released.

c. To be representative of the quantities and concentrations of radioactive materials in liquid effluent, samples shall be collected continuously in proportion to the rate of flow of the effluent stream. Prior to analyses, all samples taken for the composite shall be thoroughly mixed in order for the composite sample to be representative of the effluent release.

Table 3.2-1

d. A batch release is the discharge of liquid wastes of a discrete volume. Prior to sampling for analyses, each batch shall be isolated, and be thoroughly mixed, by a method described in the ODCM, to assure representative sampling.

e. A continuous release is the discharge of liquid wastes of a non- discrete volume; e.g., from a volume or system that has an input flow during the continuous release.

f. The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144. This list does not mean that only these nuclides are to be considered. Other gamma peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radioactive Effluent Release Report pursuant to TS 6.9.4.

g. The gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, and Xe-135. This list does not mean that only these nuclides are to be considered. Other gamma peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Effluent Release Report pursuant to T.S. 6.9.4.

3.2.2 Gaseous Effluents

SURVEILLANCE REQUIREMENTS

3.2.2.1 Dose Rates

3.2.2.1.1 The dose rate due to noble gases in gaseous effluents shall be determined to be within the limits of Control 2.2.2.1.a in accordance with the methods and procedures of the ODCM.

3.2.2.1.2 The dose rate of radioactive materials, other than noble gases, in gaseous effluents shall be determined to be within the limits of Control 2.2.2.1.b in accordance with methods and procedures of the ODCM by obtaining representative samples and performing analyses in accordance with the sampling and analysis program, specified in Table 3.2-2.

3.2.2.2 Dose, Noble Gas

3.2.2.2.1 Cumulative dose contributions from noble gas effluents for the current calendar quarter and current calendar year shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM) monthly.

3.2.2.3 Dose, Iodine-131, Iodine-133, Tritium, and Radionuclides In Particulate Form

3.2.2.3.1 Cumulative dose contributions from Iodine-131, Iodine-133, Tritium, and radionuclides in particulate form with half lives greater than 8 days for the current calendar quarter and current calendar year shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM) monthly.

3.2.2.4 Gaseous Waste Treatment

3.2.2.4.1 Doses due to gaseous releases from the unit shall be projected monthly in accordance with the ODCM.

3.2.2.5 Explosive Gas Mixture

3.2.2.5.1 The concentrations of hydrogen and oxygen in the waste gas holdup system shall be determined to be within the limits of Control 2.2.2.5 by monitoring the waste gases in the Waste Gas Holdup System with the hydrogen and oxygen monitors covered in Table 2.1-2 of Control 2.1.2.

3.2.2.6 Waste Gas Decay Tank

3.2.2.6.1 The concentration of radioactivity contained in the vent header shall be determined weekly. If the concentration of the vent header exceeds 10.7 Ci/cc, daily samples shall be taken of each waste gas decay tank being added to, to determine if the tank(s) is less than or equal to 8800 Ci/tank.

Table 3.2-2

Radioactive Gaseous Waste Sampling and Analysis Program

	Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) µCi/ml) (Note a)
A.	Waste Gas Decay Tank	P Each Tank Grab Sample	P Each Tank	Principal Gamma Emitters (Note g)	1×10^{-4}
B.	Containment Purge	P (Note b) Each Purge Grab Sample	P (Note b) Each Purge	H-3 Principal Gamma Emitters (Note g)	1×10^{-6} 1×10^{-4}
C.	Auxiliary and Fuel Handling Building Air Treatment System	M (Notes c, e) Grab Sample	M	H-3 Principal Gamma Emitters (Note g)	1×10^{-6} 1×10^{-4}
D.	Fuel Handling Building ESF Air Treatment System	M (during System Operation) Grab Sample	M (during System Operation)	H-3 Principal Gamma Emitters (Note g)	1×10^{-6} 1×10^{-4}
E.	Condenser Vacuum Pumps Exhaust (Note h)	M (Note h) Grab Sample	M (Note h)	H-3 Principal Gamma Emitters (Note g)	1×10^{-6} 1×10^{-4}
F.	Chemical Cleaning Building Air Treatment System	M (Note l) Grab Sample	M	H-3 Principal Gamma Emitters (Note g)	1×10^{-6} 1×10^{-4}
G.	Waste Handling and Packaging Facility Air Treatment System	See Section I of this table	See Section I of this table	See Section I of this table	See Section I of this table
H.	Respirator and Laundry Maintenance Facility Air Treatment System	See Section I of this table	See Section I of this table	See Section I of this table	See Section I of this table

Table 3.2-2

Radioactive Gaseous Waste Sampling and Analysis Program

	Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) (μCi/ml) (Note a)
I.	All Release Types as Listed Above in B, C, D, F, G, and H (During System Operation) (Note i)	Continuous (Note f)	W (Note d) Charcoal Sample	I-131	1×10^{-12}
		Continuous (Note f)	W (Note d) Particulate	Principal Gamma Emitters (Note g) (I-131, Others)	1×10^{-11}
		Continuous (Note f)	Q Composite Particulate Sample	Gross Alpha	1×10^{-11}
		Continuous (Note f)	Q Composite Particulate Sample	Sr-89, Sr-90	1×10^{-11}
J.	Condenser Vent Stack Continuous Iodine Sampler (Note j)	Continuous (Note k)	W (Note d) Charcoal Sample	I-131	1×10^{-12}

Table 3.2-2

Table Notation

a. The LLD is defined, for purposes of this surveillance, as the smallest concentration of radioactive material in a sample that will yield a net count above system background that will be detected with 95% probability with 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66\,S_b}{E \times V \times 2.22 \times 10^6 \times Y \times \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above (as microcurie per unit mass or volume),

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute),

E is the counting efficiency (as counts per disintegration),

V is the sample size (in units of mass or volume),

2.22×10^6 is the number of disintegrations per minute per microcurie,

Y is the fractional radiochemical yield (when applicable),

λ is the radioactive decay constant for the particular radionuclide, and

Δt is the elapsed time between midpoint of sample collection and time of counting.

Typical values of E, V, Y and Δt shall be used in the calculation.

It should be recognized that the LLD is defined as an "a priori" (before the fact) limit representing the capability of a measurement system and not as an "a posteriori" (after the fact) limit for a particular measurement.

b. Sampling and analysis shall also be performed following shutdown, startup, or a THERMAL POWER change exceeding 15 percent of RATED THERMAL POWER within one hour unless (1) analysis shows that the DOSE EQUIVALENT I-131 concentration in the primary coolant has not increased more than a factor of 3; and (2) the noble gas activity monitor shows that effluent activity has not increased by more than a factor of 3.

c. Tritium grab samples from the spent fuel pool area shall be taken at least once per 24 hours when the refueling canal is flooded.

Table 3.2-2

d. Charcoal cartridges and particulate filters shall be changed at least once per 7 days and analyses shall be completed within 48 hours after changing (or after removal from sampler).

e. Tritium grab samples shall be taken weekly from the spent fuel pool area whenever spent fuel is in the spent fuel pool.

f. The ratio of the sample flow rate to the sampled stream flow rate shall be known for the time period covered by each dose or dose rate calculation made in accordance with Controls 2.2.2.1, 2.2.2.2, and 2.2.2.3.

g. The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135 and Xe-138 for gaseous emissions and Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-137, Ce-141 and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be considered. Other gamma peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radioactive Effluent Release Report pursuant to TS 6.9.4.

h. Applicable only when condenser vacuum is established. Sampling and analysis shall also be performed following shutdown, startup, or a THERMAL POWER change exceeding 15 percent of RATED THERMAL POWER within one hour unless (1) analysis shows that the DOSE EQUIVALENT I-131 concentration in the primary coolant has not increased more than a factor of 3; and (2) the noble gas activity monitor shows that effluent activity has not increased by more than a factor of 3.

i. Gross Alpha, Sr-89, and Sr-90 analyses do not apply to the Fuel Handling Building ESF Air Treatment System.

j. If the Condenser Vent Stack Continuous Iodine Sampler is unavailable, then alternate sampling equipment will be placed in service within 48 hours or a report will be prepared and submitted within 30 days from the time the sampler is found or made inoperable which identifies (a) the cause of the inoperability, (b) the action taken to restore representative sampling capability, (c) the action taken to prevent recurrence, and (d) quantification of the release via the pathway during the period and comparison to the limits prescribed by Control 2.2.2.1.b.

k. Applicable only when condenser vacuum is established.

l. Applicable when liquid radwaste is moved or processed within the facility.

m. Iodine samples only required in the Chemical Cleaning Building when TMI-1 liquid radwaste is stored or processed in the facility.

3.2.3 Total Radioactive Effluents

3.2.3.1 Dose Calculation

3.2.3.1.1 Cumulative annual dose contributions from liquid and gaseous effluents shall be determined in accordance with Surveillances 3.2.1.2.1, 3.2.2.2.1, and 3.2.2.3.1, including direct radiation contributions from the Unit and from outside storage tanks, and in accordance with the methodology contained in the ODCM.

4.0 PART I REFERENCES

4.1 Title 10, Code of Federal Regulations, "Energy"

4.2 Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routing Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I, "Revision 1, October 1977

4.3 TMI-1 Technical Specifications, attached to Facility Operating License No. DPR-50

4.4 TMI-1 FSAR

PART II

TMI-2 RADIOLOGICAL EFFLUENT CONTROLS

PART II

Definitions

1.0 DEFINITIONS

DEFINED TERMS

1.1 The DEFINED TERMS of this section appear in capitalized type and are applicable throughout Part II of the ODCM.

PDMS

1.2 Post-Defueling Monitored Storage (PDMS) is that condition where TMI-2 defueling has been completed, the core debris removed from the reactor during the clean-up period has been shipped off-site and the facility has been placed in a stable, safe and secure condition.

ACTION

1.3 ACTION shall be those additional requirements specified as corollary statements to each control and shall be part of the controls.

OPERABLE - OPERABILITY

1.4 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment, that are required for the system, subsystem, train, component or device to perform its function(s), are also capable of performing their related support function(s).

CHANNEL CALIBRATION

1.5 A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds with necessary range and accuracy to known values of the parameter which the channel monitors. The CHANNEL CALIBRATION shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the CHANNEL FUNCTIONAL TEST. CHANNEL CALIBRATION may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.

CHANNEL CHECK

1.6 A CHANNEL CHECK shall be the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.

CHANNEL FUNCTIONAL TEST

1.7 A CHANNEL FUNCTIONAL TEST shall be:

a. Analog channels - the injection of a simulated signal into the channel as close to the primary sensor as practicable to verify OPERABILITY including alarm and/or trip functions.

b. Bistable channels - the injection of a simulated signal into the channel sensor to verify OPERABILITY including alarm and/or trip functions.

SOURCE CHECK

1.8 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

COMPOSITE SAMPLE

1.9 A COMPOSITE SAMPLE is a combination of individual samples obtained at regular intervals over a time period. Either the volume of each individual sample is proportional to the flow rate discharge at the time of sampling or the number of equal volume samples is proportional to the time period used to produce the composite.

GRAB SAMPLE

1.10 A GRAB SAMPLE is an individual sample collected in less than fifteen minutes.

BATCH RELEASE

1.11 A BATCH RELEASE is the discharge of fluid waste of a discrete volume.

CONTINUOUS RELEASE

1.12 A CONTINUOUS RELEASE is the discharge of fluid waste of a non-discrete volume, e.g., from a volume or system that has an input flow during the CONTINUOUS RELEASE.

SITE BOUNDARY

1.13 The SITE BOUNDARY used as the basis for the limits on the release of gaseous effluents is as defined in Section 2.1.2.2 and shown on Figure 2.1-3 of the TMI-1 FSAR. This boundary line includes portions of the Susquehanna River surface between the east bank of the river and Three Mile Island and between Three Mile Island and Shelley Island.

The SITE BOUNDARY used as the basis for the limits on the release of liquid effluents is as shown in Figure 1.1 in Part I of this ODCM.

FREQUENCY NOTATION

1.14 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.1. All Surveillance Requirements shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval.

TABLE 1.1

Frequency Notation

NOTATION	FREQUENCY
S (Shiftly)	At least once per 12 hours.
D (Daily)	At least once per 24 hours.
W (Weekly)	At least once per 7 days.
M (Monthly)	At least once per 31 days.
Q (Quarterly)	At least once per 92 days.
SA (Semi-Annually)	At least once per 184 days.
A (Annually)	At least once per 12 months.
E	At least once per 18 months.
N.A.	Not applicable.
P	Completed prior to each release

2.0 CONTROLS AND BASES

2.0.1 Controls and ACTION requirements shall be applicable during the conditions specified for each control.

2.0.2 Adherence to the requirements of the Control and/or associated ACTION within the specified time interval shall constitute compliance with the control. In the event the Control is restored prior to expiration to the specified time interval, completion of the ACTION statement is not required.

2.0.3 In the event the Control and associated ACTION requirements cannot be satisfied because of circumstances in excess of those addressed in the Control, initiate appropriate actions to rectify the problem to the extent possible under the circumstances, and submit a special report to the Commission pursuant to TMI-2 PDMS Technical Specification (Tech. Spec.) Section 6.8.2 within 30 days unless otherwise specified.

2.1 Radioactive Effluent Instrumentation

2.1.1 Radioactive Liquid Effluent Instrumentation

Radioactive Liquid Effluent Instrumentation is common between TMI-1 and TMI-2. Controls, applicability, and actions are specified in ODCM Part I, Control 2.1.1

2.1.2 Radioactive Gaseous Process and Effluent Monitoring Instrumentation

CONTROL:

The radioactive gaseous process and effluent monitoring instrumentation channels shown in Table 2.1-2 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Control 2.2.2.1 are not exceeded. The alarm/trip setpoints of these channels shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

APPLICABILITY: As shown in Table 2.1-2.

ACTION:

a. With a radioactive gaseous process or effluent monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above control, immediately suspend the release of radioactive effluent monitored by the affected channel or declare the channel inoperable.

b. With less than the minimum number of radioactive gaseous process or effluent monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 2.1-2. Exert best efforts to return the instrumentation to OPERABLE status within 30 days and, if unsuccessful, explain in the next Annual Effluent Release Report why the inoperability was not corrected in a timely manner.

BASES

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluent during actual or potential releases. The alarm/trip setpoints for these instruments shall be calculated in accordance with NRC approved methods in the ODCM to provide reasonable assurance that the annual releases are within the limits specified in 10 CFR 20.1301.

Table 2.1-2

Radioactive Gaseous Process and Effluent Monitoring Instrumentation

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
1.	Containment Purge Monitoring System			
a.	Noble Gas Activity Monitor (2HP-R-225)	1	NOTE 1	NOTE 2
b.	Particulate Monitor (2HP-R-225)	1	NOTE 1	NOTE 2
c.	Effluent System Flow Rate Measuring Device (2AH-FR-5907 Point 1)	1	NOTE 1	NOTE 3
2.	Station Ventilation System			
a.	Noble Gas Activity Monitor (2HP-R-219) or (2HP-R-219A)	1	NOTE 1	NOTE 2
b.	Particulate Monitor (2HP-R-219) or (2HP-R-219A)	1	NOTE 1	NOTE 2
c.	Effluent System Flow Rate Monitoring Device (2AH-FR-5907 Point 6)	1	NOTE 1	NOTE 3

NOTES:

1. During operation of the monitored system.

2. With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, secure Reactor Building Purge if in progress.

3. With flow rate monitoring instrumentation out of service, flow rates from the Auxiliary (2AH-FR-5907 Point 2), Fuel Handling (2AH-FR-5907 Point 4), Soiled Exhaust System (2AH-FR-5907 Point 5), and Reactor Buildings (2AH-FR-5907 Point 1) may be summed individually. Under these conditions, the flow rate monitoring device is considered operable. If the flow rates cannot be summed individually, they may be estimated using the maximum design flow for the exhaust fans, and the reporting requirements of Control 2.1.2.b are applicable.

2.2 Radioactive Effluent Controls

2.2.1 Liquid Effluent Controls

2.2.1.1 *Liquid Effluent Concentration*

CONTROL:

The concentration of radioactive material released at anytime from the unit to unrestricted areas shall be limited to ten times the concentrations specified in 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2.

APPLICABILITY: At all times

ACTION:

With the concentration of radioactive material released from the unit to unrestricted areas exceeding the above limits, immediately restore concentrations within the above limits.

BASES

This control is provided to ensure that the concentration of radioactive materials released in liquid waste effluent from the unit to unrestricted areas will be less than ten times the concentration levels specified in 10 CFR Part 20.1001-20.2401, Appendix B, Table 2. These Controls permit flexibility under unusual conditions, which may temporarily result in higher than normal releases, but still within ten times the concentrations, specified in 10 CFR 20. It is expected that by using this flexibility under unusual conditions, and exerting every effort to keep levels of radioactive material in liquid wastes as low as practicable, the annual releases will not exceed a small fraction of the annual average concentrations specified in 10 CFR 20. As a result, this Control provides reasonable assurance that the resulting annual exposure to an individual in off-site areas will not exceed the design objectives of Section II.A of Appendix I to 10 CFR Part 50, which were established as requirements for the cleanup of TMI-2 in the NRC's Statement of Policy of April 27, 1981.

2.2.1.2 Liquid Effluent Dose

CONTROL

The dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from the unit to the SITE BOUNDARY shall be limited:

a. During any calendar quarter to less than or equal to 1.5 mrem to the total body and to less than or equal to 5 mrem to any organ.

b. During any calendar year to less than or equal to 3 mrem to the total body and to less than or equal to 10 mrem to any organ.

APPLICABILITY: At all times

ACTION:

a. *With the calculated dose from the release of radioactive materials* in liquid effluents exceeding any of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions to be taken to reduce the releases of radioactive materials in liquid effluents during the remainder of the current calendar quarter and during the subsequent 3 calendar quarters so that the cumulative dose or dose commitment to any individual from such releases during these four calendar quarters is within 3 mrem to the total body and 10 mrem to any organ. This Special Report shall also include (1) the result of radiological analyses of the drinking water source, and (2) the radiological impact on finished drinking water supplies with regard to the requirements of 40 CFR 141, Safe Drinking Water Act.

BASES

This Control requires that the dose to offsite personnel be limited to the design objectives of Appendix I of 10 CFR Part 50. This will assure the dose received by the public during PDMS is equivalent to or less than that from a normal operating reactor. The limits also assure that the environmental impacts are consistent with those assessed in NUREG-0683, the TMI-2 Programmatic Environmental Impact Statement (PEIS). The ACTION statements provide the required flexibility under unusual conditions and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in liquid effluents will be kept "as low as is reasonably achievable". The dose calculations in the ODCM implement the requirements in Section III.A. of Appendix I that conformance with the guides of Appendix I is to be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The equations specified in the ODCM for calculating the doses due to the actual release rates of radioactive materials in liquid effluents are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of *Reactor Effluents for the Purpose of Evaluating Compliance with* 10 CFR Part 50, Appendix I," Revision 1, October, 1977, and Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I," April, 1977. NUREG-0133 provides methods for dose calculations consistent with Regulatory Guides 1.109 and 1.113.

2.2.1.3 Liquid Radwaste Treatment System

CONTROL:

The appropriate portions of the liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent from the unit to unrestricted areas would exceed 0.06 mrem to the total body or 0.2 mrem to any organ in any calendar month.

APPLICABILITY: At all times

ACTION:

a. With radioactive liquid waste being discharged without treatment and in excess of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which includes the following information:

1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystems, and the reason for inoperability,

2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and,

3. A summary description of action(s) taken to prevent a recurrence.

BASES

The requirement that the appropriate portions of this system (shared with TMI-1) be used, when specified, provides assurance that the releases of radioactive materials in liquid effluents will be kept as low as is reasonably achievable. This control implements the requirements of 10 CFR Part 50.36a, General Design Criterion 60 of Appendix A to 10 CFR Part 50 and the design objective given in Section II.D of Appendix I to 10 CFR Part 50. The intent of Section II.D. is to reduce effluents to as low as is reasonably achievable in a cost effective manner. This control satisfies this intent by establishing a dose limit which is a small fraction (25%) of Section II.A of Appendix I, 10 CFR Part 50 dose requirements. This margin, a factor of 4, constitutes a reasonable reduction.

2.2.2 Gaseous Effluent Controls

2.2.2.1 Gaseous Effluent Dose Rate

CONTROL:

The dose rate due to radioactive materials released in gaseous effluent from the site shall be limited to the following:

a. For noble gases: less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin, and

b. For tritium and all radionuclides in particulate form with half lives greater than 8 days: less than or equal to 1500 mrem/yr to any organ.

APPLICABILITY: At all times.

ACTION:

With the release rate(s) exceeding the above limits, immediately decrease the release rate to comply with the above limit(s).

BASES

The control provides reasonable assurance that the annual dose at the SITE BOUNDARY from gaseous effluent from all units on the site will be within the annual dose limits of 10 CFR Part 20 for unrestricted areas. At the same time, these Controls permit flexibility under unusual conditions, which may temporarily result in higher than the design objective levels, but still within the dose limits specified in 10 CFR 20 and within the design objectives of Appendix I to 10 CFR 50. It is expected that using this flexibility under unusual conditions, and by exerting every effort to keep levels of radioactive material in gaseous wastes as low as practicable, the annual releases will not exceed a small fraction of the annual dose limits specified in 10 CFR 20 and will not result in doses which exceed the design objectives of Appendix I to 10 CFR 50, which were endorsed as limits for the cleanup of TMI-2 by the NRC's Statement of Policy of April 27, 1981. These gaseous release rates provide reasonable assurance that radioactive material discharged in gaseous effluent will not result in the exposure of a MEMBER OF THE PUBLIC in an unrestricted area, either within or outside the SITE BOUNDARY, to annual average concentrations exceeding the values specified in Appendix B, Table 2 of 10 CFR Part 20. For MEMBERS OF THE PUBLIC who may at times be within the SITE BOUNDARY, the occupancy of the MEMBER OF THE PUBLIC will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the exclusion area boundary.

The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to a MEMBER OF THE PUBLIC at or beyond the SITE BOUNDARY to less than or equal to 500 mrem/year to the total body or to less than or equal to 3000 mrem/year to the skin. The absence of iodine ensures that the corresponding thyroid dose rate above background to a child via the inhalation pathway is less than or equal to 1500 mrem/yr (NUREG 1301), thus there is no need to specify dose rate limits for these nuclides.

2.2.2.2 Gaseous Effluents Dose-Noble Gases

CONTROL:

The air dose due to noble gases released in gaseous effluents from the unit to areas at and beyond the SITE BOUNDARY shall be limited to the following:

a. During any calendar quarter: less than or equal to 5 mrad for gamma radiation and less than or equal to 10 mrad for beta radiation and,

b. During any calendar year: less than or equal to 10 mrad for gamma radiation and less than or equal to 20 mrad for beta radiation.

APPLICABILITY: At all times.

ACTION:

a. With the calculated air dose from radioactive noble gases in gaseous effluents exceeding any of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

BASES

This control applies to the release of radioactive materials in gaseous effluents from TMI-2.

This control and associated action is provided to implement the requirements of Section II.B, III.A and IV.A of Appendix I, 10 CFR Part 50. The Control implements the guides set forth in Section II.B of Appendix I. The ACTION statements provide flexibility under unusual conditions and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents will be kept "as low as is reasonably achievable." The Surveillance Requirements implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through

the appropriate pathways is unlikely to be substantially underestimated. The dose calculation methodology and parameters established in the ODCM for calculating the doses due to the actual release rates of radioactive noble gases in gaseous effluents are consistent with the methodology provided in *Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine* Release of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water Cooled Reactors," Revision 1, July 1977. The ODCM equations provided for determining the air doses at and beyond the SITE BOUNDARY are based upon the historical average atmospheric conditions. NUREG-0133 provides methods for dose calculations consistent with Regulatory Guides 1.109 and 1.111.

2.2.2.3 Dose - Iodine-131, Iodine-133, Tritium, and Radionuclides In Particulate Form

CONTROL:

The dose to a MEMBER OF THE PUBLIC from Tritium and all radionuclides in particulate form with half lives greater than 8 days, in gaseous effluents released from the unit to areas at and beyond the SITE BOUNDARY shall be limited to the following:

a. During any calendar quarter: less than or equal to 7.5 mrem to any organ, and

b. During any calendar year: less than or equal to 15 mrem to any organ.

APPLICABILITY: At all times.

ACTION:

With the calculated dose from the release of Tritium and radionuclides in particulate form with half lives greater than 8 days, in gaseous effluents exceeding any of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which identifies the cause(s) for exceeding the limit and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

BASES

This control applies to the release of radioactive materials in gaseous effluents from TMI-2.

This control and associated action is provided to implement the requirements of Section II.C, III.A and IV.A of Appendix I, 10 CFR Part 50. The Controls are the guides set forth in Section II.C of Appendix I. The ACTION statement provides flexibility during unusual conditions and at the same time implements the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive materials in gaseous effluents will be kept "as low as is reasonably achievable." The ODCM calculational methods specified in the surveillance requirements implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of a MEMBER OF THE PUBLIC through appropriate pathways is unlikely to be substantially underestimated. The ODCM calculational methodology and parameters for calculating the doses due to the actual release rates of the subject materials are consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October, 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors," Revision 1, July, 1977. These equations also provide for determining the actual doses based upon the historical average atmospheric conditions. The release rate controls for iodine-131, iodine-133, tritium and radionuclides in particulate form with half lives greater than 8 days are dependent upon the existing radionuclide pathways to man, in areas at and beyond the SITE BOUNDARY. The pathways that were examined in the development of these calculations were: 1) individual inhalation of airborne radionuclides, 2) deposition of radionuclides onto green leafy vegetation with subsequent consumption by man, 3) deposition onto grassy areas where milk animals and meat producing animals graze with consumption of the milk and meat by man, and 4) deposition on the ground with subsequent exposure of man. The absence of iodines at the site eliminates the need to specify dose limits for these nuclides.

2.2.2.4 Ventilation Exhaust Treatment System

CONTROL

The VENTILATION EXHAUST TREATMENT SYSTEM shall be OPERABLE. The appropriate portions of the VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the monthly projected doses due to gaseous effluent releases from the site would exceed 0.3 mrem to any organ.

APPLICABILITY: At all times.

ACTION:

a. With the VENTILATION EXHAUST TREATMENT SYSTEM inoperable for more than a month or with gaseous waste being discharged without treatment and in excess of the above limits, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which includes the following information:

1. Identification of the inoperable equipment or subsystems and the reason for inoperability,

2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and

3. A summary description of action(s) taken to prevent a recurrence.

BASES

The use of the VENTILATION EXHAUST TREATMENT SYSTEM ensures that gaseous effluents are treated as appropriate prior to release to the environment. The appropriate portions of this system provide reasonable assurance that the releases of radioactive materials in gaseous effluents will be kept "as low as is reasonably achievable." This control implements the requirements of 10 CFR Part 50.36a, General Design Criterion 60 of Appendix A to 10 CFR Part 50, and the design objectives given in Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the systems were specified as a suitable fraction of the guide set forth in Sections II.B and II.C of Appendix I, 10 CFR Part 50, for gaseous effluents.

2.2.3 Total Radioactive Effluent Controls

2.2.3.1 Total Dose

CONTROL:

The annual (calendar year) dose or dose commitment to any MEMBER OF THE PUBLIC, due to releases of radioactivity and to radiation from uranium fuel cycle sources shall be limited to less than or equal to 25 mrem to the total body or any organ except the thyroid, which shall be limited to less than or equal to 75 mrem.

APPLICABILITY: At all times.

ACTION:

With the calculated dose from the release of radioactive materials in liquid or gaseous effluents exceeding twice the limits of Controls 2.2.1.2.a, 2.2.1.2.b, 2.2.2.2.a, 2.2.2.2.b, 2.2.2.3.a, or, 2.2.2.3.b, calculations should be made including direct radiation contributions from the unit and from outside storage tanks to determine whether the above limits of Control 2.2.3.1 have been exceeded. If such is the case, prepare and submit to the NRC Region I Administrator within 30 days, a Special Report which defines the corrective action to be taken to reduce subsequent releases to prevent recurrence of exceeding the above limits and includes the schedule for achieving conformance with the above limits. This Special Report, as defined in 10 CFR Part 20.2203(b), shall include an analysis which estimates the radiation exposure (dose) to a MEMBER OF THE PUBLIC from uranium fuel cycle sources, including all effluent pathways and direct radiation, for the calendar year that includes the release(s) covered by this report. It shall also describe levels of radiation and concentrations of radioactive material involved, and the cause of the exposure levels or concentrations. If the estimated dose(s) exceed the above limits, and if the release condition resulting in violation of 40 CFR 190 has not already been corrected, the Special Report shall include a request for a variance in accordance with the provisions of 40 CFR 190. Submittal of the report is considered a timely request, and a variance is granted until staff action on the request is complete.

BASES

This control is provided to meet the dose limitations of 40 CFR Part 190 that have been incorporated into 10 CFR Part 20.1301(d). This control requires the preparation and submittal of a Special Report whenever the calculated doses from plant generated radioactive effluents and direct radiation exceed 25 mrem to the total body or any organ, except the thyroid, which shall be limited to less than or equal to 75 mrem. For sites containing up to 4 reactors, it is highly unlikely that the resultant dose to a MEMBER OF THE PUBLIC will exceed the dose limits of 40 CFR Part 190 if the individual reactors remain within twice the dose design objectives of Appendix I, and if direct radiation doses from the reactor units and outside storage tanks are kept small. The Special Report will describe a course of action that should result in the limitation of the annual dose to a MEMBER OF THE PUBLIC to within the 40 CFR Part 190 limits. For the purposes of the Special Report, it may be assumed that the dose commitment to the member of the public from other uranium fuel cycle sources is negligible, with the exception that dose contributions from other nuclear fuel cycle facilities at the same site or within a radius of 8 km must be considered. If the dose to any member of the public is estimated to exceed the requirements of 40 CFR Part 190, the Special Report with a request for a variance (provided the release conditions resulting in violation of 40 CFR Part 190 have not already been corrected), in accordance with the provisions of 40 CFR Part 190.11 and 10 CFR Part 20.2203(b), is considered to be a timely request and fulfills the requirements of 40 CFR Part 190 until NRC staff action is completed. The variance only relates to the limits of 40 CFR Part 190, and does not apply in any way to the other requirements for dose limitation of 10 CFR Part 20, as addressed in Controls 2.2.1.1 and 2.2.2.1. An individual is not considered a MEMBER OF THE PUBLIC during any period in which he/she is engaged in carrying out any operation that is part of the nuclear fuel cycle.

3.0 SURVEILLANCES

3.0.1 Surveillance Requirements shall be applicable during the conditions specified for individual Controls unless otherwise stated in an individual Surveillance Requirement. The Surveillance Requirements shall be performed to demonstrate compliance with the OPERABILITY requirements of the Control.

3.0.2 Each Surveillance Requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval.

3.0.3 Failure to perform a Surveillance Requirement within the time interval specified in Section 3.0.2 shall constitute non-compliance with OPERABILITY requirements for a Control. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.

3.1 Radioactive Effluent Instrumentation

3.1.1 Radioactive Liquid Effluent Instrumentation

Surveillance Requirements

Radioactive Liquid Effluent Instrumentation is common between TMI-1 and TMI-2. Surveillances for this instrumentation are specified in ODCM Part I, Surveillance 3.1.1.

3.1.2 Radioactive Gaseous Process and Effluent Monitoring Instrumentation

SURVEILLANCE REQUIREMENTS

3.1.2.1 Each radioactive gaseous process or effluent monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, SOURCE CHECK, CHANNEL CALIBRATION, and CHANNEL TEST operations at the frequencies shown in Table 3.1-2.

Table 3.1-2

Radioactive Gaseous Process and Effluent Monitoring Instrumentation Surveillance Requirements

	INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	APPLICABILITY
1.	Containment Purge Monitoring System				
a.	Noble Gas Activity Monitor (2HP-R-225)	D	E	M	NOTE 1
b.	Particulate Sampler (2HP-R-225)	W	N/A	N/A	NOTE 1
2.	Station Ventilation Monitoring System				
a.	Noble Gas Activity Monitor (2HP-R-219) and (2HP-R-219A)	D	E	M	NOTE 1
b.	Particulate Sampler (2HP-R-219) and (2HP-R-219A)	W	N/A	N/A	NOTE 1

NOTES:

1. During operation of the monitored system.

3.2 Radioactive Effluents

3.2.1 Liquid Effluents

SURVEILLANCE REQUIREMENTS

3.2.1.1 Concentration

3.2.1.1.1 The radioactivity content of each batch of radioactive liquid waste shall be determined by sampling and analysis in accordance with Table 3.2-1. The results of analyses shall be used with the calculational methods in the ODCM to assure that the concentration at the point of release is maintained within the limits of Control 2.2.1.1.

3.2.1.1.2 Analysis of samples composited from batch releases shall be performed in accordance with Table 3.2-1. The results of the analysis shall be used with the calculational methods in the ODCM to assure that the concentrations at the point of release were maintained within the limits of Control 2.2.1.1.

3.2.1.1.3 The radioactivity concentration of liquids discharged from continuous release points shall be determined by collection and analysis of samples in accordance with Table 3.2-1. The results of the analysis shall be used with the calculational methods of the ODCM to assure that the concentration at the point of release is maintained within the limits of Control 2.2.1.1.

3.2.1.2 Dose Calculations

3.2.1.2.1 Cumulative dose contributions from liquid effluents shall be determined in accordance with the Offsite Dose Calculation Manual (ODCM) at least once a month.

3.2.1.3 Dose Projections

3.2.1.3.1 Doses due to liquid releases shall be projected at least once a month, in accordance with the ODCM.

TABLE 3.2-1

Radioactive Liquid Waste Sampling and Analysis (4, 5)

A. Liquid Releases

Sampling Frequency	Type of Activity Analysis	Detectable Concentration (3)
P Each Batch	Individual Gamma H-3	5E-7 μCi/ml (2) 1E-5 μCi/ml
Q Quarterly Composite (1)	Gross Alpha Sr-90	1E-7 μCi/ml 5E-8 μCi/ml

NOTES:

(1) A COMPOSITE SAMPLE is one in which the quantity of liquid sampled is proportional to the quantity of liquid waste discharged from the plant.

(2) For certain mixtures of gamma emitters, it may not be possible to measure radionuclides in concentrations near this sensitivity limit when other nuclides are present in the sample in much greater concentrations. Under these circumstances, it will be more appropriate to calculate the concentrations of such radionuclides using measured ratios with those radionuclides which are routinely identified and measured.

(3) The detectability limits for radioactivity analysis are based on the technical feasibility and on the potential significance in the environment of the quantities released. For some nuclides, lower detection limits may be readily achievable and when nuclides are measured below the stated limits, they should also be reported.

(4) The results of these analyses should be used as the basis for recording and reporting the quantities of radioactive material released in liquid effluents during the sampling period. In estimating releases for a period when analyses were not performed, the average of the two adjacent data points spanning this period should be used. Such estimates should be included in the effluent records and reports; however, they should be clearly identified as estimates, and the method used to obtain these data should be described.

(5) Deviations from the sampling/analysis regime will be noted in the report specified in ODCM Part IV.

3.2.2 Gaseous Effluents

SURVEILLANCE REQUIREMENTS

3.2.2.1 Dose Rates

3.2.2.1.1 The dose rate due to noble gases in gaseous effluents shall be determined to be within the limits of Control 2.2.2.1.a in accordance with the methods and procedures of the ODCM.

3.2.2.1.2 The dose rate of radioactive materials, other than noble gases, in gaseous effluents shall be determined to be within the limits of Control 2.2.2.1.b in accordance with methods and procedures of the ODCM by obtaining representative samples and performing analyses in accordance with the sampling and analysis program, specified in Table 3.2-2.

3.2.2.2 Dose, Noble Gas

3.2.2.2.1 Cumulative dose contributions from noble gas effluents for the current calendar quarter and current calendar year shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM) monthly.

3.2.2.3 Dose, Tritium and Radionuclides In Particulate Form

3.2.2.3.1 Cumulative dose contributions from Tritium and radionuclides in particulate form with half lives greater than 8 days for the current calendar quarter and current calendar year shall be determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM) monthly.

3.2.2.4 Ventilation Exhaust Treatment

3.2.2.4.1 Doses due to gaseous releases from the unit shall be projected monthly in accordance with the ODCM.

TABLE 3.2-2

Radioactive Gaseous Waste Sampling and Analysis (3)

SAMPLE POINT	SAMPLE TYPE	SAMPLING FREQUENCY	TYPE OF ACTIVITY ANALYSIS	DETECTABLE CONCENTRATION(1)(a)
Reactor Building Purge Releases	Gas	P	H-3	1E-6 μCi/cc
		Each Purge	Individual Gamma Emitters	1E-4 μCi/cc (2)
Unit Exhaust Vent Release Points	Gas	M	H-3	1E-6 μCi/cc
		Monthly	Individual Gamma Emitters	1E-4 μCi/cc (2)
	Particulates	W Weekly	Individual (b) Gamma Emitters	1E-10 μCi/cc (2)
		M Monthly Composite	Sr-90	1E-11 μCi/cc
		M Monthly Composite	Gross Alpha Emitters	1E-11 μCi/cc
Reactor Building Breather	Particulates	SA Semi-Annually	Indv. Gamma Emitters (b)	1E-10 μCi/cc (2)
			Sr-90	1E-11 μCi/cc
			Gross Alpha Emitters	1E-11 μCi/cc

(1) The above detectability limits are based on technical feasibility and on the potential significance in the environment of the quantities released. For some nuclides, lower detection limits may be readily achievable and when nuclides are measured below the stated limits, they should also be reported.

(2) For certain mixtures of gamma emitters, it may be possible to measure radionuclides at levels near their sensitivity limits when other nuclides are present in the sample at much higher levels. Under these circumstances, it will be more appropriate to calculate the levels of such radionuclides using observed ratios in the gaseous component in the reactor coolant for those radionuclides which are measurable.

(3) Deviations from the sampling and analysis regime will be noted in the report specified in ODCM Part IV.

TABLE 3.2-2

Radioactive Gaseous Waste Sampling and Analysis Program

Table Notation

a. The LLD is the smallest concentration of radioactive material in a sample that will be detected with 95% probability with 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66\ S_b}{E \times V \times 2.22 \times 10^6 \times Y \times \exp(-\lambda \Delta t)}$$

Where

LLD is the lower limit of detection as defined above (as picocurie per unit mass or volume).

S_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute).

E is the counting efficiency (as counts per transformation),

V is the sample size (in units of mass or volume),

2.22 is the number of transformations per minute per picocurie,

Y is the fractional radiochemical yield (when applicable),

λ is the radioactive decay constant for the particular radionuclide, and

Δt is the elapsed time between midpoint of sample collection and time of counting (for plant effluents, not environmental samples),

The value of S_b used in the calculation of the LLD for a detection system shall be based on the actual observed variance of the background counting rate or of the counting rate of the blank samples (as appropriate) rather than on an unverified theoretically predicted variance. In calculating the LLD for a radionuclide determined by gamma-ray spectrometry, the background shall include the typical contributions of other radionuclides normally present in the samples. Typical values of E, V, Y, and Δt shall be used in the calculation. The background count rate is calculated from the background counts that are determined to be with ± one FWHM (Full-Width-at-Half-Maximum) energy band about the energy of the gamma-ray peak used for the quantitative analysis for that radionuclide.

TABLE 3.2-2

b. The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141 and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD for the analyses shall be reported as "less than" the nuclide's LLD and shall not be reported as being present at the LLD level for that nuclide. The "less than" values shall not be used in the required dose calculations.

3.2.3 Total Radioactive Effluents

3.2.3.1 *Dose Calculation*

3.2.3.1.1 Cumulative annual dose contributions from liquid and gaseous effluents shall be determined in accordance with Surveillances 3.2.1.2.1, 3.2.2.2.1, and 3.2.2.3.1, including direct radiation contributions from the Unit and from outside storage tanks, and in accordance with the methodology contained in the ODCM.

4.0 PART II REFERENCES

4.1 *NUREG-0683, "Final Programmatic Environmental Impact Statement related to decontamination and disposal of radioactive wastes resulting from March 28, 1979, accident Three Mile Island Nuclear Station, Unit 2," March 1981, and its supplements.*

4.2 TMI-2 PDMS Technical Specifications, attached to Facility License No. DPR-73

4.3 Title 10, Code of Federal Regulations, "Energy"

4.4 *"Statement of Policy Relative to the NRC Programmatic Environmental Impact Statement on the* Cleanup of Three Mile Island Unit 2," dated April 27, 1981

4.5 Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977

4.6 DOE/TIC-27601, Atmospheric Science and Power Reduction

4.7 TMI-1 Technical Specifications, attached to Facility Operating License No. DPR-50

4.8 PDMS - SAR

PART III

EFFLUENT DATA AND CALCULATIONAL METHODOLOGIES

1.0 LIQUID EFFLUENT MONITORS

1.1 TMI-1 and TMI-2 Liquid Radiation Monitor Set Points

The liquid effluent off-line monitors are set such that the concentration(s) of radionuclides in the liquid effluents will not exceed ten times the concentrations specified in 10 CFR 20, Appendix B Table 2, Col 2. Table 1.1 lists the Liquid Effluent Release Points and their parameters; Figure 1.1 provides a Liquid Release Pathway Diagram.

To meet the above limit, the alarm/trip set points for liquid effluent monitors and flow measuring devices are set in accordance with the following equation:

$$\frac{c * f}{F + f} \leq C \qquad \text{(eq 1.1)}$$

where:

C = ten times the effluent concentration of 10 CFR 20 for the site, in μCi/ml.

c = the set point, in μCi/ml, of the liquid effluent monitor measuring the radioactivity concentration in the effluent line prior to dilution and release. The set point is inversely proportional to the maximum volumetric flow of the effluent line and proportional to the minimal volumetric flow of the dilution stream plus the effluent stream. The alert set point value is set to ensure that advance warning occurs prior to exceeding any limits. The high alarm set point value is such that if it were exceeded, it would result in concentrations exceeding ten times the 10 CFR 20 concentrations for the unrestricted area.

f = flow set point as measured at the radiation monitor location, in volume per unit time, but in the same units as F below.

F = flow rate of dilution water measured prior to the release point, in volume per unit time.

The set point concentration is reduced such that concentration contributions from multiple release points would not combine to exceed ten times 10 CFR 20 concentrations. The set point concentration is converted to set point scale units using appropriate radiation monitor calibration factors.

This section of the ODCM is implemented by the Radiation Monitor System Set Points procedure and, for batch releases, the Releasing Radioactive Liquid Waste procedure.

1.2 TMI Liquid Effluent Release Points and Liquid Radiation Monitor Data

TMI-1 has two required liquid radiation monitors. These are RM-L6 and RM-L12. These liquid release point radiation monitors and sample points are shown in Table 1.1. (The TMI outfall radiation monitor, RM-L7, is also listed for information only.)

TMI-2 does not have any required liquid radiation monitors, but does utilize RM-L12, and RM-L7 for release of liquid waste.

1.2.1 RM-L6

RM-L6 is an off-line system, monitoring radioactive batch discharges from the TMI-1 liquid radwaste system (see Figure 1.1). These batch releases are sampled and analyzed per site procedures prior to release. The release rate is based on releasing one of two Waste Evaporator Condensate Storage Tanks (WECST) at a flow which will add less than 10%, of ten times the 10 CFR 20 concentrations [20% for H-3] to radionuclide concentrations in the unrestricted area, including conservative default values for Sr-89, Sr-90, and Fe-55.

The release flow rate used is the most restrictive of two flow rates calculated for each liquid batch release, per the approved plant procedure.

Two Dilution Factors (DF) are calculated to ultimately calculate the batch release flow rate. These two DF's are calculated to insure each radionuclide released to the unrestricted area is less than 10 percent of ten times the 10CFR20 radionuclide concentrations, (20% for H-3), and to ensure each liquid batch release boron concentration to the river will not exceed 0.7 ppm.

The maximum release flow rate is then calculated by dividing the most restrictive (largest) DF into 90 percent of the current dilution flow rate of the Mechanical Draft Cooling Tower (MDCT). This conservative flow rate is then multiplied by 0.9 for the allowable flow rate.

- ***Calculation of the 10CFR20 concentration DF:***

  $DF_1 = \Sigma_i (SA_i) \div$ (10% [20% for H-3] of ten times the 10CFR20 concentration)

  SA = Specific Activity of each identified radionuclide

- **Calculation of Boron DF:**

  DF_2 = Actual Tank Boron Concentration ÷ 0.7.

- **Maximum release flow rate calculation:**

  Max Flow = [(MDCT flow gpm * 0.9) ÷ (Most Restrictive DF)] * 0.9

The dilution flow rate used is the current flow rate at the site. The minimum dilution flow rate is 5000 gpm per the TMI-1 FSAR. This ensures this batch release will meet the following equation.

$$\Sigma(C_i/X_i) + (C_{H\text{-}3}/2X_{H\text{-}3}) \leq 0.1, \qquad \text{(eq 1.2)}$$

where: C_i = diluted concentration of the i^{th} radionuclide, other than H-3

X_i = Ten times the concentration for that radionuclide in the unrestricted area (10 CFR 20, App. B, Table 2, Col. 2). A value of 3E-3 μCi/ml for dissolved and entrained noble gases shall be used.

C_{H-3} =diluted concentration of H-3

X_{H-3} =Ten times the concentration for H-3 in the restricted area (10 CFR 20, App. B, Table 2, Col. 2).

The set points for RM-L6 are based on the maximum release rate (30 gpm), a minimum dilution flow (5000 gpm), and 25% of ten times the 10CFR20 concentration for Cs-137, which is the most limiting radionuclide at a concentration of 1.0E-5 uCi/ml. These inputs are used in Equation 1.1 to determine the RM-L-6 High Alarm setpoint for all radionuclides being released. A high alarm on RM-L-6 will close valve WDL-V-257 and terminate any WECST releases to the environment.

1.2.2 RM-L12

RM-L12 is an off-line system, monitoring periodic combined releases from the Industrial Waste Treatment System/Industrial Waste Filtration System (IWTS/IWFS). The input to IWTS/IWFS originates in TMI-2 sumps, (see Figures 1.1 and 1.2) and the TMI-1 Turbine Building sump (see Figure 1.1). The set points are based on the maximum release rate from both IWTS and IWFS simultaneously, (see Figure 1.1) a minimum dilution flow rate, and 50% of ten times the 10CFR20 concentration for Cs-137, which is the most limiting radionuclide at a concentration of 1E-5 μCi/ml. These inputs are used in equation 1.1 to determine the RM-L12 High Alarm set point for all radionuclides being released. A high alarm on RM-L12 will close IWTS and IWFS release valves and trip release pumps to stop the release.

1.2.3 RM-L10

RM-L10 was a NaI detector submerged in the TMI-1 Turbine Building Sump. This detector has been removed from service.

1.2.4 RM-L7

RM-L7 is not an ODCM required liquid radiation monitor. RM-L7 is an off-line system, monitoring the TMI outfall to the Susquehanna River (see Figures 1.1 and 1.2). This monitor is the final radiation monitor for TMI-1 and TMI-2 normal liquid effluent releases.

1.3 Control of Liquid Releases

TMI liquid effluent releases are controlled to less than ten times the 10CFR20 concentrations by limiting the percentage of this limit allowable from the two TMI liquid release points. RM-L6 and effluent sampling limit batch releases to less than or equal to 25% for all radionuclides, and RM-L12 and effluent sampling limit releases from TMI-1 and TMI-2 to less than or equal to 50% for Cs-137.

These radiation monitor set points also include built in meter error factors to further ensure that TMI liquid effluent releases are less than ten times the 10CFR20 concentrations to the environment.

The radioactivity content of each batch of radioactive liquid waste is determined prior to release by sampling and analysis in accordance with ODCM Part I Table 3.2-1 or ODCM Part II, Table 3.2-1. The results of analyses are used with the calculational methods in Section 1.1, to assure that the concentration at the point of release is maintained within the ODCM Part I Control 2.2.1.1, and ODCM Part II Control 2.2.1.1.

Post-release analysis of samples composited from batch releases are performed in accordance with ODCM Part I Table 3.2-1 or ODCM Part II Table 3.2-1. The results of the previous post-release analysis shall be used with the calculational methods in the ODCM to assure that the concentrations at the point of release were maintained within the ODCM Part I Control 2.2.1.1, and ODCM Part II Control 2.2.1.1.

The radioactivity concentration of liquids discharged from continuous release points are determined by collection and analysis of samples in accordance with ODCM Part I Table 3.2-1, or ODCM Part II Table 3.2-1. The results of the analysis are used with the calculational methods of the ODCM to assure that the concentration at the point of release is maintained within the ODCM Part I Control 2.2.1.1, and ODCM Part II Control 2.2.1.1.

TABLE 1.1

TMI Liquid Release Point and Liquid Radiation Monitor Data

LIQUID RADIATION MONITOR (DETECTOR)	LOCATION	LIQUID RELEASE POINT (Maximum Volume)	DISCHARGE FLOW RECORDER	RELEASE TERMINATION INTERLOCK (YES/NO) VALVES
RM-L6 (NaI)	281' Elevation TMI-1 Auxiliary Bldg	WECST Batch Releases (8000 gal.)	FT-84	YES WDL-V257
RM-L7 (NaI) **	South end of TMI-1 MDCT	Station Discharge TMI-1 and TMI-2,	FT-146	YES WDL-V257 *WDL-R-1311
RM-L12 (NaI)	IWFS Building NW Corner	IWTS/IWFS Continuous Releases (300,000/ 80,000 gal.)	FT-342/ FT-373	YES IW-V73, IW-P16,17,18 IW-V279, IW-P29,30

* WDL-R-1311 has been flanged off as a TMI-2 liquid outfall.
** *RM-L7 is not an ODCM required liquid radiation monitor.*

TABLE 1.2

TMI-2 Sump Capacities

Sump	Total Capacity Gallons	Gallons per Inch
Turbine Building Sump	1346	22.43
Circulating Water Pump House Sump	572	10.59
Control Building Area Sump	718	9.96
Tendon Access Galley Sump	538	9.96
Control to Service Building Sump	1346	22.43
Contaminated Drain Tank Room Sump	135	3.80
Chlorinator House Sump	---	---
Water Treatment Sump**	1615	22.43
Air Intake Tunnel Normal Sump	700	---
Air Intake Tunnel Emergency Sump	100000	766.00
Condensate Polisher Sump*	2617	62.31
Sludge Collection Sump**	1106	26.33
Heater Drain Sump	---	---
Solid Waste Staging Facility Sump	1476	24.00
Auxiliary Building Sump	10102	202.00
Decay Heat Vault Sump	479	10.00
Building Spray Vault Sump	479	10.00

* Condensate Polisher Sump is deactivated and in PDMS condition.
** The Water Treatment and Sludge Collection Sumps will be deactivated for PDMS.

Figure 1.1
TMI-1 Liquid Effluent Pathways

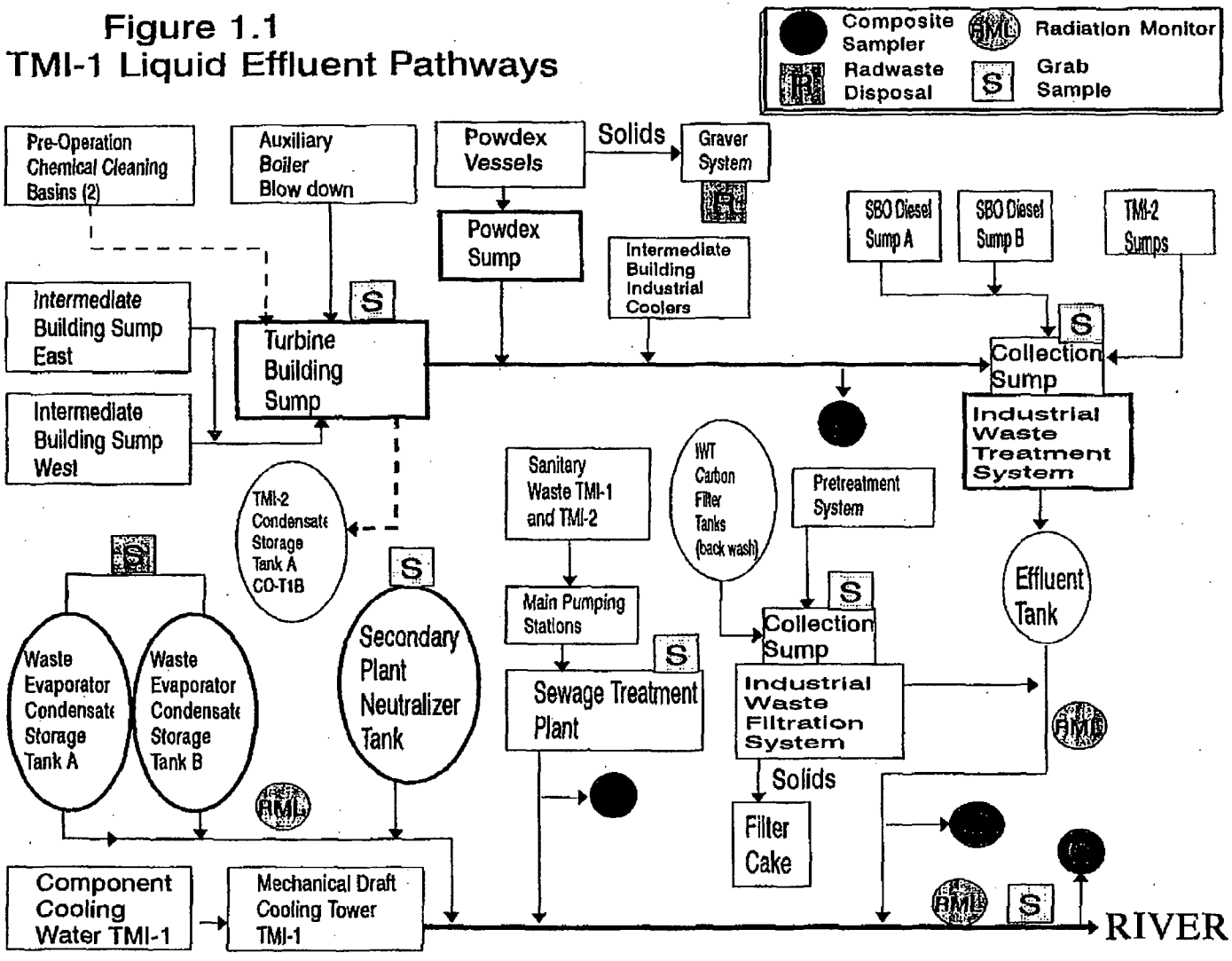
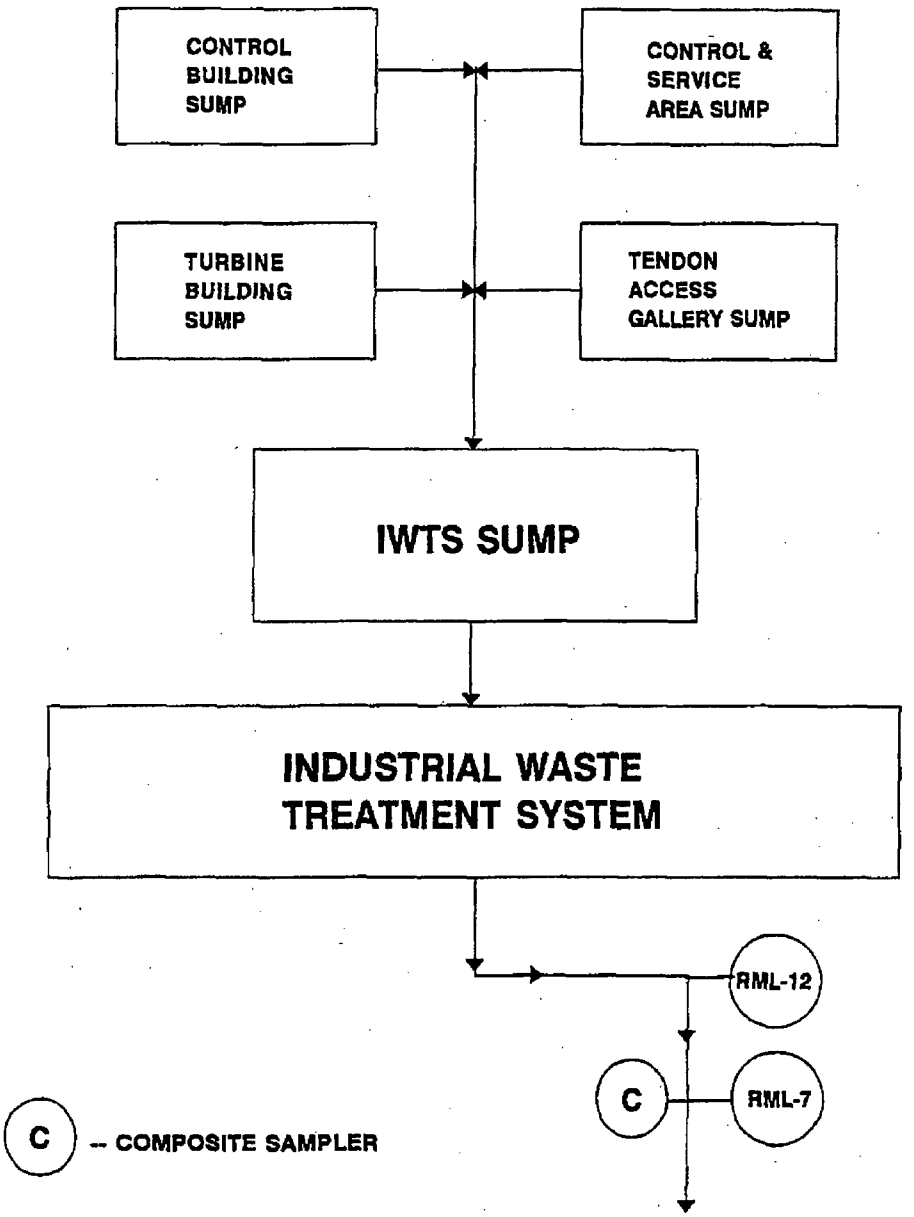


FIGURE 1.2

TMI-2 Liquid Effluent Pathways



2.0 LIQUID EFFLUENT DOSE ASSESSMENT

2.1 Liquid Effluents - 10 CFR 50 Appendix I

The dose from liquid effluents results from the consumption of fish and drinking water. The location of the nearest potable water intake is PP&L Brunner Island Steam Electric Station located downstream of TMI. The use of the flow of the Susquehanna River as the dilution flow is justified based on the complete mixing in the river prior to the first potable water supply, adequately demonstrated by flume tracer die studies and additional liquid effluent release studies conducted using actual TMI-1 tritium releases. Other pathways contribute negligibly at Three Mile Island. The dose contribution from all radionuclides in liquid effluents released to the unrestricted area is calculated using the following expression:

$$\text{Dose } j = \sum_i (\Delta t)\, X\, (C_i)\, X \left[\left(AW_{ij}\, X\, \frac{f}{FR} \right) + \left(AF_{ij}\, X\, \frac{f}{FD}\, X\, \frac{1}{DF} \right) \right] \qquad \text{(eq 2.1)}$$

where:

Dose j = the cumulative dose commitment to the total body or any organ, j, from the liquid effluents for the total time period, in mrem.

Δt = the length of the time period of actual releases, over which C_i and f are averaged for all liquid releases, in hours.

C_i = the average concentration of radionuclide, i, in undiluted liquid effluent during time period Δt from any liquid release, in μCi/ml.

NOTE

For Fe-55, Sr-89, Sr-90, prior to batch releases conservative concentration values will be used in the initial dose calculation based on similar past plant conditions. LLD values are not used in dose calculations.

f = undiluted liquid waste flow, in gpm.

FD = plant dilution water flowrate during the period of release, in gpm

FR = actual river flowrate during the period of release or average river flowrate for the month the release is occurring, in gpm.

DF = dilution factor as a result of mixing effects in the near field of the discharge structure of 0.2 (NUREG 0133) or taken to be 5 based on the inverse of 0.2.

AWij and AFij = the site-related ingestion dose commitment factor to the total body or any organ, j, for each identified principle gamma and beta emitter, in mrem/hr per μCi/ml. AW is the factor for the water pathway and AF is the factor for the fish pathway.

Values for AWij are determined by the following equation:

$$AW_{ij} = (1.14E5) \times (U_w) \times (DF_{ij}) \qquad \text{(eq 2.2)}$$

where:

1.14E5 = (1.0E6 pCi/μCi) x (1.0E3 ml/kg) ÷ (8760 hr/yr)

Uw = Water consumption rate for adult is 730 kg/yr (Reg. Guide 1.109, Rev. 1).

DFij = ingestion dose conversion factor for radionuclide, i, for adults total body and for "worst case" organ, j, in mrem/pCi, from Table 2.1 (Reg. Guide 1.109)

Values for AF_{ij} are determined by the following equation:

AFij = (1.14E5) x (Uf) x (DFij) x (BFi) (eq 2.2.2)

where:

1.14E5 = defined above

Uf = adult fish consumption, assumed to be 21 kg/yr (Reg. Guide 1.109, Rev. 1).

DFij = ingestion dose conversion factor for radionuclide, i, for adult total body and for "worst case" organ, j, in mrem/pCi, from Table 2.1 (Reg. Guide 1.109, Rev. 1).

BFi = Bioaccumulation factor for radionuclide, i, in fish, in pCi/kg per pCi/L from Table 2.2 (Reg. Guide 1.109, Rev. 1).

2.2 TMI Liquid Radwaste System Dose Calcs Once/Month

ODCM Part I Control 2.2.1.3 and TMI-2 PDMS Tech Spec Section 6.7.4.a.6 requires that appropriate portions of the liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the monthly projected doses due to the liquid effluent releases from each unit to unrestricted areas would exceed 0.06 mrem to the total body or 0.2 mrem to any organ in any calendar month. The following calculational method is provided for performing this dose projection.

At least once per month, the total dose from all liquid releases for the quarter-to-date will be divided by the number of days into the quarter and multiplied by 31. Also, this dose projection shall include the estimated dose due to any anticipated unusual releases during the period for which the projection is made. If this projected dose exceeds 0.06 mrem total body or 0.2 mrem any organ, appropriate portions of the Liquid Radwaste Treatment System, as defined in Section 3.1, shall be used to reduce radioactivity levels prior to release.

At the discretion of Radiological Engineering, time periods other than the current quarter-to-date may be used to project doses if the dose per day in the current quarter-to-date is not believed to be representative of the dose per day projected for the next month.

2.3 Alternative Liquid Dose Calculational Methodology

As an alternative, models in, or based upon, those presented in Regulatory Guide 1.109 (Rev. 1) may be used to make a comprehensive dose assessment. Default parameter values from Reg. Guide 1.109 (Rev. 1) and/or actual site specific data are used where applicable.

As an alternative dose calculational methodology TMI calculates doses using SEEDS (simplified environmental effluent dosimetry system).

The onsite and SEEDS calculational models use actual liquid release data with actual monthly Susquehanna River flow data to assess the dispersion of effluents in the river.

TABLE 2.1

Liquid Dose Conversion Factors (DCF): DF_{ij}

Ingestion Dose Factors for Adults*
(MREM Per PCI Ingested)

NUCLIDE		BONE	LIVER	T. BODY	THYROID	KIDNEY	LUNG	GI-LLI
H	3	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07
C	14	2.84E-06	5.68E-07	5.68E-07	5.68E-07	5.68E-07	5.68E-07	5.68E-07
NA	24	1.70E-06	1.70E-06	1.70E-06	1.70E-06	1.70E-06	1.70E-06	1.70E-06
CR	51	NO DATA	NO DATA	2.66E-09	1.59E-09	5.86E-10	3.53E-09	6.69E-07
MN	54	NO DATA	4.57E-06	8.72E-07	NO DATA	1.36E-06	NO DATA	1.40E-05
MN	56	NO DATA	1.15E-07	2.04E-08	NO DATA	1.46E-07	NO DATA	3.67E-06
FE	55	2.75E-06	1.90E-06	4.43E-07	NO DATA	NO DATA	1.06E-06	1.09E-06
FE	59	4.34E-06	1.02E-05	3.91E-06	NO DATA	NO DATA	2.85E-06	3.40E-05
CO	58	NO DATA	7.45E-07	1.67E-06	NO DATA	NO DATA	NO DATA	1.51E-05
CO	60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05
NI	63	1.30E-04	9.01E-06	4.36E-06	NO DATA	NO DATA	NO DATA	1.88E-06
NI	65	5.28E-07	6.86E-08	3.13E-08	NO DATA	NO DATA	NO DATA	1.74E-06
CU	64	NO DATA	8.33E-08	3.91E-08	NO DATA	2.10E-07	NO DATA	7.10E-06
ZN	65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06
ZN	69	1.03E-08	1.97E-08	1.37E-09	NO DATA	1.28E-08	NO DATA	2.96E-09
BR	83	NO DATA	NO DATA	4.02E-08	NO DATA	NO DATA	NO DATA	5.79E-08
BR	84	NO DATA	NO DATA	5.21E-08	NO DATA	NO DATA	NO DATA	4.09E-13
BR	85	NO DATA	NO DATA	2.14E-09	NO DATA	NO DATA	NO DATA	LT E-24
RB	86	NO DATA	2.11E-05	9.83E-06	NO DATA	NO DATA	NO DATA	4.16E-06
RB	88	NO DATA	6.05E-08	3.21E-08	NO DATA	NO DATA	NO DATA	8.36E-19
RB	89	NO DATA	4.01E-08	2.82E-08	NO DATA	NO DATA	NO DATA	2.33E-21
SR	89	3.08E-04	NO DATA	8.84E-06	NO DATA	NO DATA	NO DATA	4.94E-05
SR	90	7.58E-03	NO DATA	1.86E-03	NO DATA	NO DATA	NO DATA	2.19E-04
SR	91	5.67E-06	NO DATA	2.29E-07	NO DATA	NO DATA	NO DATA	2.70E-05
SR	92	2.15E-06	NO DATA	9.30E-08	NO DATA	NO DATA	NO DATA	4.26E-05
Y	90	9.62E-09	NO DATA	2.58E-10	NO DATA	NO DATA	NO DATA	1.02E-04

TABLE 2.1

Liquid Dose Conversion Factors (DCF): DF_{ij}

Ingestion Dose Factors for Adults*
(MREM Per PCI Ingested)

NUCLIDE		BONE	LIVER	T. BODY	THYROID	KIDNEY	LUNG	GI-LLI
Y	91M	9.09E-11	NO DATA	3.52E-12	NO DATA	NO DATA	NO DATA	2.67E-10
Y	91	1.41E-07	NO DATA	3.77E-09	NO DATA	NO DATA	NO DATA	7.76E-05
Y	92	8.45E-10	NO DATA	2.47E-11	NO DATA	NO DATA	NO DATA	1.48E-05
Y	93	2.68E-09	NO DATA	7.40E-11	NO DATA	NO DATA	NO DATA	8.50E-05
ZR	95	3.04E-08	9.75E-09	6.60E-09	NO DATA	1.53E-08	NO DATA	3.09E-05
ZR	97	1.68E-09	3.39E-10	1.55E-10	NO DATA	5.12E-10	NO DATA	1.05E-04
NB	95	6.22E-09	3.46E-09	1.86E-09	NO DATA	3.42E-09	NO DATA	2.10E-05
MO	99	NO DATA	4.31E-06	8.20E-07	NO DATA	9.76E-06	NO DATA	9.99E-06
TC	99M	2.47E-10	6.98E-10	8.89E-09	NO DATA	1.06E-08	3.42E-10	4.13E-07
TC	101	2.54E-10	3.66E-10	3.59E-09	NO DATA	6.59E-09	1.87E-10	1.10E-21
RU	103	1.85E-07	NO DATA	7.97E-08	NO DATA	7.06E-07	NO DATA	2.16E-05
RU	105	1.54E-08	NO DATA	6.08E-09	NO DATA	1.99E-07	NO DATA	9.42E-06
RU	106	2.75E-06	NO DATA	3.48E-07	NO DATA	5.31E-06	NO DATA	1.78E-04
AG	110M	1.60E-07	1.48E-07	8.79E-08	NO DATA	2.91E-07	NO DATA	6.04E-05
SB	125	1.79E-06	2.00E-08	4.26E-07	1.82E-09	0.0	1.38E-06	1.97E-05
TE	125M	2.68E-06	9.71E-07	3.59E-07	8.06E-07	1.09E-05	NO DATA	1.07E-05
TE	127M	6.77E-06	2.42E-06	8.25E-07	1.73E-06	2.75E-05	NO DATA	2.27E-05
TE	127	1.10E-07	3.95E-08	2.38E-08	8.15E-08	4.48E-07	NO DATA	8.68E-06
TE	129M	1.15E-05	4.29E-06	1.82E-06	3.95E-06	4.80E-05	NO DATA	5.79E-05
TE	129	3.14E-08	1.18E-08	7.65E-09	2.41E-08	1.32E-07	NO DATA	2.37E-08
TE	131M	1.73E-06	8.46E-07	7.05E-07	1.34E-06	8.57E-06	NO DATA	8.40E-05
TE	131	1.97E-08	8.23E-09	6.22E-09	1.62E-08	8.63E-08	NO DATA	2.79E-09
TE	132	2.52E-06	1.63E-06	1.53E-06	1.80E-06	1.57E-05	NO DATA	7.71E-05
I	130	7.56E-07	2.23E-06	8.80E-07	1.89E-04	3.48E-06	NO DATA	1.92E-06
I	131	4.16E-06	5.95E-06	3.41E-06	1.95E-03	1.02E-05	NO DATA	1.57E-06
I	132	2.03E-07	5.43E-07	1.90E-07	1.90E-05	8.65E-07	NO DATA	1.02E-07
I	133	1.42E-06	2.47E-06	7.53E-07	3.63E-04	4.31E-06	NO DATA	2.22E-06
I	134	1.06E-07	2.88E-07	1.03E-07	4.99E-06	4.58E-07	NO DATA	2.51E-10

TABLE 2.1

Liquid Dose Conversion Factors (DCF): DF_{ij}

Ingestion Dose Factors for Adults*
(MREM Per PCI Ingested)

NUCLIDE		BONE	LIVER	T. BODY	THYROID	KIDNEY	LUNG	GI-LLI
I	135	4.43E-07	1.16E-06	4.28E-07	7.65E-05	1.86E-06	NO DATA	1.31E-06
CS	134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06
CS	136	6.51E-06	2.57E-05	1.85E-05	NO DATA	1.43E-05	1.96E-06	2.92E-06
CS	137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06
CS	138	5.52E-08	1.09E-07	5.40E-08	NO DATA	8.01E-08	7.91E-09	4.65E-13
BA	139	9.70E-08	6.91E-11	2.84E-09	NO DATA	6.46E-11	3.92E-11	1.72E-07
BA	140	2.03E-05	2.55E-08	1.33E-06	NO DATA	8.67E-09	1.46E-08	4.18E-05
BA	141	4.71E-08	3.56E-11	1.59E-09	NO DATA	3.31E-11	2.02E-11	2.22E-17
BA	142	2.13E-08	2.19E-11	1.34E-09	NO DATA	1.85E-11	1.24E-11	3.00E-26
LA	140	2.50E-09	1.26E-09	3.33E-10	NO DATA	NO DATA	NO DATA	9.25E-05
LA	142	1.28E-10	5.82E-11	1.45E-11	NO DATA	NO DATA	NO DATA	4.25E-07
CE	141	9.36E-09	6.33E-09	7.18E-10	NO DATA	2.94E-09	NO DATA	2.42E-05
CE	143	1.65E-09	1.22E-06	1.35E-10	NO DATA	5.37E-10	NO DATA	4.56E-05
CE	144	4.88E-07	2.04E-07	2.62E-08	NO DATA	1.21E-07	NO DATA	1.65E-04
PR	143	9.20E-09	3.69E-09	4.56E-10	NO DATA	2.13E-09	NO DATA	4.03E-05
PR	144	3.01E-11	1.25E-11	1.53E-12	NO DATA	7.05E-12	NO DATA	4.33E-18
ND	147	6.29E-09	7.27E-09	4.35E-10	NO DATA	4.25E-09	NO DATA	3.49E-05
W	187	1.03E-07	8.61E-08	3.01E-08	NO DATA	NO DATA	NO DATA	2.82E-05
NP	239	1.19E-09	1.17E-10	6.45E-11	NO DATA	3.65E-10	NO DATA	2.40E-05

* Dose factors of internal exposure are for continuous intake over a one-year period and include the dose commitment over a 50-year period; from Reg. Guide 1.109 (Rev. 1). Additional dose factors for nuclides not included in this table may be obtained from NUREG-0172.

TABLE 2.2

Bioaccumulation Factors, BF_i

Bioaccumulation Factors to be Used in the Absence of Site-Specific Data*
(pCi/kg per pCi/liter)

ELEMENT	FRESHWATER	
	FISH	INVERTEBRATE
H	9.0E-01	9.0E-01
C	4.6E+03	9.1E+03
NA	1.0E+02	2.0E+02
CR	2.0E+02	2.0E+03
MN	4.0E+02	9.0E+04
FE	1.0E+02	3.2E+03
C0	5.0E+01	2.0E+02
NI	1.0E+02	1.0E+02
CU	5.0E+01	4.0E+02
ZN	2.0E+03	1.0E+04
BR	4.2E+02	3.3E+02
RB	2.0E+03	1.0E+03
SR	3.0E+01	1.0E+02
Y	2.5E+01	1.0E+03
ZR	3.3E+00	6.7E+00
NB	3.0E+04	1.0E+02
MO	1.0E+01	1.0E+01
TC	1.5E+01	5.0E+00
RU	1.0E+01	3.0E+02
RH	1.0E+01	3.0E+02
***AG-110m	2.30E+1	7.70E+2
**SB	1.0E+00	1.0E+00
TE	4.0E+02	6.1E+03
I	1.5E+01	5.0E+00
CS	2.0E+03	1.0E+03
BA	4.0E+00	2.0E+02
LA	2.5E+01	1.0E+03
CE	1.0E+00	1.0E+03
PR	2.5E+01	1.0E+03
ND	2.5E+01	1.0E+03
W	1.2E+03	1.0E+01
NP	1.0E+01	4.0E+02

* Bioaccumulation factor values are taken from Reg. Guide 1.109 (Rev. 1), Table A-1j.

** Sb bioaccumulation factor value is taken from EPRI NP-3840.

*** Ag bioaccumulation factor value is taken from Reg. Guide 1.109 (Rev. 0), Table A-8.

3.0 TMI LIQUID EFFLUENT WASTE TREATMENT SYSTEMS

3.1 TMI-1 Liquid Effluent Waste Treatment System

3.1.1 *Description of the Liquid Radioactive Waste Treatment System* (see Figure 3.1)

Reactor Coolant Train

a. Water Sources - (3) Reactor Coolant Bleed Tanks (RCBT)
- (1) Reactor Coolant Drain Tank (RCDT)

b. Liquid Processing - Reactor Coolant Waste Evaporator (see Figure 3.2)
- Demineralizers prior to release

c. Liquid Effluent for Release - (2) Waste Evaporator Condensate Storage Tanks
- (WECST)

d. *Dilution* - Mechanical Draft Cooling Tower (0-38k gpm)
- River Flow (2E7 gpm average)

Miscellaneous Waste Train

a. Water sources: - Auxiliary Building Sump
- Reactor Building Sump
- Miscellaneous Waste Storage Tank
- Laundry Waste Storage Tank
- Neutralizer Mixing Tank
- Neutralizer Feed Tank
- Used Precoat Tank
- Borated Water Tank Tunnel Sump
- Heat Exchanger Vault Sump
- Tendon Access Galley Sump
- Spent Fuel Pool Room Sump
- TMI-2 Miscellaneous Waste Holdup Tank

b. Liquid Processing - Miscellaneous Waste Evaporator, MWE (see Figure 3.2)
- Demineralizers prior to release

c. Liquid Effluent for Release - (2) Waste Evaporator Condensate Storage Tanks
- (WECST)

d. Dilution - Mechanical Draft Cooling Tower (0-38k gpm)
- River Flow (2E7 gpm average)

3.2 Operability of the TMI-1 Liquid Effluent Waste Treatment System

3.2.1 The TMI-1 Liquid Waste Treatment System as described in Section 11 of the TMI-1 Final Safety Analysis Report is considered to be operable when one of each of the following pieces of equipment is available to perform its intended function:

a) Miscellaneous Waste Evaporator (WDL-Z1B) or Reactor Coolant Evaporator (WDL-Z1A)

b) Waste Evaporator Condensate Demineralizer (WDL-K3 A or B)

c) Waste Evaporator Condensate Storage Tank (WDL-T 11 A or B)

d) Evaporator Condensate Pumps (WDL-P 14 A or B)

3.2.2 TMI-1 Representative Sampling Prior to Discharge

All liquid releases from the TMI-1 Liquid Waste Treatment System are made through the Waste Evaporator Condensate Storage Tanks. To provide thorough mixing and a representative sample, the contents of the tank are recirculated using one of the Waste Evaporator Condensate Transfer Pumps.

3.3 TMI-2 Liquid Effluent Waste Treatment System

3.3.1 Description of the TMI-2 Liquid Radioactive Waste Treatment System

The TMI-2 Liquid Radioactive Waste Treatment System has been out of service since the TMI-2 Accident in 1979. TMI-2 Liquid Radioactive Waste is processed by the TMI-1 system described in Section 3.1 prior to release. In addition, TMI-2 releases water from various sumps and tanks to the river (see Figures 1.1 and 1.2). These processes are governed by plant procedures that encompass proper sampling, sample analysis, and radiation monitoring techniques.

FIGURE 3.1

TMI-1 Liquid Radwaste

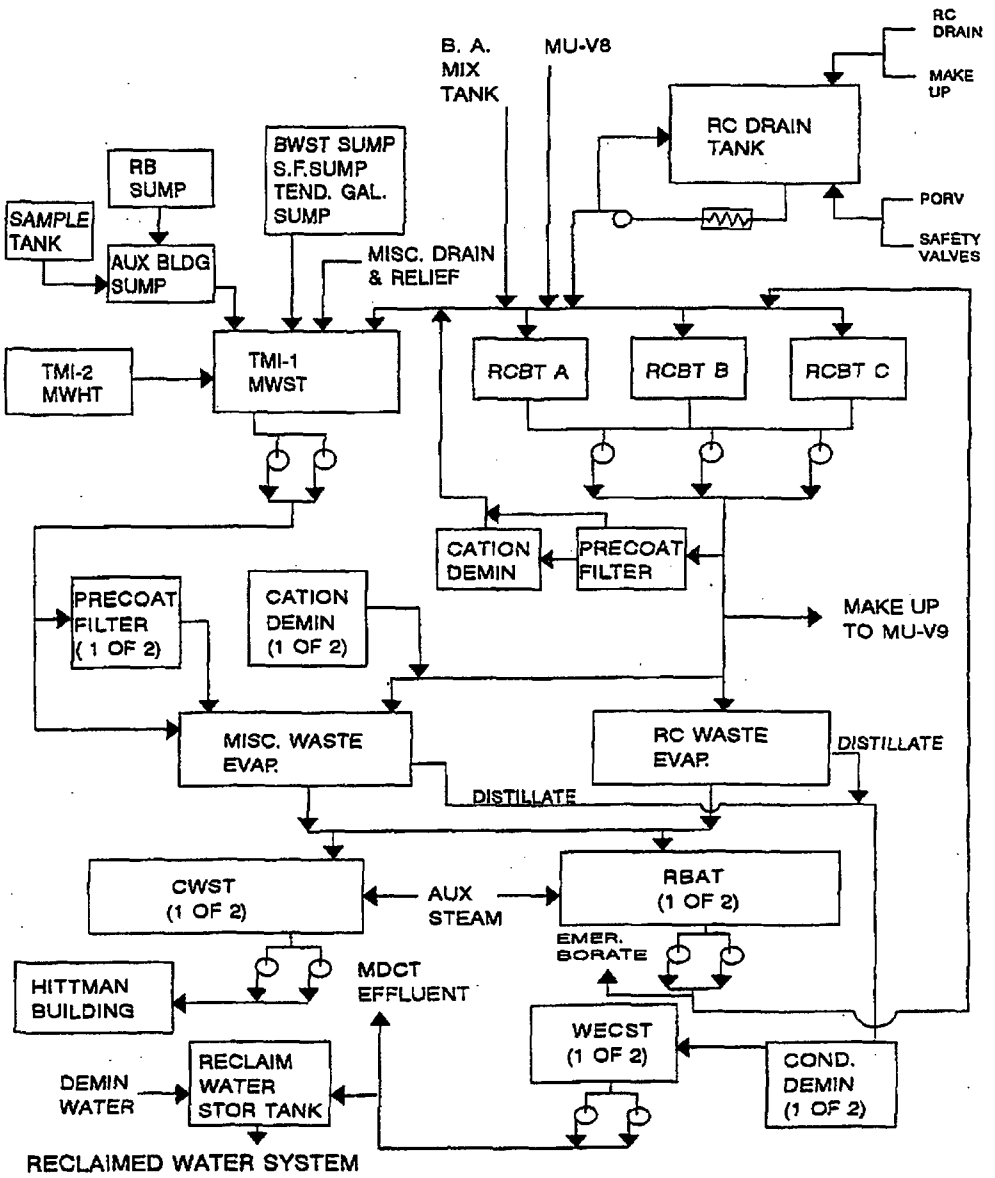
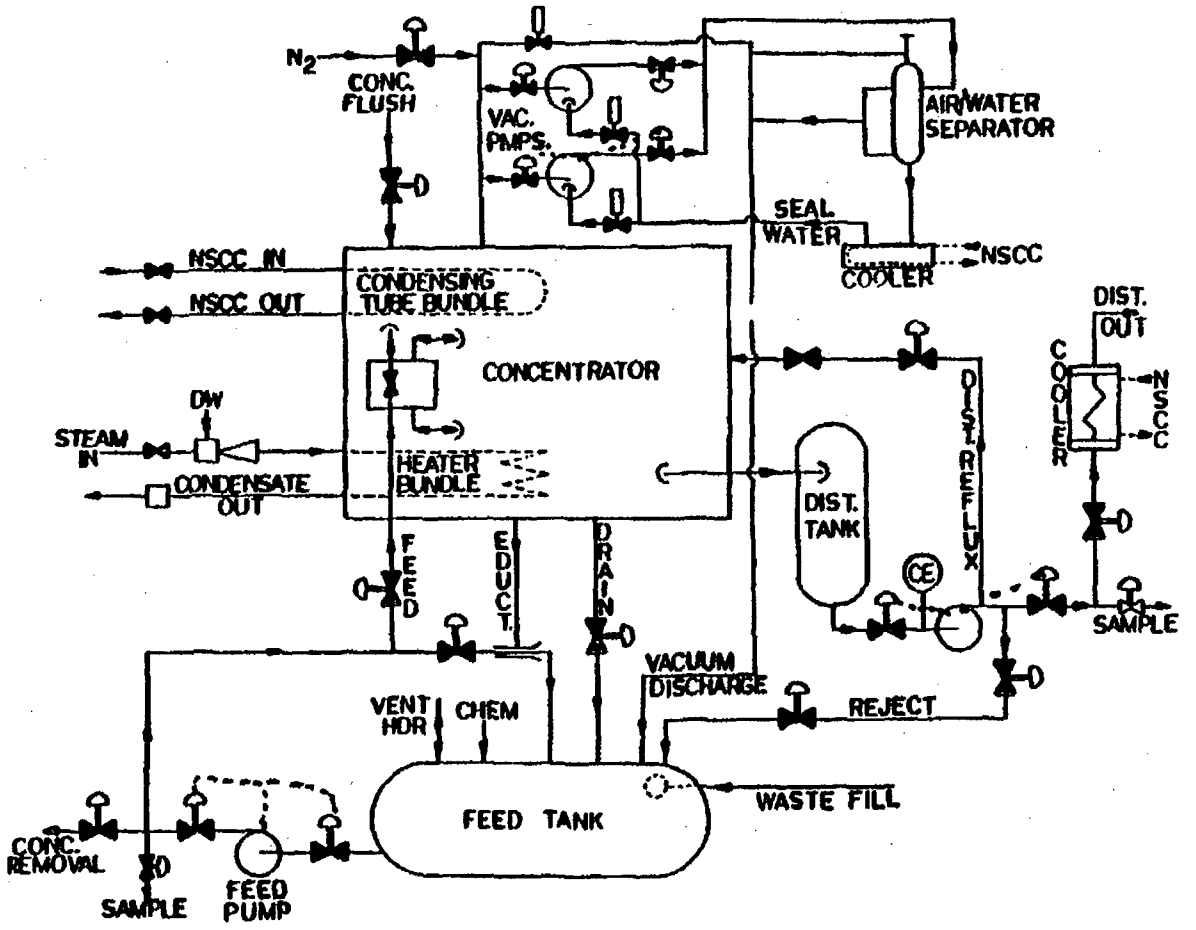


FIGURE 3.2

TMI-1 Liquid Waste Evaporators



4.0 GASEOUS EFFLUENT MONITORS

4.1 TMI-1 Noble Gas Monitor Set Points

The gaseous effluent monitor set points are established for each gaseous effluent radiation monitor to assure concentrations of radionuclides in gaseous effluents do not exceed the limits set forth in ODCM Part I Control 2.2.2.1. Table 4.1 lists Gaseous Effluent Release Points and their associated parameters; Figure 4.1 provides a Gaseous Effluent Release Pathway Diagram.

The set points are established to satisfy the more restrictive set point concentration in the following two equations:

$$500 > \sum_i (c_i)(F)(K_i)(Dv) \qquad \text{(eq 4.1.1)}$$

and

$$3000 > \sum_i (c_i)(L_i + 1.1\,M_i)(Dv)(F) \qquad \text{(eq 4.1.2)}$$

where:

c_i = set point concentration based on Xe-133 equivalent, in μCi/cc

F = gaseous effluent flowrate at the monitor, in cc/sec

K_i = total body dose factor, in mrem/yr per μCi/m^3 from Table 4.3

Dv = highest sector annual average gaseous atmospheric dispersion factor (X/Q) at or beyond the unrestricted area boundary, in sec/m^3, from Table 4.4 for station vent releases and Table 4.5 for all other releases, (Condenser off gas, ESF FHB, and ground releases). Maximum values presently used are 7.17E-7 sec/m^3 at sector NNE for station vent, and 1.16E-5 sec/m^3 at sectors N and WNW for all other releases.

L_i = skin dose factor due to beta emissions from radionuclide i, in mrem/yr per μCi/m^3 from Table 4.3.

M_i = air dose factor due to gamma emissions from radionuclide i, in mrad/yr per μCi/m^3 from Table 4.3.

1.1 = mrem skin dose per mrad air dose.

500 = annual whole body dose rate limit for unrestricted areas, in mrem/yr.

3000 = annual skin dose rate limit for unrestricted areas, in mrem/yr.

The set point concentration is further reduced such that the concentration contributions from multiple release points would not combine to exceed ODCM Control limits.

The set point concentration is converted to set point scale units on each radiation monitor using appropriate calibration factors.

This section of the ODCM is implemented by the Radiation Monitor System Set Points procedure and the procedure for Releasing Radioactive Gaseous Waste.

4.2 TMI-1 Particulate and Radioiodine Monitor Set Points

Set points for monitors which detect radionuclides other than noble gases are also established to assure that concentrations of these radionuclides in gaseous effluents do not exceed the limits of ODCM Part I Control 2.2.2.1.

Set points are established so as to satisfy the following equations:

$$1500 > \sum_i (c_i)(F)(P_i)(Dv) \qquad \text{(eq 4.2)}$$

where:

c_i = set point concentration based on I-131 equivalent, in μCi/cc

F = gaseous effluent flow rate at the monitor, in cc/sec

P_i = pathway dose parameter, in mrem/yr per μCi/m3 for the inhalation pathway from Table 4.6. The dose factors are based on the actual individual organ and most restrictive age group (child) (NUREG-0133).

NOTE

Appendix A contains P_i calculational methodology.

1500 = annual dose rate limit to any organ from particulates and radioiodines and radionuclides (other than noble gases) with half lives greater than eight days in mrem/yr.

Dv = highest sector annual average gaseous dispersion factor (X/Q or D/Q) at or beyond the unrestricted area boundary from Table 4.4 for releases from the station vent and Table 4.5 for all other releases. X/Q is used for the inhalation pathway. Maximum values of X/Q presently used are 7.17E-7 sec/m3 for station vent, at sector SE, and 1.16E-5 sec/m3 for all other releases, at sectors N and WNW.

The set point concentration is further reduced such that concentration contributions from multiple release points would not combine to exceed ODCM Control limits.

The set point concentration is converted to set point scale units on each radiation monitor using appropriate calibration factors.

This section of the ODCM is implemented by the Radiation Monitor Systems Set Points procedure and the procedure for Releasing Radioactive Gaseous Waste.

4.3 TMI-2 Gaseous Radiation Monitor Set Points

TMI-2 Gaseous Radiation Monitors have their set points described in TMI Plant Procedure 1101-2.1. Figure 4.5 provides a gaseous effluent release pathway diagram. Table 4.2 provides TMI-2 Radiation Monitor Data.

These set points are set in accordance with the Controls delineated in Part II of this ODCM.

4.4 TMI-1 Gaseous Effluent Release Points and Gaseous Radiation Monitor Data

TMI-1 has eleven (11) required effluent gaseous radiation monitors. These are RM-A4, RM-A5, RM-A15, RM-A6, RM-A7, RM-A8, RM-A9, RM-A14, ALC-RMI-18, WHP-RIT-1, and RLM-RM-1. These gaseous release points, radiation monitors, and sample points are shown in Table 4.1.

4.4.1 RM-A4/RM-A6 Fuel Handling and Auxiliary Building Exhaust

RM-A4 is the radiation monitor for the TMI-1 Fuel Handling Building Ventilation (see Figures 4.1 and 4.2). RM-A6 is the radiation monitor for the TMI-1 Auxiliary Building Ventilation (see Figures 4.1 and 4.2). High alarms on RM-A4 or RM-A6 noble gas channels will initiate shutdown of the related building ventilation air supply system. These two radiation monitors concurrently will satisfy requirements for the Station Vent release point in place of RM-A8.

4.4.2 RM-A8 Station Ventilation Exhaust

RM-A8 is the particulate, radioiodine and gaseous radiation monitor for the TMI-1 Station Ventilation (see Figures 4.1 and 4.2). This in plant effluent radiation monitor also has an associated sampling panel with sampling lines located before the sample filters. High alarm on RM-A8 noble gas low channel will initiate shutdown of the Station Ventilation air supply systems. (The Fuel Handling and Auxiliary Building Ventilation). This radiation monitor satisfies requirements for the Station Vent release point in place of RM-A4 and RM-A6.

4.4.3 RM-A5/RM-A15 Condenser Off Gas Exhaust

RM-A5 is the gaseous radiation monitor for the TMI-1 Condenser Off Gas exhaust (see Figures 4.1 and 4.4). RM-A15 is the back up gaseous radiation monitor for the TMI-1 Condenser Off Gas exhaust (see Figures 4.1 and 4.4). High alarms on RM-A5 low channel or RM-A15 noble gas channels will initiate the MAP-5 Radioiodine Processor Station. These two radiation monitors together satisfy requirements for the Condenser Off Gas release point.

4.4.4 RM-A7 Waste Gas Decay Tank Exhaust

RM-A7 is the gaseous radiation monitor for the TMI-1 Waste Gas Decay tanks (see Figures 4.1 and 4.2). This in plant effluent radiation monitor also has an associated sampling panel. High alarm on RM-A7 noble gas channel will initiate shutdown of the Waste Gas Decay Tank release in progress. This radiation monitor satisfies requirements for batch gaseous releases to the Station Vent release point.

4.4.5 RM-A9 Reactor Building Purge Exhaust

RM-A9 is the particulate, radioiodine and gaseous radiation monitor for the TMI-1 Reactor Building Purge system (see Figures 4.1 and 4.3). This in plant effluent radiation monitor also has an associated sampling panel with sampling lines located before the sample filters. High alarm on RM-A9 noble gas low channel will initiate shutdown of the Reactor Building Purge System. This radiation monitor satisfies requirements for the Reactor Building Purge System release point.

4.4.6 RM-A14 ESF FHB Ventilation System

RM-A14 is the gaseous radiation monitor for the TMI-1 Emergency Safeguards Features (ESF) Fuel Handling Building Exhaust system (see Figures 4.1 and 4.2). This in plant effluent radiation monitor also has an associated sampling panel with sampling lines located before the sampler filters. High alarm on RM-A14 noble gas channel will initiate shutdown of the ESF Fuel Handling Building Exhaust System. This radiation monitor satisfies requirements for the ESF Fuel Handling Building Exhaust System release point.

4.4.7 ALC-RMI-18 Chemical Cleaning Facility (CCF) Ventilation Exhaust

ALC-RMI-18 is an Victoreen particulate, radioiodine, and gaseous radiation monitor for the Chemical Cleaning building exhaust. This monitor is located in the Chemical Cleaning building on the ground floor, and has an associated sample panel. Sampling for particulate activity is performed off of the monitor.

4.4.8 WHP-RIT-1 Waste Handling and Packaging Facility (WHPF) Exhaust

WHP-RIT-1 is an Eberline AMS-3 particulate radiation monitor for the TMI WHPF. The monitor is located in the Mechanical Equipment Room in the WHPF. Sampling for particulate activity is performed off of the monitor. A high alarm will initiate shutdown of the ventilation air exhaust system.

4.4.9 RLM-RM-1 Respirator Cleaning and Laundry Maintenance (RLM) Facility

RLM-RM-1 is an Eberline AMS-3 particulate radiation monitor for the TMI RLM Facility. The monitor is located in the Mechanical Equipment Room in the RLM. Sampling for particulate activity is performed off of the monitor.

4.5 TMI-2 Gaseous Effluent Release Points and Gaseous Radiation Monitor Data

TMI-2 has three (3) regulatory required gaseous effluent radiation monitors. These are HP-R-219, HP-R-219A and HP-R-225. These gaseous release points, radiation monitors, and sample points are shown in Table 4.2, and various gaseous effluent pathways are depicted in Figure 4.5.

4.5.1 HP-R-219 Station Ventilation Exhaust

HP-R-219 is a Victoreen particulate and gaseous radiation monitor for the TMI-2 ventilation exhaust. This in-plant effluent radiation monitor is located in the TMI-2 Auxiliary Building 328 foot elevation and has an associated sample panel.

4.5.2 HP-R-219A Station Ventilation Exhaust

HP-R-219A is a Victoreen particulate and gaseous radiation monitor for the TMI-2 ventilation exhaust. This in-plant effluent radiation monitor is located in the TMI-2 Auxiliary Building 328 foot elevation.

4.5.3 HP-R-225 Reactor Building Purge Air Exhaust Duct "A"

HP-R-225 is a Victoreen particulate and gaseous radiation monitor for the TMI-2 Reactor Building Purge Air Exhaust System. This in-plant effluent radiation monitor is located in the TMI-2 Auxiliary Building 328' elevation area.

4.6 Control of Gaseous Effluent Releases

TMI gaseous effluent combined releases are controlled (per ODCM Part I for TMI-1 and ODCM Part II for TMI-2) by effluent sampling and radiation monitor set points. These measures assure that releases from the various vents do not combine to produce dose rates at the site boundary exceeding the most restrictive of 500 mrem per year to the total body or 3000 mrem per year to the skin, and 1500 mrem per year to the thyroid. This is done by restricting simultaneous releases and by limiting the dose rates that may be contributed by the various vents at any time. The various vent radiation monitor set points are each based on fractions of the above limits and do not exceed the above limits when summed together. These effluent radiation monitor set points are calculated using the methodology described in equations 4.1.1, or 4.1.2 and 4.2. The actual set points are then listed in TMI-1 Operations Procedure 1101-2.1.

The radioactive content of each batch of gaseous waste is determined prior to release by sampling and analyses in accordance with ODCM Part I for TMI-1 and ODCM Part II for TMI-2. The results of pre-release analyses are used with the calculational methods in Sections 4.1 and 4.2 to assure that the dose rates at the site boundary are maintained below the limits in ODCM Part I for TMI-1 and ODCM Part II for TMI-2.

Post-release analyses of samples composited from batch and continuous releases are performed in accordance with ODCM Part I for TMI-1 and ODCM Part II for TMI-2. The results of the analyses are used to assure that the dose rates at the site boundary are maintained within the limits of ODCM Part I for TMI-1 and ODCM Part II for TMI-2.

TABLE 4.1

TMI-1 Gaseous Release Point and Gaseous Radiation Monitor Data

GASEOUS RADIATION MONITOR (DETECTOR)	LOCATION	GASEOUS RELEASE POINT	(F) FLOW RECORDER	RELEASE TERMINATION INTERLOCK (YES/NO) VALVES
RM-A4	306' Elevation Auxiliary Bldg.	Fuel Hand. Building Exhaust	FR-149	YES AH-E-10 AH-D-120 AH-D-121 AH-D-122
RM-A6	306' Elevation Auxiliary Bldg.	Auxiliary Building Exhaust	FR-150	YES AH-E-11
RM-A8	RMA-8/9 Bldg. Near BWST Exhaust	Station Vent	FR-149 & FR-150	YES WDG-V47 AH-E-10 AH-E-11 Starts MAP-5 Radioiodine Sampler
RM-A5	322' Elevation Second Floor Turbine Bldg.	Condenser Off Gas Exhaust	FR-1113	YES Starts MAP-5 Radioiodine Sampler
RM-A15	322' Elevation Second Floor Turbine Bldg.	Condenser Off Gas Exhaust	FR-1113	YES Starts MAP-5 Radioiodine Sampler
RM-A7	306' Elevation Auxiliary Bldg.	Waste Gas Decay Tanks (A,B,C)	FR-123	YES WDG-V47
RM-A9	RMA-8/9 Bldg. Near BWST	Reactor Building Purge Exhaust	FR-909/ FR-148	YES AH-V-1A/B/C/D WDG-534/535 Starts MAP-5 Radioiodine Sampler
RM-A14	331' Elevation ESF FHB Outside Chem. Addition Bldg.	ESF Fuel Handling Building Exhaust	FR-1104A/B	NO Manual Actions

TABLE 4.1

TMI-1 Gaseous Release Point and Gaseous Radiation Monitor Data

GASEOUS RADIATION MONITOR (DETECTOR)	LOCATION	GASEOUS RELEASE POINT	RELEASE TERMINATION INTERLOCK (YES/NO) VALVES
ALC-RMI-18	Chemical Cleaning Bldg. 304' Elevation	CCB Exhaust System (Typical flow rate is 10,000 cfm)	NONE
WHP-RIT-1	WHPF Mechanical Equipment Room	WHPF Exhaust System (Typical flow rate is 7,500 cfm)	YES WHPF Ventilation Trips
RLM-RM-1	RLM-Mechanical Equipment Room	RLM Exhaust System (Typical flow rate is 900 cfm)	NONE

TABLE 4.2

TMI-2 Gaseous Release Point and Gaseous Radiation Monitor Data

GASEOUS RADIATION MONITOR (DETECTOR)	LOCATION	GASEOUS RELEASE POINT	RELEASE TERMINATION INTERLOCK (YES/NO) VALVES
HP-R-219	328' Elevation Auxiliary Building	Station Vent Exhaust	NONE
HP-R-219A	328' Elevation Auxiliary Building	Station Vent Exhaust	NONE
HP-R-225	328' Elevation Auxiliary Building	Reactor Bldg Purge Exhaust Duct "A"	NONE

TABLE 4.3

Dose Factors for Noble Gases and Daughters*

Radionuclide	Gamma Total Body Dose Factor(a) K_i (mrem/yr per $\mu Ci/m^3$)	Beta Skin Dose Factor(b) L_i (mrem/yr per $\mu Ci/m^3$)	Gamma Air Dose Factor M_i (mrad/yr per $\mu Ci/m^3$)	Beta Air Dose Factor N_i (mrad/yr per $\mu Ci/m^3$)
Kr-83m	7.56E-02**	---	1.93E+01	2.88E+02
Kr-85m	1.17E+03	1.46E+03	1.23E+03	1.97E+03
Kr-85	1.61E+01	1.34E+03	1.72E+01	1.95E+03
Kr-87	5.92E+03	9.73E+03	6.17E+03	1.03E+04
Kr-88	1.47E+04	2.37E+03	1.52E+04	2.93E+03
Kr-89	1.66E+04	1.01E+04	1.73E+04	1.06E+04
Kr-90	1.56E+04	7.29E+03	1.63E+04	7.83E+03
Xe-131m	9.15E+01	4.76E+02	1.56E+02	1.11E+03
Xe-133m	2.51E+02	9.94E+02	3.27E+02	1.48E+03
Xe-133	2.94E+02	3.06E+02	3.53E+02	1.05E+03
Xe-135m	3.12E+03	7.11E+02	3.36E+03	7.39E+02
Xe-135	1.81E+03	1.86E+03	1.92E+03	2.46E+03
Xe-137	1.42E+03	1.22E+04	1.51E+03	1.27E+04
Xe-138	8.83E+03	4.13E+03	9.21E+03	4.75E+03
Ar-41	8.84E+03	2.69E+03	9.30E+03	3.28E+03

* Dose factors are for immersion exposure in uniform semi-infinite cloud of noble gas radionuclides that may be detected in gaseous effluents. Dose factor values are taken from Regulatory Guide 1.109 (Rev. 1), Table B-1.

** 7.56E-02 = 7.56×10^{-2}.

(a) Total body dose factor for gamma penetration depth of 5 cm into the body.

(b) Skin dose factor at a tissue depth or tissue density thickness of 7 mg/cm^2.

TABLE 4.4

Atmospheric Dispersion Factors for Three Mile Island

• STATION VENT
• SECTOR AVERAGE X/Q (IN SEC/M^3)

DISTANCE (IN METERS)

SEASON - ANNUAL

SECTOR	610	2413	4022	5631	7240	12067	24135	40225	56315	72405
N	1.18E-07	5.32E-07	2.95E-07	1.93E-07	1.39E-07	5.52E-08	1.91E-08	5.02E-09	1.88E-09	1.09E-09
NNE	1.70E-07	7.17E-07	3.45E-07	2.00E-07	1.39E-07	5.58E-08	1.70E-08	4.77E-09	1.98E-09	9.69E-10
NE	1.12E-07	1.75E-07	3.26E-07	1.86E-07	1.21E-07	5.00E-08	1.67E-08	4.67E-09	1.85E-09	9.93E-10
ENE	1.09E-07	2.13E-07	2.67E-07	1.53E-07	1.05E-07	4.31E-08	1.42E-08	4.42E-09	1.59E-09	8.64E-10
E	2.31E-07	1.71E-07	1.52E-07	1.49E-07	1.06E-07	4.63E-08	1.52E-08	5.19E-09	2.48E-09	1.50E-09
ESE	3.50E-07	2.12E-07	2.50E-07	1.48E-07	9.48E-08	3.98E-08	1.50E-08	5.92E-09	2.92E-09	1.93E-09
SE	4.19E-07	3.79E-07	2.53E-07	1.55E-07	1.11E-07	4.82E-08	1.81E-08	6.84E-09	3.30E-09	2.22E-09
SSE	2.90E-07	3.62E-07	2.55E-07	1.49E-07	1.11E-07	5.02E-08	1.98E-08	6.97E-09	2.94E-09	1.70E-09
S	1.87E-07	6.47E-08	2.16E-07	1.30E-07	8.65E-08	4.09E-08	1.40E-08	4.96E-09	1.99E-09	1.04E-09
SSW	6.13E-08	4.16E-08	1.56E-07	1.03E-07	6.81E-08	2.72E-08	9.74E-09	3.01E-09	1.50E-09	8.23E-10
SW	5.76E-08	1.14E-07	1.70E-07	1.05E-07	6.93E-08	2.51E-08	9.34E-09	2.72E-09	1.33E-09	8.33E-10
WSW	8.52E-08	3.75E-07	2.14E-07	1.26E-07	7.74E-08	3.08E-08	1.02E-08	3.28E-09	1.39E-09	9.69E-10
W	1.15E-07	5.80E-07	2.88E-07	1.63E-07	1.18E-07	5.23E-08	1.72E-08	5.06E-09	1.98E-09	1.25E-09
WNW	1.41E-07	6.28E-07	3.30E-07	2.19E-07	1.48E-07	5.68E-08	1.95E-08	6.32E-09	2.16E-09	1.34E-09
NW	1.42E-07	5.67E-07	3.17E-07	1.93E-07	1.30E-07	5.67E-08	2.06E-08	5.90E-09	2.70E-09	1.45E-09
NNW	1.00E-07	5.77E-07	3.18E-07	1.80E-07	1.27E-07	5.20E-08	1.77E-08	4.82E-09	2.01E-09	1.22E-09

• STATION VENT
• SECTOR AVERAGE D/Q (IN M^2)

DISTANCE (IN METERS)

SEASON - ANNUAL

SECTOR	610	2413	4022	5631	7240	12067	24135	40225	56315	72405
N	2.51E-09	8.72E-10	4.84E-10	2.98E-10	2.50E-10	8.57E-11	2.51E-11	4.98E-12	1.57E-12	7.84E-13
NNE	3.89E-09	1.98E-09	9.54E-10	4.99E-10	3.38E-10	1.10E-10	2.89E-11	6.06E-12	2.10E-12	8.89E-13
NE	2.58E-09	6.70E-10	9.13E-10	4.91E-10	2.97E-10	1.04E-10	2.87E-11	6.01E-12	1.99E-12	9.23E-13
ENE	2.15E-09	5.85E-10	5.54E-10	3.06E-10	2.08E-10	8.30E-11	2.32E-11	5.41E-12	1.63E-12	7.64E-13
E	5.54E-09	1.23E-09	6.17E-10	4.59E-10	3.63E-10	1.34E-10	3.66E-11	9.44E-12	3.77E-12	1.97E-12
ESE	9.17E-09	2.05E-09	1.51E-09	8.66E-10	5.11E-10	1.82E-10	5.77E-11	1.72E-11	7.07E-12	4.07E-12
SE	1.22E-08	2.88E-09	1.84E-09	1.02E-09	6.85E-10	2.60E-10	8.30E-11	2.34E-11	9.42E-12	5.51E-12
SSE	7.50E-09	1.62E-09	1.08E-09	5.89E-10	4.49E-10	1.87E-10	6.16E-11	1.61E-11	5.67E-12	2.83E-12
S	3.86E-09	6.53E-10	6.27E-10	3.59E-10	2.32E-10	1.06E-10	3.05E-11	8.10E-12	2.73E-12	1.23E-12
SSW	1.13E-09	2.94E-10	4.19E-10	2.53E-10	1.56E-10	5.38E-11	1.68E-11	3.91E-12	1.64E-12	7.84E-13
SW	1.19E-09	3.84E-10	4.96E-10	2.80E-10	1.70E-10	5.24E-11	1.65E-11	3.62E-12	1.49E-12	8.12E-13
WSW	1.77E-09	8.31E-10	6.49E-10	3.50E-10	1.99E-10	6.73E-11	1.89E-11	4.58E-12	1.63E-12	9.90E-13
W	2.41E-09	1.29E-09	6.81E-10	3.65E-10	2.96E-10	1.12E-10	3.11E-11	6.90E-12	2.26E-12	1.25E-12
WNW	3.20E-09	1.39E-09	7.73E-10	5.91E-10	3.66E-10	1.19E-10	3.43E-11	8.36E-12	2.39E-12	1.29E-12
NW	3.25E-09	1.23E-09	7.39E-10	4.22E-10	2.77E-10	1.14E-10	7.28E-11	7.61E-12	2.92E-12	1.36E-12
NNW	1.98E-09	9.88E-10	5.71E-10	3.05E-10	2.23E-10	8.21E-11	2.41E-11	4.93E-12	1.72E-12	9.03E-13

DATA FROM 1/1/78 THROUGH 12/31/86 USED IN CALCULATIONS

TABLE 4.5

Atmospheric Dispersion Factors for Three Mile Island

• GROUND RELEASE
• SECTOR AVERAGE X/Q (IN SEC/M^3)

DISTANCE (IN METERS)

SEASON - ANNUAL

SECTOR	610	2413	4022	5631	7240	12067	24135	40225	56315	72405
N	1.16E-05	1.13E-06	5.94E-07	3.80E-07	2.38E-07	9.74E-08	3.45E-08	9.28E-09	3.52E-09	2.05E-09
NNE	1.08E-05	1.10E-06	5.66E-07	3.41E-07	2.38E-07	9.55E-08	3.11E-08	8.94E-09	3.74E-09	1.84E-09
NE	7.02E-06	9.81E-07	5.42E-07	3.17E-07	2.10E-07	9.01E-08	3.10E-08	8.87E-09	3.54E-09	1.91E-09
ENE	7.14E-06	9.64E-07	4.92E-07	2.85E-07	1.97E-07	7.82E-08	2.64E-08	8.38E-09	3.04E-09	1.66E-09
E	8.49E-06	1.09E-06	5.48E-07	2.91E-07	1.87E-07	8.40E-08	2.82E-08	9.85E-09	4.75E-09	2.87E-09
ESE	6.91E-06	9.02E-07	4.49E-07	2.57E-07	1.67E-07	7.20E-08	2.77E-08	1.12E-08	5.54E-09	3.68E-09
SE	6.70E-06	9.06E-07	4.53E-07	2.81E-07	2.03E-07	8.94E-08	3.33E-08	1.28E-08	6.19E-09	4.18E-09
SSE	7.26E-06	9.25E-07	4.91E-07	2.87E-07	2.08E-07	9.18E-08	3.72E-08	1.32E-08	5.62E-09	3.26E-09
S	8.70E-06	9.08E-07	3.99E-07	2.41E-07	1.61E-07	7.31E-08	2.57E-08	9.23E-09	3.74E-09	1.95E-09
SSW	6.05E-06	7.01E-07	2.75E-07	1.86E-07	1.24E-07	5.06E-08	1.82E-08	5.71E-09	2.87E-09	1.58E-09
SW	5.94E-06	5.71E-07	2.86E-07	1.81E-07	1.22E-07	4.50E-08	1.72E-08	5.12E-09	2.53E-09	1.59E-09
WSW	8.00E-06	7.02E-07	3.60E-07	2.15E-07	1.34E-07	5.50E-08	1.87E-08	6.12E-09	2.62E-09	1.83E-09
W	1.02E-05	1.07E-06	5.30E-07	3.02E-07	2.05E-07	9.31E-08	3.15E-08	9.48E-09	3.74E-09	2.38E-09
WNW	1.16E-05	1.13E-06	5.98E-07	3.67E-07	2.53E-07	1.00E-07	3.56E-08	1.18E-08	4.07E-09	2.54E-09
NW	1.13E-05	1.06E-06	5.70E-07	3.53E-07	2.40E-07	1.02E-07	3.82E-08	1.11E-08	5.14E-09	2.78E-09
NNW	1.08E-05	1.04E-06	5.72E-07	3.27E-07	2.22E-07	9.06E-08	3.20E-08	8.89E-09	3.75E-09	2.29E-09

• GROUND RELEASE
• SECTOR AVERAGE D/Q (IN M^{-2})

DISTANCE (IN METERS)

SEASON - ANNUAL

SECTOR	610	2413	4022	5631	7240	12067	24135	40225	56315	72405
N	2.30E-08	1.88E-09	8.93E-10	4.82E-10	2.70E-10	8.96E-11	2.53E-11	4.98E-12	1.57E-12	7.84E-13
NNE	2.66E-08	2.25E-09	1.06E-09	5.42E-10	3.38E-10	1.10E-10	2.89E-11	6.06E-12	2.10E-12	8.89E-13
NE	1.75E-08	2.00E-09	1.01E-09	5.04E-10	2.98E-10	1.04E-10	2.88E-11	6.01E-12	1.99E-12	9.23E-13
ENE	1.68E-08	1.85E-09	8.65E-10	4.28E-10	2.65E-10	8.57E-11	2.33E-11	5.41E-12	1.63E-12	7.64E-13
E	2.88E-08	2.99E-09	1.39E-09	6.34E-10	3.67E-10	1.35E-10	3.68E-11	9.42E-12	3.77E-12	1.97E-12
ESE	3.59E-08	3.80E-09	1.77E-09	8.79E-10	5.15E-10	1.83E-10	5.78E-11	1.71E-11	7.06E-12	4.06E-12
SE	4.12E-08	4.55E-09	2.13E-09	1.15E-09	7.50E-10	2.72E-10	8.31E-11	2.34E-11	9.42E-12	5.50E-12
SSE	3.12E-08	3.23E-09	1.59E-09	8.00E-10	5.20E-10	1.88E-10	6.18E-11	1.61E-11	5.66E-12	2.83E-12
S	2.65E-08	2.21E-09	9.07E-10	4.75E-10	2.86E-10	1.07E-10	3.06E-11	8.10E-12	2.73E-12	1.23E-12
SSW	1.45E-08	1.30E-09	4.80E-10	2.82E-10	1.70E-10	5.71E-11	1.69E-11	3.91E-12	1.64E-12	7.84E-13
SW	1.42E-08	1.10E-09	5.15E-10	2.82E-10	1.71E-10	5.24E-11	1.65E-11	3.62E-12	1.49E-12	8.12E-13
WSW	2.01E-08	1.41E-09	6.82E-10	3.54E-10	2.00E-10	6.76E-11	1.89E-11	4.58E-12	1.63E-12	9.90E-13
W	2.55E-08	2.16E-09	1.00E-09	4.91E-10	3.01E-10	1.12E-10	3.11E-11	6.90E-12	2.27E-12	1.25E-12
WNW	2.88E-08	2.30E-09	1.13E-09	5.93E-10	3.67E-10	1.19E-10	3.43E-11	8.36E-12	2.39E-12	1.29E-12
NW	2.78E-08	2.15E-09	1.06E-09	5.58E-10	3.41E-10	1.19E-10	3.57E-11	7.61E-12	2.92E-12	1.36E-12
NNW	2.17E-08	1.75E-09	8.75E-10	4.24E-10	2.57E-10	8.55E-11	2.42E-11	4.93E-12	1.72E-12	9.03E-13

DATA FROM 1/1/78 THROUGH 12/31/86 USED IN CALCULATIONS

TABLE 4.6

Dose Parameters for Radioiodines and Radioactive Particulate in Gaseous Effluents*

NUCLIDE	CRITICAL ORGAN	ORGAN FACTOR	Pi***	NUCLIDE	CRITICAL ORGAN	ORGAN FACTOR	Pi***
H-3**	TOTAL BODY	3.04E-07	1.12E+03	RU-103	LUNG	1.79E-04	6.62E+05
C-14	BONE	9.70E-06	3.59E+04	RU-105	GI-LLI	2.69E-05	9.95E+04
NA-24	TOTAL BODY	4.35E-06	1.61E+04	RU-106	LUNG	3.87E-03	1.43E+07
P-32	BONE	7.04E-04	2.60E+06	AG-110M	LUNG	1.48E-03	5.48E+06
CR-51	LUNG	4.59E-06	1.70E+04	TE-125M	LUNG	1.29E-04	4.77E+05
MN-54	LUNG	4.26E-04	1.58E+06	SB-125	LUNG	6.27E-04	2.32E+06
MN-56	GI-LLI	3.33E-05	1.23E+05	TE-127M	LUNG	4.00E-04	1.48E+06
FE-55	LUNG	3.00E-05	1.11E+05	TE-127	GI-LLI	1.52E-05	5.62E+04
FE-59	LUNG	3.43E-04	1.27E+06	TE-129M	LUNG	4.76E-04	1.76E+06
CO-58	LUNG	2.99E-04	1.11E+06	TE-129	GI-LLI	6.89E-06	2.55E+04
CO-60	LUNG	1.91E-03	7.07E+06	TE-131M	GI-LLI	8.32E-05	3.08E+05
NI-63	BONE	2.22E-04	8.21E+05	TE-131	LUNG	5.55E-07	2.05E+03
NI-65	GI-LLI	2.27E-05	8.40E+04	TE-132	LUNG	1.02E-04	3.77E+05
CU-64	GI-LLI	9.92E-06	3.67E+04	I-130	THYROID	4.99E-04	1.85E+06
ZN-65	LUNG	2.69E-04	9.95E+05	I-131	THYROID	4.39E-03	1.62E+07
ZN-69	GI-LLI	2.75E-06	1.02E+04	I-132	THYROID	5.23E-05	1.94E+05
BR-83	TOTAL BODY	1.28E-07	4.74E+02	I-133	THYROID	1.04E-03	3.85E+06
BR-84	TOTAL BODY	1.48E-07	5.48E+02	I-134	THYROID	1.37E-05	5.07E+04
BR-85	TOTAL BODY	6.84E-09	2.53E+01	I-135	THYROID	2.14E-04	7.92E+05
RB-86	LIVER	5.36E-05	1.98E+05	CS-134	LIVER	2.74E-04	1.01E+06
RB-88	LIVER	1.52E-07	5.62E+02	CS-136	LIVER	4.62E-05	1.71E+05
RB-89	LIVER	9.33E-08	3.45E+02	CS-137	BONE	2.45E-04	9.07E+05
SR-89	LUNG	5.89E-04	2.16E+06	CS-138	LIVER	2.27E-07	8.40E+02
SR-90	BONE	2.73E-02	1.01E+08	BA-139	GI-LLI	1.56E-05	5.77E+04
SR-91	GI-LLI	4.70E-05	1.74E+05	BA-140	LUNG	4.71E-04	1.74E+06
SR-92	GI-LLI	6.55E-05	2.42E+05	BA-141	LUNG	7.89E-07	2.92E+03
Y-90	GI-LLI	7.24E-05	2.68E+05	BA-142	LUNG	4.44E-07	1.64E+03
Y-91M	LUNG	7.60E-07	2.81E+03	LA-140	GI-LLI	6.10E-05	2.26E+05
Y-91	LUNG	7.10E-04	2.63E+06	LA-142	GI-LLI	2.05E-05	7.59E+04
Y-92	GI-LLI	6.46E-05	2.39E+05	CE-141	LUNG	1.47E-04	5.44E+05
Y-93	GI-LLI	1.05E-04	3.89E+05	CE-143	GI-LLI	3.44E-05	1.27E+05
ZR-95	LUNG	6.03E-04	2.23E+06	CE-144	LUNG	3.23E-03	1.20E+07
ZR-97	GI-LLI	9.49E-05	3.51E+05	PR-143	LUNG	1.17E-04	4.33E+05
NB-95	LUNG	1.66E-04	6.14E+05	PR-144	LUNG	4.23E-07	1.57E+03
MO-99	LUNG	3.66E-05	1.35E+05	ND-147	LUNG	8.87E-05	3.28E+05
TC-99M	GI-LLI	1.30E-06	4.81E+03	W-187	GI-LLI	2.46E-05	9.10E+04
TC-101	LUNG	1.58E-07	5.85E+02	NP-239	GI-LLI	1.73E-05	6.40E+04

* The listed dose parameters are for radionuclides, other than noble gases that may be detected in gaseous effluents. Pi factors include all nonatmospheric pathway transport parameters, the receptor's usage of pathway media, and are based on the most restrictive age group (child) critical organ. Additional dose parameters for nuclides not included in this Table may be calculated using the methodology described in NUREG-0133.

** Tritium dose factors include an increase of 50% to account for the additional amount of this nuclide absorbed through the skin.

*** mrem/year per $\mu Ci/m^3$.

FIGURE 4.1

TMI-1 Gaseous Effluent Pathways

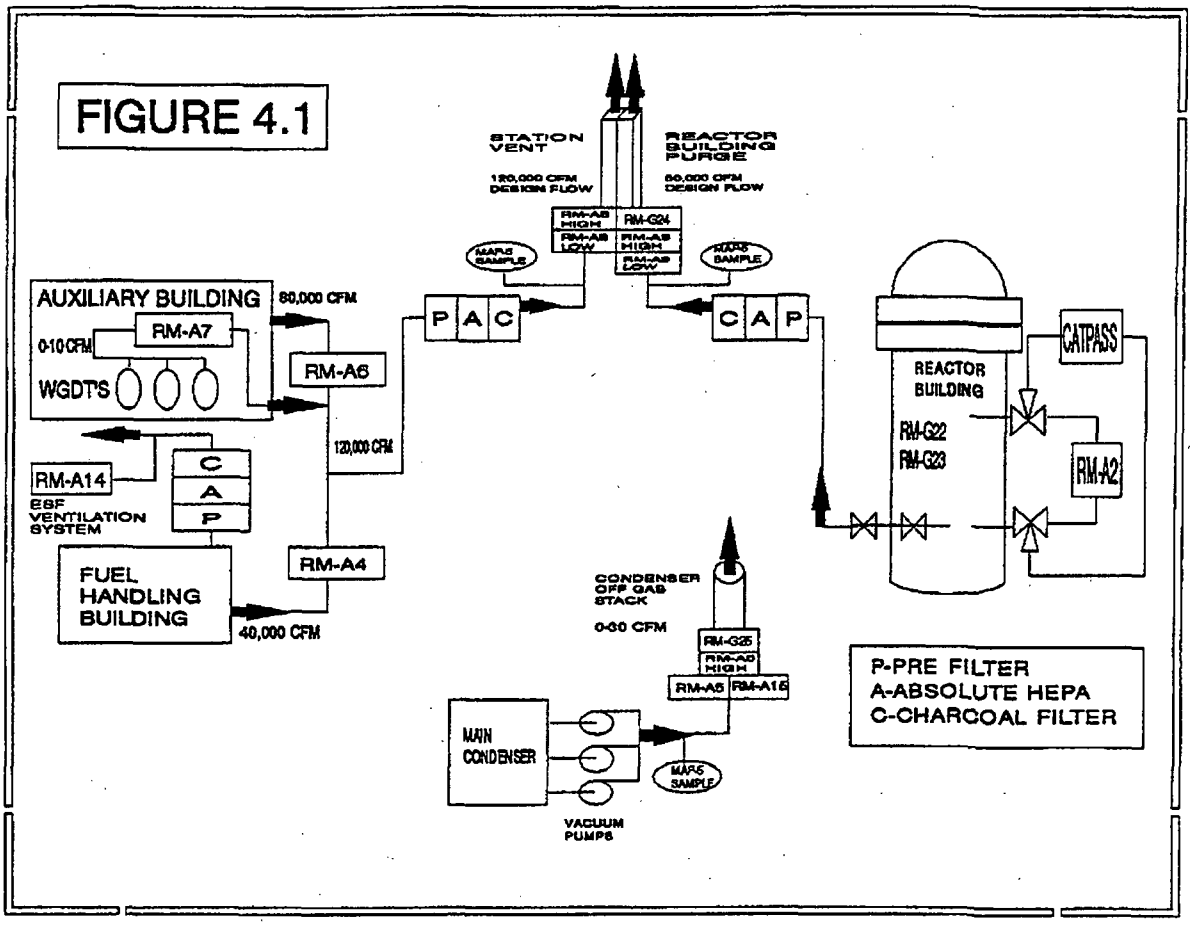


FIGURE 4.2

TMI-1 Auxiliary and Fuel Handling Buildings Effluent Pathways

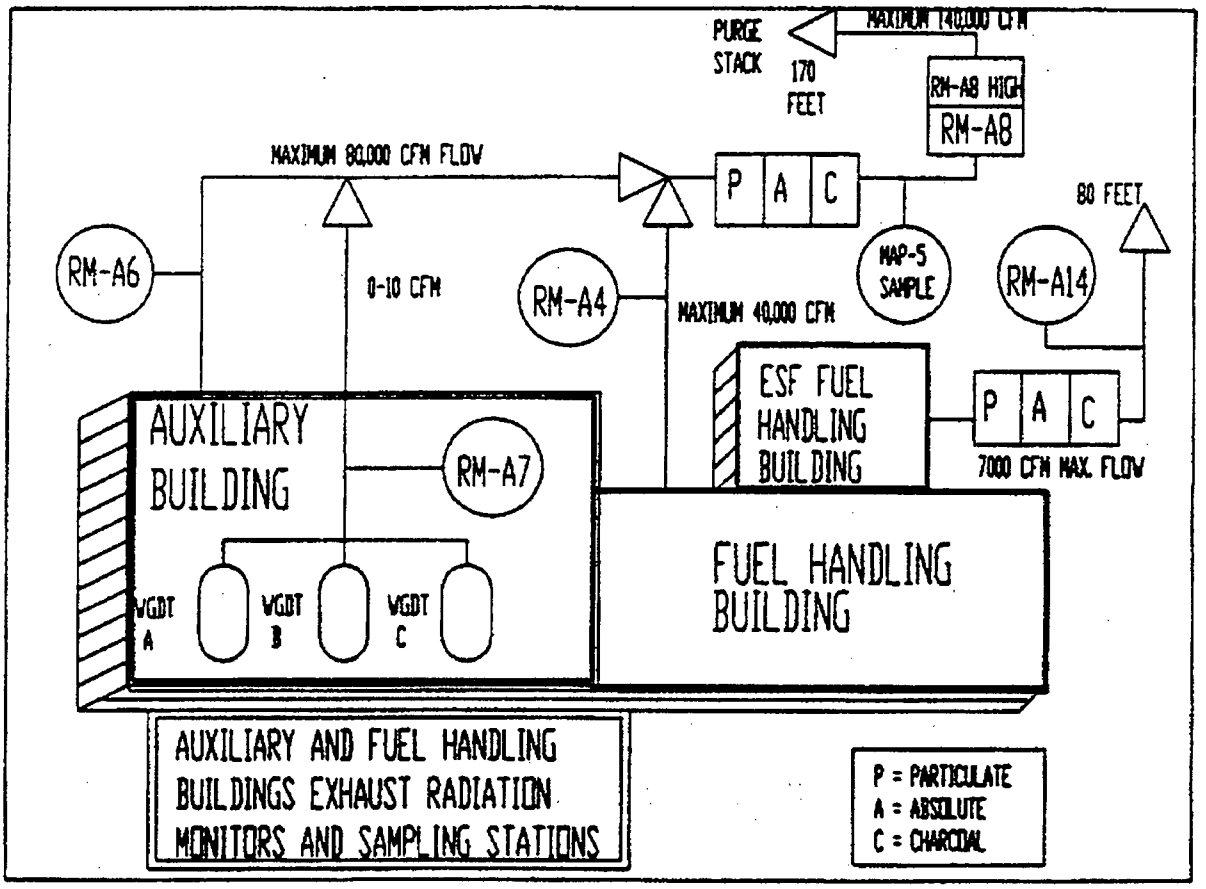


FIGURE 4.3

TMI-1 Reactor Building Effluent Pathway

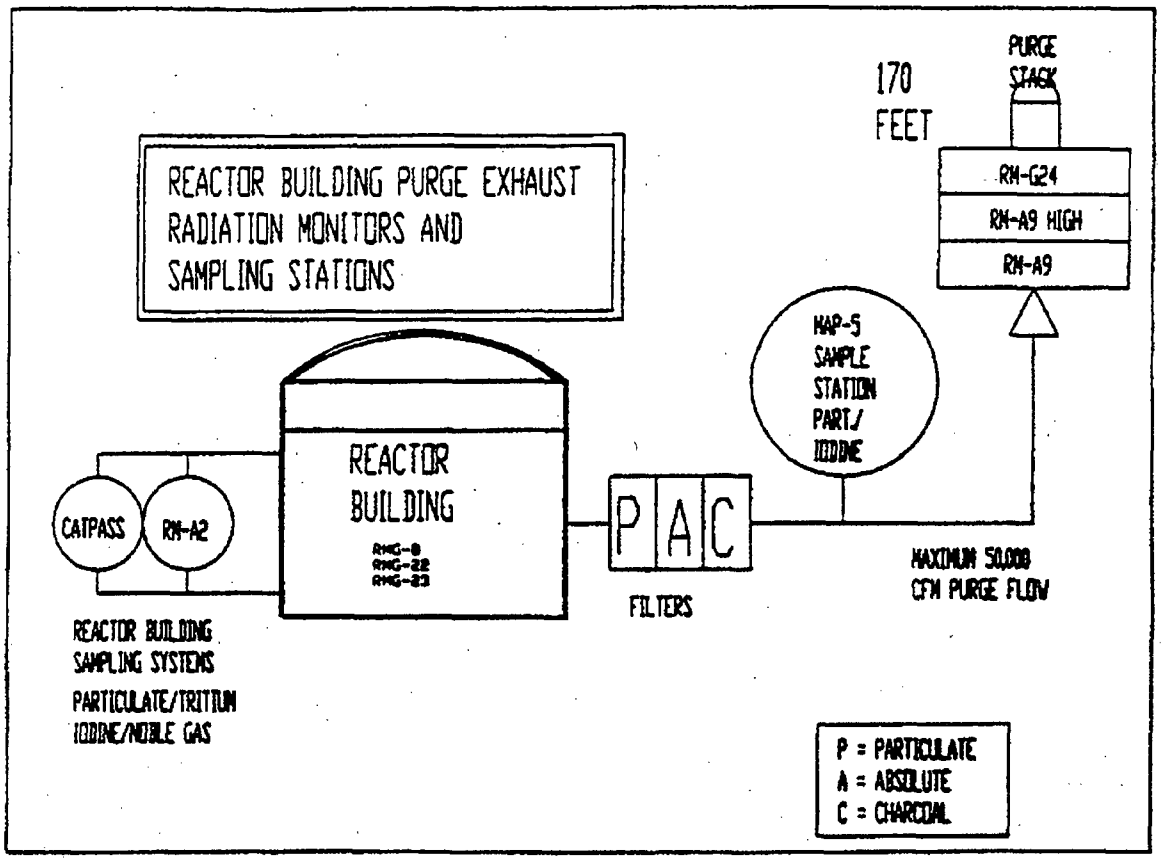


FIGURE 4.4

TMI-1 Condenser Offgas Effluent Pathway

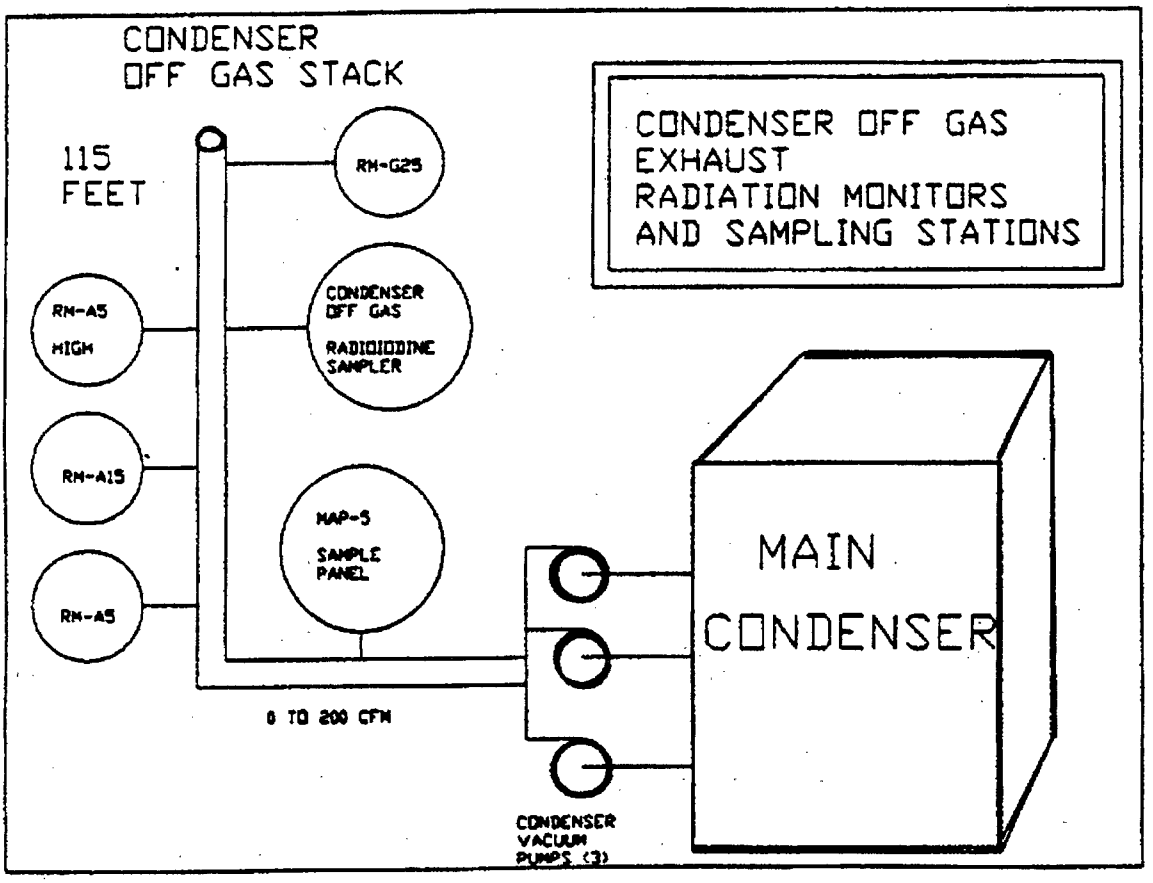
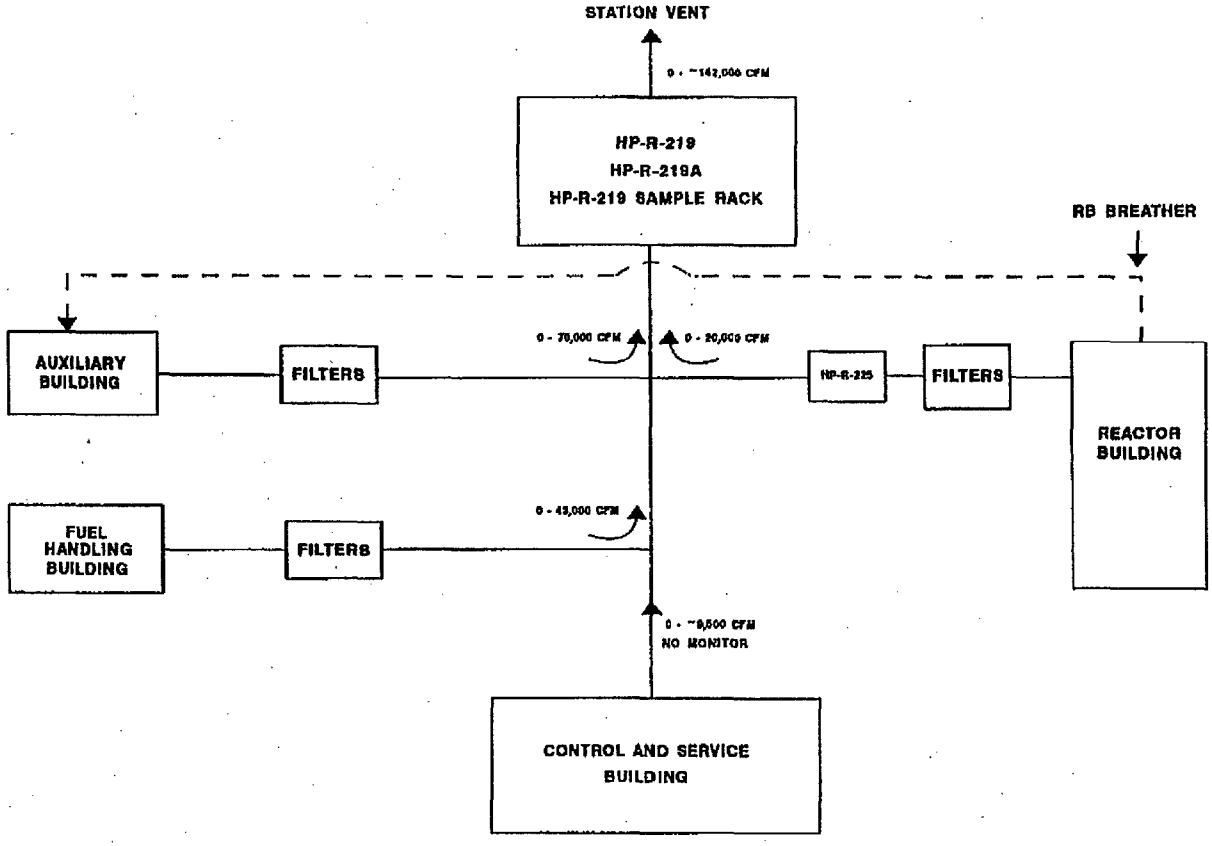


FIGURE 4.5
TMI-2 Gaseous Effluent Filtration System/Pathways



UNIT 2 EXHAUST AIR FLOW AND RMS SCHEMATIC

5.0 GASEOUS EFFLUENT DOSE ASSESSMENT

5.1 Gaseous Effluents - Instantaneous Release Limits

5.1.1 Noble Gases

For noble gases, the following equations apply for total body and skin dose rate at the unrestricted area boundary:

5.1.1.1 Total Body

$$\text{Dose Rate}_{tb} = \sum_i (K_i) \times (Dv) \times (Q_i) \qquad \text{(eq 5.1.1.1)}$$

where:

Dose Ratet b = instantaneous total body dose rate limit, at the site boundary, in mrem/yr.

K_i = total body dose factor due to gamma emissions for each identified noble gas radionuclide, in mrem/yr per $\mu Ci/m^3$ from Table 4.3.

Dv = highest sector annual average gaseous dispersion factor (X/Q) at or beyond the unrestricted area boundary, in sec/m^3, from Table 4.4 for station vent releases; and Table 4.5 for all other releases (Condenser Off Gas, ESF FHB, and ground releases). Maximum values presently in use are 7.17E-7 sec/m^3 at sector NNE for station vent, and 1.16E-5 sec/m^3 for all other releases, at sectors N and WNW.

Q_i = Release rate of radionuclide, i, in $\mu Ci/sec$ as determined by sampling and analysis. Calculated using the concentration of noble gas radionuclide, i, in $\mu Ci/cc$, times the release pathway flow rate, in cc/second.

5.1.1.2 Skin

$$\text{Dose Rate}_{sk} = \sum_i (L_i + 1.1\ M_i)\ X\ (Dv)\ X\ (Q_i) \qquad \text{(eq 5.1.1.2)}$$

where:

Dose Rate$_{sk}$ = instantaneous mrem/year skin dose rate limit, at the site boundary, in mrem/yr.

L_i = skin dose factor due to beta emissions for each identified noble gas radionuclide, in mrem/yr per $\mu Ci/m^3$ from Table 4.3.

M_i = air dose factor due to gamma emissions for each identified noble gas radionuclide, in mrad/yr per $\mu Ci/m^3$ from Table 4.3.

1.1 = mrem skin dose per mrad air dose. Converts air dose to skin dose.

Q_i = release rate of radionuclide, i, in $\mu Ci/sec$, as determined by sampling and analysis. Calculated using the concentration of noble gas radionuclide, i, in $\mu Ci/cc$, times the release pathway flow rate, in cc/second.

Dv = highest sector annual average gaseous dispersion factor (X/Q) at or beyond the unrestricted area boundary, in sec/m^3, from Table 4.4 for station vent releases; and Table 4.5 for all other releases (Condenser Off Gas, ESF FHB, and ground releases). Maximum values presently in use are 7.17E-7 sec/m^3 at sector NNE for station vent, and 1.16E-5 sec/m^3 for all other releases, at sectors N and WNW.

5.1.2 Iodine-131, Iodine-133, Tritium and Radionuclides in Particulate Form, with Half-Lives Greater than 8 Days

For I-131, I-133, Tritium and Radionuclides in Particulate Form, with half-lives greater than 8 days, the following equation applies:

$$\text{Dose Rate}_{IP} = \sum_i (P_i)(D_v)(Q_i) \qquad \text{(eq 5.1.2)}$$

where:

Dose Rate$_{IP}$ = mrem/year organ dose rate.

P_i = dose parameter for I-131, I-133, Tritium and Radionuclides in Particulate Form, with half-lives greater than 8 days, for the inhalation pathway, in mrem/yr per $\mu Ci/m^3$, from Table 4.6. The dose factors are based on the critical individual organ and most restrictive age group (child).

Dv = highest sector annual average gaseous dispersion factor (X/Q or D/Q) at or beyond the unrestricted area boundary, in sec/m^3, from Table 4.4 for the station vent releases and Table 4.5 for all other releases. X/Q is used for the inhalation pathway. Maximum values of X/Q presently used are 7.17E-7 sec/m^3 for station vent, at sector NNE, and 1.16E-5 sec/m^3 for all other releases, at sectors N and WNW.

Q_i = release rate of each radionuclide, i, in $\mu Ci/sec$. Calculated using the concentration of each radionuclide, i, in $\mu Ci/cc$, times the release pathway flow rate, in cc/second.

5.2 Gaseous Effluents - 10 CFR 50 Appendix I

5.2.1 Noble Gases

The air dose in an unrestricted area due to noble gases released in gaseous effluents from the site is determined using the following expressions:

$$\text{Dose } \Gamma = (3.17\text{E-}8) \times \sum_i (M_i) \times (Dv) \times (Qi) \qquad \text{(eq 5.2.1)}$$

and

$$\text{Dose } \beta = (3.17\text{E-}8) \times \sum_i (N_i) \times (Dv) \times (Qi) \qquad \text{(eq 5.2.2)}$$

where:

Dose Γ = mrad gamma air dose due to gamma emissions from noble gas radionuclides.

Dose β = mrad beta air dose due to beta emissions from noble gas radionuclides.

M_i = air dose factor due to gamma emissions for each identified noble gas radionuclide, in mrad/yr per $\mu Ci/m^3$, from Table 4.3.

N_i = air dose factor due to beta emissions for each identified noble gas radionuclide, in mrad/yr per $\mu Ci/m^3$, from Table 4.3.

Dv = highest sector annual average gaseous dispersion factor, X/Q, at or beyond the unrestricted area boundary, in sec/m^3. Values may be read or interpolated from Table 4.4 for releases from the station vent and Table 4.5 for all other releases. Maximum values of X/Q presently used are 7.17E-7 sec/m^3 for station vent at sector NNE, and 1.16E-5 sec/m^3 for all other releases at sectors N or WNW.

Q_i = release of noble gas radionuclide, i, in μCi, over the specified time period, (μCi/second * seconds).

3.17E-8 = inverse of the number of seconds in a year.

NOTE

If the methodology in this section is used in determining dose to an individual, rather than air dose due to noble gases, substitute Ki, from Table 4.3, for Mi, and (Li + 1.1 Mi) for Ni.

5.2.2 Iodine-131, Iodine-133, Tritium and Radionuclides in Particulate Form, with Half-Lives Greater than 8 Days

The dose to an individual from I-131, I-133, Tritium and Radionuclides in Particulate Form with half-lives greater than 8 days in gaseous effluents released from the site to an unrestricted area is determined by solving the following expression:

$$\text{Dose}_o = \sum_i (3.17\text{E-}8) \times \sum_i (R_i)(Dv)(Q_i) \qquad \text{(eq 5.2.2)}$$

where:

Dose_o = dose to all real pathways, p, to organ, o, of an individual in age group, a, from I-131, I-133, Tritium and Radionuclides in Particulate Form, with half-lives greater than 8 days, in mrem, during any desired time period.

R_i = the dose factor for each identified radionuclide, i, pathway, p, age group, a, and organ, o, in mrem/yr per $\mu Ci/m^3$ for the inhalation pathway and m^2 - mrem/yr per $\mu Ci/sec$ for other pathways, from Tables 5.2 to 5.7.

NOTE

Since there is minimal or no elemental iodine released from the condenser off-gas air ejector (see NUREG-0017) all Iodine R_i values for all pathways, except the inhalation pathway, are considered to be zero when performing dose calculations for releases from the condenser off-gas air ejector. Only calculate the dose due to the inhalation pathway for condenser off-gas air ejector iodines.

NOTE

Tritium, H-3, dose factor is mrem/year per $\mu Ci/m^3$ for all pathways.

Dv = highest sector annual average gaseous dispersion factor (X/Q) at or beyond the unrestricted area boundary, in sec/m^3, for the inhalation pathway, and D/Q, in m^2, for other pathways. Table 4.4 is used to derive the values for station vent releases and Table 4.5 is used to derive the values for all other releases. The values used to calculate site boundary and critical receptor doses are as follows:

Station Vent Releases - Boundary			
Inhalation X/Q	7.17 E-7		
Meat D/Q	1.22 E-8	Ground D/Q	1.22 E-8
Cow/Milk/Infant D/Q	1.22 E-8	Vegetation D/Q	1.22 E-8
Station Vent Releases - Critical Receptor			
Inhalation X/Q	7.2 E-7		
Meat D/Q	4.6 E-9	Ground D/Q	7.8 E-9
Cow/Milk/Infant D/Q	3.1 E-9	Vegetation D/Q	8.9 E-9

Ground or Other Releases - Boundary			
Inhalation X/Q	1.16 E-5		
Meat D/Q	4.12 E-8	Ground D/Q	4.12 E-8
Cow/Milk/Infant D/Q	4.12 E-8	Vegetation D/Q	4.12 E-8
Ground or Other Releases - Critical Receptor			
Inhalation X/Q	1.2 E-5		
Meat D/Q	9.2 E-9	Ground D/Q	3.0 E-8
Cow/Milk/Infant D/Q	6.3 E-9	Vegetation D/Q	2.6 E-8

Dv(H-3) = In the case of H-3 only the X/Q's above are used for all pathways.

Q_i = release of I-131, I-133, Tritium and Radionuclides, i, in Particulate Form with half-lives greater than 8 days, in μCi, cumulative over the specified time period (μCi/second * seconds).

3.17E-8 = inverse of the number of seconds in a year.

5.3 Gaseous Radioactive System Dose Calculations Once per Month

ODCM Part I Control 2.2.2.4 and TMI-2 PDMS Tech Spec Section 6.7.4.a.6 requires that appropriate subsystem of the Gaseous Radwaste Treatment System shall be used to reduce the radioactive materials in gaseous waste prior to their discharge. When the monthly projected doses due to the gaseous effluent releases from the site would exceed:

0.2 mrad to air from gamma radiation; or
0.4 mrad to air from beta radiation; or
0.3 mrem to any organ.

The following calculational method is provided for performing this dose projection.

At least once per month the gamma air dose, beta air dose and the maximum organ dose for the quarter-to-date will be divided by the number of days into the quarter and multiplied by 31. Also, this dose projection shall include the estimated dose due to any anticipated unusual release during the period for which the projection is made. If these projected doses exceed any of the values listed above, appropriate portions of the TMI-1 Gaseous Waste Treatment System, as defined in Section 6.0, or appropriate portions of the TMI-2 Gaseous Effluent Filtration System as shown on Figure 4.5, shall be used to reduce radioactivity levels prior to release.

At the discretion of Radiological Engineering, time periods other than the current quarter-to-date may be used to project doses if the dose per day in the current quarter-to-date is not believed to be representative of the dose per day projected for the next month.

5.4 *Alternative Dose Calculational Methodologies for Gaseous Effluents*

As an alternative to the methods described above, the models in/or based upon, those presented in Regulatory Guide 1.109 (Rev. 1) may be used to make a comprehensive dose assessment. Default parameter values from Regulatory Guide 1.109 (Rev. 1) and/or actual site specific data can be used where applicable.

The onsite, on-line computerized system for tracking gaseous effluent dose uses annual average gaseous dispersion factors. As an alternative dose calculational methodology. TMI calculates doses using an advanced class "A" dispersion model called SEEDS (simplified environmental effluent dosimetry system).

This model incorporates the guidelines and methodology set forth in USNRC Regulatory Guide 1.109, and uses actual hourly meteorological information matched to the time of releases to more accurately assess the dispersion of effluents in the atmosphere. Combining this assessment of dispersion with TMI effluent data for each unit, postulated maximum hypothetical doses to the public are calculated.

TABLE 5.2.1

Pathway Dose Factors, Ri

AGE GROUP: INFANT PATHWAY: INHALATION

NUCLIDE	ORGAN DOSE FACTORS; mrem/year per $\mu Ci/m^3$						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	6.47E+02	6.47E+02	6.47E+02	6.47E+02	6.47E+02	6.47E+02
C-14	2.65E+04	5.31E+03	5.31E+03	5.31E+03	5.31E+03	5.31E+03	5.31E+03
CR-51	0.00E+00	0.00E+00	8.95E+01	5.75E+01	1.32E+01	1.28E+04	3.57E+02
MN-54	0.00E+00	2.53E+04	4.98E+03	0.00E+00	4.98E+03	1.00E+06	7.06E+03
FE-55	1.97E+04	1.17E+04	3.33E+03	0.00E+00	0.00E+00	8.69E+04	1.09E+03
FE-59	1.36E+04	2.35E+04	9.48E+03	0.00E+00	0.00E+00	1.02E+06	2.48E+04
CO-58	0.00E+00	1.22E+03	1.82E+03	0.00E+00	0.00E+00	7.77E+05	1.11E+04
CO-60	0.00E+00	8.02E+03	1.18E+04	0.00E+00	0.00E+00	4.51E+06	3.19E+04
NI-63	3.39E+05	2.04E+04	1.16E+04	0.00E+00	0.00E+00	2.09E+05	2.42E+03
ZN-65	1.93E+04	6.26E+04	3.11E+04	0.00E+00	3.25E+04	6.47E+05	5.14E+04
RB-86	0.00E+00	1.90E+05	8.82E+04	0.00E+00	0.00E+00	0.00E+00	3.04E+03
SR-89	3.98E+05	0.00E+00	1.14E+04	0.00E+00	0.00E+00	2.03E+06	6.40E+04
SR-90	4.09E+07	0.00E+00	2.59E+06	0.00E+00	0.00E+00	1.12E+07	1.31E+05
Y-91	5.88E+05	0.00E+00	1.57E+04	0.00E+00	0.00E+00	2.45E+06	7.03E+04
ZR-95	1.15E+05	2.79E+04	2.03E+04	0.00E+00	3.11E+04	1.75E+06	2.17E+04
NB-95	1.57E+04	6.43E+03	3.78E+03	0.00E+00	4.72E+03	4.79E+05	1.27E+04
RU-103	2.02E+03	0.00E+00	6.79E+02	0.00E+00	4.24E+03	5.52E+05	1.61E+04
RU-106	8.68E+04	0.00E+00	1.09E+04	0.00E+00	1.07E+05	1.16E+07	1.64E+05
AG-110M	9.98E+03	7.22E+03	5.00E+03	0.00E+00	1.09E+04	3.67E+06	3.30E+04
TE-125M	4.76E+03	1.99E+03	6.58E+02	1.62E+03	0.00E+00	4.47E+05	1.29E+04
TE-127M	1.67E+04	6.90E+03	2.07E+03	4.87E+03	3.75E+04	1.31E+06	2.73E+04
TE-129M	1.41E+04	6.09E+03	2.23E+03	5.47E+03	3.18E+04	1.68E+06	6.90E+04
I-131	3.79E+04	4.44E+04	1.96E+04	1.48E+07	5.18E+04	0.00E+00	1.06E+03
I-133	1.32E+04	1.92E+04	5.60E+03	3.56E+06	2.24E+04	0.00E+00	2.16E+03
CS-134	3.96E+05	7.03E+05	7.45E+04	0.00E+00	1.90E+05	7.97E+04	1.33E+03
CS-136	4.83E+04	1.35E+05	5.29E+04	0.00E+00	5.64E+04	1.18E+04	1.43E+03
CS-137	5.49E+05	6.12E+05	4.55E+04	0.00E+00	1.72E+05	7.13E+04	1.33E+03
BA-140	5.60E+04	5.60E+01	2.90E+03	0.00E+00	1.34E+01	1.60E+06	3.84E+04
CE-141	2.77E+04	1.67E+04	1.99E+03	0.00E+00	5.25E+03	5.17E+05	2.16E+04
CE-144	3.19E+06	1.21E+06	1.76E+05	0.00E+00	5.38E+05	9.84E+06	1.48E+05
PR-143	1.40E+04	5.24E+03	6.99E+02	0.00E+00	1.97E+03	4.33E+05	3.72E+04
ND-147	7.94E+03	8.13E+03	5.00E+02	0.00E+00	3.15E+03	3.22E+05	3.12E+04

TABLE 5.2.2

Pathway Dose Factors, Ri

AGE GROUP: CHILD PATHWAY: INHALATION

NUCLIDE	ORGAN DOSE FACTORS; mrem/year per μCi/m³						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	1.12E+03	1.12E+03	1.12E+03	1.12E+03	1.12E+03	1.12E+03
C-14	3.59E+04	6.73E+03	6.73E+03	6.73E+03	6.73E+03	6.73E+03	6.73E+03
CR-51	0.00E+00	0.00E+00	1.54E+02	8.55E+01	2.43E+01	1.70E+04	1.08E+03
MN-54	0.00E+00	4.29E+04	9.51E+03	0.00E+00	1.00E+04	1.58E+06	2.29E+04
FE-55	4.74E+04	2.52E+04	7.77E+03	0.00E+00	0.00E+00	1.11E+05	2.87E+03
FE-59	2.07E+04	3.34E+04	1.67E+04	0.00E+00	0.00E+00	1.27E+06	7.07E+04
CO-58	0.00E+00	1.77E+03	3.16E+03	0.00E+00	0.00E+00	1.11E+06	3.44E+04
CO-60	0.00E+00	1.31E+04	2.26E+04	0.00E+00	0.00E+00	7.07E+06	9.62E+04
NI-63	8.21E+05	4.63E+04	2.80E+04	0.00E+00	0.00E+00	2.75E+05	6.33E+03
ZN-65	4.26E+04	1.13E+05	7.03E+04	0.00E+00	7.14E+04	9.95E+05	1.63E+04
RB-86	0.00E+00	1.98E+05	1.14E+05	0.00E+00	0.00E+00	0.00E+00	7.99E+03
SR-89	5.99E+05	0.00E+00	1.72E+04	0.00E+00	0.00E+00	2.16E+06	1.67E+05
SR-90	1.01E+08	0.00E+00	6.44E+06	0.00E+00	0.00E+00	1.48E+07	3.43E+05
Y-91	9.14E+05	0.00E+00	2.44E+04	0.00E+00	0.00E+00	2.63E+06	1.84E+05
ZR-95	1.90E+05	4.18E+04	3.70E+04	0.00E+00	5.96E+04	2.23E+06	6.11E+04
NB-95	2.35E+04	9.18E+03	6.55E+03	0.00E+00	8.62E+03	6.14E+05	3.70E+04
RU-103	2.79E+03	0.00E+00	1.07E+03	0.00E+00	7.03E+03	6.62E+05	4.48E+04
RU-106	1.36E+05	0.00E+00	1.69E+04	0.00E+00	1.84E+05	1.43E+07	4.29E+05
AG-110M	1.69E+04	1.14E+04	9.14E+03	0.00E+00	2.12E+04	5.48E+06	1.00E+05
TE-125M	6.73E+03	2.33E+03	9.14E+02	1.92E+03	0.00E+00	4.77E+05	3.38E+04
TE-127M	2.49E+04	8.55E+03	3.02E+03	6.07E+03	6.36E+04	1.48E+06	7.14E+04
TE-129M	1.92E+04	6.85E+03	3.04E+03	6.33E+03	5.03E+04	1.76E+06	1.82E+05
I-131	4.81E+04	4.81E+04	2.73E+04	1.62E+07	7.88E+04	0.00E+00	2.84E+03
I-133	1.66E+04	2.03E+04	7.70E+03	3.85E+06	3.38E+04	0.00E+00	5.48E+03
CS-134	6.51E+05	1.01E+06	2.25E+05	0.00E+00	3.30E+05	1.21E+05	3.85E+03
CS-136	6.51E+04	1.71E+05	1.16E+05	0.00E+00	9.55E+04	1.45E+04	4.18E+03
CS-137	9.07E+05	8.25E+05	1.28E+05	0.00E+00	2.82E+05	1.04E+05	3.62E+03
BA-140	7.40E+04	6.48E+01	4.33E+03	0.00E+00	2.11E+01	1.74E+06	1.02E+05
CE-141	3.92E+04	1.95E+04	2.90E+03	0.00E+00	8.55E+03	5.44E+05	5.66E+04
CE-144	6.77E+06	2.12E+06	3.61E+05	0.00E+00	1.17E+06	1.20E+07	3.89E+05
PR-143	1.85E+04	5.55E+03	9.14E+02	0.00E+00	3.00E+03	4.33E+05	9.73E+04
ND-147	1.08E+04	8.73E+03	6.81E+02	0.00E+00	4.81E+03	3.28E+05	8.21E+04

TABLE 5.2.3

Pathway Dose Factors, Ri

AGE GROUP: TEEN PATHWAY: INHALATION

NUCLIDE	ORGAN DOSE FACTORS; mrem/year per μCi/m^3						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	1.27E+03	1.27E+03	1.27E+03	1.27E+03	1.27E+03	1.27E+03
C-14	2.60E+04	4.87E+03	4.87E+03	4.87E+03	4.87E+03	4.87E+03	4.87E+03
CR-51	0.00E+00	0.00E+00	1.35E+02	7.50E+01	3.07E+01	2.10E+04	3.00E+03
MN-54	0.00E+00	5.11E+04	8.40E+03	0.00E+00	1.27E+04	1.98E+06	6.68E+04
FE-55	3.34E+04	2.38E+04	5.54E+03	0.00E+00	0.00E+00	1.24E+05	6.39E+03
FE-59	1.59E+04	3.70E+04	1.43E+04	0.00E+00	0.00E+00	1.53E+06	1.78E+05
CO-58	0.00E+00	2.07E+03	2.78E+03	0.00E+00	0.00E+00	1.34E+06	9.52E+04
CO-60	0.00E+00	1.51E+04	1.98E+04	0.00E+00	0.00E+00	8.72E+06	2.59E+05
NI-63	5.80E+05	4.34E+04	1.98E+04	0.00E+00	0.00E+00	3.07E+05	1.42E+04
ZN-65	3.86E+04	1.34E+05	6.24E+04	0.00E+00	8.64E+04	1.24E+06	4.66E+04
RB-86	0.00E+00	1.90E+05	8.40E+04	0.00E+00	0.00E+00	0.00E+00	1.77E+04
SR-89	4.34E+05	0.00E+00	1.25E+04	0.00E+00	0.00E+00	2.42E+06	3.71E+05
SR-90	1.08E+08	0.00E+00	6.68E+06	0.00E+00	0.00E+00	1.65E+07	7.65E+05
Y-91	6.61E+05	0.00E+00	1.77E+04	0.00E+00	0.00E+00	2.94E+06	4.09E+05
ZR-95	1.46E+05	4.58E+04	3.15E+04	0.00E+00	6.74E+04	2.69E+06	1.49E+05
NB-95	1.86E+04	1.03E+04	5.66E+03	0.00E+00	1.00E+04	7.51E+05	9.68E+04
RU-103	2.10E+03	0.00E+00	8.96E+02	0.00E+00	7.43E+03	7.83E+05	1.09E+05
RU-106	9.84E+04	0.00E+00	1.24E+04	0.00E+00	1.90E+05	1.61E+07	9.60E+05
AG-110M	1.38E+04	1.31E+04	7.99E+03	0.00E+00	2.50E+04	6.75E+06	2.73E+05
TE-125M	4.88E+03	2.24E+03	6.67E+02	1.40E+03	0.00E+00	5.36E+05	7.50E+04
TE-127M	1.80E+04	8.16E+03	2.18E+03	4.38E+03	6.54E+04	1.66E+06	1.59E+05
TE-129M	1.39E+04	6.58E+03	2.25E+03	4.58E+03	5.19E+04	1.98E+06	4.05E+05
I-131	3.54E+04	4.91E+04	2.64E+04	1.46E+07	8.40E+04	0.00E+00	6.49E+03
I-133	1.22E+04	2.05E+04	6.22E+03	2.92E+06	3.59E+04	0.00E+00	1.03E+04
CS-134	5.02E+05	1.13E+06	5.49E+05	0.00E+00	3.75E+05	1.46E+05	9.76E+03
CS-136	5.15E+04	1.94E+05	1.37E+05	0.00E+00	1.10E+05	1.78E+04	1.09E+04
CS-137	6.70E+05	8.48E+05	3.11E+05	0.00E+00	3.04E+05	1.21E+05	8.48E+03
BA-140	5.47E+04	6.70E+01	3.52E+03	0.00E+00	2.28E+01	2.03E+06	2.29E+05
CE-141	2.84E+04	1.90E+04	2.17E+03	0.00E+00	8.88E+03	6.14E+05	1.26E+05
CE-144	4.89E+06	2.02E+06	2.62E+05	0.00E+00	1.21E+06	1.34E+07	8.64E+05
PR-143	1.34E+04	5.31E+03	6.62E+02	0.00E+00	3.09E+03	4.83E+05	2.14E+05
ND-147	7.86E+03	8.56E+03	5.13E+02	0.00E+00	5.02E+03	3.72E+05	1.82E+05

TABLE 5.2.4

Pathway Dose Factors, Ri

AGE GROUP: ADULT PATHWAY: INHALATION

NUCLIDE	ORGAN DOSE FACTORS; mrem/year per μCi/m^3						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03	1.26E+03
C-14	1.82E+04	3.41E+03	3.41E+03	3.41E+03	3.41E+03	3.41E+03	3.41E+03
CR-51	0.00E+00	0.00E+00	1.00E+02	5.95E+01	2.28E+01	1.44E+04	3.32E+03
MN-54	0.00E+00	3.96E+04	6.30E+03	0.00E+00	9.84E+03	1.40E+06	7.74E+04
FE-55	2.46E+04	1.70E+04	3.94E+03	0.00E+00	0.00E+00	7.21E+04	6.03E+03
FE-59	1.18E+04	2.78E+04	1.06E+04	0.00E+00	0.00E+00	1.02E+06	1.88E+05
CO-58	0.00E+00	1.58E+03	2.07E+03	0.00E+00	0.00E+00	9.28E+05	1.06E+05
CO-60	0.00E+00	1.15E+04	1.48E+04	0.00E+00	0.00E+00	5.97E+06	2.85E+05
NI-63	4.32E+05	3.14E+04	1.45E+04	0.00E+00	0.00E+00	1.78E+05	1.34E+04
ZN-65	3.24E+04	1.03E+05	4.66E+04	0.00E+00	6.90E+04	8.64E+05	5.34E+04
RB-86	0.00E+00	1.35E+05	5.90E+04	0.00E+00	0.00E+00	0.00E+00	1.66E+04
SR-89	3.04E+05	0.00E+00	8.72E+03	0.00E+00	0.00E+00	1.40E+06	3.50E+05
SR-90	9.92E+07	0.00E+00	6.10E+06	0.00E+00	0.00E+00	9.60E+06	7.22E+05
Y-91	4.62E+05	0.00E+00	1.24E+04	0.00E+00	0.00E+00	1.70E+06	3.85E+05
ZR-95	1.07E+05	3.44E+04	2.33E+04	0.00E+00	5.42E+04	1.77E+06	1.50E+05
NB-95	1.41E+04	7.82E+03	4.21E+03	0.00E+00	7.74E+03	5.05E+05	1.04E+05
RU-103	1.53E+03	0.00E+00	6.58E+02	0.00E+00	5.83E+03	5.05E+05	1.10E+05
RU-106	6.91E+04	0.00E+00	8.72E+03	0.00E+00	1.34E+05	9.36E+06	9.12E+05
AG-110M	1.08E+04	1.00E+04	5.94E+03	0.00E+00	1.97E+04	4.63E+06	3.02E+05
TE-125M	3.42E+03	1.58E+03	4.67E+02	1.05E+03	1.24E+04	3.14E+05	7.06E+04
TE-127M	1.26E+04	5.77E+03	1.57E+03	3.29E+03	4.58E+04	9.60E+05	1.50E+05
TE-129M	*9.76E+03*	*4.67E+03*	*1.58E+03*	*3.44E+03*	*3.66E+04*	*1.16E+06*	*3.83E+05*
I-131	2.52E+04	3.58E+04	2.05E+04	1.19E+07	6.13E+04	0.00E+00	6.28E+03
I-133	8.64E+03	1.48E+04	4.52E+03	2.15E+06	2.58E+04	0.00E+00	8.88E+03
CS-134	3.73E+05	8.48E+05	7.28E+05	0.00E+00	2.87E+05	9.76E+04	1.04E+04
CS-136	3.90E+04	1.46E+05	1.10E+05	0.00E+00	8.56E+04	1.20E+04	1.17E+04
CS-137	4.78E+05	6.21E+05	4.28E+05	0.00E+00	2.22E+05	7.52E+04	8.40E+03
BA-140	3.90E+04	4.90E+01	2.57E+03	0.00E+00	1.67E+01	1.27E+06	2.18E+05
CE-141	1.99E+04	1.35E+04	1.53E+03	0.00E+00	6.26E+03	3.62E+05	1.20E+05
CE-144	3.43E+06	1.43E+06	1.84E+05	0.00E+00	8.48E+05	7.78E+06	8.16E+05
PR-143	9.36E+03	3.75E+03	4.64E+02	0.00E+00	2.16E+03	2.81E+05	2.00E+05
ND-147	5.27E+03	6.10E+03	3.65E+02	0.00E+00	3.56E+03	2.21E+05	1.73E+05

TABLE 5.3.1

Pathway Dose Factors, Ri

AGE GROUP: ALL PATHWAY: GROUND PLANE

NUCLIDE	ORGAN DOSE FACTORS*	
	T.BODY	SKIN
H-3	0.00E+00	0.00E+00
C-14	0.00E+00	0.00E+00
CR-51	4.65E+06	5.50E+06
MN-54	1.39E+09	1.62E+09
FE-55	0.00E+00	0.00E+00
FE-59	2.73E+08	3.21E+08
CO-58	3.79E+08	4.44E+08
CO-60	2.15E+10	2.53E+10
NI-63	0.00E+00	0.00E+00
ZN-65	7.47E+08	8.59E+08
RB-86	8.97E+06	1.03E+07
SR-89	2.16E+04	2.51E+04
SR-90	0.00E+00	0.00E+00
Y-91	1.07E+06	1.21E+06
ZR-95	2.45E+08	2.84E+08
NB-95	1.37E+08	1.61E+08
RU-103	1.08E+08	1.26E+08
RU-106	4.22E+08	5.06E+08
AG-110M	3.44E+09	4.01E+09
TE-125M	1.55E+06	2.13E+06
TE-127M	9.17E+04	1.08E+05
TE-129M	1.98E+07	2.31E+07
I-131	1.72E+07	2.09E+07
I-133	2.45E+06	2.98E+06
CS-134	6.86E+09	8.00E+09
CS-136	1.51E+08	1.71E+08
CS-137	1.03E+10	1.20E+10
BA-140	2.06E+07	2.36E+07
CE-141	1.37E+07	1.54E+07
CE-144	6.96E+07	8.05E+07
PR-143	0.00E+00	0.00E+00
ND-147	8.39E+06	1.01E+07

* m^2 - mrem/year per µCi/sec.

TABLE 5.4.1

Pathway Dose Factors, Ri

AGE GROUP: INFANT PATHWAY: GRASS-COW-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	2.38E+03	2.38E+03	2.38E+03	2.38E+03	2.38E+03	2.38E+03
C-14	2.34E+09	5.00E+08	5.00E+08	5.00E+08	5.00E+08	5.00E+08	5.00E+08
CR-51	0.00E+00	0.00E+00	1.61E+05	1.05E+05	2.30E+04	2.05E+05	4.70E+06
MN-54	0.00E+00	3.91E+07	8.85E+06	0.00E+00	8.65E+06	0.00E+00	1.43E+07
FE-55	1.35E+08	8.74E+07	2.34E+07	0.00E+00	0.00E+00	4.27E+07	1.11E+07
FE-59	2.25E+08	3.93E+08	1.55E+08	0.00E+00	0.00E+00	1.16E+08	1.88E+08
CO-58	0.00E+00	2.43E+07	6.06E+07	0.00E+00	0.00E+00	0.00E+00	6.05E+07
CO-60	0.00E+00	8.83E+07	2.08E+08	0.00E+00	0.00E+00	0.00E+00	2.10E+08
NI-63	3.50E+10	2.16E+09	1.21E+09	0.00E+00	0.00E+00	0.00E+00	1.08E+08
ZN-65	5.56E+09	1.91E+10	8.79E+09	0.00E+00	9.24E+09	0.00E+00	1.61E+10
RB-86	0.00E+00	2.23E+10	1.10E+10	0.00E+00	0.00E+00	0.00E+00	5.70E+08
SR-89	1.26E+10	0.00E+00	3.62E+08	0.00E+00	0.00E+00	0.00E+00	2.59E+08
SR-90	1.22E+11	0.00E+00	3.10E+10	0.00E+00	0.00E+00	0.00E+00	1.52E+09
Y-91	7.34E+04	0.00E+00	1.95E+03	0.00E+00	0.00E+00	0.00E+00	5.26E+06
ZR-95	6.81E+03	1.66E+03	1.18E+03	0.00E+00	1.79E+03	0.00E+00	8.27E+05
NB-95	5.94E+05	2.45E+05	1.41E+05	0.00E+00	1.75E+05	0.00E+00	2.07E+08
RU-103	8.68E+03	0.00E+00	2.90E+03	0.00E+00	1.81E+04	0.00E+00	1.06E+05
RU-106	1.91E+05	0.00E+00	2.38E+04	0.00E+00	2.25E+05	0.00E+00	1.45E+06
AG-110M	3.86E+08	2.82E+08	1.87E+08	0.00E+00	4.03E+08	0.00E+00	1.46E+10
TE-125M	1.51E+08	5.05E+07	2.04E+07	5.08E+07	0.00E+00	0.00E+00	7.19E+07
TE-127M	4.22E+08	1.40E+08	5.10E+07	1.22E+08	1.04E+09	0.00E+00	1.70E+08
TE-129M	5.58E+08	1.91E+08	8.59E+07	2.14E+08	1.39E+09	0.00E+00	3.33E+08
I-131	2.72E+09	3.21E+09	1.41E+09	1.05E+12	3.75E+09	0.00E+00	1.15E+08
I-133	3.63E+07	5.29E+07	1.55E+07	9.62E+09	6.22E+07	0.00E+00	8.96E+06
CS-134	3.65E+10	6.81E+10	6.88E+09	0.00E+00	1.75E+10	7.19E+09	1.85E+08
CS-136	1.98E+09	5.83E+09	2.18E+09	0.00E+00	2.32E+09	4.75E+08	8.85E+07
CS-137	5.15E+10	6.03E+10	4.27E+09	0.00E+00	1.62E+10	6.55E+09	1.89E+08
BA-140	2.42E+08	2.42E+05	1.25E+07	0.00E+00	5.75E+04	1.49E+05	5.94E+07
CE-141	4.34E+04	2.65E+04	3.12E+03	0.00E+00	8.17E+03	0.00E+00	1.37E+07
CE-144	2.33E+06	9.53E+05	1.30E+05	0.00E+00	3.85E+05	0.00E+00	1.34E+08
PR-143	1.49E+03	5.56E+02	7.37E+01	0.00E+00	2.07E+02	0.00E+00	7.84E+05
ND-147	8.83E+02	9.07E+02	5.55E+01	0.00E+00	3.50E+02	0.00E+00	5.75E+05

TABLE 5.4.2

Pathway Dose Factors, Ri

AGE GROUP: CHILD PATHWAY: GRASS-COW-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	1.57E+03	1.57E+03	1.57E+03	1.57E+03	1.57E+03	1.57E+03
C-14	1.20E+09	2.39E+08	2.39E+08	2.39E+08	2.39E+08	2.39E+08	2.39E+08
CR-51	0.00E+00	0.00E+00	1.02E+05	5.65E+04	1.54E+04	1.03E+05	5.40E+06
MN-54	0.00E+00	2.10E+07	5.59E+06	0.00E+00	5.89E+06	0.00E+00	1.76E+07
FE-55	1.12E+08	5.94E+07	1.84E+07	0.00E+00	0.00E+00	3.36E+07	1.10E+07
FE-59	1.20E+08	1.95E+08	9.70E+07	0.00E+00	0.00E+00	5.65E+07	2.03E+08
CO-58	0.00E+00	1.21E+07	3.72E+07	0.00E+00	0.00E+00	0.00E+00	7.08E+07
CO-60	0.00E+00	4.32E+07	1.27E+08	0.00E+00	0.00E+00	0.00E+00	2.39E+08
NI-63	2.97E+10	1.59E+09	1.01E+09	0.00E+00	0.00E+00	0.00E+00	1.07E+08
ZN-65	4.14E+09	1.10E+10	6.86E+09	0.00E+00	6.95E+09	0.00E+00	1.94E+09
RB-86	0.00E+00	8.78E+09	5.40E+09	0.00E+00	0.00E+00	0.00E+00	5.65E+08
SR-89	6.63E+09	0.00E+00	1.89E+08	0.00E+00	0.00E+00	0.00E+00	2.57E+08
SR-90	1.12E+11	0.00E+00	2.84E+10	0.00E+00	0.00E+00	0.00E+00	1.51E+09
Y-91	3.91E+04	0.00E+00	1.05E+03	0.00E+00	0.00E+00	0.00E+00	5.21E+06
ZR-95	3.84E+03	8.43E+02	7.51E+02	0.00E+00	1.21E+03	0.00E+00	8.80E+05
NB-95	3.18E+05	1.24E+05	8.86E+04	0.00E+00	1.16E+05	0.00E+00	2.29E+08
RU-103	4.29E+03	0.00E+00	1.65E+03	0.00E+00	1.08E+04	0.00E+00	1.11E+05
RU-106	9.25E+04	0.00E+00	1.15E+04	0.00E+00	1.25E+05	0.00E+00	1.44E+06
AG-110M	2.09E+08	1.41E+08	1.13E+08	0.00E+00	2.63E+08	0.00E+00	1.68E+10
TE-125M	7.39E+07	2.00E+07	9.85E+06	2.07E+07	0.00E+00	0.00E+00	7.13E+07
TE-127M	2.08E+08	5.61E+07	2.47E+07	4.98E+07	5.94E+08	0.00E+00	1.69E+08
TE-129M	2.72E+08	7.59E+07	4.22E+07	8.76E+07	7.98E+08	0.00E+00	3.31E+08
I-131	1.31E+09	1.31E+09	7.46E+08	4.34E+11	2.16E+09	0.00E+00	1.17E+08
I-133	1.72E+07	2.13E+07	8.05E+06	3.95E+09	3.55E+07	0.00E+00	8.58E+06
CS-134	2.27E+10	3.72E+10	7.85E+09	0.00E+00	1.15E+10	4.14E+09	2.01E+08
CS-136	1.01E+09	2.79E+09	1.80E+09	0.00E+00	1.49E+09	2.21E+08	9.80E+07
CS-137	3.23E+10	3.09E+10	4.56E+09	0.00E+00	1.01E+10	3.62E+09	1.93E+08
BA-140	1.18E+08	1.03E+05	6.86E+06	0.00E+00	3.35E+04	6.14E+04	5.96E+07
CE-141	2.19E+04	1.09E+04	1.62E+03	0.00E+00	4.79E+03	0.00E+00	1.36E+07
CE-144	1.63E+06	5.09E+05	8.67E+04	0.00E+00	2.82E+05	0.00E+00	1.33E+08
PR-143	7.18E+02	2.16E+02	3.56E+01	0.00E+00	1.17E+02	0.00E+00	7.75E+05
ND-147	4.45E+02	3.61E+02	2.79E+01	0.00E+00	1.98E+02	0.00E+00	5.71E+05

TABLE 5.4.3

Pathway Dose Factors, RI

AGE GROUP: TEEN PATHWAY: GRASS-COW-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	9.93E+02	9.93E+02	9.93E+02	9.93E+02	9.93E+02	9.93E+02
C-14	4.86E+08	9.73E+07	9.73E+07	9.73E+07	9.73E+07	9.73E+07	9.73E+07
CR-51	0.00E+00	0.00E+00	4.99E+04	2.77E+04	1.09E+04	7.13E+04	8.39E+06
MN-54	0.00E+00	1.40E+07	2.78E+06	0.00E+00	4.19E+06	0.00E+00	2.88E+07
FE-55	4.46E+07	3.16E+07	7.37E+06	0.00E+00	0.00E+00	2.01E+07	1.37E+07
FE-59	5.19E+07	1.21E+08	4.68E+07	0.00E+00	0.00E+00	3.82E+07	2.86E+08
CO-58	0.00E+00	7.94E+06	1.83E+07	0.00E+00	0.00E+00	0.00E+00	1.10E+08
CO-60	0.00E+00	2.78E+07	6.27E+07	0.00E+00	0.00E+00	0.00E+00	3.62E+08
NI-63	1.18E+10	8.36E+08	4.01E+08	0.00E+00	0.00E+00	0.00E+00	1.33E+08
ZN-65	2.11E+09	7.32E+09	3.42E+09	0.00E+00	4.69E+09	0.00E+00	3.10E+09
RB-86	0.00E+00	4.73E+09	2.22E+09	0.00E+00	0.00E+00	0.00E+00	7.00E+08
SR-89	2.68E+09	0.00E+00	7.67E+07	0.00E+00	0.00E+00	0.00E+00	3.19E+08
SR-90	6.62E+10	0.00E+00	1.63E+10	0.00E+00	0.00E+00	0.00E+00	1.86E+09
Y-91	1.58E+04	0.00E+00	4.24E+02	0.00E+00	0.00E+00	0.00E+00	6.48E+06
ZR-95	1.65E+03	5.21E+02	3.58E+02	0.00E+00	7.65E+02	0.00E+00	1.20E+06
NB-95	1.41E+05	7.82E+04	4.30E+04	0.00E+00	7.58E+04	0.00E+00	3.34E+08
RU-103	1.81E+03	0.00E+00	7.75E+02	0.00E+00	6.39E+03	0.00E+00	1.51E+05
RU-106	3.76E+04	0.00E+00	4.73E+03	0.00E+00	7.24E+04	0.00E+00	1.80E+06
AG-110M	9.64E+07	9.12E+07	5.55E+07	0.00E+00	1.74E+08	0.00E+00	2.56E+10
TE-125M	3.01E+07	1.08E+07	4.02E+06	8.40E+06	0.00E+00	0.00E+00	8.87E+07
TE-127M	8.45E+07	3.00E+07	1.00E+07	2.01E+07	3.42E+08	0.00E+00	2.11E+08
TE-129M	1.10E+08	4.09E+07	1.74E+07	3.56E+07	4.61E+08	0.00E+00	4.14E+08
I-131	5.38E+08	7.53E+08	4.05E+08	2.20E+11	1.30E+09	0.00E+00	1.49E+08
I-133	7.08E+06	1.20E+07	3.66E+06	1.68E+09	2.11E+07	0.00E+00	9.09E+06
CS-134	9.83E+09	2.31E+10	1.07E+10	0.00E+00	7.35E+09	2.81E+09	2.88E+08
CS-136	4.49E+08	1.77E+09	1.19E+09	0.00E+00	9.63E+08	1.52E+08	1.42E+08
CS-137	1.34E+10	1.78E+10	6.21E+09	0.00E+00	6.06E+09	2.36E+09	2.54E+08
BA-140	4.87E+07	5.97E+04	3.14E+06	0.00E+00	2.02E+04	4.01E+04	7.51E+07
CE-141	8.89E+03	5.94E+03	6.82E+02	0.00E+00	2.80E+03	0.00E+00	1.70E+07
CE-144	6.59E+05	2.73E+05	3.54E+04	0.00E+00	1.63E+05	0.00E+00	1.66E+08
PR-143	2.90E+02	1.16E+02	1.44E+01	0.00E+00	6.73E+01	0.00E+00	9.55E+05
ND-147	1.81E+02	1.97E+02	1.18E+01	0.00E+00	1.16E+02	0.00E+00	7.12E+05

TABLE 5.4.4

Pathway Dose Factors, Ri

AGE GROUP: ADULT PATHWAY: GRASS-COW-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	7.62E+02	7.62E+02	7.62E+02	7.62E+02	7.62E+02	7.62E+02
C-14	2.63E+08	5.26E+07	5.26E+07	5.26E+07	5.26E+07	5.26E+07	5.26E+07
CR-51	0.00E+00	0.00E+00	2.85E+04	1.70E+04	6.28E+03	3.78E+04	7.17E+06
MN-54	0.00E+00	8.40E+06	1.60E+06	0.00E+00	2.50E+06	0.00E+00	2.57E+07
FE-55	2.51E+07	1.73E+07	4.04E+06	0.00E+00	0.00E+00	9.66E+06	9.93E+06
FE-59	2.97E+07	6.97E+07	2.67E+07	0.00E+00	0.00E+00	1.95E+07	2.32E+08
CO-58	0.00E+00	4.71E+06	1.05E+07	0.00E+00	0.00E+00	0.00E+00	9.54E+07
CO-60	0.00E+00	1.64E+07	3.61E+07	0.00E+00	0.00E+00	0.00E+00	3.08E+08
NI-63	6.72E+09	4.65E+08	2.25E+08	0.00E+00	0.00E+00	0.00E+00	9.71E+07
ZN-65	1.37E+09	4.36E+09	1.97E+09	0.00E+00	2.91E+09	0.00E+00	2.74E+09
RB-86	0.00E+00	2.59E+09	1.21E+09	0.00E+00	0.00E+00	0.00E+00	5.10E+08
SR-89	1.45E+09	0.00E+00	4.16E+07	0.00E+00	0.00E+00	0.00E+00	2.32E+08
SR-90	4.67E+10	0.00E+00	1.15E+10	0.00E+00	0.00E+00	0.00E+00	1.35E+09
Y-91	8.57E+03	0.00E+00	2.29E+02	0.00E+00	0.00E+00	0.00E+00	4.72E+06
ZR-95	9.41E+02	3.02E+02	2.04E+02	0.00E+00	4.74E+02	0.00E+00	9.57E+05
NB-95	8.24E+04	4.58E+04	2.46E+04	0.00E+00	4.53E+04	0.00E+00	2.78E+08
RU-103	1.02E+03	0.00E+00	4.38E+02	0.00E+00	3.88E+03	0.00E+00	1.19E+05
RU-106	2.04E+04	0.00E+00	2.58E+03	0.00E+00	3.93E+04	0.00E+00	1.32E+06
AG-110M	5.81E+07	5.38E+07	3.19E+07	0.00E+00	1.06E+08	0.00E+00	2.19E+10
TE-125M	1.63E+07	5.89E+06	2.18E+06	4.89E+06	6.61E+07	0.00E+00	6.49E+07
TE-127M	4.57E+07	1.63E+07	5.57E+06	1.17E+07	1.86E+08	0.00E+00	1.53E+08
TE-129M	6.01E+07	2.24E+07	9.51E+06	2.06E+07	2.51E+08	0.00E+00	3.02E+08
I-131	2.96E+08	4.23E+08	2.42E+08	1.39E+11	7.25E+08	0.00E+00	1.12E+08
I-133	3.87E+06	6.73E+06	2.05E+06	9.88E+08	1.17E+07	0.00E+00	6.04E+06
CS-134	5.64E+09	1.34E+10	1.10E+10	0.00E+00	4.34E+09	1.44E+09	2.35E+08
CS-136	2.63E+08	1.04E+09	7.48E+08	0.00E+00	5.78E+08	7.92E+07	1.18E+08
CS-137	7.37E+09	1.01E+10	6.60E+09	0.00E+00	3.42E+09	1.14E+09	1.95E+08
BA-140	2.69E+07	3.38E+04	1.76E+06	0.00E+00	1.15E+04	1.94E+04	5.54E+07
CE-141	4.84E+03	3.27E+03	3.71E+02	0.00E+00	1.52E+03	0.00E+00	1.25E+07
CE-144	3.57E+05	1.49E+05	1.92E+04	0.00E+00	8.85E+04	0.00E+00	1.21E+08
PR-143	1.57E+02	6.32E+01	7.81E+00	0.00E+00	3.65E+01	0.00E+00	6.90E+05
ND-147	9.40E+01	1.09E+02	6.50E+00	0.00E+00	6.35E+01	0.00E+00	5.22E+05

TABLE 5.5.1

Pathway Dose Factors, Ri

AGE GROUP: INFANT PATHWAY: GRASS-GOAT-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	4.86E+03	4.86E+03	4.86E+03	4.86E+03	4.86E+03	4.86E+03
C-14	2.34E+09	5.00E+08	5.00E+08	5.00E+08	5.00E+08	5.00E+08	5.00E+08
CR-51	0.00E+00	0.00E+00	1.94E+04	1.26E+04	2.76E+03	2.46E+04	5.64E+05
MN-54	0.00E+00	4.68E+06	1.06E+06	0.00E+00	1.04E+06	0.00E+00	1.72E+06
FE-55	1.76E+06	1.14E+06	3.03E+05	0.00E+00	0.00E+00	5.55E+05	1.44E+05
FE-59	2.92E+06	5.10E+06	2.01E+06	0.00E+00	0.00E+00	1.51E+06	2.44E+06
CO-58	0.00E+00	2.91E+06	7.26E+06	0.00E+00	0.00E+00	0.00E+00	7.25E+06
CO-60	0.00E+00	1.06E+07	2.50E+07	0.00E+00	0.00E+00	0.00E+00	2.52E+07
NI-63	4.19E+09	2.59E+08	1.46E+08	0.00E+00	0.00E+00	0.00E+00	1.29E+07
ZN-65	6.67E+08	2.29E+09	1.05E+09	0.00E+00	1.11E+09	0.00E+00	1.93E+09
RB-86	0.00E+00	2.67E+09	1.32E+09	0.00E+00	0.00E+00	0.00E+00	6.83E+07
SR-89	2.65E+10	0.00E+00	7.59E+08	0.00E+00	0.00E+00	0.00E+00	5.44E+08
SR-90	2.55E+11	0.00E+00	6.50E+10	0.00E+00	0.00E+00	0.00E+00	3.19E+09
Y-91	8.80E+03	0.00E+00	2.34E+02	0.00E+00	0.00E+00	0.00E+00	6.31E+05
ZR-95	8.17E+02	1.99E+02	1.41E+02	0.00E+00	2.15E+02	0.00E+00	9.91E+04
NB-95	7.13E+04	2.93E+04	1.70E+04	0.00E+00	2.10E+04	0.00E+00	2.48E+07
RU-103	1.04E+03	0.00E+00	3.48E+02	0.00E+00	2.17E+03	0.00E+00	1.27E+04
RU-106	2.28E+04	0.00E+00	2.85E+03	0.00E+00	2.70E+04	0.00E+00	1.73E+05
AG-110M	4.63E+07	3.38E+07	2.24E+07	0.00E+00	4.84E+07	0.00E+00	1.75E+09
TE-125M	1.81E+07	6.05E+06	2.45E+06	6.09E+06	0.00E+00	0.00E+00	8.62E+06
TE-127M	5.06E+07	1.68E+07	6.12E+06	1.46E+07	1.24E+08	0.00E+00	2.04E+07
TE-129M	6.69E+07	2.29E+07	1.03E+07	2.57E+07	1.67E+08	0.00E+00	3.99E+07
I-131	3.27E+09	3.85E+09	1.69E+09	1.27E+12	4.50E+09	0.00E+00	1.37E+08
I-133	4.36E+07	6.35E+07	1.86E+07	1.15E+10	7.46E+07	0.00E+00	1.07E+07
CS-134	1.09E+11	2.04E+11	2.06E+10	0.00E+00	5.26E+10	2.15E+10	5.55E+08
CS-136	5.94E+09	1.75E+10	6.52E+09	0.00E+00	6.96E+09	1.42E+09	2.65E+08
CS-137	1.54E+11	1.81E+11	1.28E+10	0.00E+00	4.85E+10	1.96E+10	5.65E+08
BA-140	2.90E+07	2.90E+04	1.50E+06	0.00E+00	6.89E+03	1.78E+04	7.13E+06
CE-141	5.21E+03	3.18E+03	3.74E+02	0.00E+00	9.79E+02	0.00E+00	1.64E+06
CE-144	2.79E+05	1.14E+05	1.56E+04	0.00E+00	4.62E+04	0.00E+00	1.60E+07
PR-143	1.78E+02	6.66E+01	8.83E+00	0.00E+00	2.48E+01	0.00E+00	9.40E+04
ND-147	1.06E+02	1.09E+02	6.66E+00	0.00E+00	4.19E+01	0.00E+00	6.89E+04

TABLE 5.5.2

Pathway Dose Factors, Ri

AGE GROUP: CHILD PATHWAY: GRASS-GOAT-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	3.20E+03	3.20E+03	3.20E+03	3.20E+03	3.20E+03	3.20E+03
C-14	1.20E+09	2.39E+08	2.39E+08	2.39E+08	2.39E+08	2.39E+08	2.39E+08
CR-51	0.00E+00	0.00E+00	1.22E+04	6.78E+03	1.85E+03	1.24E+04	6.48E+05
MN-54	0.00E+00	2.52E+06	6.71E+05	0.00E+00	7.06E+05	0.00E+00	2.11E+06
FE-55	1.45E+06	7.71E+05	2.39E+05	0.00E+00	0.00E+00	4.36E+05	1.43E+05
FE-59	1.56E+06	2.53E+06	1.26E+06	0.00E+00	0.00E+00	7.34E+05	2.64E+06
CO-58	0.00E+00	1.46E+06	4.46E+06	0.00E+00	0.00E+00	0.00E+00	8.49E+06
CO-60	0.00E+00	5.18E+06	1.53E+07	0.00E+00	0.00E+00	0.00E+00	2.87E+07
NI-63	3.56E+09	1.91E+08	1.21E+08	0.00E+00	0.00E+00	0.00E+00	1.28E+07
ZN-65	4.96E+08	1.32E+09	8.22E+08	0.00E+00	8.33E+08	0.00E+00	2.32E+08
RB-86	0.00E+00	1.05E+09	6.47E+08	0.00E+00	0.00E+00	0.00E+00	6.77E+07
SR-89	1.39E+10	0.00E+00	3.97E+08	0.00E+00	0.00E+00	0.00E+00	5.39E+08
SR-90	2.35E+11	0.00E+00	5.95E+10	0.00E+00	0.00E+00	0.00E+00	3.16E+09
Y-91	4.69E+03	0.00E+00	1.25E+02	0.00E+00	0.00E+00	0.00E+00	6.24E+05
ZR-95	4.60E+02	1.01E+02	9.00E+01	0.00E+00	1.45E+02	0.00E+00	1.05E+05
NB-95	3.82E+04	1.49E+04	1.06E+04	0.00E+00	1.40E+04	0.00E+00	2.75E+07
RU-103	5.14E+02	0.00E+00	1.98E+02	0.00E+00	1.29E+03	0.00E+00	1.33E+04
RU-106	1.11E+04	0.00E+00	1.38E+03	0.00E+00	1.50E+04	0.00E+00	1.73E+05
AG-110M	2.51E+07	1.69E+07	1.35E+07	0.00E+00	3.15E+07	0.00E+00	2.01E+09
TE-125M	8.86E+06	2.40E+06	1.18E+06	2.49E+06	0.00E+00	0.00E+00	8.55E+06
TE-127M	2.50E+07	6.72E+06	2.96E+06	5.97E+06	7.12E+07	0.00E+00	2.02E+07
TE-129M	3.26E+07	9.10E+06	5.06E+06	1.05E+07	9.56E+07	0.00E+00	3.97E+07
I-131	1.57E+09	1.57E+09	8.95E+08	5.21E+11	2.58E+09	0.00E+00	1.40E+08
I-133	2.06E+07	2.55E+07	9.66E+06	4.74E+09	4.25E+07	0.00E+00	1.03E+07
CS-134	6.80E+10	1.12E+11	2.35E+10	0.00E+00	3.46E+10	1.24E+10	6.01E+08
CS-136	3.04E+09	8.36E+09	5.41E+09	0.00E+00	4.45E+09	6.64E+08	2.94E+08
CS-137	9.68E+10	9.26E+10	1.37E+10	0.00E+00	3.02E+10	1.09E+10	5.80E+08
BA-140	1.41E+07	1.24E+04	8.23E+05	0.00E+00	4.02E+03	7.37E+03	7.15E+06
CE-141	2.63E+03	1.31E+03	1.95E+02	0.00E+00	5.74E+02	0.00E+00	1.63E+06
CE-144	1.95E+05	6.11E+04	1.04E+04	0.00E+00	3.38E+04	0.00E+00	1.59E+07
PR-143	8.61E+01	2.59E+01	4.27E+00	0.00E+00	1.40E+01	0.00E+00	9.29E+04
ND-147	5.34E+01	4.33E+01	3.35E+00	0.00E+00	2.37E+01	0.00E+00	6.85E+04

TABLE 5.5.3

Pathway Dose Factors, Ri

AGE GROUP: TEEN PATHWAY: GRASS-GOAT-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	2.04E+03	2.04E+03	2.04E+03	2.04E+03	2.04E+03	2.04E+03
C-14	4.86E+08	9.72E+07	9.72E+07	9.72E+07	9.72E+07	9.72E+07	9.72E+07
CR-51	0.00E+00	0.00E+00	5.99E+03	3.33E+03	1.31E+03	8.55E+03	1.01E+06
MN-54	0.00E+00	1.68E+06	3.34E+05	0.00E+00	5.02E+05	0.00E+00	3.45E+06
FE-55	5.79E+05	4.11E+05	9.58E+04	0.00E+00	0.00E+00	2.61E+05	1.78E+05
FE-59	6.74E+05	1.57E+06	6.08E+05	0.00E+00	0.00E+00	4.96E+05	3.72E+06
CO-58	0.00E+00	9.53E+05	2.20E+06	0.00E+00	0.00E+00	0.00E+00	1.31E+07
CO-60	0.00E+00	3.34E+06	7.52E+06	0.00E+00	0.00E+00	0.00E+00	4.35E+07
NI-63	1.42E+09	1.00E+08	4.81E+07	0.00E+00	0.00E+00	0.00E+00	1.60E+07
ZN-65	2.53E+08	8.78E+08	4.10E+08	0.00E+00	5.62E+08	0.00E+00	3.72E+08
RB-86	0.00E+00	5.67E+08	2.67E+08	0.00E+00	0.00E+00	0.00E+00	8.40E+07
SR-89	5.62E+09	0.00E+00	1.61E+08	0.00E+00	0.00E+00	0.00E+00	6.69E+08
SR-90	1.39E+11	0.00E+00	3.43E+10	0.00E+00	0.00E+00	0.00E+00	3.90E+09
Y-91	1.90E+03	0.00E+00	5.09E+01	0.00E+00	0.00E+00	0.00E+00	7.78E+05
ZR-95	1.98E+02	6.25E+01	4.30E+01	0.00E+00	9.18E+01	0.00E+00	1.44E+05
NB-95	1.69E+04	9.38E+03	5.16E+03	0.00E+00	9.09E+03	0.00E+00	4.01E+07
RU-103	2.17E+02	0.00E+00	9.29E+01	0.00E+00	7.66E+02	0.00E+00	1.82E+04
RU-106	4.50E+03	0.00E+00	5.68E+02	0.00E+00	8.69E+03	0.00E+00	2.16E+05
AG-110M	1.16E+07	1.09E+07	6.65E+06	0.00E+00	2.09E+07	0.00E+00	3.07E+09
TE-125M	3.61E+06	1.30E+06	4.82E+05	1.01E+06	0.00E+00	0.00E+00	1.06E+07
TE-127M	1.01E+07	3.59E+06	1.20E+06	2.41E+06	4.11E+07	0.00E+00	2.52E+07
TE-129M	1.32E+07	4.90E+06	2.09E+06	4.26E+06	5.53E+07	0.00E+00	4.96E+07
I-131	6.45E+08	9.03E+08	4.85E+08	2.64E+11	1.56E+09	0.00E+00	1.79E+08
I-133	8.49E+06	1.44E+07	4.40E+06	2.01E+09	2.53E+07	0.00E+00	1.09E+07
CS-134	2.95E+10	6.93E+10	3.22E+10	0.00E+00	2.20E+10	8.41E+09	8.62E+08
CS-136	1.35E+09	5.30E+09	3.56E+09	0.00E+00	2.89E+09	4.55E+08	4.27E+08
CS-137	4.02E+10	5.34E+10	1.86E+10	0.00E+00	1.82E+10	7.07E+09	7.60E+08
BA-140	5.84E+06	7.16E+03	3.76E+05	0.00E+00	2.43E+03	4.81E+03	9.01E+06
CE-141	1.07E+03	7.12E+02	8.18E+01	0.00E+00	3.35E+02	0.00E+00	2.04E+06
CE-144	7.90E+04	3.27E+04	4.25E+03	0.00E+00	1.95E+04	0.00E+00	1.99E+07
PR-143	3.48E+01	1.39E+01	1.73E+00	0.00E+00	8.08E+00	0.00E+00	1.15E+05
ND-147	2.18E+01	2.37E+01	1.42E+00	0.00E+00	1.39E+01	0.00E+00	8.54E+04

TABLE 5.5.4

Pathway Dose Factors, Ri

AGE GROUP: ADULT PATHWAY: GRASS-GOAT-MILK

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	1.56E+03	1.56E+03	1.56E+03	1.56E+03	1.56E+03	1.56E+03
C-14	2.64E+08	5.27E+07	5.27E+07	5.27E+07	5.27E+07	5.27E+07	5.27E+07
CR-51	0.00E+00	0.00E+00	3.43E+03	2.05E+03	7.56E+02	4.55E+03	8.63E+05
MN-54	0.00E+00	1.01E+06	1.93E+05	0.00E+00	3.01E+05	0.00E+00	3.10E+06
FE-55	3.27E+05	2.26E+05	5.26E+04	0.00E+00	0.00E+00	1.26E+05	1.30E+05
FE-59	3.87E+05	9.09E+05	3.48E+05	0.00E+00	0.00E+00	2.54E+05	3.03E+06
CO-58	0.00E+00	5.66E+05	1.27E+06	0.00E+00	0.00E+00	0.00E+00	1.15E+07
CO-60	0.00E+00	1.97E+06	4.35E+06	0.00E+00	0.00E+00	0.00E+00	3.70E+07
NI-63	8.08E+08	5.60E+07	2.71E+07	0.00E+00	0.00E+00	0.00E+00	1.17E+07
ZN-65	1.65E+08	5.24E+08	2.37E+08	0.00E+00	3.51E+08	0.00E+00	3.30E+08
RB-86	0.00E+00	3.12E+08	1.45E+08	0.00E+00	0.00E+00	0.00E+00	6.14E+07
SR-89	3.05E+09	0.00E+00	8.76E+07	0.00E+00	0.00E+00	0.00E+00	4.89E+08
SR-90	9.84E+10	0.00E+00	2.41E+10	0.00E+00	0.00E+00	0.00E+00	2.84E+09
Y-91	1.03E+03	0.00E+00	2.76E+01	0.00E+00	0.00E+00	0.00E+00	5.68E+05
ZR-95	1.13E+02	3.63E+01	2.46E+01	0.00E+00	5.70E+01	0.00E+00	1.15E+05
NB-95	9.92E+03	5.52E+03	2.97E+03	0.00E+00	5.45E+03	0.00E+00	3.35E+07
RU-103	1.22E+02	0.00E+00	5.27E+01	0.00E+00	4.67E+02	0.00E+00	1.43E+04
RU-106	2.45E+03	0.00E+00	3.10E+02	0.00E+00	4.73E+03	0.00E+00	1.59E+05
AG-110M	6.99E+06	6.47E+06	3.84E+06	0.00E+00	1.27E+07	0.00E+00	2.64E+09
TE-125M	1.96E+06	7.09E+05	2.62E+05	5.89E+05	7.96E+06	0.00E+00	7.81E+06
TE-127M	5.50E+06	1.97E+06	6.70E+05	1.41E+06	2.23E+07	0.00E+00	1.84E+07
TE-129M	7.23E+06	2.70E+06	1.14E+06	2.48E+06	3.02E+07	0.00E+00	3.64E+07
I-131	3.56E+08	5.09E+08	2.92E+08	1.67E+11	8.73E+08	0.00E+00	1.34E+08
I-133	4.65E+06	8.10E+06	2.47E+06	1.19E+09	1.41E+07	0.00E+00	7.28E+06
CS-134	1.70E+10	4.04E+10	3.30E+10	0.00E+00	1.31E+10	4.34E+09	7.07E+08
CS-136	7.92E+08	3.13E+09	2.25E+09	0.00E+00	1.74E+09	2.38E+08	3.55E+08
CS-137	2.22E+10	3.03E+10	1.99E+10	0.00E+00	1.03E+10	3.42E+09	5.87E+08
BA-140	3.24E+06	4.07E+03	2.12E+05	0.00E+00	1.38E+03	2.33E+03	6.67E+06
CE-141	5.82E+02	3.94E+02	4.47E+01	0.00E+00	1.83E+02	0.00E+00	1.51E+06
CE-144	4.30E+04	1.80E+04	2.31E+03	0.00E+00	1.07E+04	0.00E+00	1.45E+07
PR-143	1.90E+01	7.60E+00	9.40e-01	0.00E+00	4.39E+00	0.00E+00	8.30E+04
ND-147	1.13E+01	1.31E+01	7.82e-01	0.00E+00	7.65E+00	0.00E+00	6.28E+04

TABLE 5.6.1

Pathway Dose Factors, Ri

AGE GROUP: INFANT PATHWAY: GRASS-COW-MEAT

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
C-14	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CR-51	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
MN-54	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
FE-55	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
FE-59	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CO-58	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CO-60	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
NI-63	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ZN-65	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
RB-86	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
SR-89	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
SR-90	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Y-91	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ZR-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
NB-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
RU-103	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
RU-106	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
AG-110M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
TE-125M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
TE-127M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
TE-129M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-133	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CS-134	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CS-136	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CS-137	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BA-140	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CE-141	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CE-144	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
PR-143	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ND-147	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

TABLE 5.6.2

Pathway Dose Factors, Ri

AGE GROUP: CHILD PATHWAY: GRASS-COW-MEAT

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	2.34E+02	2.34E+02	2.34E+02	2.34E+02	2.34E+02	2.34E+02
C-14	3.84E+08	7.67E+07	7.67E+07	7.67E+07	7.67E+07	7.67E+07	7.67E+07
CR-51	0.00E+00	0.00E+00	8.78E+03	4.88E+03	1.33E+03	8.90E+03	4.66E+05
MN-54	0.00E+00	8.01E+06	2.13E+06	0.00E+00	2.25E+06	0.00E+00	6.73E+06
FE-55	4.57E+08	2.43E+08	7.52E+07	0.00E+00	0.00E+00	1.37E+08	4.49E+07
FE-59	3.77E+08	6.10E+08	3.04E+08	0.00E+00	0.00E+00	1.77E+08	6.35E+08
CO-58	0.00E+00	1.64E+07	5.03E+07	0.00E+00	0.00E+00	0.00E+00	9.58E+07
CO-60	0.00E+00	6.93E+07	2.04E+08	0.00E+00	0.00E+00	0.00E+00	3.84E+08
NI-63	2.91E+10	1.56E+09	9.91E+08	0.00E+00	0.00E+00	0.00E+00	1.05E+08
ZN-65	3.76E+08	1.00E+09	6.22E+08	0.00E+00	6.31E+08	0.00E+00	1.76E+08
RB-86	0.00E+00	5.76E+08	3.54E+08	0.00E+00	0.00E+00	0.00E+00	3.71E+07
SR-89	4.82E+08	0.00E+00	1.38E+07	0.00E+00	0.00E+00	0.00E+00	1.87E+07
SR-90	1.04E+10	0.00E+00	2.64E+09	0.00E+00	0.00E+00	0.00E+00	1.40E+08
Y-91	1.80E+06	0.00E+00	4.82E+04	0.00E+00	0.00E+00	0.00E+00	2.40E+08
ZR-95	2.66E+06	5.86E+05	5.21E+05	0.00E+00	8.38E+05	0.00E+00	6.11E+08
NB-95	3.10E+06	1.21E+06	8.63E+05	0.00E+00	1.13E+06	0.00E+00	2.23E+09
RU-103	1.55E+08	0.00E+00	5.96E+07	0.00E+00	3.90E+08	0.00E+00	4.01E+09
RU-106	4.44E+09	0.00E+00	5.54E+08	0.00E+00	6.00E+09	0.00E+00	6.91E+10
AG-110M	8.39E+06	5.67E+06	4.53E+06	0.00E+00	1.06E+07	0.00E+00	6.74E+08
TE-125M	5.69E+08	1.54E+08	7.59E+07	1.60E+08	0.00E+00	0.00E+00	5.49E+08
TE-127M	1.78E+09	4.78E+08	2.11E+08	4.25E+08	5.06E+09	0.00E+00	1.44E+09
TE-129M	1.79E+09	5.00E+08	2.78E+08	5.77E+08	5.26E+09	0.00E+00	2.18E+09
I-131	1.66E+07	1.67E+07	9.48E+06	5.52E+09	2.74E+07	0.00E+00	1.48E+06
I-133	5.72e-01	7.08e-01	2.68e-01	1.31E+02	1.18E+00	0.00E+00	2.85e-01
CS-134	9.23E+08	1.51E+09	3.19E+08	0.00E+00	4.69E+08	1.68E+08	8.16E+06
CS-136	1.63E+07	4.48E+07	2.90E+07	0.00E+00	2.39E+07	3.56E+06	1.57E+06
CS-137	1.33E+09	1.28E+09	1.89E+08	0.00E+00	4.16E+08	1.50E+08	8.00E+06
BA-140	4.42E+07	3.87E+04	2.58E+06	0.00E+00	1.26E+04	2.31E+04	2.24E+07
CE-141	2.22E+04	1.11E+04	1.65E+03	0.00E+00	4.86E+03	0.00E+00	1.38E+07
CE-144	2.32E+06	7.26E+05	1.24E+05	0.00E+00	4.02E+05	0.00E+00	1.89E+08
PR-143	3.33E+04	1.00E+04	1.65E+03	0.00E+00	5.42E+03	0.00E+00	3.60E+07
ND-147	1.17E+04	9.48E+03	7.34E+02	0.00E+00	5.20E+03	0.00E+00	1.50E+07

TABLE 5.6.3

Pathway Dose Factors, Ri

AGE GROUP: TEEN PATHWAY: GRASS-COW-MEAT

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	1.93E+02	1.93E+02	1.93E+02	1.93E+02	1.93E+02	1.93E+02
C-14	2.04E+08	4.08E+07	4.08E+07	4.08E+07	4.08E+07	4.08E+07	4.08E+07
CR-51	0.00E+00	0.00E+00	5.63E+03	3.13E+03	1.23E+03	8.03E+03	9.46E+05
MN-54	0.00E+00	7.00E+06	1.39E+06	0.00E+00	2.09E+06	0.00E+00	1.44E+07
FE-55	2.38E+08	1.69E+08	3.94E+07	0.00E+00	0.00E+00	1.07E+08	7.31E+07
FE-59	2.12E+08	4.95E+08	1.91E+08	0.00E+00	0.00E+00	1.56E+08	1.17E+09
CO-58	0.00E+00	1.40E+07	3.24E+07	0.00E+00	0.00E+00	0.00E+00	1.94E+08
CO-60	0.00E+00	5.83E+07	1.31E+08	0.00E+00	0.00E+00	0.00E+00	7.60E+08
NI-63	1.52E+10	1.07E+09	5.15E+08	0.00E+00	0.00E+00	0.00E+00	1.71E+08
ZN-65	2.50E+08	8.68E+08	4.05E+08	0.00E+00	5.56E+08	0.00E+00	3.68E+08
RB-86	0.00E+00	4.06E+08	1.91E+08	0.00E+00	0.00E+00	0.00E+00	6.00E+07
SR-89	2.55E+08	0.00E+00	7.29E+06	0.00E+00	0.00E+00	0.00E+00	3.03E+07
SR-90	8.04E+09	0.00E+00	1.99E+09	0.00E+00	0.00E+00	0.00E+00	2.26E+08
Y-91	9.54E+05	0.00E+00	2.56E+04	0.00E+00	0.00E+00	0.00E+00	3.91E+08
ZR-95	1.50E+06	4.73E+05	3.25E+05	0.00E+00	6.95E+05	0.00E+00	1.09E+09
NB-95	1.79E+06	9.95E+05	5.48E+05	0.00E+00	9.64E+05	0.00E+00	4.25E+09
RU-103	8.56E+07	0.00E+00	3.66E+07	0.00E+00	3.02E+08	0.00E+00	7.15E+09
RU-106	2.36E+09	0.00E+00	2.97E+08	0.00E+00	4.54E+09	0.00E+00	1.13E+11
AG-110M	5.06E+06	4.78E+06	2.91E+06	0.00E+00	9.13E+06	0.00E+00	1.34E+09
TE-125M	3.03E+08	1.09E+08	4.05E+07	8.46E+07	0.00E+00	0.00E+00	8.94E+08
TE-127M	9.41E+08	3.34E+08	1.12E+08	2.24E+08	3.81E+09	0.00E+00	2.35E+09
TE-129M	9.49E+08	3.52E+08	1.50E+08	3.06E+08	3.97E+09	0.00E+00	3.56E+09
I-131	8.93E+06	1.25E+07	6.72E+06	3.65E+09	2.15E+07	0.00E+00	2.47E+06
I-133	3.08e-01	5.22e-01	1.59e-01	7.29E+01	9.16e-01	0.00E+00	3.95e-01
CS-134	5.23E+08	1.23E+09	5.71E+08	0.00E+00	3.91E+08	1.49E+08	1.53E+07
CS-136	9.43E+06	3.71E+07	2.49E+07	0.00E+00	2.02E+07	3.18E+06	2.99E+06
CS-137	7.24E+08	9.63E+08	3.35E+08	0.00E+00	3.28E+08	1.27E+08	1.37E+07
BA-140	2.39E+07	2.93E+04	1.54E+06	0.00E+00	9.94E+03	1.97E+04	3.69E+07
CE-141	1.18E+04	7.87E+03	9.05E+02	0.00E+00	3.71E+03	0.00E+00	2.25E+07
CE-144	1.23E+06	5.08E+05	6.60E+04	0.00E+00	3.03E+05	0.00E+00	3.09E+08
PR-143	1.76E+04	7.03E+03	8.76E+02	0.00E+00	4.08E+03	0.00E+00	5.79E+07
ND-147	6.23E+03	6.78E+03	4.06E+02	0.00E+00	3.98E+03	0.00E+00	2.44E+07

TABLE 5.6.4

Pathway Dose Factors, Ri

AGE GROUP: ADULT PATHWAY: GRASS-COW-MEAT

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	3.24E+02	3.24E+02	3.24E+02	3.24E+02	3.24E+02	3.24E+02
C-14	2.42E+08	4.83E+07	4.83E+07	4.83E+07	4.83E+07	4.83E+07	4.83E+07
CR-51	0.00E+00	0.00E+00	7.04E+03	4.21E+03	1.55E+03	9.35E+03	1.77E+06
MN-54	0.00E+00	9.18E+06	1.75E+06	0.00E+00	2.73E+06	0.00E+00	2.81E+07
FE-55	2.93E+08	2.03E+08	4.73E+07	0.00E+00	0.00E+00	1.13E+08	1.16E+08
FE-59	2.66E+08	6.25E+08	2.39E+08	0.00E+00	0.00E+00	1.75E+08	2.08E+09
CO-58	0.00E+00	1.82E+07	4.09E+07	0.00E+00	0.00E+00	0.00E+00	3.70E+08
CO-60	0.00E+00	7.52E+07	1.66E+08	0.00E+00	0.00E+00	0.00E+00	1.41E+09
NI-63	1.89E+10	1.31E+09	6.33E+08	0.00E+00	0.00E+00	0.00E+00	2.73E+08
ZN-65	3.56E+08	1.13E+09	5.12E+08	0.00E+00	7.57E+08	0.00E+00	7.13E+08
RB-86	0.00E+00	4.87E+08	2.27E+08	0.00E+00	0.00E+00	0.00E+00	9.59E+07
SR-89	3.02E+08	0.00E+00	8.66E+06	0.00E+00	0.00E+00	0.00E+00	4.84E+07
SR-90	1.24E+10	0.00E+00	3.05E+09	0.00E+00	0.00E+00	0.00E+00	3.60E+08
Y-91	1.13E+06	0.00E+00	3.03E+04	0.00E+00	0.00E+00	0.00E+00	6.24E+08
ZR-95	1.87E+06	6.01E+05	4.07E+05	0.00E+00	9.43E+05	0.00E+00	1.90E+09
NB-95	2.30E+06	1.28E+06	6.87E+05	0.00E+00	1.26E+06	0.00E+00	7.76E+09
RU-103	1.05E+08	0.00E+00	4.53E+07	0.00E+00	4.02E+08	0.00E+00	1.23E+10
RU-106	2.80E+09	0.00E+00	3.54E+08	0.00E+00	5.41E+09	0.00E+00	1.81E+11
AG-110M	6.68E+06	6.18E+06	3.67E+06	0.00E+00	1.22E+07	0.00E+00	2.52E+09
TE-125M	3.59E+08	1.30E+08	4.81E+07	1.08E+08	1.46E+09	0.00E+00	1.43E+09
TE-127M	1.12E+09	3.99E+08	1.36E+08	2.85E+08	4.53E+09	0.00E+00	3.74E+09
TE-129M	1.13E+09	4.23E+08	1.79E+08	3.89E+08	4.73E+09	0.00E+00	5.71E+09
I-131	1.08E+07	1.54E+07	8.82E+06	5.04E+09	2.64E+07	0.00E+00	4.06E+06
I-133	3.68e-01	6.41e-01	1.95e-01	9.42E+01	1.12E+00	0.00E+00	5.76e-01
CS-134	6.58E+08	1.57E+09	1.28E+09	0.00E+00	5.07E+08	1.68E+08	2.74E+07
CS-136	1.21E+07	4.78E+07	3.44E+07	0.00E+00	2.66E+07	3.65E+06	5.43E+06
CS-137	8.72E+08	1.19E+09	7.82E+08	0.00E+00	4.05E+08	1.35E+08	2.31E+07
BA-140	2.90E+07	3.64E+04	1.90E+06	0.00E+00	1.24E+04	2.08E+04	5.96E+07
CE-141	1.41E+04	9.51E+03	1.08E+03	0.00E+00	4.42E+03	0.00E+00	3.64E+07
CE-144	1.46E+06	6.10E+05	7.83E+04	0.00E+00	3.62E+05	0.00E+00	4.93E+08
PR-143	2.09E+04	8.40E+03	1.04E+03	0.00E+00	4.85E+03	0.00E+00	9.17E+07
ND-147	7.08E+03	8.18E+03	4.90E+02	0.00E+00	4.78E+03	0.00E+00	3.93E+07

TABLE 5.7.1

Pathway Dose Factors, Ri

AGE GROUP: INFANT PATHWAY: VEGETATION

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
C-14	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CR-51	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
MN-54	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
FE-55	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
FE-59	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CO-58	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CO-60	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
NI-63	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ZN-65	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
RB-86	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
SR-89	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
SR-90	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Y-91	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ZR-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
NB-95	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
RU-103	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
RU-106	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
AG-110M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
TE-125M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
TE-127M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
TE-129M	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-133	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CS-134	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CS-136	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CS-137	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BA-140	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CE-141	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
CE-144	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
PR-143	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ND-147	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00

TABLE 5.7.2

Pathway Dose Factors, Ri

AGE GROUP: CHILD PATHWAY: VEGETATION

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	4.02E+03	4.02E+03	4.02E+03	4.02E+03	4.02E+03	4.02E+03
C-14	8.89E+08	1.78E+08	1.78E+08	1.78E+08	1.78E+08	1.78E+08	1.78E+08
CR-51	0.00E+00	0.00E+00	1.17E+05	6.49E+04	1.77E+04	1.18E+05	6.20E+06
MN-54	0.00E+00	6.65E+08	1.77E+08	0.00E+00	1.86E+08	0.00E+00	5.58E+08
FE-55	8.01E+08	4.25E+08	1.32E+08	0.00E+00	0.00E+00	2.40E+08	7.87E+07
FE-59	3.98E+08	6.44E+08	3.21E+08	0.00E+00	0.00E+00	1.87E+08	6.71E+08
CO-58	0.00E+00	6.44E+07	1.97E+08	0.00E+00	0.00E+00	0.00E+00	3.76E+08
CO-60	0.00E+00	3.78E+08	1.12E+09	0.00E+00	0.00E+00	0.00E+00	2.10E+09
NI-63	3.95E+10	2.11E+09	1.34E+09	0.00E+00	0.00E+00	0.00E+00	1.42E+08
ZN-65	8.12E+08	2.16E+09	1.35E+09	0.00E+00	1.36E+09	0.00E+00	3.80E+08
RB-86	0.00E+00	4.51E+08	2.77E+08	0.00E+00	0.00E+00	0.00E+00	2.90E+07
SR-89	3.60E+10	0.00E+00	1.03E+09	0.00E+00	0.00E+00	0.00E+00	1.39E+09
SR-90	1.24E+12	0.00E+00	3.15E+11	0.00E+00	0.00E+00	0.00E+00	1.67E+10
Y-91	1.87E+07	0.00E+00	4.99E+05	0.00E+00	0.00E+00	0.00E+00	2.49E+09
ZR-95	3.86E+06	8.48E+05	7.55E+05	0.00E+00	1.21E+06	0.00E+00	8.85E+08
NB-95	4.11E+05	1.60E+05	1.14E+05	0.00E+00	1.50E+05	0.00E+00	2.96E+08
RU-103	1.53E+07	0.00E+00	5.90E+06	0.00E+00	3.86E+07	0.00E+00	3.97E+08
RU-106	7.45E+08	0.00E+00	9.30E+07	0.00E+00	1.01E+09	0.00E+00	1.16E+10
AG-110M	3.21E+07	2.17E+07	1.73E+07	0.00E+00	4.04E+07	0.00E+00	2.58E+09
TE-125M	3.51E+08	9.50E+07	4.67E+07	9.84E+07	0.00E+00	0.00E+00	3.38E+08
TE-127M	1.32E+09	3.56E+08	1.57E+08	3.16E+08	3.77E+09	0.00E+00	1.07E+09
TE-129M	8.40E+08	2.35E+08	1.30E+08	2.71E+08	2.47E+09	0.00E+00	1.02E+09
I-131	1.43E+08	1.44E+08	8.18E+07	4.76E+10	2.36E+08	0.00E+00	1.28E+07
I-133	3.53E+06	4.37E+06	1.65E+06	8.12E+08	7.28E+06	0.00E+00	1.76E+06
CS-134	1.60E+10	2.63E+10	5.55E+09	0.00E+00	8.15E+09	2.93E+09	1.42E+08
CS-136	8.28E+07	2.28E+08	1.47E+08	0.00E+00	1.21E+08	1.81E+07	8.00E+06
CS-137	2.39E+10	2.29E+10	3.38E+09	0.00E+00	7.46E+09	2.68E+09	1.43E+08
BA-140	2.79E+08	2.44E+05	1.63E+07	0.00E+00	7.96E+04	1.46E+05	1.41E+08
CE-141	6.57E+05	3.28E+05	4.86E+04	0.00E+00	1.44E+05	0.00E+00	4.09E+08
CE-144	1.27E+08	3.99E+07	6.79E+06	0.00E+00	2.21E+07	0.00E+00	1.04E+10
PR-143	1.45E+05	4.36E+04	7.21E+03	0.00E+00	2.36E+04	0.00E+00	1.57E+08
ND-147	7.15E+04	5.79E+04	4.49E+03	0.00E+00	3.18E+04	0.00E+00	9.18E+07

TABLE 5.7.3

Pathway Dose Factors, Ri

AGE GROUP: TEEN PATHWAY: VEGETATION

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	2.59E+03	2.59E+03	2.59E+03	2.59E+03	2.59E+03	2.59E+03
C-14	3.69E+08	7.38E+07	7.38E+07	7.38E+07	7.38E+07	7.38E+07	7.38E+07
CR-51	0.00E+00	0.00E+00	6.16E+04	3.42E+04	1.35E+04	8.79E+04	1.03E+07
MN-54	0.00E+00	4.54E+08	9.01E+07	0.00E+00	1.36E+08	0.00E+00	9.32E+08
FE-55	3.26E+08	2.31E+08	5.39E+07	0.00E+00	0.00E+00	1.47E+08	1.00E+08
FE-59	1.80E+08	4.19E+08	1.62E+08	0.00E+00	0.00E+00	1.32E+08	9.91E+08
CO-58	0.00E+00	4.36E+07	1.01E+08	0.00E+00	0.00E+00	0.00E+00	6.01E+08
CO-60	0.00E+00	2.49E+08	5.60E+08	0.00E+00	0.00E+00	0.00E+00	3.24E+09
NI-63	1.61E+10	1.13E+09	5.45E+08	0.00E+00	0.00E+00	0.00E+00	1.81E+08
ZN-65	4.24E+08	1.47E+09	6.86E+08	0.00E+00	9.42E+08	0.00E+00	6.23E+08
RB-86	0.00E+00	2.73E+08	1.28E+08	0.00E+00	0.00E+00	0.00E+00	4.04E+07
SR-89	1.52E+10	0.00E+00	4.34E+08	0.00E+00	0.00E+00	0.00E+00	1.80E+09
SR-90	7.51E+11	0.00E+00	1.85E+11	0.00E+00	0.00E+00	0.00E+00	2.11E+10
Y-91	7.84E+06	0.00E+00	2.10E+05	0.00E+00	0.00E+00	0.00E+00	3.22E+09
ZR-95	1.72E+06	5.43E+05	3.73E+05	0.00E+00	7.98E+05	0.00E+00	1.25E+09
NB-95	1.92E+05	1.07E+05	5.87E+04	0.00E+00	1.03E+05	0.00E+00	4.56E+08
RU-103	6.82E+06	0.00E+00	2.92E+06	0.00E+00	2.41E+07	0.00E+00	5.70E+08
RU-106	3.09E+08	0.00E+00	3.90E+07	0.00E+00	5.97E+08	0.00E+00	1.48E+10
AG-110M	1.52E+07	1.43E+07	8.72E+06	0.00E+00	2.74E+07	0.00E+00	4.03E+09
TE-125M	1.48E+08	5.34E+07	1.98E+07	4.14E+07	0.00E+00	0.00E+00	4.37E+08
TE-127M	5.52E+08	1.96E+08	6.56E+07	1.31E+08	2.24E+09	0.00E+00	1.37E+09
TE-129M	3.61E+08	1.34E+08	5.72E+07	1.17E+08	1.51E+09	0.00E+00	1.36E+09
I-131	7.69E+07	1.08E+08	5.78E+07	3.14E+10	1.85E+08	0.00E+00	2.13E+07
I-133	1.94E+06	3.29E+06	1.00E+06	4.59E+08	5.77E+06	0.00E+00	2.49E+06
CS-134	7.10E+09	1.67E+10	7.75E+09	0.00E+00	5.31E+09	2.03E+09	2.08E+08
CS-136	4.39E+07	1.73E+08	1.16E+08	0.00E+00	9.41E+07	1.48E+07	1.39E+07
CS-137	1.01E+10	1.35E+10	4.69E+09	0.00E+00	4.59E+09	1.78E+09	1.92E+08
BA-140	1.39E+08	1.71E+05	8.97E+06	0.00E+00	5.78E+04	1.15E+05	2.15E+08
CE-141	2.83E+05	1.89E+05	2.17E+04	0.00E+00	8.90E+04	0.00E+00	5.41E+08
CE-144	5.28E+07	2.18E+07	2.83E+06	0.00E+00	1.30E+07	0.00E+00	1.33E+10
PR-143	6.99E+04	2.79E+04	3.48E+03	0.00E+00	1.62E+04	0.00E+00	2.30E+08
ND-147	3.62E+04	3.94E+04	2.36E+03	0.00E+00	2.31E+04	0.00E+00	1.42E+08

TABLE 5.7.4

Pathway Dose Factors, Ri

AGE GROUP: ADULT PATHWAY: VEGETATION

NUCLIDE	ORGAN DOSE FACTORS; m^2 - mrem/year per µCi/sec						
	BONE	LIVER	T.BODY	THYROID	KIDNEY	LUNG	GI-LLI
H-3	0.00E+00	2.26E+03	2.26E+03	2.26E+03	2.26E+03	2.26E+03	2.26E+03
C-14	2.28E+08	4.55E+07	4.55E+07	4.55E+07	4.55E+07	4.55E+07	4.55E+07
CR-51	0.00E+00	0.00E+00	4.64E+04	2.77E+04	1.02E+04	6.15E+04	1.17E+07
MN-54	0.00E+00	3.13E+08	5.97E+07	0.00E+00	9.31E+07	0.00E+00	9.58E+08
FE-55	2.10E+08	1.45E+08	3.38E+07	0.00E+00	0.00E+00	8.08E+07	8.31E+07
FE-59	1.26E+08	2.97E+08	1.14E+08	0.00E+00	0.00E+00	8.29E+07	9.89E+08
CO-58	0.00E+00	3.07E+07	6.89E+07	0.00E+00	0.00E+00	0.00E+00	6.23E+08
CO-60	0.00E+00	1.67E+08	3.69E+08	0.00E+00	0.00E+00	0.00E+00	3.14E+09
NI-63	1.04E+10	7.21E+08	3.49E+08	0.00E+00	0.00E+00	0.00E+00	1.50E+08
ZN-65	3.17E+08	1.01E+09	4.56E+08	0.00E+00	6.75E+08	0.00E+00	6.36E+08
RB-86	0.00E+00	2.19E+08	1.02E+08	0.00E+00	0.00E+00	0.00E+00	4.32E+07
SR-89	9.98E+09	0.00E+00	2.86E+08	0.00E+00	0.00E+00	0.00E+00	1.60E+09
SR-90	6.05E+11	0.00E+00	1.48E+11	0.00E+00	0.00E+00	0.00E+00	1.75E+10
Y-91	5.12E+06	0.00E+00	1.37E+05	0.00E+00	0.00E+00	0.00E+00	2.82E+09
ZR-95	1.17E+06	3.77E+05	2.55E+05	0.00E+00	5.91E+05	0.00E+00	1.19E+09
NB-95	1.42E+05	7.92E+04	4.26E+04	0.00E+00	7.83E+04	0.00E+00	4.81E+08
RU-103	4.77E+06	0.00E+00	2.06E+06	0.00E+00	1.82E+07	0.00E+00	5.57E+08
RU-106	1.93E+08	0.00E+00	2.44E+07	0.00E+00	3.72E+08	0.00E+00	1.25E+10
AG-110M	1.05E+07	9.75E+06	5.79E+06	0.00E+00	1.92E+07	0.00E+00	3.98E+09
TE-125M	9.66E+07	3.50E+07	1.29E+07	2.90E+07	3.93E+08	0.00E+00	3.86E+08
TE-127M	3.49E+08	1.25E+08	4.26E+07	8.93E+07	1.42E+09	0.00E+00	1.17E+09
TE-129M	2.51E+08	9.37E+07	3.97E+07	8.63E+07	1.05E+09	0.00E+00	1.26E+09
I-131	8.08E+07	1.16E+08	6.62E+07	3.79E+10	1.98E+08	0.00E+00	3.05E+07
I-133	2.09E+06	3.63E+06	1.11E+06	5.34E+08	6.33E+06	0.00E+00	3.26E+06
CS-134	4.67E+09	1.11E+10	9.08E+09	0.00E+00	3.59E+09	1.19E+09	1.94E+08
CS-136	4.28E+07	1.69E+08	1.22E+08	0.00E+00	9.41E+07	1.29E+07	1.92E+07
CS-137	6.36E+09	8.70E+09	5.70E+09	0.00E+00	2.95E+09	9.81E+08	1.68E+08
BA-140	1.29E+08	1.62E+05	8.47E+06	0.00E+00	5.52E+04	9.29E+04	2.66E+08
CE-141	1.97E+05	1.33E+05	1.51E+04	0.00E+00	6.20E+04	0.00E+00	5.10E+08
CE-144	3.29E+07	1.38E+07	1.77E+06	0.00E+00	8.16E+06	0.00E+00	1.11E+10
PR-143	6.25E+04	2.51E+04	3.10E+03	0.00E+00	1.45E+04	0.00E+00	2.74E+08
ND-147	3.34E+04	3.85E+04	2.31E+03	0.00E+00	2.25E+04	0.00E+00	1.85E+08

6.0 TMI-1 GASEOUS WASTE TREATMENT SYSTEM

6.1 Description of the TMI-1 Gaseous Radwaste Treatment System (see Figure 6.1)

6.1.1 Waste Gas System

a. Reactor Building:

- Reactor Coolant Drain Tank (RCDT) header

b. Auxiliary Building:

- Vent Header from
  1. Miscellaneous Waste Storage Tank (MWST)
  2. Three (3) Reactor Coolant Bleed Tanks (RCBT)
- Waste Gas Delay Tank
- Two (2) Waste Gas Compressors
- Three (3) Waste Gas Decay Tanks (WGDT)

c. Filtration and dilution provided by the Station Ventilation System.

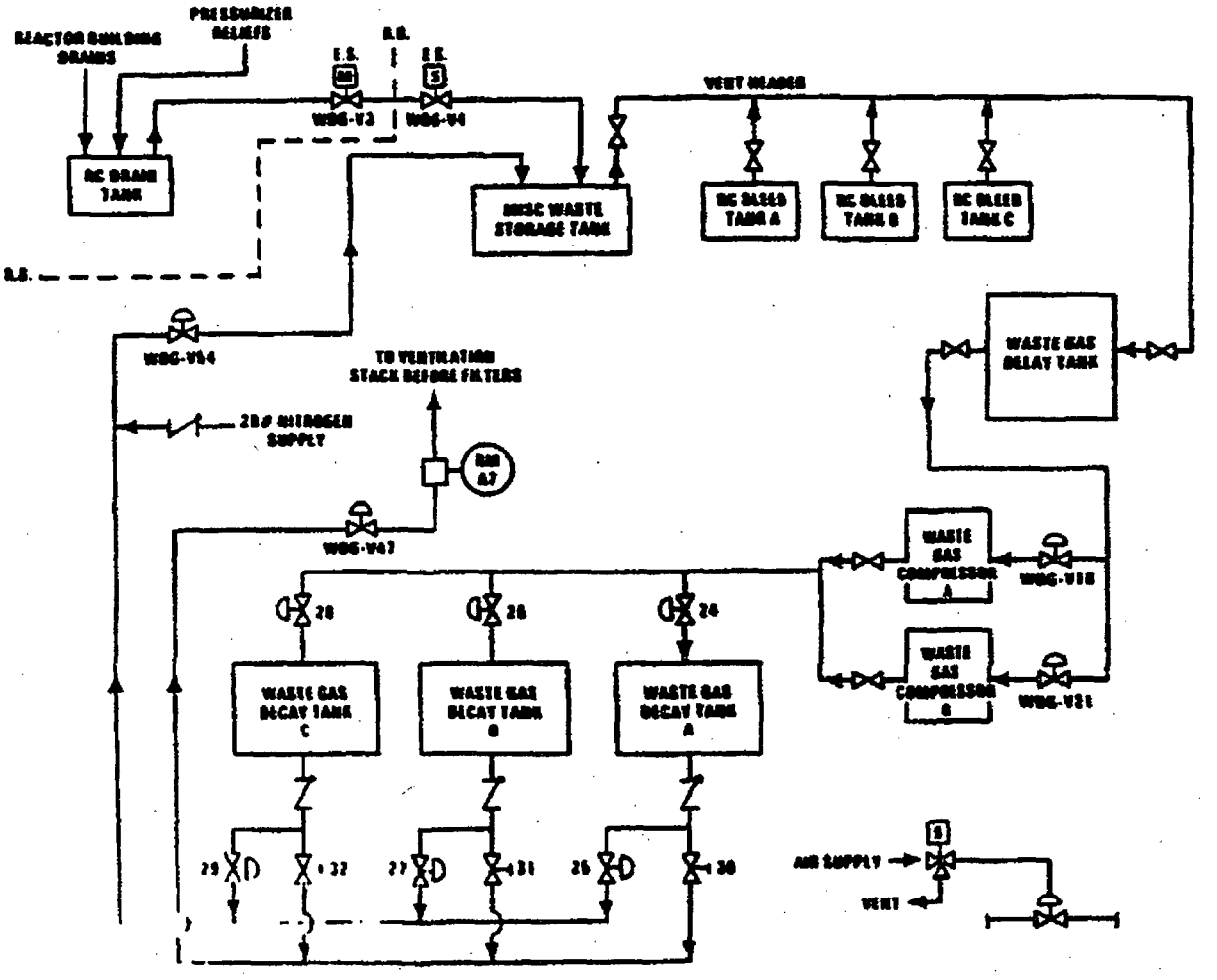
6.2 Operability

Operability of the Gaseous Waste Treatment System is defined as the ability to remove gas from the vent header/tank gas spaces and store it under a higher pressure in the Waste Gas Decay Tanks for subsequent release.

Except for initiating the make up tank sample and waste gas venting and the recycle or disposal of compressed waste gases stored in the waste gas decay tanks, the operation of the waste gas system is entirely automatic. One waste gas compressor comes on automatically, removing gases from the vent header system as required, to maintain the pressure in the system at a maximum of about 16.4 psia.

FIGURE 6.1

Waste Gas System



7.0 EFFLUENT TOTAL DOSE ASSESSMENT

7.1 Total Dose Calculation

The annual (calendar year) dose or dose commitment to any member of the public, due to releases of radioactivity and to radiation from uranium fuel cycle sources shall be limited to less than or equal to 25 mrem to the total body or any organ except the thyroid, which shall be limited to less than or equal to 75 mrem. This control is provided in order to meet the dose limitations of 40 CFR 190.

The total dose from TMI-1 and TMI-2 (uranium fuel cycle facilities within 8 kilometers) is calculated by summing the calculated annual doses to critical organs of a real individual for liquid effluent using Section 2.1 methodology, for gaseous effluent using Section 5.2.1 and 5.2.2 methodology, and the direct radiation from the site from the environmental monitoring program's direct radiation (TLD) monitors.

8.0 TMINS RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM (REMP)

8.1 Monitoring Program Requirements

8.1.1 Controls

In accordance with the TMI-1 Tech. Specs. and TMI-2 PDMS Tech. Specs., the radiological environmental monitoring program shall be conducted as specified in Table 8.1.

8.1.2 Applicability

At all times.

8.1.3 Action

a. With the radiological environmental monitoring program not being conducted as specified in Table 8.1, prepare and submit to the Commission in the Annual Radiological Environmental Operating Report, a description of the reasons for not conducting the program as required and the plans for preventing a recurrence.

b. *With the level of radioactivity as the result of plant effluents in an* environmental sampling medium exceeding the reporting levels of Table 8.2 when averaged over any calendar quarter, prepare and submit to the Commission within 30 days from the end of the affected calendar quarter, a special report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions to be taken to reduce radioactive effluents so that the potential annual dose* to a member of the public is less than the calendar year limits of ODCM Part I Controls 2.2.1.2, 2.2.2.2 and 2.2.2.3 and ODCM Part II Controls 2.2.1.2, 2.2.2.2 and 2.2.2.3. When more than one of the radionuclides in Table 8.2 are detected as the result of plant effluents in the sampling medium, this report shall be submitted if:

$$\frac{\text{concentration (1)}}{\text{reportinglevel (1)}} + \frac{\text{concentration (2)}}{\text{reporting level (2)}} + \geq 1.0$$

When radionuclides other than those in Table 8.2 are detected and are the result of plant effluents, this report shall be submitted if the potential annual dose* to a member of the public is equal to or greater than the calendar year limits of ODCM Part I Controls 2.2.1.2, 2.2.2.2 and 2.2.2.3 and ODCM Part II, Controls 2.2.1.2, 2.2.2.2 and 2.2.2.3. This report is not required if the measured level of radioactivity was not the result of plant effluents; however, in such an event, the condition shall be reported and described in the Annual Radiological Environmental Operating Report.

---

* The methodology and parameters used to estimate the potential annual dose to a member of the public shall be indicated in this report.

c. With milk or fresh leafy vegetation samples unavailable from one or more of the sample locations required by Table 8.1, identify specific locations for obtaining replacement samples and add them within 30 days to the Radiological Environmental Monitoring Program given in the ODCM. The specific locations from which samples were unavailable may then be deleted from the monitoring program. Pursuant to TMI-1 Tech. Spec. 6.14 and TMI-2 PDMS Tech. Spec. 6.12, submit in the next Annual Radioactive Effluent Release Report documentation for a change in the ODCM including a revised figure(s) and table for the ODCM reflecting the new location(s) with supporting information identifying the cause of the unavailability of samples and justifying the selection of the new location(s) for obtaining samples.

8.1.4 Bases

The radiological monitoring program required by this control provides representative measurements of radiation and of radioactive materials in those exposure pathways and for those radionuclides which lead to the highest potential radiation exposures of members of the general public resulting from the station operation. This monitoring program implements Section IV B.2 of Appendix I to 10CFR50 and thereby supplements the radiological effluent monitoring program by verifying that the measurable concentrations of radioactive materials and levels of radiation are not higher than expected on the basis of the effluent measurements and modeling of the environmental exposure pathways. Guidance for this monitoring is provided by the Radiological Assessment Branch Technical Position on Environmental Monitoring (Revision 1, November 1979). Program changes may be initiated based on operational experience.

8.1.5 Surveillance Requirements

The radiological environmental monitoring samples shall be collected pursuant to Table 8.1, from the specific locations given in Tables 8.4 through 8.10 and Maps 8.1 through 8.3, and shall be analyzed pursuant to the requirements of Table 8.1 and the detection capabilities required by Table 8.3.

8.2 Land Use Census

8.2.1 Controls

In accordance with the TMI-1 Tech. Specs. and TMI-2 PDMS Tech. Specs., a Land Use Census shall be conducted and shall identify within a distance of 8 km (5 miles) the location in each of the 16 meteorological sectors of the nearest milk animal, the nearest residence, and the nearest garden* of greater than 50 m^2 (500 ft^2) producing broad leaf vegetation.

8.2.2 Applicability

At all times.

8.2.3 Action

a. With a Land Use Census identifying a location(s) that yields a calculated dose or dose commitment greater than the values currently being calculated in ODCM Part I Surveillance 3.2.2.3.1, pursuant to ODCM, Part IV, Section 2.0, identify the new location(s) in the next Annual Radioactive Effluent Release Report.

b. With a Land Use Census identifying a location(s) that yields a calculated dose or dose commitment (via the same exposure pathway) 20% greater than at a location from which samples are currently being obtained in accordance with Table 8.1, add the new location(s) within 30 days to the Radiological Environmental Monitoring Program given in the ODCM. The sampling location(s), excluding the control station location, having the lowest calculated dose or dose commitment(s), via the same exposure pathway, may be deleted from this monitoring program after October 31 of the year in which this Land Use Census was conducted. Pursuant to TMI-1 Tech. Spec. 6.14 and TMI-2 PDMS Tech. Spec. 6.12, submit in the next Annual Radioactive Effluent Release Report documentation for a change in the ODCM including a revised figure(s) and table(s) for the ODCM reflecting the new location(s) with information supporting the change in sampling locations.

* Broad leaf vegetation sampling of at least three different kinds of vegetation may be performed at the site boundary in each of two different sectors with the highest predicted D/Qs in lieu of the garden census. Requirements for broad leaf sampling in Table 8.1 shall be followed, including analysis of control samples.

8.2.4 Bases

This Control is provided to ensure that changes in the use of unrestricted areas are identified and modifications to the monitoring program are made if required by the results of this census. The best information from the door-to-door survey, aerial surveys, or consulting with local agricultural authorities shall be used. This census satisfies the requirements of Section IV.B.3 of Appendix I to 10 CFR 50. Restricting the census to gardens of greater than 500 square feet (50 m^2) provides assurance that significant exposure pathways via leafy vegetables will be identified and monitored since a garden of this size is the minimum required to produce the quantity (26 kg/yr) of leafy vegetables assumed in Regulatory Guide 1.109 for consumption by a child. To determine this minimum garden size, the following assumptions were used: 1) that 20% of the garden was used for growing broad leaf vegetation (i.e., similar to lettuce and cabbage), and 2) a vegetation yield of 2 kg/square meter.

8.2.5 Surveillance Requirements

The Land Use Census shall be conducted during the growing season at least once per 12 months, using that information that will provide the best results, such as by a door-to-door survey, aerial survey, or by consulting local agricultural authorities. The results of the Land Use Census shall be included in the Annual Radiological Environmental Operating Report pursuant to ODCM, Part IV, Section 1.0.

8.3 Interlaboratory Comparison Program

8.3.1 Controls

In accordance with the TMI-1 Tech. Specs. and TMI-2 PDMS Tech. Specs., analyses shall be performed on radioactive materials supplied as part of an Interlaboratory Comparison Program which has been approved by the Commission (NRC). Only those samples and analyses which are required by Table 8.1 shall be performed.

8.3.2 Applicability

At all times.

8.3.3 Action

With analysis not being performed as required above, report the corrective action taken to prevent a recurrence to the Commission in the Annual Radiological Environmental Operating Report.

8.3.4 Bases

The requirement for participation in an approved Interlaboratory Comparison Program is *provided to ensure that independent checks on precision and accuracy* of the measurements of radioactive material in environmental sample matrices are performed as part of a quality assurance program for environmental monitoring in order to demonstrate that the results are reasonably valid for the purpose of Section IV, B.2 of Appendix I to 10 CFR 50.

8.3.5 Surveillance Requirements

A summary of the Interlaboratory Comparison Program results shall be included in the Annual Radiological Environmental Operating Report.

TABLE 8.1

Sample Collection and Analysis Requirements

Exposure Pathway and/or Sample	Number of Samples and Sample Locations[a]	Sampling and Collection Frequency[b]	Type and Frequency of Analysis[b]
1. Airborne Radioiodine and Particulates	Samples from 5 locations from Table 8.4.	Continuous sampler operation with sample collection weekly, or more frequently if required by dust loading.	Radioiodine Canister: Analyze weekly for I-131. Particulate Filter: Analyze for gross beta radioactivity following filter change[d]. Perform gamma isotopic analysis[e] on composite (by location) sample quarterly.
2. Direct Radiation[c]	Samples from 40 locations from Table 8.5 (using either 2 dosimeters or at least 1 instrument for continuously measuring and recording dose rate at each location).	Sample Quarterly	Analyze for gamma dose quarterly.

TABLE 8.1

Sample Collection and Analysis Requirements

Exposure Pathway and/or Sample	Number of Samples and Sample Locations[a]	Sampling and Collection Frequency[b]	Type and Frequency of Analysis[b]
3. Waterborne			
a. Surface[f]	Samples from 2 locations from Table 8.6. • 1 sample from downstream (indicator) location • 1 sample from upstream (control) location (or location not influenced by the station discharge)	Composite[g] sample over 1 monthly period.	Perform gamma isotopic analysis[e] monthly. Composite for tritium analysis quarterly.
b. Drinking	Samples from 2 locations from Table 8.6. • 1 sample at the location of the nearest water supply that could be affected by the station discharge. • 1 sample from a control location.	Composite[g] sample over 1 monthly period.	Perform gross beta and gamma isotopic analysis[e] monthly. Perform Sr-90 analysis if gross beta of monthly composite >10 times control. Composite for tritium analysis quarterly.
c. Sediment from Shoreline	Samples from 2 locations (1 Control and 1 Indicator) from Table 8.7.	Sample twice per year (Spring and Fall)	Perform gamma isotopic analysis[e] on each sample.

TABLE 8.1

Sample Collection and Analysis Requirements

Exposure Pathway and/or Sample	Number of Samples and Sample Locations[a]	Sampling and Collection Frequency[b]	Type and Frequency of Analysis[b]
4. Ingestion			
a. Milk	Samples from 4 locations from Table 8.8.	Sample semimonthly when animals are on pasture; monthly at other times.	Perform gamma isotopic analysis[e] and I-131 analysis on each sample. Composite for Sr-90 analysis quarterly.
b. Fish	Samples from 2 locations from Table 8.9. • 1 sample of recreationally important bottom feeders and 1 sample of recreationally important predators in the vicinity of the station discharge. • 1 sample of recreationally important bottom feeders and 1 sample of recreationally important predators from an area not influenced by the station discharge.	Sample twice per year (Spring and Fall).	Perform gamma isotopic[e] and Sr-90 analysis on edible portions.
c. Food Products	Samples from 2 locations from Table 8.10 (when available) • 1 sample of green leafy vegetables or leafy vegetation at a location in the immediate vicinity of the station. (indicator) • 1 sample of same species or group from a location not influenced by the station discharge.	Sample at time of harvest.	Perform gamma isotopic[e], I-131, and Sr-90 analysis on edible portions.

TABLE 8.1

Sample Collection and Analysis Requirements

Table Notation

a. Sampling locations are provided in Tables 8.4 through 8.10. They are depicted in Maps 8.1 through 8.3. Deviations are permitted from the required sampling schedule if specimens are unobtainable due to hazardous conditions, seasonal unavailability, malfunction of automatic sampling equipment and other legitimate reasons. All deviations from the sampling schedule shall be explained in the Annual Radiological Environmental Operating Report.

b. Frequency notation: weekly (7 days), semimonthly (15 days), monthly (31 days), and quarterly (92 days). All surveillance requirements shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval. A total maximum combined interval time for any 4 consecutive tests shall not exceed 3.25 times the specified collection or analysis interval.

c. One or more instruments, such as a pressurized ion chamber for measuring and recording dose rate continuously, may be used in place of, or in addition to, integrating dosimeters. For the purpose of this table, a thermoluminescent dosimeter (TLD) is considered to be one phosphor; two or more phosphors in a packet are considered as two or more dosimeters. Film badges shall not be used as dosimeters for measuring direct radiation.

d. Airborne particulate sample filters shall be analyzed for gross beta radioactivity 24 hours or more after sampling to allow for radon and thoron daughter decay. If gross beta activity in an air particulate sample(s) is greater than ten times the calendar year mean of control samples, Sr-90 and gamma isotopic analysis shall be performed on the individual sample(s).

e. Gamma isotopic analysis means the identification and quantification of gamma-emitting radionuclides that may be attributable to the effluents from the facility.

f. The "upstream sample" shall be taken at a distance beyond significant influence of the discharge. The "downstream sample" shall be taken in an area beyond but near the mixing zone.

g. Composite sample aliquots shall be collected at time intervals that are short (e.g., hourly) relative to the compositing period (e.g., monthly) in order to assure obtaining a representative sample.

TABLE 8.2

Reporting Levels for Radioactivity Concentrations in Environmental Samples

Analysis	Water (pCi/L)	Airborne Particulate or gas (pCi/m^3)	Fish (pCi/kg,wet)	Milk (pCi/L)	Food Products (pCi/kg, wet)
H-3	20,000[(a)]				
Mn-54	1000		30,000		
Fe-59	400		10,000		
Co-58	1000		30,000		
Co-60	300		10,000		
Zn-65	300		20,000		
Sr-90	8	0.1	100	8	100
Zr-Nb-95	400				
I-131	2	0.9		3	100
Cs-134	30	10	1000	60	1000
Cs-137	50	20	2000	70	2000
Ba-La-140	200			300	

[(a)] For drinking water samples. This is 40 CFR Part 141 value.

TABLE 8.3

Detection Capabilities for Environmental Sample Analysis[a]

Lower Limit of Detection (LLD)[b,c]

Analysis	Water (pCi/L)	Airborne Particulate or Gas (pCi/m^3)	Fish (pCi/kg, wet)	Milk (pCi/L)	Food Products (pCi/kg,wet)	Sediment (pCi/kg, dry)
Gross Beta	4	0.01				
H-3	2000					
Mn-54	15		130			
Fe-59	30		260			
Co-58, 60	15		130			
Zn-65	30		260			
Zr-95	30					
Sr-90	2	0.01	10	2	10	
Nb-95	15					
I-131	1[d]	0.07		1	60	
Cs-134	15	0.05	130	15	60	150
Cs-137	18	0.06	150	18	80	180
Ba-140	60			60		
La-140	15			15		

TABLE 8.3

Detection Capabilities for Environmental Sample Analysis[a]
Table Notation

a. This list does not mean that only these nuclides are to be considered. Other peaks that are identifiable, which may be related to plant operations, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radiological Environmental Operating Report.

b. Required detection capabilities for thermoluminescent dosimeters used for environmental measurements are given in Regulatory Guide 4.13 (Rev. 1).

c. The LLD is defined, for purposes of these controls, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$LLD = \frac{4.66\, S_b}{E \bullet V \bullet 2.22 \bullet Y \bullet \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above, as picocuries per unit mass or volume.

s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate, as counts per minute,

E is the counting efficiency, as counts per disintegration,

V is the sample size in units of mass or volume,

2.22 is the number of disintegrations per minute per picocurie,

Y is the fractional radiochemical yield (when applicable),

λ is the radioactive decay constant for the particular radionuclide and

Δt for environmental samples is the elapsed time between sample collection, or end of the sample collection period, and time of counting.

Typical values of E, V, Y and Δt should be used in the calculation.

It should be recognized that the LLD is defined as an "a priori" (before the fact) limit representing the capability of a measurement system and not as an "a posteriori" (after the fact) limit for a particular measurement. Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. Occasionally background fluctuations, unavoidable small samples sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors shall be identified and described in the Annual Radiological Environmental Operating Report.

d. LLD for drinking water.

TABLE 8.4

TMINS REMP Station Locations-Air Particulate and Air Iodine

Station Code	Distance (miles)	Azimuth (°)	Map No.
E1-2	0.4	97	2
F1-3	0.6	112	70
G2-1	1.4	126	74
M2-1	1.3	256	3
A3-1	2.7	357	4
H3-1	2.2	160	5
Q15-1	13.4	309	8

TABLE 8.5

TMINS REMP Station Locations-Direct Radiation (TLD)

Station Code	Distance (miles)	Azimuth (°)	Map No.
A1-4	0.3	6	9
B1-1	0.6	25	10
B1-2	0.4	23	11
C1-2	0.3	50	13
D1-1	0.2	76	14
E1-2	0.4	97	2
E1-4	0.2	97	16
F1-2	0.2	112	17
G1-3	0.2	130	18
H1-1	0.5	167	19
J1-1	0.8	176	21
J1-3	0.3	189	22
K1-4	0.2	209	24
L1-1	0.1	236	26
M1-1	0.1	250	27
N1-3	0.1	274	28
P1-1	0.4	303	29
P1-2	0.1	292	30

TABLE 8.5

TMINS REMP Station Locations-Direct Radiation (TLD)

Station Code	Distance (miles)	Azimuth (°)	Map No.
Q1-2	0.2	321	31
R1-1	0.2	335	32
C2-1	1.5	44	33
K2-1	1.2	200	34
M2-1	1.3	256	3
A3-1	2.7	357	4
H3-1	2.2	160	5
R3-1	2.6	341	35
B5-1	4.9	19	36
C5-1	4.7	43	37
E5-1	4.7	82	38
F5-1	4.7	109	39
G5-1	4.8	131	40
H5-1	4.1	158	41
J5-1	4.9	181	42
K5-1	4.9	202	43
L5-1	4.1	228	44
M5-1	4.3	249	45
N5-1	5.0	268	46
P5-1	5.0	284	47
Q5-1	5.0	317	48
R5-1	4.9	339	49
D6-1	5.2	66	50
E7-1	6.7	88	51
Q9-1	8.5	310	52
B10-1	9.2	21	53
G10-1	9.7	128	6
G15-1	14.4	126	54
J15-1	12.6	183	7
Q15-1	13.4	309	8

TABLE 8.6

TMINS REMP Station Locations-Surface Water

Station Code	Distance (miles)	Azimuth (°)	Map No.
J1-2 (R)	0.5	188	57
A3-2 (R)	2.7	356	59
Q9-1 (F)	8.5	310	52
Q9-1 (R)	8.5	310	52
G15-2 (F)	13.3	129	62
G15-3 (F)	15.7	124	63
F15-1 (R)	12.6	122	65

(R) = Raw Water
(F) = Finished Water

TABLE 8.7

TMINS REMP Station Locations-Aquatic Sediment

Station Code	Distance (miles)	Azimuth (°)	Map No.
A1-3	0.5	359	67
G1-1	0.3	137	68
K1-3	0.2	212	69
J2-1	1.4	179	58
J1-2	0.5	188	57

TABLE 8.8

TMINS REMP Station Locations-Milk

Station Code	Distance (miles)	Azimuth (°)	Map No.
D2-1	1.1	62	72
E2-2	1.1	96	73
F4-1	3.2	104	61
G2-1	1.4	126	74
P7-1	6.7	293	77
K15-3	14.4	205	78

TABLE 8.9

TMINS REMP Station Locations-Fish

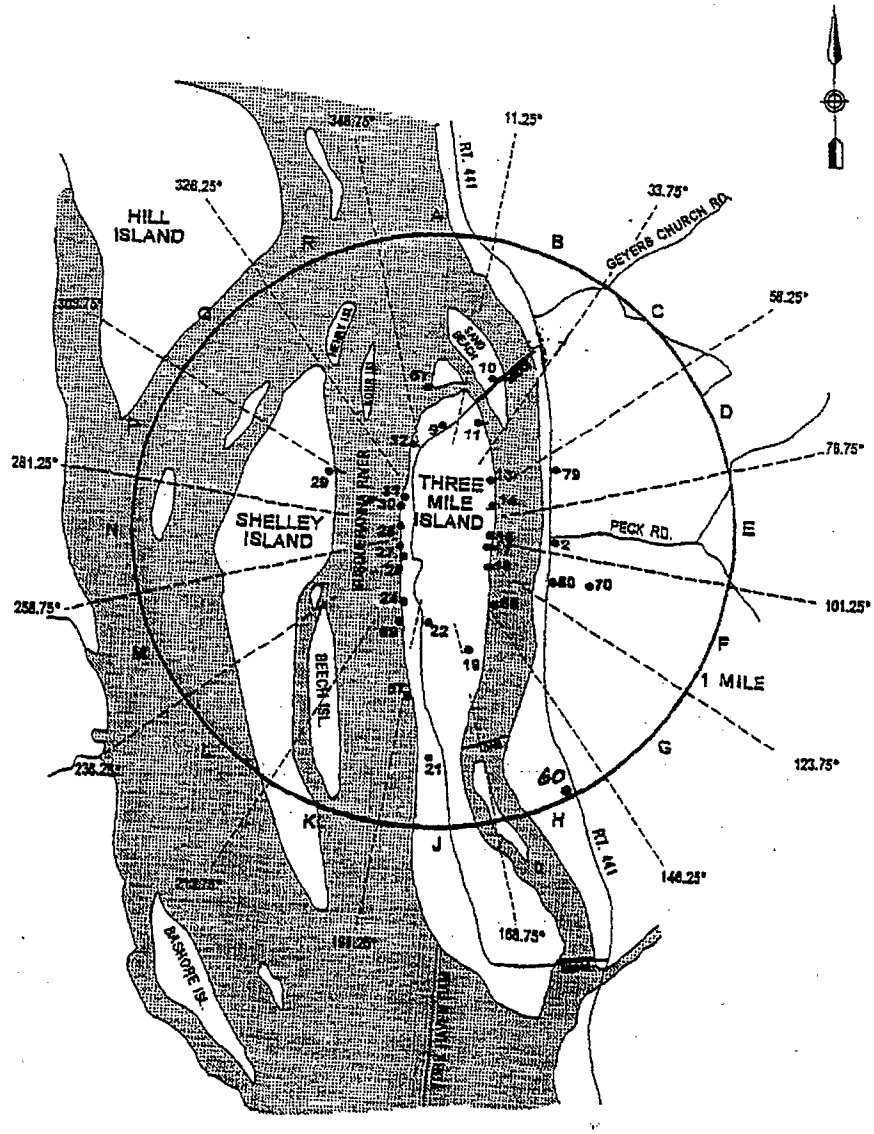
Station Code	Station Location
IND	Downstream of Station Discharge
BKG	Upstream of Station Discharge

TABLE 8.10

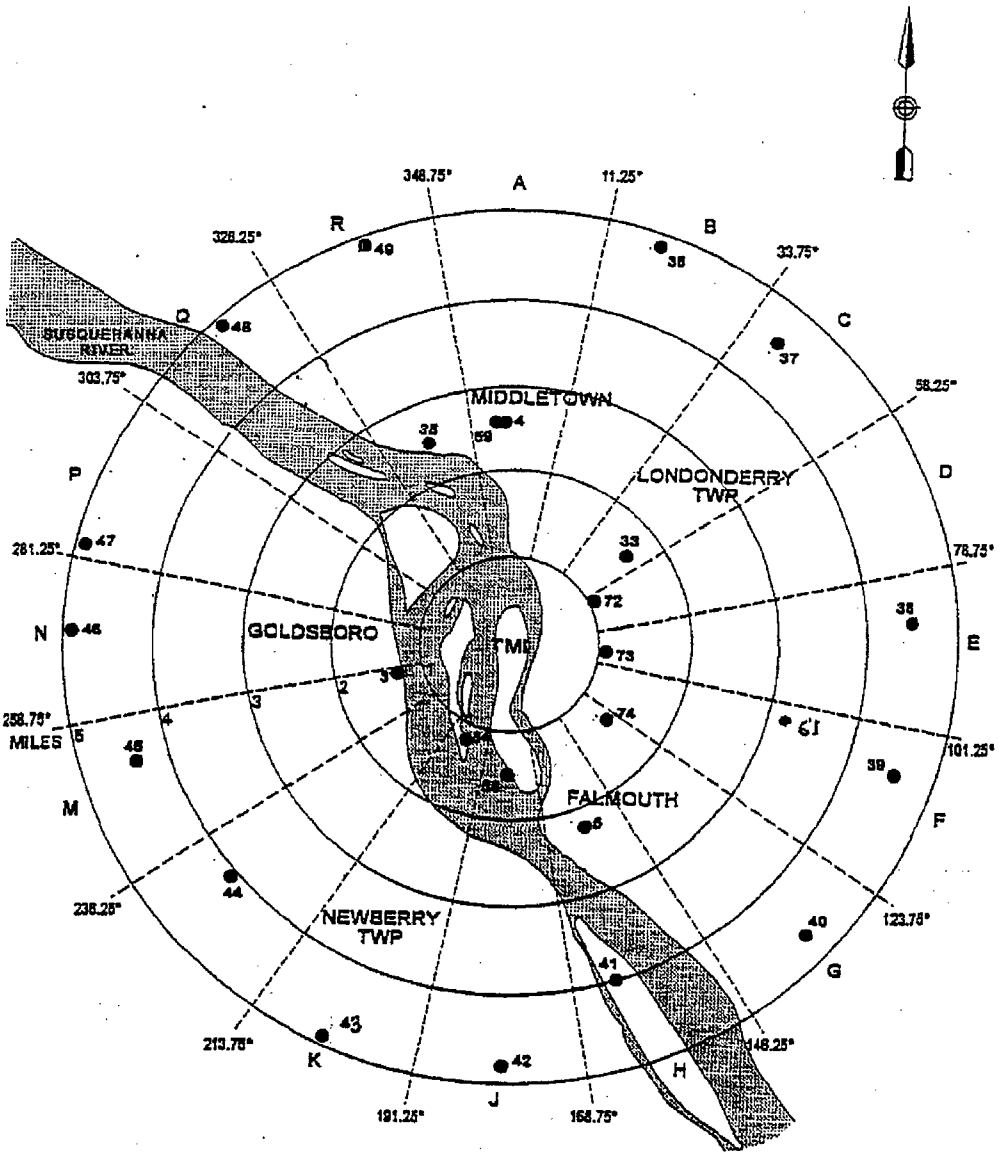
TMINS REMP Station Locations-Food Products

Station Code	*Distance (miles)*	*Azimuth (°)*	*Map No.*
D1-3	0.5	65	79
E1-2	0.4	97	2
F1-1	0.5	117	80
H1-2	1.0	151	60
B10-2	10.0	31	55

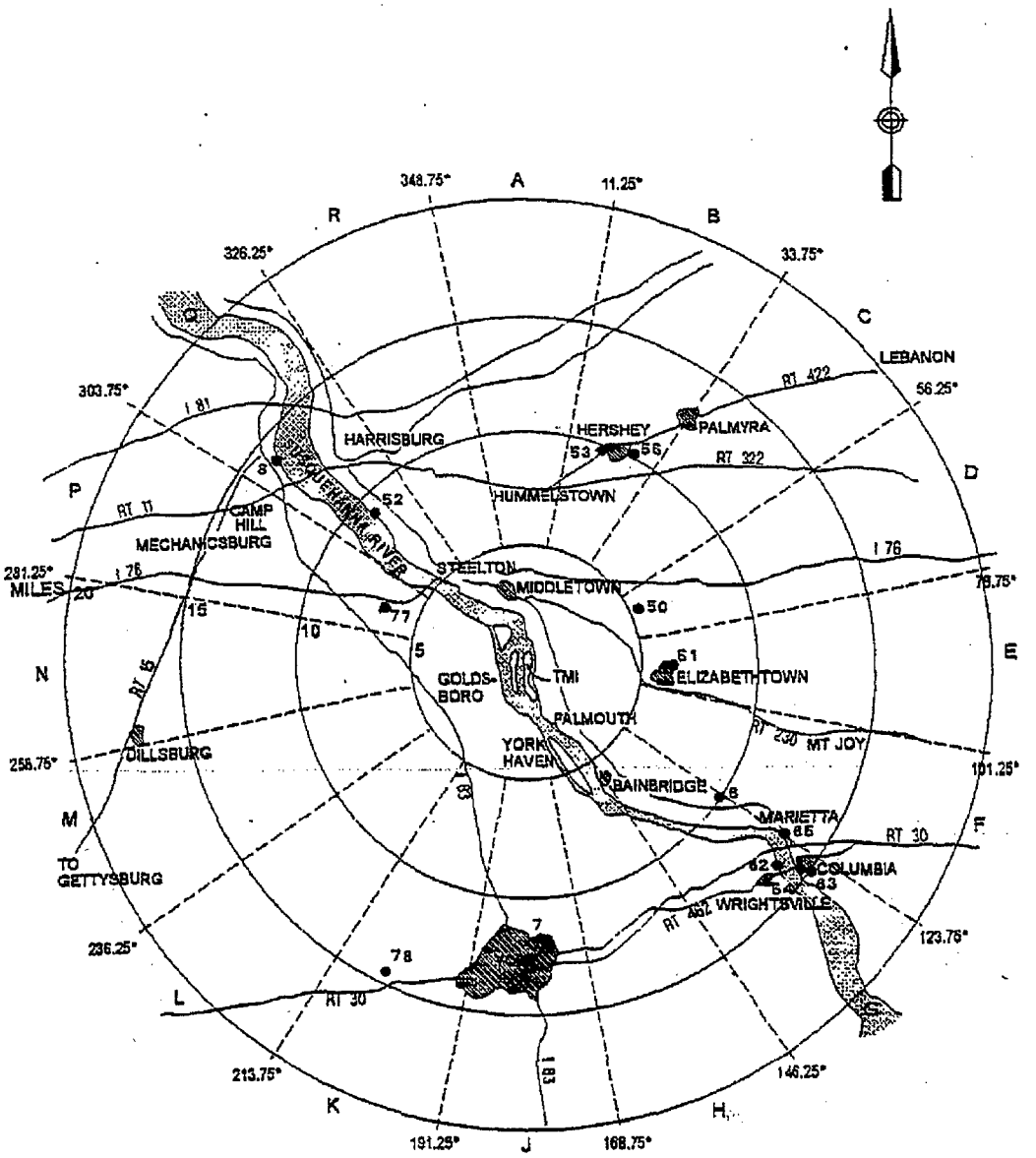
MAP 8.1
THREE MILE ISLAND NUCLEAR STATION
LOCATIONS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
STATIONS WITHIN 1 MILE OF THE SITE



MAP 8.2
THREE MILE ISLAND NUCLEAR STATION
LOCATIONS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
STATIONS WITHIN 5 MILES OF THE SITE



MAP 8.3
THREE MILE ISLAND NUCLEAR STATION
LOCATIONS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM
STATIONS GREATER THAN 5 MILES FROM THE SITE



9.0 PART III REFERENCES

1. EPRI NP-3840, RP 1560-3 Final Report, "Environmental Radiation Doses From Difficult-To-Measure Nuclides," January 1985

2. "Evaluation of the Three Mile Island Nuclear Station Unit 1 to Demonstrate Conformance to the Design Objectives of 10 CFR 50, Appendix I," Nuclear Safety Associates, May 1976

3. TMI-1 Final Safety Analysis Report (FSAR)

4. TMI-2 Final Safety Analysis Report (FSAR)

5. Meteorological Information and Dose Assessment System (MIDAS)

6. NUREG-0017, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from PWR," Revision 1, 1985

7. NUREG-0133, "Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants," October 1978

8. NUREG-0172, "Age-Specific Radiation Dose Commitment Factors For A One-Year Chronic Intake," November 1977

9. Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water Cooled Nuclear Power Plants," Revision 1, June 1974

10. Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR 50, Appendix I," Revision 1, October 1977

11. Simplified Environmental Effluent Dosimetry System (SEEDS)

12. TMI Recirculation Factor Memos, April 12, 1988 and March 17, 1988

13. TMI-1 Operations Procedure, 1101-2.1, "Radiation Monitor Set Points"

14. Title 10, Code of Federal Regulations, "Energy"

15. TMI-1 Technical Specifications, attached to Facility Operating License No. DPR-50

16. Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors," Revision 1, July 1977

17. TMI-2 PDMS Technical Specifications, attached to Facility License No. DPR-73

18. Radiological Assessment Branch Technical Position on Environmental Monitoring, Revision 1, November 1979

19. Title 40, Code of Federal Regulations, "Protection of Environment"

20. Regulatory Guide 4.13, "Performance, Testing, and Procedural Specifications for Thermoluminescence Dosimetry: Environmental Applications," Revision 1, July 1977

21. Post-Defueling Monitored Storage Safety Analysis Report (PDMS SAR)

PART IV

REPORTING REQUIREMENTS

PART IV

Reporting Requirements

1.0 TMI ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT**

1.1 Routine Radiological Environmental Operating Reports covering the operation of the unit during the previous calendar year shall be submitted to the Commission prior to May 1 of each year.

1.2 The Annual Radiological Environmental Operating Reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental monitoring activities for the report period, including a comparison with pre-operational studies, with operational controls as appropriate, and with previous environmental monitoring reports, and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of Land Use Censuses required by Part III, Section 8.2.

1.3 The Annual Radiological Environmental Operating Reports shall include the summarized tabulated results of analysis of all radiological environmental samples and environmental radiation measurements required by Part III Table 8.1 taken during the period pursuant to the locations specified in the tables and figures in this ODCM, as well as summarized and tabulated results of these analyses and measurements in a format similar to the table in the Radiological Assessment Branch Technical Position, Revision 1, November 1979. In the event that some individual results are not available for inclusion with the report, the report shall be submitted explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

1.4 The reports shall also include the following: a summary description of the radiological environments monitoring program; a map(s) of all sampling locations keyed to a table giving distances and directions from a point that is midway between the Reactor Buildings of TMI-1 and TMI-2; the results of licensee participation in the Interlaboratory Comparison Program, required by Part III, Section 8.3; discussion of all deviations from the sampling schedule of Part III, Table 8.1; discussion of all the required analyses in which the LLD required by Part III, Table 8.3 was not achievable.

**A single submittal may be made for the station.

2.0 TMI ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

NOTE

A single submittal may be made for the station. The submittal should combine those sections that are common to both units at the station however, for units with separate radwaste systems, the submittal shall specify the release of radioactive material from each unit.

2.1 Routine Radioactive Effluent Release Reports covering the operations of the unit during the previous 12 months of operation shall be submitted prior to May 1 for TMI-1 and TMI-2.

2.2 The following information shall be included in both Radioactive Effluent Release Reports to be submitted each year:

The Radioactive Effluent Release Reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Reg. Guide 1.21, Rev. 1, with data summarized on a quarterly basis following the format of Appendix B thereof.

2.3 The Radioactive Effluent Release Reports shall include the following information for each type of solid waste shipped offsite during the report period:

a. container volume,

b. total curie quantity (specify whether determined by measurement or estimate),

c. principal radionuclides (specify whether determined by measurement or estimate),

d. type of waste (e.g., spent resin, compacted dry waste, evaporator bottoms),

e. type of shipment (e.g., LSA, Type A, Type B) and

f. solidification agent (e.g., cement).

2.4 The Radioactive Effluent Release Reports shall include a summary of unplanned releases from the site to unrestricted areas of radioactive materials in gaseous and liquid effluents made during the reporting period.

2.5 The Radioactive Effluent Release Reports shall include any changes made during the reporting period to the PROCESS CONTROL PROGRAM (PCP) documents and to the OFFSITE DOSE CALCULATION MANUAL (ODCM), as well as a listing of new locations for dose calculations and/or environmental monitoring identified by the land use census pursuant to Part III Section 8.2.

2.6 The Radioactive Effluent Release Reports shall include the instrumentation not returned to OPERABLE status within 30 days per ODCM Part I Controls 2.1.1b and 2.1.2b, and ODCM Part II Control 2.1.2b.

2.7 The Radioactive Effluent Release Report to be submitted shall include an annual summary of hourly meteorological data collected over the previous year. This annual summary may be either in the form of an hour-by-hour listing of wind speed, wind direction, atmosphere stability, and precipitation (if measured) on magnetic tape, or in the form of joint frequency distribution of wind speed, wind direction, and atmospheric stability.

2.8 The Radioactive Effluent Release Report shall include an assessment of the radiation doses to MEMBERS OF THE PUBLIC due to the radioactive liquid and gaseous effluents released from the unit or station during the previous calendar year. The meteorological conditions concurrent with the time of release of radioactive materials in gaseous effluents (as determined by sampling frequency and measurement) shall be used for determining the gaseous pathway doses. The assessment of radiation doses shall be performed in accordance with this ODCM.

2.9 The Radioactive Effluent Release Report shall include an assessment of the radiation doses from radioactive liquid and gaseous effluents to MEMBERS OF THE PUBLIC due to their activities inside the SITE BOUNDARY during the report period, to verify compliance with the limits of 10CFR20.1301(a)(1). All assumptions used in making these assessments (i.e., specific activity, exposure time and location) shall be included in these reports.

2.10 The Radioactive Effluent Release Report shall also include an assessment of radiation doses to the likely most exposed real individual from reactor releases and other nearby uranium fuel cycle sources including doses from primary effluent pathways and direct radiation for the previous 12 consecutive months to show conformance with 40 CFR 190 "Environmental Radiation Protection Standards for Nuclear Power Operation." Acceptable methods for calculating the dose contributions from liquid and gaseous effluents are given in Regulatory Guide 1.109, Rev. 1.

3.0 PART IV REFERENCES

3.1 Radiological Assessment Branch Technical Position, Revision 1, November 1979

3.2 Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, June 1974

3.3 TMI-1 Technical Specifications, attached to Facility Operating License No. DPR-50

3.4 Title 40, Code of Federal Regulations, "Protection of Environment"

3.5 Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977

3.6 Title 10, Code of Federal Regulations, "Energy"

3.7 Regulatory Guide 1.111, "Methods of Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors," Revision 1, July 1977

3.8 Regulatory Guide 1.112, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors," Revision O-R, April 1976

3.9 Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I," Revision 1, April 1977

APPENDIX A

P_i - Pathway Dose Rate Parameter

P_i (inhalation) = k' (BR) DFA_i (Eq A-1)

Where:

P_i = the pathway dose rate parameter for radionuclide, i, (other than noble gases) for the inhalation pathway, in mrem/yr per microcurie/m^3. The dose factors are based on the critical individual organ for the child age group.

k' = conversion factor, 1E6 pCi/microcurie

BR = 3700 m^3/yr, breathing rate for child (Reg. Guide 1.109, Rev. 1, Table E-5)

DFA_i = the maximum organ inhalation dose factor for the infant age group for the ith radionuclide (mRem/pCi). Values are taken from Table E-10, Reg. Guide 1.109 (Rev. 1), or NUREG-0172.

Resolution of the units yields: (ODCM Part III Table 4.6)

P_i (inhalation) = 3.7E9 DFA_i (mrem/yr per $\mu Ci/m^3$) (Eq A-2)

NOTE

The latest NRC Guidance has deleted the requirement to determine P_i (ground plane) and P_i (food). In addition, the critical age group has been changed from infant to child.

APPENDIX B

RI - Inhalation Pathway Dose Factor

$R_i = k' (BR) (DFA_{i,a,o})$ *(mrem/yr per microcurie/m^3)* (Eq B-1)

Where:

k' = conversion factor, 1E6 pCi/microcurie

BR = breathing rate, 1400, 3700, 8000, 8000 m^3/yr for infant, child, teenager, and adult age groups, respectively. (Reg. Guide 1.109, Rev. 1, Table E-5)

$DFA_{i,a,o}$ = the inhalation dose factor for organ, o, of the receptor of a given age group, a, and for the ith radionuclide, in mrem/pCi. The total body is considered as an organ in the selection of $DFA_{i,a,o}$. Values are taken from Tables E-7 through E-10, Reg. Guide 1.109 (Rev. 1), or NUREG 0172.

Resolutions of the units yields:

Ri = (1.4E9) ($DFA_{i,a,o}$) infant (ODCM Part III Table 5.2.1)

Ri = (3.7E9) ($DFA_{i,a,o}$) child (ODCM Part III Table 5.2.2)

Ri = (8.0E9) ($DFA_{i,a,o}$) teen and adult (ODCM Part III Tables 5.2.3 and 5.2.4)

APPENDIX C

Ri - Ground Plane Pathway Dose Factor

$$R_i = k' k'' (SF) (DFG_i) [(1-e^{-\lambda_i t})/\lambda_i] \quad \text{(Eq C-1)}$$

Where:

k' = conversion factor, 1E6 pCi/microcurie

k" = conversion factor, 8760 hr/yr

λ_i = decay constant for the i^{th} radionuclide, sec^{-1}

t = the exposure time (this calculation assumes that decay is the only operating removal mechanism) 4.73×10^8 sec. (15 yrs), Reg. Guide 1.109 (Rev. 1), Appendix C

DFG_i = the ground plane dose conversion factor for the i^{th} radionuclide (mrem/hr per pCi/m^2). Values are taken from Table E-6, Reg. Guide 1.109 (Rev. 1), or NUREG 0172. These values apply to all age groups.

SF = 0.7, shielding factor, from Table E-15 Reg. Guide 1.109 (Rev. 1)

Reference ODCM Part III Table 5.3.1

APPENDIX D

Ri - Grass Cow-Milk Pathway Dose Factor

$$R_i = k' [(Q_F \times U_{AP}) / (\lambda_i + \lambda_w)] \times (F_m) \times (r) \times (DFL_{i,a,o}) \times [((f_p \times f_s)/Y_p) + ((1-f_p \times f_s) e^{-\lambda_i t_h})/Y_s] e^{-\lambda_i t_f} \quad \text{(Eq D-1)}$$

Where:

k' = conversion factor, 1E6 picocurie/microcurie (pCi/μci)

Q_F = cow consumption rate, 50 kg/day, (Reg. Guide 1.109, Rev. 1)
goat consumption rate, 6 kg/day, (Reg. Guide 1.109, Rev. 1, Table E-2)

U_{AP} = Receptor's milk consumption rate; 330, 330, 400, 310 liters/yr for infant, child, teenager, and adult age groups, respectively (Reg. Guide 1.109, Rev. 1)

Y_P = agricultural productivity by unit area of pasture feed grass, 0.7 kg/m^2 (NUREG-0133)

Y_s = agricultural productivity by unit area of stored feed, 2.0 kg/m^2 (NUREG-0133)

F_m = stable element transfer coefficient (Table E-1, Reg. Guide 1.109, Rev. 1)

r = fraction of deposited activity retained in cow's feed grass, 0.2 for particulates, 1.0 for radioiodine (Table E-15, Reg. Guide 1.109, Rev. 1)

$DFL_{i,a,o}$ = the ingestion dose factor for organ, o, and the ith radionuclide for each respective age group, a (Tables E-11 to E-14, Reg. Guide 1.109, Rev. 1), or NUREG 0172.

λ_i = decay constant for the ith radionuclide, sec^{-1}

λ_w = decay constant for weathering, 5.73 x 10^{-7} sec^{-1} (NUREG-0133); based on a 14 day half life

t_f = 1.73 x 10^5 sec, the transport time from pasture to cow to milk to receptor (Table E-15, Reg. Guide 1.109, Rev. 1), or 2 days

t_h = 7.78 x 10^6 sec, the transport time from pasture to harvest to cow to milk to receptor (Table E-15, Reg. Guide 1.109, Rev. 1), or 90 days

f_p = 1.0, the fraction of the year that the cow is on pasture

f_s = 1.0, the fraction of the cow feed that is pasture grass while the cow is on pasture

The concentration of tritium in milk is based on the airborne concentration rather than the deposition. Therefore, R_i is based on (X/Q):

$$R^{C}_{t,a,o} = k'k''' F_m Q_F U_{AP} DFL_{t,a,o} (.75 [.5/H]) \quad \text{(Eq D-2)}$$

Where:

$k''' =$ 1E3 grams/kg

$H =$ 8 grams/m^3, absolute humidity of the atmosphere

$.75 =$ fraction of the total feed grass mass that is water

$.5 =$ ratio of the specific activity of the feed grass water to the atmospheric water (NUREG-0133)

$DFL_{t,a,o} =$ the ingestion dose factor for tritium and organ, o, for each respective age group, a (Tables E-11 to E-14, Reg. Guide 1.109, Rev. 1), or NUREG 0172.

All other parameters and values are as given above.

NOTE

Goat-milk pathway factor, R_i, will be computed using the cow-milk pathway factor equation. F_m factor for goat-milk will be from Table E-2 Reg. Guide 1.109, Rev. 1.

Reference: ODCM Part III Tables 5.4.1 to 5.4.4

APPENDIX E

Ri - Cow-Meat Pathway Dose Factor

$$R_i = k' [(Q_F \times U_{AP}) / (\lambda_i + \lambda_w)] \times (F_f) \times (r) \times (DFL_{i,a,o}) \times [((f_p \times f_s)/Y_p) + ((1-f_p f_s)\, e^{-\lambda_i t_h})/Y_s] \times e^{-\lambda_i t_f} \quad \text{(Eq E-1)}$$

Where:

k' = conversion factor, 1E6 picocurie/microcurie (pCi/μci)

Q_F = cow consumption rate, 50 kg/day, (Reg. Guide 1.109, Rev. 1)

U_{AP} = Receptor's meat consumption rate; 0, 41, 65, 110 kg/yr for infant, child, teenager, and adult age groups, respectively (Reg. Guide 1.109, Rev. 1)

F_f = the stable element transfer coefficients, days/kg (Table E-1, Reg. Guide 1.109, Rev. 1)

r = fraction of deposited activity retained in cow's feed grass, 0.2 for particulates, 1.0 for radioiodine (Table E-15, Reg. Guide 1.109, Rev. 1)

$DFL_{i,a,o}$ = the ingestion dose factor for organ, o, and the ith radionuclide for each respective age group, a *(Tables E-11 to E-14, Reg. Guide 1.109, Rev. 1), or NUREG 0172.*

λ_i = decay constant for the radionuclide i, sec^{-1}

λ_w = decay constant for weathering, 5.73×10^{-7} sec^{-1} (NUREG-0133), based on a 14 day half life

t_f = 1.73×10^6 sec, the transport time from pasture to receptor (NUREG-0133)

t_h = 7.78×10^6 sec, the transport time from crop to receptor (NUREG-0133)

Y_P = agricultural productivity by unit area of pasture feed grass, 0.7 kg/m^2 (NUREG-0133)

Y_s = agricultural productivity by unit area of stored feed, 2.0 kg/m^2 (NUREG-0133)

f_p = 1.0, the fraction of the year that the cow is on pasture

f_s = 1.0, the fraction of the cow feed that is pasture grass while the cow is on pasture

The concentration of tritium in meat is based on the airborne concentration rather than the deposition. Therefore, R_i is based on (X/Q):

$$R_{t,a,o} = k'k''' F_f Q_F U_{AP} (DFL_{t,a,o}) \times 0.75 \times (0.5/H) \quad \text{(Eq E-2)}$$

Where:

All terms are as defined above and in Appendix D.

Reference: ODCM Part III, Tables 5.6.1 to 5.6.4

APPENDIX F

Ri - Vegetation Pathway Dose Factor

$$R_i = k' \times [r/(Y_v(\lambda_i + \lambda_w))] \times (DFL_{i,a,o}) \times [(U^L_A) f_L e^{-\lambda_i t^L} + U^S_A f_g e^{-\lambda_i t^h}] \quad \text{(Eq F-1)}$$

Where:

k' = 1E6 picocurie/microcurie (pCi/µci)

U^L_A = the consumption rate of fresh leafy vegetation, 0, 26, 42, 64 kg/yr for infant, child, teenager, or adult age groups, respectively (Reg. Guide 1.109, Rev. 1)

U^S_A = the consumption rate of stored vegetation, 0, 520, 630, 520 kg/yr for infant, child, teenager, or adult age groups respectively (Reg. Guide 1.109, Rev. 1)

f_L = the fraction of the annual intake of fresh leafy vegetation grown locally, = 1.0 (NUREG-0133)

f_g = the fraction of the stored vegetation grown locally = 0.76 (NUREG-0133)

t_L = the average time between harvest of leafy vegetation and its consumption, 8.6×10^4 seconds [Table E-15, Reg. Guide 1.109, Rev. 1 (24 hrs)]

t_h = the average time between harvest of stored leafy vegetation and its consumption, 5.18×10^6 seconds, [Table E-15, Reg. Guide 1.109, Rev. 1 (60 days)]

y_v = the vegetation area density, 2.0 kg/m^2 (Table E-15, Reg. Guide 1.109, Rev. 1)

All other parameters are as previously defined.

The concentration of tritium in vegetation is based on the airborne concentration rather than the deposition. Therefore, R_i is based on (X/Q)

$$R_{t,a,o} = k'k''' [U^L_A f_L + U^S_A f_g] (DFL_{t,a,o}) \{.75 [.5/H]\} \quad \text{(Eq F-2)}$$

Where:

All terms are as defined above and in Appendix D.

Reference: ODCM Part III, Tables 5.7.1 to 5.7.4.

APPENDIX A-F REFERENCES

(Page 1 of 4)

Parameters Used in Dose Factor Calculations

Parameter	Value	Origin of Value		
		Table in R.G. 1.109	Section of NUREG-0133	Site-Specific
	***** For P_i *****			
DFA_i	Each radionuclide	E-9		Note 1
BR	3700 m^3/yr (child)	E-5		
	*****For Ri (Vegetation)*****			
r	*Each element type*	E-1		
Y_v	2.0 kg/m^2	E-15		
λw	5.73 E-7 sec^{-1}		5.3.1.3	
DFL_i	Each age group and radionuclide	E-11 thru E-14		Note 1
U_a^L	Each age group	E-5		
f_L	1.0		5.3.1.5	
t_L	8.6 E + 4 seconds	E-15		
U_a^S	Each age group	E-5		
f_g	0.76		5.3.1.5	
t_h	5.18 E + 6 seconds	E-15		
H	8.0 grams/kg		5.2.1.3	
	*****For Ri (Inhalation)*****			
BR	Each age group	E-5		
DFA_i	Each age group and nuclide	E-7 thru E-10		Note 1

APPENDIX A-F REFERENCES

(Page 2 of 4)

Parameters Used in Dose Factor Calculations

Parameter	Value	Origin of Value		
		Table in R.G. 1.109	Section of NUREG-0133	Site-Specific
	***** For R_i (Ground Plane) *****			
SF	0.7	E-15		
DFG_i	Each radionuclide	E-6		
t	4.73 E + 8 sec		5.3.1.2	
	***** For Ri (Grass/Animal/Meat) *****			
Q_F(Cow)	50 kg/day	E-3		
Q_F (Goat)	6 kg/day	E-3		Ref. Only
U_{ap}	Each age group	E-5		
λw	5.73 E-7 sec^{-1}		5.3.1.3	
F_f (Both)	Each element	E-1		
r	Each element type	E-15		
DFL_i	Each age group and nuclide	E-11 thru E-14		Note 1
f_p	1.0		5.3.1.3	Note 2
f_s	1.0		5.3.1.3	Note 2
Y_p	0.7 kg/m^3	E-15		
t_h	7.78 E + 6 sec	E-15		
Y_s	2.0 kg/m^2	E-15		
t_f	1.73 E + 6 sec	E-15		
H	8.0 grams/kg		5.2.1.3	

APPENDIX A-F REFERENCES

(Page 3 of 4)

Parameters Used in Dose Factor Calculations

Parameter	Value	Origin of Value		
		Table in R.G. 1.109	Section of NUREG-0133	Site-Specific
	***** For R_i (Grass/Cow/Milk) *****			
Q_f	50 kg/day	E-3		
U_{ap}	Each age group	E-5		
λw	5.73 E-7 sec^{-1}		5.3.1.3	
F_m	Each element	E-1		
r	Each element type	E-15		
DFL_1	Each age group and nuclide	E-11 thru E-14		Note 1
Y_p	0.7 kg/m^2	E-15		
t_h	7.78 E + 6 sec	E-15		
Y_s	2.0 kg/m^2	E-15		
t_f	1.73 E + 5 sec	E-15		
f_p	1.0		5.3.1.3	
f_s	1.0		5.3.1.3	
H	8.0 grams/kg		5.2.1.3	

APPENDIX A-F REFERENCES

(Page 4 of 4)

NOTES

1. Inhalation and ingestion dose factors were taken from the indicated source. For each age group, for each nuclide, the organ dose factor used was the highest dose factor for that nuclide and age group in the referenced table.

2. Typically beef cattle are raised all year on pasture. Annual land surveys have indicated that the small number of goats raised within 5 miles typically are used for grass control and not food or milk. Nevertheless, the goats can be treated as full meat sources where present, despite the fact that their numbers cannot sustain the meat consumption rates of Table E-5, NUREG-0133.

REFERENCES

1. Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, October 1977.

2. TMI-1 Technical Specifications, attached to Facility Operating License No. DPR-50.

3. NUREG-0133, "Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants," October 1978.