
Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants

A Guidance Manual for Users of
Standard Technical Specifications

**U.S. Nuclear Regulatory
Commission**

Office of Nuclear Reactor Regulation

J. S. Boegli, R. R. Bellamy,
W. L. Britz, R. L. Waterfield



NOTICE

Availability of Reference Materials Cited in NRC Publications

Most documents cited in NRC publications will be available from one of the following sources:

1. The NRC Public Document Room, 1717 H Street, N.W.
Washington, DC 20555
2. The Superintendent of Documents, U.S. Government Printing Office, Post Office Box 37082,
Washington, DC 20013-7082
3. The National Technical Information Service, Springfield, VA 22161

Although the listing that follows represents the majority of documents cited in NRC publications, it is not intended to be exhaustive.

Referenced documents available for inspection and copying for a fee from the NRC Public Document Room include NRC correspondence and internal NRC memoranda; NRC Office of Inspection and Enforcement bulletins, circulars, information notices, inspection and investigation notices; Licensee Event Reports; vendor reports and correspondence; Commission papers; and applicant and licensee documents and correspondence.

The following documents in the NUREG series are available for purchase from the GPO Sales Program: formal NRC staff and contractor reports, NRC-sponsored conference proceedings, and NRC booklets and brochures. Also available are Regulatory Guides, NRC regulations in the *Code of Federal Regulations*, and *Nuclear Regulatory Commission Issuances*.

Documents available from the National Technical Information Service include NUREG series reports and technical reports prepared by other federal agencies and reports prepared by the Atomic Energy Commission, forerunner agency to the Nuclear Regulatory Commission.

Documents available from public and special technical libraries include all open literature items, such as books, journal and periodical articles, and transactions. *Federal Register* notices, federal and state legislation, and congressional reports can usually be obtained from these libraries.

Documents such as theses, dissertations, foreign reports and translations, and non-NRC conference proceedings are available for purchase from the organization sponsoring the publication cited.

Single copies of NRC draft reports are available free, to the extent of supply, upon written request to the Division of Information Support Services, Distribution Section, U.S. Nuclear Regulatory Commission, Washington, DC 20555.

Copies of industry codes and standards used in a substantive manner in the NRC regulatory process are maintained at the NRC Library, 7920 Norfolk Avenue, Bethesda, Maryland, and are available there for reference use by the public. Codes and standards are usually copyrighted and may be purchased from the originating organization or, if they are American National Standards, from the American National Standards Institute, 1430 Broadway, New York, NY 10018.

PREPARATION OF
RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATIONS
FOR NUCLEAR POWER PLANTS

A Guidance Manual for Users of Standard Technical Specifications

Editors

J. S. Boegli
R. R. Bellamy
W. L. Britz
R. L. Waterfield

Principal Investigators

W. C. Burke
F. J. Congel

Other Participating Staff

J. E. Fairbent	K. F. Eckerman
P. G. Stoddart	J. H. Osloond
L. G. Bell	F. M. Akstolewicz
F. P. Cardile	D. L. Ondish
J. T. Collins	W. E. Kreger

Manuscript Completed: October 1978
Date Published: October 1978

Division of Site Safety and Environmental Analysis
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission

TABLE OF CONTENTS

<u>Chapter</u>	<u>Page</u>
1 INTRODUCTION.....	1
1.1 Purpose.....	1
1.2 Background.....	1
2 DEFINITIONS AND STAFF POSITIONS.....	5
2.1 Definitions.....	5
2.2 Staff Positions Augmenting Standard Definitions.....	5
3 SPECIAL CONSIDERATIONS.....	7
3.1 Multi-Unit Sites with Shared Radioactive Waste Management Systems.....	7
3.2 Population Exposure.....	7
3.3 Meteorological Data.....	8
3.4 National Interim Primary Drinking Water Regulations.....	8
3.5 Solid Waste Management System - Process Control Program.....	9
3.6 Offsite Dose Calculation Manual (ODCM).....	9
3.7 Identification of Radionuclides in Effluents.....	9
3.8 Environmental Radiation Protection Standards for Nuclear Power Operations.....	10
4 LIQUID EFFLUENTS.....	12
4.1 Instrumentation.....	12
4.2 Requirement for Implementing 10 CFR Part 20.....	13
4.3 Requirement for Implementing 10 CFR Part 50.....	14
4.4 Specification on Radioactivity Contents in Liquid-Containing Tanks.....	17
4.5 Specification on the Use of Liquid Radioactive Waste Management System....	18
5 GASEOUS EFFLUENTS.....	20
5.1 Instrumentation.....	20
5.2 Dose Limit for Implementing 10 CFR Part 20.....	21
5.3 Dose Limit for Implementing 10 CFR Part 50.....	27
5.4 Specification on the Use of Gaseous Radioactive Waste Management System...	36
5.5 Specification on Explosive Gas Mixture Limitation.....	37
5.6 Specification Unique to LWR Design Features.....	38
REFERENCES.....	42
APPENDIX A.....	A-1
APPENDIX B.....	B-1
APPENDIX C.....	C-1
APPENDIX D.....	D-1
ADDENDUM.....	AA-1

CHAPTER 1

INTRODUCTION

1.1 PURPOSE

The purpose of this manual is to describe methods found acceptable to the staff of the U.S. Nuclear Regulatory Commission (NRC) for the calculation of certain key values required in the preparation of proposed radiological effluent Technical Specifications using the Standard Technical Specifications for light-water-cooled nuclear power plants. This manual also provides guidance to applicants for operating licenses for nuclear power plants in the preparation of proposed radiological effluent Technical Specifications or in preparing requests for changes to existing radiological effluent Technical Specifications for operating licenses. The manual additionally describes current staff positions on the methodology for estimating radiation exposure due to the release of radioactive materials in effluents and on the administrative control of radioactive waste treatment systems.

1.2 BACKGROUND

Section 50.36, "Technical Specifications," of 10 CFR Part 50, "Licensing of Production and Utilization Facilities" (Ref. 1), requires that each nuclear power reactor operating license issued by the NRC contain Technical Specifications that set forth limits, operating conditions, and other regulatory requirements imposed on the facility operation for the protection of the health and safety of the public. Conditions and limitations corresponding to certain key values which are system-dependent and site-related are to be incorporated in these Technical Specifications for compliance with 10 CFR 50.36a, "Technical Specifications on Effluents from Nuclear Power Reactors." Under the provisions of Section 50.36, each applicant for an operating license is required to submit proposed Technical Specifications for his facility, including the supporting bases. These are reviewed by the NRC staff to assure that the proposed Technical Specifications contain such conditions and limitations as deemed appropriate and necessary; approved Technical Specifications are then included as Appendix A of the operating license.

Standard Technical Specifications have been developed by the staff for each appropriate nuclear steam supply system (NSSS) vendor to provide guidance to applicants for the preparation of proposed Technical Specifications. These are as follows:

- NUREG-0212 - "Standard Technical Specifications for Combustion Engineering Pressurized Water Reactors"
- NUREG-0103 - "Standard Technical Specifications for Babcock and Wilcox Pressurized Water Reactors"

NUREG-0452 - "Standard Technical Specifications for Westinghouse Pressurized Water Reactors"

NUREG-0123 - "Standard Technical Specifications for General Electric Boiling Water Reactors"

Current Standard Technical Specifications are available from the National Technical Information Service (NTIS), Springfield, Virginia, 22161. These Standard Technical Specifications will contain the radiological effluent Technical Specifications to be used by the applicant for an operating license. In the interim, model radioactive effluent Technical Specifications have been provided in NUREG-0472 (Ref. 2) for pressurized water reactors, and NUREG-0473 (Ref. 3) for boiling water reactors. Table 1.1 provides a summary of those applicable sections in the Standard Technical Specifications which contain conditions and limitations relative to the radiological effluent Technical Specifications to be discussed in this manual.

The Standard Technical Specifications contain the limiting conditions for operation necessary for complying with the Commission's regulations and are in a format that is acceptable to the NRC staff. The reporting requirements reflect the guidelines provided in Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications," Revision 4, August 1975 (Ref. 4) and Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations," Revision 3, May 1977 (Ref. 5).

The methodology discussed in this manual and used to implement the requirements of 10 CFR Part 50, Appendix I, "Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criterion 'As Low As Practicable' for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents" (Ref. 1), is consistent with the Regulatory Guides used in the staff's safety evaluations pursuant to 10 CFR 50.34a(c). These guides are as follows:

Regulatory Guide 1.109 - "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I" (Revision 1), October 1977. (Ref. 6)

Regulatory Guide 1.110 - "Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors," March 1976. (Ref. 7)

Regulatory Guide 1.111 - "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors" (Revision 1), July 1977. (Ref. 8)

Regulatory Guide 1.112 - "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors," April 1976. (Ref. 9)

TABLE 1.1
SUMMARY OF APPLICABLE SECTIONS IN THE STANDARD TECHNICAL SPECIFICATIONS
CONSIDERED IN THIS MANUAL

PLANT TYPE	Standard Technical Specification or Model Technical Specification in NUREG-0472 (Ref. 2) or NUREG-0473 (Ref. 3)		SECTION TITLES	DEFINITIONS	SECTION REFERENCE IN THIS MANUAL
	TABES	BASES			
PWR BWR	1.0	Yes	No	Definitions	2.0
PWR BWR	3/4.3.3.8	Yes	Yes	Radioactive Liquid Effluent Instrumentation	4.1
PWR BWR	3/4.3.3.9	Yes	Yes	Radioactive Gaseous Effluent Instrumentation	5.1
PWR BWR	3/4.11.1.1	Yes	Yes	Liquid Effluents, Concentration	4.2
PWR BWR	3/4.11.1.2	No	Yes	Liquid Effluents, Dose	4.3
PWR BWR	3/4.11.1.3	No	Yes	Liquid Effluents, Liquid Waste Treatment	4.5
PWR BWR	3/4.11.1.4	No	Yes	Liquid Effluents, Liquid Holdup Tanks	4.4
PWR BWR	3/4.11.2.1	Yes	Yes	Gaseous Effluents, Dose Rate	5.2
PWR BWR	3/4.11.2.2	No	Yes	Dose, Noble Gases	5.3
PWR BWR	3/4.11.2.3	No	Yes	Dose Radiotodines, Radioactive Material in Particulate Form and Radionuclides Other than Noble Gases	5.3
PWR BWR	3/4.11.2.4	No	Yes	Gaseous Effluents, Gaseous Waste Treatment	5.4
PWR BWR	3/4.11.2.5	No	Yes	Gaseous Effluents, Dose	3.8
PWR BWR	3/4.11.2.6A	No	Yes	Explosive Gas Mixtures (Systems designed to withstand a hydrogen explosion)	5.5
PWR BWR	3/4.11.2.6B	No	Yes	Explosive Gas Mixtures (Systems not designed to withstand a hydrogen explosion)	5.5
PWR BWR	3/4.11.2.7	No	Yes	Gaseous Effluents, Gas Storage Tanks	5.6
BWR	3/4.11.2.7	No	Yes	Gaseous Effluents, Main Condenser	5.6
BWR	3/4.11.3.8	No	Yes	Gaseous Effluents, Mark I or II Containment (Optional)	3.0
PWR BWR	3/4.11.3.1	No	Yes	Solid Radioactive Waste	3.0
PWR BWR	3/4.12.1	Yes	Yes	Monitoring Program	4.3, 5.3
PWR BWR	3/4.12.2	No	Yes	Land Use Census	4.3, 5.3
PWR BWR	Fig. 3.11-1	No	No	Unrestricted Area Boundary for Liquid Effluents	2.0
PWR BWR	Fig. 5.1-1	No	No	Unrestricted Area Boundary for Gaseous Effluents	2.0
PWR BWR	6.9	No	No	Reporting Requirements	General

Regulatory Guide 1.113 - "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I" (Revision 1), April 1977. (Ref. 10)

Computer codes used to determine certain values and parameters used with the Regulatory Guides listed above are described in the following documents:

NUREG-0017 - "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Pressurized Water Reactors (PWR-GALE Code)," April 1976 (Ref. 11)

NUREG-0016 - "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Boiling Water Reactors (BWR-GALE Code) " April 1976 (Ref. 12)

NUREG-0324 - "XOQDOQ, Program for the Meteorological Evaluation of Routine Effluent Releases at Nuclear Power Stations," September 1977 (Ref. 13)

These guides and computer codes provide acceptable methods of complying with the Commission's regulations; however, conformance with the staff's guidelines on Standard Technical Specifications, Regulatory Guides, and dose calculation methodology is not required. If the proposed Technical Specifications include mathematical models and parameters that differ from the methodology used by the staff to calculate set points and release rates or estimate doses due to releases of radioactive materials in effluents, the parameters and calculations used shall be substantiated in the Offsite Dose Calculation Manual.

Based on the findings of the Environmental and Safety Hearings, the Commission may impose certain additional or alternative limiting conditions for operation based on values that are more restrictive than those determined using this manual or provided in the Standard Technical Specifications. In such cases, the limiting conditions will be based on the decisions of the Atomic Safety and Licensing Board rather than the limiting values determined by this manual or included in the Standard Technical Specifications.

CHAPTER 2

DEFINITIONS AND STAFF POSITIONS

2.1 DEFINITIONS

Section 1.0, DEFINITIONS, of the Standard Technical Specifications provides standard definitions of terms and phrases which appear capitalized throughout the specifications. Standard definitions are provided to assure licensing consistency. When a term or phrase is used in a limited subject area of the Standard Technical Specifications, it is defined in the limited subject area and referenced by Specification number, table, figure or footnote.

2.2 STAFF POSITIONS AUGMENTING STANDARD DEFINITIONS

In certain circumstances, terms used in the Standard Technical Specifications are defined or specified in applicable regulations, such as 10 CFR Part 50 (Ref. 1). Staff positions clarifying certain of these definitions are as discussed below.

LIMITING CONDITIONS FOR OPERATION (LCO) is defined in 10 CFR 50.36(c)(2) as "the lowest functional capability or performance levels of equipment required for safe operation of the facility." This definition is applicable to the components of the radioactive waste management systems during normal reactor operations, including anticipated operational occurrences. When an LCO for a nuclear power reactor is not met, the licensee shall either shut down the reactor or follow any remedial action permitted by the Technical Specifications until the LCO can be met. Remedial action by the licensee may include processing by normal or alternate modes of operation for the control of radioactive effluents using such existing equipment as may be installed in the radioactive waste management systems.

MAINTENANCE AND USE of the equipment installed in the radioactive waste management systems is required in 10 CFR 50.34a(c) and in 10 CFR 50.36a(a)(1). The term, MAINTENANCE AND USE, is applicable to the installed components of the liquid, gaseous, and solid radioactive waste management systems and to instrumentation installed for the monitoring and control of potentially radioactive effluents. MAINTENANCE AND USE does not require the installation of fully redundant systems; however, prudent management procedures, such as scheduled standby and maintenance periods should be employed. The Standard Technical Specifications specify levels or values above which equipment installed in the radioactive waste management systems shall be used to enable the licensee to show that he is exerting his best efforts to maintain levels of radioactive effluents "as low as is reasonably achievable," in accordance with 10 CFR 50.36a.

UNRESTRICTED AREA is defined in 10 CFR 20.3(a)(17) (Ref. 14), as "any area access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, and any area used for residential quarters."

For purposes of implementation, the definition of UNRESTRICTED AREA has been expanded as follows: "any area at or beyond the site boundary access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, and any area within the site boundary used for residential quarters or industrial, commercial, institutional and recreational facilities". The UNRESTRICTED AREA boundary may coincide with the exclusion (fenced) area boundary, as defined in 10 CFR 100.3(a) (Ref. 15), may include land areas owned by the licensee, provided that occupancy is controlled by the licensee for the purposes of meeting the requirements of 10 CFR Part 20, but does not include areas over water bodies.

To assure that the UNRESTRICTED AREA boundary is defined in each license, the Standard Technical Specifications require two maps, one for liquid effluents (Figure 3.11-1)* and one for gaseous effluents (Figure 5.1-1)* locating all points surrounding the facility at which the licensee shall comply with the expanded definition, given above. These boundaries shall be consistent with those established in the Safety Analysis Report, or the Final Hazard Summary for the facility. The UNRESTRICTED AREAS, established at or beyond these boundaries, are also considered in the LIMITING CONDITIONS FOR OPERATION to keep levels of radioactive materials in effluents as low as is reasonably achievable, pursuant to 10 CFR 50.36a.

RELEASE RATE used in this manual is defined as, "the discharge of radioactive materials in liquid or gaseous effluents per unit time." The "second" is used as the practical reporting time unit for establishing release rates to show compliance with the requirements of 10 CFR Part 20 and for establishing instantaneous limitations based on potentially radioactive releases. The "hour" is used as the practical reporting time unit in establishing average release rates to show conformance with the requirements of 10 CFR Part 50 for noble gas releases, for gaseous radioactive effluents other than noble gases (radioiodines and particulates) and for radioactive materials released in liquid effluents. Liquid releases are further subdivided into batch and continuous releases. Gaseous releases are subdivided into short- and long-term releases. These gaseous release subdivisions classify cumulative releases as being either less than or greater than 500 hrs/year, respectively, for gaseous effluents. Further discussion is provided in Sections 3.3, 4.2 and 5.2 of this manual.

RADIOACTIVE WASTE MANAGEMENT SYSTEMS are defined as all process and control equipment provided to reduce the amount or concentration of radioactive materials (in any form) released from the facility. The overall systems may be divided into subsystems to handle the radioactive materials contained in liquid and gaseous streams and in solid waste. The Standard Technical Specifications have adopted nomenclature for systems and components which are in common use in the industry. In preparing proposed Technical Specifications, the system and component names may be changed to correspond to the terminology used in the Final Safety Analysis Report (FSAR) or the Final Hazard Summary, if applicable. Engineered Safety Feature (ESF) atmospheric cleanup systems are not considered to be radioactive waste management system components and, therefore, are not within the scope of this manual.

*This Figure is to be included in proposed Technical Specifications.

CHAPTER 3

SPECIAL CONSIDERATIONS

3.1 MULTI-UNIT SITES WITH SHARED RADIOACTIVE WASTE MANAGEMENT SYSTEMS

The Standard Technical Specifications are written on a "per unit" basis, since this is the format in which operating licenses are issued. When shared radioactive waste management systems are used by more than one reactor unit on a site, the wastes from all units are mixed for shared treatment; by such mixing, the effluent releases cannot accurately be ascribed to a specific reactor unit. The licensee should estimate the contributions from each unit based on input conditions, e.g., flow rates and radioactivity concentrations, or, if not practicable, the treated effluent releases may be allocated equally to each of the radioactive waste producing reactors sharing the treatment system. For determining conformance to LCOs, these allocations from shared treatment systems are to be added to the releases specifically attributed to each unit to obtain the total releases per unit.

It is preferred that discharge lines leading to common release points be separately controlled and monitored, and the licensee should consider instrumentation which will provide volumetric recording and radiological effluent monitoring, sampling and analyses on a unit basis. This has been accomplished for some units by continuous representative sampling and analysis of the streams prior to mixing and continuously monitored and controlled at the common release point. Multi-unit sites with common release points, but without shared radioactive waste management systems, are also required to determine the alarm/trip setpoints in Sections 4.1 and 5.1 of this manual to assure compliance with the requirements of 10 CFR Part 20 (Ref. 14) at each release point.

In preparing Technical Specifications for units at adjacent sites (multi-unit stations with a common boundary), the sites should be considered as a multi-unit site.

3.2 POPULATION EXPOSURE

The Standard Technical Specifications 3.11.1.3 and 3.11.2.4 require the use of equipment installed in the radioactive waste management system to meet the requirements of 10 CFR 50.36a (Ref. 1).

To assure that the Technical Specifications consider the population radiation doses, the use of the installed radioactive waste management system is required when the projected cumulative doses exceed an appropriate fraction of the individual dose limitations. This method of establishing use of the radwaste equipment assures that the staff's Appendix I cost-benefit analysis in the safety evaluation is not invalidated. Sections 4.5 and 5.4 of this manual provide these values for the Standard Technical Specifications. Guidance on reporting population exposures is provided in Regulatory Guide 1.21 (Ref. 17).

3.3 METEOROLOGICAL DATA

The Standard Technical Specifications consider the historical annual average atmospheric dispersion condition rather than real time dispersion conditions in determining the LCO for radioactive materials in gaseous effluents.

Releases are characterized as "long" or "short" term, depending on the frequency and duration of the releases. This characterization permits the matching of the releases to more appropriate atmospheric diffusion, dispersion and decay conditions.

"Long-term" refers to releases that are generally continuous and stable in release rate with some anticipated variation (i.e., <50%, based on a running monthly average) in release rate, such as is experienced in normal ventilation system effluents at nuclear power plants. Determination of doses due to long-term releases should use the historical annual average relative concentration (χ/Q) based on meteorological data summarized, as recommended in Regulatory Guide 1.111 (Ref. 8):

"Short-term" refers to releases that are intermittent in radionuclide concentrations or flow, such as releases from PWR gas storage tanks, PWR containment ventings and purges, BWR drywell purges (See Standard Technical Specification 3.11.3.8), BWR mechanical vacuum pump exhausts, and systems or components with infrequent use. Short-term releases may be due to operational variations which result in radioactive releases greater than 50% of the releases normally considered as long-term. Short-term releases from these sources during normal operation, including anticipated operational occurrences, are defined as those which occur for a total of 500 hours or less in a calendar year but not more than 150 hours in any quarter. Determination of doses due to short-term releases can use the annual average relative concentration (long-term) if it can be demonstrated that past short-term releases were sufficiently random in both time of day and duration (e.g., the short-term release periods were not dependent solely on atmospheric conditions or time of day) to be represented by the annual average dispersion conditions. Otherwise, the short-term relative concentration value should be calculated in accordance with the guidelines provided in NUREG-0324 (Ref. 13) for short-term release.

Even though "annual average" atmospheric dispersion conditions are used as basis for the Standard Technical Specifications, "real time" meteorological data should be summarized hour-by-hour and coupled with the corresponding releases, and the summary should be included in the SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT.

3.4 NATIONAL INTERIM PRIMARY DRINKING WATER REGULATIONS

Operators of nuclear power plants located on fresh water bodies which are used as sources of water for drinking water supply systems, are required to make a special report concerning the impact on the water supply system due to liquid effluent releases into the water bodies which are above the value(s) permitted in Specification 3.11.1.2 of the Standard Technical Specifications. The NRC has no legal responsibility to implement 40 CFR Part 141, "National Interim Primary Drinking Water Regulations" (Ref. 16), or to assure routine conformance to the Act since this is the responsibility of the Environmental Protection Agency and the

water plant operator. This special report is intended for public information and as a tool to assure awareness by the licensee of the impact of radioactive liquid releases on the community's water supply system. The impact within the water supply system is dependent on treatment given to the water taken into the system. The water plant operator is responsible for providing appropriate treatment to assure that 40 CFR Part 141 requirements are met. While the operator of the nuclear power plant is not responsible for meeting the requirements of 40 CFR Part 141 in the water supply system, his success in meeting the requirements of Specification 3.11.1.2 will assure an environmentally acceptable impact on the water supply system. The non-radiological impact is separately considered in the Appendix B Technical Specifications.

3.5 SOLID WASTE MANAGEMENT SYSTEM - PROCESS CONTROL PROGRAM

Standard Technical Specification 3.11.3.1 requires the operator of each nuclear power plant to establish a PROCESS CONTROL PROGRAM for the solid radioactive waste management system. The purpose of the PROCESS CONTROL PROGRAM is to provide reasonable assurance of the complete SOLIDIFICATION of processed wastes and of the absence of free water in the processed waste. At the time the applicant submits proposed Technical Specifications, he should submit the PROCESS CONTROL PROGRAM for NRC review and approval prior to implementation. The PROCESS CONTROL PROGRAM should consist of the processing steps and a set of established process parameters, which include but are not limited to pH, oil content, ratio of solidification agent to influent waste, water content, and ratio of solidification agent to chemical additive for each type of anticipated waste (filter sludges, spent resins, evaporator bottoms, boric acid solutions, sodium sulfate solutions and filter media). The surveillance requirements in the Standard Technical Specifications provide the steps to be taken to assure that operation is within the parameters established by the PROCESS CONTROL PROGRAM. Packaging procedures should demonstrate conformance with Specification 3.11.3.1. The PROCESS CONTROL PROGRAM required by the Standard Technical Specifications is to be documented in the operating procedures for each reactor and available for review by the NRC inspector. A summary of changes to the PROCESS CONTROL PROGRAM shall be provided in the SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT.

3.6 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Standard Technical Specifications 3.3.3.8 and 3.3.3.9 require the operator of each nuclear power plant to establish alarm and trip action setpoints for each radioactive liquid and gaseous effluent release point in maintained, auditable records, determined in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM). The ODCM shall contain the methodology and parameters to be used in the calculation of offsite doses due to radioactive liquid and gaseous effluents pursuant to Specifications 3.11.1.2, 3.11.2.2 and 3.11.2.3, and the established limits of Specifications 3.11.1.1 and 3.11.2.1. The ODCM shall be submitted to the NRC with the proposed Technical Specifications for review and approval by the NRC. Changes to the ODCM shall be provided in the SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT.

3.7 IDENTIFICATION OF RADIONUCLIDES IN EFFLUENTS

In order to determine the radiological impact associated with the release of radioactive materials in liquid and gaseous effluents, the principal radionuclides contributing to the

dose must be identified. Tables 4.11-1 and 4.11-2** of the Standard Technical Specifications contain the sampling and analysis programs required for identifying principal radionuclides in effluents. These tables were compiled using the guidelines of Regulatory Guide 1.21 (Ref. 17) and reflect current radiochemical analytical methods. Other methods may be necessary to enhance identification and analysis, as provided by the footnotes to Tables 4.11-1 and 4.11-2. In lieu of sample-analysis, if the applicant does not consider that the collection, radiochemical separation, and analytical methods are technically feasible or practical at the specified LLD, then the dose limitations in Specifications 3.11.1.2, 3.11.2.2, and 3.11.2.3 should be proportionally reduced by assuming the continued presence and release concentrations of those radionuclides as determined by the source term (GALE Code, Ref. 11 or 12). For example, the dose LCO may be reduced based on predicted radioactive materials in gaseous effluents from PWR turbine buildings if sampling is not provided. For BWRs and PWRs it may be reduced if carbon-14 analysis is not provided.

3.8 ENVIRONMENTAL RADIATION PROTECTION STANDARDS FOR NUCLEAR POWER OPERATIONS

Standard Technical Specification 3.11.2.5 specifies in the Action that when the calculated doses associated with the effluent releases exceed twice* the limits of any one of the Specifications 3.11.1.2, 3.11.2.2 or 3.11.2.3, the licensee shall prepare and submit a Special Report to the Commission and limit subsequent releases such that the dose or dose commitment to a real individual from all uranium fuel cycle sources is limited to ≤ 25 mrem to the total body or any organ (except the thyroid, which is limited to ≤ 75 mrem) over 12 consecutive months. This Special Report shall include an analysis which demonstrates that radiation exposures to all real individuals from all uranium fuel cycle sources (including all liquid and gaseous effluent pathways and direct radiation) are less than the standards in 40 CFR Part 190, Environmental Radiation Protection Standards for Nuclear Power Operations (Ref. 18). If analysis indicates that releases resulting in doses that exceed the 40 CFR 190 Standard could occur, obtain a variance from the Commission to permit such releases. The Standard Technical Specifications 3.11.1.2, 3.11.2.2 and 3.11.2.3 consider doses to a real individual and apply to each reactor but do not include any other portion of the uranium fuel cycle or direct shine from the reactor.

The "Uranium fuel cycle" is defined in 40 CFR Part 190.02(b) as:

"Uranium fuel cycle means the operations of milling of uranium ore, chemical conversion of uranium, isotopic enrichment of uranium, fabrication of uranium fuel, generation of electricity by a light-water-cooled nuclear power plant using uranium fuel, and reprocessing of spent uranium fuel, to the extent that these directly support the production of electrical power for public use utilizing nuclear energy, but excludes mining operations, operations at waste disposal sites, transportation of any radioactive material in support of these operations, and the reuse of recovered non-uranium special nuclear and by-product materials from the cycle."

The following general guidelines are presented for preparation of the Special Report:

- 1) determine which uranium fuel cycle facilities or operations, in addition to the nuclear power reactor units at the site, contribute to the annual dose to the maximum exposed member of the public. The maximum exposed member of the public for

*This value may be reduced for multi-unit sites depending on staff analysis.

**These Tables are to be included in proposed Technical Specifications.

this evaluation may or may not correspond to the individual considered in the Technical Specification;

- 2) determine the total annual dose to this person from all existing pathways and sources of radioactivity and radiation using the methodologies described in this NUREG document and applicable references. Where additional information on pathways and nuclides is needed, the best available information should be used and documented;
- 3) include direct radiation from the site in the dose determination. An acceptable method for calculating radiation from the N-16 component of direct radiation is: SKYSHINE, A Computer Procedure for Evaluating Effects of Structure Design on N-16 Gamma-Ray Dose Rates, Radiation Research Associates, Inc. Report RRA-T7209, November 1972 (Ref. 19).

In addition to N-16, all direct radiation from the plant and storage facilities should be considered in the dose determination. The direct dose component (including N-16) may be determined by calculation or actual measurement (e.g., high pressure ionization chamber). The calculation or actual measurement must be documented in this Special Report.

The 25 mrem and 75 mrem dose standards are effective December 1, 1979, except for doses arising from operations associated with the milling of uranium ore which is effective December 1, 1980.

Further information on the method of implementation of 40 CFR Part 190 is being developed by the NRC staff.

CHAPTER 4

LIQUID EFFLUENTS

4.1 INSTRUMENTATION

Standard Technical Specification 3.3.3.8 requires that:

"The radioactive liquid effluent monitoring instrumentation channels shown in Table 3.3-11* shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Specification 3.11.1.1 are not exceeded. The setpoints shall be determined in accordance with procedures as described in the ODCM, and shall be recorded in the station log."

Table 3.3-11* provides a list of radioactive liquid effluent monitoring instrumentation needed to comply with the requirements of General Design Criteria (GDC) 60, 63, and 64 of Appendix A to 10 CFR Part 50 (Ref. 1). The list includes instrumentation such as radioactivity monitoring and sampling devices, automatic control devices, and essential flow and level devices which are components of the monitoring channels. The list uses common nomenclature for the effluent streams; however, the names may be revised, as necessary, to conform with a particular plant's nomenclature. Deletion of any item listed should be justified. Clarification of proposed Technical Specifications should be provided by a simple drawing or sketch showing stream intersections, instrumentation, and control features. Duplicate instrumentation (i.e., instruments that measure different sensor parameters or ranges) should be listed separately. The channel logic should assure that the alarmed trip action is not negated by switching.

The plant procedures should contain a quality assurance program for instruments as recommended in Regulatory Guide 4.15 (Ref. 20).

4.1.1 Setpoint Determination to be Provided in the ODCM

The alarm and trip setpoint(s) for each instrument channel listed in Table 3.3-11* should be provided and should correspond to a value(s) which represents a safe margin of assurance that the instantaneous liquid release limit of 10 CFR Part 20 is not exceeded. If the alarm and the automatic control trip are separate devices, the alarm/trip setpoint in the ODCM should list the separate trip setpoints. The alarm/trip setpoint in the ODCM should list the alarm setpoint where any trip actions are by manual initiation. The method for calculating fixed and adjustable setpoints shall be provided in the ODCM and auditable records shall be maintained indicating the actual setpoints used at all times. For setpoint calculations, see the Addendum to this manual.

* This Table is to be included in proposed Technical Specifications.

The alarm/trip setpoint for a liquid effluent radiation monitor should be determined based on the instantaneous concentration limits of 10 CFR Part 20, Appendix B, Table II, Column 2, and are to be applied at the point of discharge for that stream, pipe or conduit into the unrestricted area, as defined by Figure 3.1-1.* The alarm/trip setpoint should not consider dilution, dispersion or decay in the unrestricted area beyond the release point. An isolation control valve or system shall be provided on the discharge line prior to the release point to permit termination of radioactive releases prior to exceeding the instantaneous concentration limits of 10 CFR Part 20. The isolation and radiation monitoring points are to be located upstream of the release point far enough to assure that the time lag between the established alarm and isolation of the release will not permit a release exceeding these limits. If the stream is diluted by non-radioactive effluents, and the stream dilution and effluent isolation control system is in the exclusion area, the monitor's alarm/trip setpoint may be determined by considering the known in-plant dilution. In-plant dilution is the ratio of the total release rate at the release point into the unrestricted area to the release rate of the undiluted stream, and should be based on continuous measurement of these liquid flows. In such cases, alarm/trip setpoints should also be provided on the flow or level instrumentation with indication in the main control room. The minimum or actual instantaneous in-plant dilution ratio on which the liquid effluent radiation monitor alarm/trip setpoint has been based, should be continuously measured to aid prompt corrective action to satisfy Specification 3.11.1.1.

Conservative assumptions may be included in establishing setpoints to account for such system variables as the control and measurement system efficiency and detection capabilities during normal and anticipated operating conditions, the effects of multiple release points with common or shared in-plant dilution, variability of dilution flow and principal radionuclide composition, and the time lag between alarm/trip action and final isolation of the radioactive effluent. A record of analyses showing current spectra of radionuclides used to calibrate radiation monitors should be maintained in the plant records.

The instruments listed in Table 3.3-11** should also be included in Table 4.3-11** to provide the instrument surveillance requirements, such as calibration, source checking, functional testing and channel checking.

4.2 REQUIREMENT FOR IMPLEMENTING 10 CFR PART 20

In preparing proposed Technical Specifications, Figure 3.11-1* should consist of a map of the site area, showing the unrestricted area boundary for liquid effluents, as defined in 10 CFR 20.3(a)(17). Guidelines for preparing the figure are contained in Section 2.1.1 of Regulatory Guide 1.70 (Ref. 21).

Standard Technical Specification 3.11.1.1 specifies that:

* This Figure is to be included in proposed Technical Specifications.

** See footnote on page 12.

"The concentration of radioactive material released from the site to unrestricted areas (see Figure 3.11-1)* shall be limited to the concentrations specified in 10 CFR Part 20, Appendix B, Table II, Column 2 for radionuclides other than noble gases and 2×10^{-4} $\mu\text{Ci/ml}$ total activity concentration for all dissolved or entrained noble gases."

The concentration limits provided in 10 CFR Part 20, Appendix B, Table II, Column 2, do not include an MPC for noble gases dissolved or entrained in liquid effluents. An MPC of 2×10^{-4} $\mu\text{Ci/ml}$ has been established, based on the assumption that xenon-135 is the controlling radionuclide; the Xe-135 MPC in air (submersion) was converted to an equivalent concentration in water using the method described in International Commission on Radiological Protection (ICRP) Publication 2 (Ref. 22). The value of 2×10^{-4} $\mu\text{Ci/ml}$ shall be used for a mixture of dissolved or entrained noble gases, not otherwise identified in liquid releases.

To demonstrate that the Specifications are being met, the surveillance requirements specify that a sampling and analysis program be implemented according to Table 4.11-1.** There are two general types of releases: batch and continuous. A batch release is the discharge of liquid waste of a discrete volume. A continuous release is the discharge of liquid wastes of a nondiscrete volume; e.g., from a volume or system that has an input flow during the continuous release. For example, releases from sample monitor tanks are batch, and releases from steam generator blowdown are continuous. The sampling and analysis frequency and the type of analysis required by the Standard Technical Specifications is given in Table 4.11-1 for each type of release. The lower limit of detection is also specified in Table 4.11-1 for typical in-plant radiochemical analysis equipment. This program meets the requirements of 10 CFR Part 50, GDC 64, and conforms to the guidelines given in Regulatory Guides 1.21 (Ref. 17) and 4.15 (Ref. 20).

4.3 REQUIREMENT FOR IMPLEMENTING 10 CFR PART 50

The Standard Technical Specification 3.11.1.2 requires that the cumulative dose contributions be determined in accordance with the ODCM at least once per 31 days. The cumulative dose contributions should consider the dose contributions from the maximum exposed individual's consumption of fish, invertebrates and potable water, as appropriate. Normally, the adult is the maximum exposed individual. All of these pathways should be considered in the calculation unless demonstrated not to be present. For many plant sites, the dose calculations may be performed assuming conservative dilution factors and receptor locations to show compliance with the Technical Specification rather than a more rigorous determination. The relationships presented below are acceptable for inclusion in the ODCM. If other methods are selected to implement this Specification, it is expected that the alternate method will include the same general features considered below.

* See footnote on page 13.

** See footnote on page 12.

The dose contributions for the total time period $\sum_{\ell=1}^m \Delta t_{\ell}$ should be determined by calculation at least once per 31 days and a cumulative summation of these total body and any organ doses should be maintained for each calendar quarter. These dose contributions should be calculated for all radionuclides identified in liquid effluents released to unrestricted areas using the following expression:

$$D_{i\tau} = \sum_i [A_{i\tau} \sum_{\ell=1}^m \Delta t_{\ell} C_{i\ell} F_{\ell}]$$

where:

D_{τ} = the cumulative dose commitment to the total body or any organ, τ , from the liquid effluents for the total time period $\sum_{\ell=1}^m \Delta t_{\ell}$, in mrem.

Δt_{ℓ} = the length of the ℓ th time period over which $C_{i\ell}$ and F_{ℓ} are averaged for all liquid releases, in hours.

$C_{i\ell}$ = the average concentration of radionuclide, i , in undiluted liquid effluent during time period Δt_{ℓ} from any liquid release, in $\mu\text{Ci/ml}$.

$A_{i\tau}$ = the site related ingestion dose commitment factor to the total body or any organ τ for each identified principal gamma and beta emitter listed in Table 4.11-1,* in mrem-ml per hr- μCi .

F_{ℓ} = the near field average dilution factor for $C_{i\ell}$ during any liquid effluent release. Defined as the ratio of the maximum undiluted liquid waste flow during release to the product of the average flow from the site discharge structure to unrestricted receiving waters times _____. (_____ is the site specific applicable factor for the mixing effect of the discharge structure.)

The term $C_{i\ell}$ is the composite undiluted concentration of radioactive material in liquid waste at the common release point determined from the Radioactive Liquid Waste Sampling and Analysis Program, Table 4.11-1* in the Standard Technical Specifications. All dilution factors beyond the sample point(s) are to be included in the F_{ℓ} and $A_{i\tau}$ terms.

The term F_{ℓ} is a near field average dilution factor, considering the combined liquid releases from each unit even if there is more than one release point to the unrestricted area per unit within one-quarter mile of each other. As described in Section 3.1 of this manual, multi-unit sites with shared radioactive waste management systems should calculate the total continuous and batch liquid release concentrations for each reactor. The value of the term F_{ℓ} should be determined as:

* See footnote on page 12.

$$F_{\ell} = \frac{\text{liquid radioactive waste flow per unit}}{\text{discharge structure exit flow per unit} \times \text{applicable factor}}$$

The liquid radioactive waste flow is the maximum flow from all continuous and batch radioactive effluent releases specified in Table 4.11-1, from all liquid radioactive waste management systems, per unit. The discharge structure exit flow is the average flow during disposal from the discharge structure release point into the receiving water body (in an unrestricted area) per unit. The definition of F_{ℓ} also requires a value to be included in Specification 3.11.1.2 for the dilution as a result of mixing effects in the near field of the discharge structure. For plants with once through cooling, the applicable factor is set equal to one, i.e., no additional dilution is considered. For plants with cooling towers, onsite ponds, or lagoons, the factor shall be a number such that the product of the average blowdown flow to the receiving water body, in cfs and the applicable factor, is 1000 cfs or less. The 1000 cfs figure was selected to correspond to a typical flow for a unit with once-through cooling water and agrees with the staff method for determining compliance with Appendix I (Ref. 1) at the OL stage. The value of this applicable factor is to be included in the blank provided for the term F_{ℓ} . The actual dilution factor value is dependent upon the dilution available in the near field of the receiving water body; however, the applicable factor is limited, as stated above.

4.3.1 Dose Factor Related to Liquid Effluents

The above equation for calculating the dose contributions requires the use of a dose factor $A_{i\tau}$ for each nuclide, i , which embodies the dose factors, pathway transfer factors (e.g., bioaccumulation factors), pathway usage factors, and dilution factors for the points of pathway origin. The adult total body dose factor and the maximum adult organ dose factor for each radionuclide will be used from Table E-11 of Regulatory Guide 1.109 (Ref. 6); thus the list should contain critical organ dose factors for various organs. The dose factor may be written:

$$A_{i\tau} = k_o (U_w/D_w + U_F BF_i + U_I BI_i) DF_i$$

where

$A_{i\tau}$ = composite dose parameter for the total body or critical organ of an adult for nuclide, i , for all appropriate pathways, mrem/hr per $\mu\text{Ci}/\text{ml}$.

k_o = units conversion factor, $1.14 \times 10^5 = 10^6 \text{ pCi}/\mu\text{Ci} \times 10^3 \text{ ml}/\text{kg} \div 8760 \text{ hr}/\text{yr}$.

U_w = 730 kg/yr, adult water consumption (fresh water site only).

U_F = 21 kg/yr, adult fish consumption (all sites).

U_I = 5 kg/yr, adult invertebrate consumption (salt water site only).

BF_i = Bioaccumulation factor for nuclide, i , in fish (fresh or salt water site, as applicable), pCi/kg per pCi/l , from Table A-1 of Regulatory Guide 1.109 (Ref. 4).

BI_i = Bioaccumulation factor for nuclide, i , in invertebrates (salt water only), pCi/kg per pCi/l , from Table A-1 of Regulatory Guide 1.109 (Ref. 4).

DF_i = Dose conversion factor for nuclide, i , for adults in pre-selected organ, τ , in mrem/pCi, from Table E-11 of Regulatory Guide 1.109 (Ref. 4).

D_w = Dilution factor from the near field area within one-quarter mile of the release point(s) to the potable water intake for the adult water consumption (fresh water site only).

Inserting the usage factors of Regulatory Guide 1.109 (Ref. 6) as appropriate into the equation gives the following expressions:

For Fresh Water sites: $A_{i\tau} = 1.14 \times 10^5 (730/D_w + 21BF_i) DF_i$

For Salt Water sites: $A_{i\tau} = 1.14 \times 10^5 (21BF_i + 5BI_i) DF_i$

As noted, all the factors required to calculate the values of $A_{i\tau}$ should be contained in the ODCM. The staff's method of calculating dilution factors for aquatic dispersion is provided in Regulatory Guide 1.113 (Ref. 10). The ODCM should include a detailed presentation of the calculation model and a tabulation of all values assigned to the parameters in expressions used to implement the Specification 3.11.1.2.

4.3.2 Special Reports

The Standard Technical Specifications 3.11.1.2, 3.11.1.3, 3.11.2.2, 3.11.2.3, 3.11.2.4 and 3.11.3.1 require that action be taken when the Specification cannot be met. The action is in the form of special reports, in lieu of licensee event reports, indicating the corrective action to be taken to reduce the dose impact due to the release of radioactive materials in liquid effluents.

These special reports should be prepared using the methodology provided in this manual to determine the dose impact. Such information in the special reports will be used by the staff in determining if the corrective action proposed by the licensee is adequate to bring the releases within the design objectives of Appendix I to 10 CFR Part 50. These special reports may also require submitting additional information as described in Section 3.4 of this manual.

4.4 SPECIFICATION ON RADIOACTIVITY CONTENTS IN LIQUID-CONTAINING TANKS

Standard Technical Specification 3.11.1.4 and Tables 3.3-11* and 4.3-11* list liquid-containing tanks outside containment that are to be analyzed periodically to verify that the radioactivity content (in curies, excluding tritium and dissolved or entrained noble gases) is below the specified value. Tanks included in this Specification are those that are not surrounded by liners, dikes or walls capable of holding the tank contents and do not have tank overflow and drains connected to the liquid radioactive waste management system. Indoor tanks are not included unless an analysis based on design basis fission product leakage from the fuel results in radionuclide concentrations in excess of the limits of 10 CFR Part 20, Appendix B, Table II, Column 2, where leaked fluid is capable of affecting the nearest existing or known future water supply** in an unrestricted area.

* See footnote on page 12.

** "Supply" means a well or surface water intake that is used as a water source for direct human consumption or indirectly through animals, crops or food processing. "Known future" water supply means potential wells or surface water intakes which are identified, or may be reasonably deduced from available information.

For those tanks that are determined to be included in Specification 3.11.1.4 and Tables 3.3-11 and 4.3-11, a curie limit should be determined based on the methodology presented in Appendices A or B of this manual, using the PWR-RATAFR Computer Code for pressurized water reactor plants, or the BWR-RATAFR Computer Code for boiling water reactor plants, respectively. The methodology is based on the calculated radionuclide inventory in the tank at 80% capacity using a design basis fission product source term of (1) 1% of the operating fission product inventory in the core being released to the primary coolant for a PWR, or (2) a fission product release consistent with a noble gas release rate of 100 $\mu\text{Ci/MWt-sec}$ at 30 minutes decay for a BWR. These Computer Codes determine the radionuclide inventory in a tank that would result in concentrations equal to the limits of 10 CFR Part 20, Appendix B, Table II, Column 2, at (1) the nearest potable water supply and (2) the nearest surface water supply in an unrestricted area.

By excluding tritium and dissolved or entrained noble gases from the surveillance analyses, since these can be estimated for any licensee event report, Specification 3.11.1.4 should include the lowest curie quantity of activation and mixed fission products determined for any tank listed in Specification 3.11.1.4 as the curie limit for all tanks included in that Specification.

Most operating reactors have required the use of temporary process and storage tanks during maintenance and service periods, or when temporary solidification equipment is used at the facility; therefore, the Specification 3.11.1.4 should indicate a "temporary tank." The curie limit for a temporary tank may be calculated by the above method, but should be limited to ≤ 10 curies, excluding tritium and dissolved or entrained gases. If the temporary tank is mobile and not used for more than a calendar quarter, it need not be included in Tables 3.3-11* and 4.3-11.*

4.5 SPECIFICATION ON THE USE OF LIQUID RADIOACTIVE WASTE MANAGEMENT SYSTEM

Standard Technical Specification 3.11.1.3 specifies that:

"The liquid radwaste treatment system shall be OPERABLE. The appropriate subsystems shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent releases to unrestricted areas (see Figure 3.11-1)* when averaged over 31 days, exceeds 0.06 mrem to the total body or 0.2 mrem to any organ."

The operability of the liquid radioactive waste management system ensures that this system will be available for use whenever liquid effluents require treatment prior to release to the environment. The term "liquid radwaste treatment system" involves all of the installed and available liquid radioactive waste management system equipment, as well as their controls, power instrumentation, and services that make the system functional. Equipment that is considered standby or redundant is also included, since their function is to assure operability. The specification also permits alternate treatment paths using alternate subsystems and equipment to be used in the event that the normal treatment is inoperable.

* See footnote on page 13.

This Specification requires maintenance and use of the liquid radioactive waste management system for conformance to 10 CFR Part 50.36a. Maintenance and use of the radioactive waste management system components are discussed in Section 2.2 of this manual.

To determine if use of the installed equipment is necessary, the licensee must project the cumulative liquid effluent releases over the ensuing 31 days. These releases should include all plant effluents from all liquid radioactive waste management and liquid waste disposal system components that are planned to be operated at the projected capacity and performance of each component used during the specified time. These releases should include a margin, based on operating data, for anticipated and unplanned operational occurrences and should use the methodology discussed in Section 4.3 of this manual. The impact from this projected cumulative release is to be compared to 0.06 mrem for the total body or 0.2 mrem for any organ. If the projection indicates these values will be exceeded, then the installed liquid radioactive waste management system components that will reduce those radioactive materials in liquid effluents and the projected impact, must be used.

The values for the projected impact, given above, correspond to approximately one forty-eighth of the design objective values of Appendix I, Section II.A of 10 CFR Part 50 in a month, and if continued at this rate for a year, they would correspond to less than one-fourth the values limited by Specification 3.11.1.2.b. The calculations of projected cumulative dose impact that could result from the proposed operation should use the methodology provided in Section 4.3 of this manual.

CHAPTER 5

GASEOUS EFFLUENTS

5.1 INSTRUMENTATION

Standard Technical Specification 3.3.3.9 requires that:

"The radioactive gaseous process and effluent monitoring instrumentation channels shown in Table 3.3-12* shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Specification 3.11.2.1 are not exceeded. The setpoints shall be determined in accordance with procedures as described in the ODCM, and shall be recorded in the station log."

Table 3.3-12* provides a list of the radioactive gaseous effluent monitoring instrumentation needed to comply with the requirements of General Design Criteria (GDC) 60, 63, and 64 of Appendix A to 10 CFR Part 50. The list includes instrumentation such as radioactivity monitoring and sampling devices, automatic control devices and essential flow and level devices that are components of the monitoring systems. The list uses common nomenclature for the effluent streams; however, the names may be revised as necessary, to conform with a particular plant's nomenclature. The list may include effluent streams which are not applicable to a given plant, or which may be duplicated and, therefore, should be tailored for the proposed Technical Specifications. Clarification of proposed Technical Specifications should be provided by a simple drawing or sketch, showing stream intersections, instrumentation and control features. Duplicate instrumentation (i.e., instruments that measure different sensor parameters or ranges) should be listed separately in Tables 3.3-12* and 4.3-12.* The channel logic should assure that the alarmed trip action is not negated by switching.

The plant procedures should contain a quality assurance program for instruments as recommended in Regulatory Guide 4.15 (Ref. 20).

5.1.1 Setpoint Determination to be Provided in the ODCM

The alarm/trip setpoint or automatic control trip setpoint for each instrument channel listed in Table 3.3-12* should be provided and should correspond to a value(s) which represents a safe margin of assurance that the instantaneous gaseous release limit of Specification 3.11.2.1(a) will not be exceeded. For channels with separate alarm and automatic control trips, the setpoint for the automatic control trip should be the established value referenced above; the corresponding setpoint for alarm/trip should be established such that an alarm trip will occur either in advance of the automatic control trip or simultaneously with the automatic control trip. For channels with alarm trips only, the setpoint for the alarm/trip should be the established value referenced above, provided that the manual or procedural response to the alarm represents a safe margin of assurance that the instantaneous gaseous release limit of 10 CFR Part 20 will not be exceeded. The alarm/trip setpoint in the ODCM should list the alarm setpoint for those channels where any trip actions are by

* This Table is to be included in the proposed Technical Specifications.

manual initiation. The method for calculating fixed or adjustable setpoints shall be provided in the ODCM and auditable records shall be maintained indicating the actual setpoints used at all times.

The alarm/trip setpoint for any gaseous effluent radiation monitor should be determined based on the instantaneous (see RELEASE RATE fundamental time units in Section 2.2 of this manual) concentration limits of 10 CFR Part 20, Appendix B, Table II, Column 1, and are to be applied at the point at which the discharge leaves the restricted area boundary into an unrestricted area, as defined by Figure 5.1-1.** The bases for each setpoint should consider the type of release at the monitor's location, e.g., long-term releases using the long-term atmospheric dispersion conditions or, if applicable (see Section 3.3 of this manual), short-term releases using the short-term atmospheric dispersion conditions. An isolation control valve or system shall be provided on the discharge line upstream of the release point to permit isolation prior to exceeding the specified release limits.

If the alarm/trip setpoints are based on predetermined factors accounting for atmospheric conditions, the elevation of the release point should be considered. The symbols used in the equations in this manual use a subscript (s) for a free-standing stack for elevated releases (or vents that take the plume out of the building wake) and a subscript (v) for vent for releases that are not completely elevated. Guidance on the staff's method for estimating atmospheric transport and dispersion of gaseous effluents in routine releases is provided in Regulatory Guide 1.111 (Ref. 8). The radiation monitor alarm/trip setpoints for each release point should be based on the radioactive noble gases in gaseous effluents. It is not considered to be practicable to apply instantaneous alarm/trip setpoints to integrating radiation monitors sensitive to radioiodines, radioactive materials in particulate form and radionuclides other than noble gases. Alarm/trip setpoints should also be provided in the main control room for flow measurement devices which are part of continuous monitoring or sampling systems and should alarm on loss-of-flow or departure from an established flow range. In all cases, conservative assumptions may be necessary in establishing these setpoints to account for system variables, such as the control and measurement system efficiency and detection capabilities during normal, anticipated, and unusual operating conditions, the variability in release flow and principal radionuclides, and the time lag between alarm/trip action and the final isolation of the radioactive effluent. The current spectrum of radionuclides used to calibrate radiation monitors should be maintained in the plant records. The instruments listed in Table 3.3-12* should also be included in Table 4.3-12,* to provide the instrument surveillance requirements, such as calibration, source checking, functional testing, and channel checking.

5.2 DOSE LIMIT FOR IMPLEMENTING 10 CFR PART 20

In preparing proposed Technical Specifications, Figure 5.1-1** should consist of a map of the site area showing the exclusion boundary, as defined in 10 CFR 100.3(a) (Ref. 15) and the unrestricted area boundary, as defined in 10 CFR 20.3(a)(17) (Ref. 14). Guidelines for this figure are contained in Section 2.1.1 of Regulatory Guide 1.70 (Ref. 21). Details on the release point locations and significant elevations should be given in Figure 5.1-1.**

*See footnote on page 20.

**This Figure is to be included in proposed Technical Specifications.

5.2.1 Implementation of 10 CFR Part 20 - Airborne Releases

The Standard Technical Specification 3.11.2.1 implements 10 CFR Part 20 as follows:

"The instantaneous dose rate in unrestricted areas (see Figure 5.1-1)** due to radioactive materials released in gaseous effluents from the site shall be limited to the following values:

- a. The dose rate limit for noble gases shall be < 500 mrem/yr to the total body and < 3000 mrem/yr to the skin, and
- b. The dose rate limit for all radioiodines and for all radioactive materials in particulate form and radionuclides other than noble gases with half lives greater than 8 days shall be < 1500 mrem/yr to any organ."

The ODCM should provide the mathematical relationships used to implement the above specification. The relationships presented below are acceptable for inclusion in the ODCM. If other methods are selected to implement the specification, it is expected that the alternative method will include the same general features considered below.

- a. Release rate limit for noble gases:

$$\sum_i [V_i \dot{Q}_{iS} + K_i ((\bar{x}/Q)_V \dot{Q}_{iV})] < 500 \text{ mrem/yr, and}$$

$$\sum_i [(L_i (\bar{x}/Q)_S + 1.1 B_i) \dot{Q}_{iS} + (L_i + 1.1 M_i)((\bar{x}/Q)_V \dot{Q}_{iV})] < 3000 \text{ mrem/yr}$$

where the terms are defined below.

- b. Release rate limit for all radionuclides and radioactive materials in particulate form and radionuclides other than noble gases:

$$\sum_i P_i [W_S \dot{Q}_{iS} + W_V \dot{Q}_{iV}] < 1500 \text{ mrem/yr}$$

where:

- K_i = The total body dose factor due to gamma emissions for each identified noble gas radionuclide, in mrem/yr per $\mu\text{Ci}/\text{m}^3$.
- L_i = The skin dose factor due to beta emissions for each identified noble gas radionuclide, in mrem/yr per $\mu\text{Ci}/\text{m}^3$.
- M_i = The air dose factor due to gamma emissions for each identified noble gas radionuclide, in mrad/yr per $\mu\text{Ci}/\text{m}^3$ (unit conversion constant of 1.1 mrem/mrad converts air dose to skin dose).
- P_i = The dose parameter for radionuclides other than noble gases for the inhalation pathway, in mrem/yr per $\mu\text{Ci}/\text{m}^3$ and for food and ground plane pathways, in $\text{m}^2(\text{mrem/yr per } \mu\text{Ci/sec})$. The dose factors are based on the critical individual organ and most restrictive age group (infant).
- V_i = The constant for each identified noble gas radionuclide accounting for the gamma radiation from the elevated finite plume, derived in accordance with the dose methodology in Regulatory Guide 1.109, Appendix B, Section 1, in mrem/yr per $\mu\text{Ci}/\text{sec}$.
- B_i = The constant for long-term releases (greater than 500 hrs/yr) for each identified noble gas radionuclide accounting for the gamma radiation from the elevated finite plume, derived in accordance with the dose methodology in Regulatory Guide 1.109, Appendix B, Section 1, in mrad/yr per $\mu\text{Ci}/\text{sec}$.

** See footnote on page 21.

- \dot{Q}_{is} = The release rate of radionuclides, i, in gaseous effluents from free-standing stack, in $\mu\text{Ci}/\text{sec}$ (per unit, unless otherwise specified.)
- \dot{Q}_{iV} = The release rate of radionuclides, i, in gaseous effluent from all vent releases, in $\mu\text{Ci}/\text{sec}$ (per unit, unless otherwise specified).
- $(\overline{x/Q})_s$ = _____ sec/m^3 . For free-standing stack releases. The highest calculated annual average relative concentration for any area at or beyond the unrestricted area boundary.
- $(\overline{x/Q})_v$ = _____ sec/m^3 . For all vent releases. The highest calculated annual average relative concentration for any area at or beyond the unrestricted area boundary.
- W_v = The highest calculated annual average dispersion parameter for estimating the dose to an individual at the controlling location due to all vent releases:
- $W_v =$ _____ sec/m^3 , for the inhalation pathway. The location is the unrestricted area boundary in the ____ sector.
- $W_v =$ _____ meters^{-2} , for the food and ground plane pathways. The location is the unrestricted area boundary in the ____ sector.
- W_s = The highest calculated annual average dispersion parameter for estimating the dose to an individual at the controlling location due to free-standing stack releases:
- $W_s =$ _____ sec/m^3 , for the inhalation pathway. The location is the unrestricted area boundary in the ____ sector.
- $W_s =$ _____ meters^{-2} , for the food and ground plane pathways. The location is the unrestricted area boundary in the ____ sector."

SPECIAL NOTES: (1) If there are no free-standing stacks, the factors denoted by the subscript, s, need not be considered. (2) In all cases, the tritium releases use the first W parameter, based on relative concentration (sec/m^3). (3) All radioiodines are assumed to be released in elemental form. If analysis includes the capability of determining elemental and nonelemental forms in all releases, the food pathway parameters may be adjusted accordingly.

The Specification is applicable to the location (unrestricted area boundary or beyond), characterized by the values of the parameters V_i , B_i or $(\overline{x/Q})$ which result in the maximum total body or skin dose commitment. In the event that the analyses indicates a different location for the total body and skin dose limitations, the location selected for consideration shall be that which minimizes the allowable release values.

The factors K_i , L_i , and M_i relate the radionuclide airborne concentrations to various dose rates assuming a semi-infinite cloud. These factors may be taken directly from Table B-1 of the Regulatory Guide 1.109 (Ref. 6), if the values therein are multiplied by 10^6 to convert picocuries $^{-1}$ to microcuries $^{-1}$ as used in the above equations. A tabulation of these factors should be included in the ODCM.

The B_i and V_i factors for the radionuclides are based on the finite plume model of Regulatory Guide 1.109 (Ref. 6). From Equation 6 of Section 6.2 of this Regulatory Guide, B_i can be expressed as:

$$B_i = \frac{K}{r_d} \sum_j \sum_k \sum_l \frac{f_{jk} A_{li} \mu_a E_l I}{u_j} \left(\frac{\text{mrad/yr}}{\mu\text{Ci/sec}} \right)$$

Where:

- I = the results of numerical integration over the plume spatial distribution of the airborne activity as defined by the meteorological condition of wind speed (u_j) and atmospheric stability class k for a particular wind direction.
- K = a numerical constant representing unit conversions, presented below.
- r_d = the distance from the release point to the receptor location, in meters.
- u_j = the mean wind speed assigned to the j th wind speed class, in meters/sec.
- f_{jk} = the joint frequency of occurrence of the j th wind speed class and k th stability class (dimensionless).
- A_{li} = the number of photons of energy corresponding to the l th energy group emitted per transformation of the i th radionuclide, in number/transformation.
- E_l = the energy assigned to the l th energy group, in Mev.
- μ_a = the energy absorption coefficient in air for photon energy E_l , in meters⁻¹.

The constant K follows from Equation 6 of Section C.2.a of Regulatory Guide 1.109 (Ref. 6), as:

$$\begin{aligned} K &= \frac{260 \text{ mrad(radians)}(\text{m}^3)(\text{transformation})}{\text{sec(Mev)}(\text{Ci})} \left(\frac{16 \text{ sectors}}{2\pi \text{ radians}} \right) (10^{-6} \frac{\text{Ci}}{\mu\text{Ci}}) (3.15 \times 10^7 \frac{\text{sec}}{\text{yr}}) \\ &= 2.1 \times 10^4 \text{ mrad} (\text{m}^3)(\text{transformation})/\text{yr}(\text{Mev})(\mu\text{Ci}). \end{aligned}$$

The V_i factor is computed with conversion from air dose to tissue depth dose, thus:

$$V_i = 1.1 \frac{K}{r_d} \sum_j \sum_k \sum_l \frac{f_{jk} A_{li} \mu_a E_l I}{u_j} [e^{-\mu_T T_d}] \left(\frac{\text{mrem/yr}}{\mu\text{Ci/sec}} \right)$$

where:

- μ_T = the tissue energy absorption coefficient for photons of energy E_l , in cm²/gm.
- T_d = the tissue density thickness taken to represent the total body dose (5gm/cm²).
- 1.1 = the ratio of the tissue to air absorption coefficients over the energy range of photons of interest. This ratio converts dose (mrad) to dose equivalent (mrem).

The parameter, P_i , contained in the radiiodine and particulates Specification 3.11.2.1.b includes pathway transport parameters of the i th radionuclide, the receptor's usage of the

pathway media and the dosimetry of the exposure. Pathway usage rates and the internal dosimetry are functions of the receptor's age; however, the youngest age group, the infant, will always receive the maximum dose under the exposure conditions for Specification 3.11.2.1. For the infant exposure, separate values of P_i may be calculated using the PARTS computer program given in Appendix D of this manual for the inhalation pathway which uses a W parameter based on (\bar{X}/Q) , and the food (milk) and ground pathway which uses a W parameter normally based on (\bar{D}/Q) , except for tritium, for application in the ODCM. The following sections provide detail on calculating these P_i values for inclusion in the ODCM.

The values of P_i are independent of vent and stack release elevation. In the case of tritium, (\bar{X}/Q) is the W parameter for the food (milk) pathway as well as the inhalation pathway. As tritium is a weak beta emitter, the ground plane contribution is zero for tritium. (NOTE: The value for the P_i (food) for tritium is 2.4×10^3 mrem/yr per $\mu\text{Ci}/\text{m}^3$.) If the controlling locations for vent and stack releases are different, the controlling location for vent releases should be used in Specification 3.11.2.1.

Omitting the subscripts for vent and stack releases, the dose rate from the *i*th radionuclide (except tritium) is:

$$P_i \text{ (inhalation)} (\bar{X}/Q) Q + [P_i \text{ (food)} + P_i \text{ (ground plane)}] (\bar{D}/Q) Q \text{ (mrem/yr)}$$

and for tritium, is:

$$P_i \text{ (inhalation)} (\bar{X}/Q) Q + P_i \text{ (food)} (\bar{X}/Q) Q = 3.0 \times 10^3 (\bar{X}/Q) Q \text{ (mrem/yr)}$$

5.2.1.1 Calculation of P_i (Inhalation)

$$P_i = K' (BR) DFA_i \text{ (mrem/yr per } \mu\text{Ci}/\text{m}^3)$$

where:

K' = a constant of unit conversion, 10^6 pCi/ μCi .

BR = the breathing rate of the infant age group, in m^3/yr .

DFA_i = the maximum organ inhalation dose factor for the infant age group for the *i*th radionuclide, in mrem/pCi. The total body is considered as an organ in the selection of DFA_i .

The age group considered is the infant group. The infant's breathing rate is taken as $1400 \text{ m}^3/\text{yr}$ from Table E-5 of Regulatory Guide 1.109 (Ref. 6). The inhalation dose factors for the infant, DFA_i are presented in Table E-10 of Regulatory Guide 1.109, in units of mrem/pCi.

Resolution of the units yields:

$$P_i \text{ (inhalation)} = 1.4 \times 10^9 DFA_i$$

5.2.1.2 Calculation of P_i (Ground Plane)

$$P_i = K'K''DFG_i (1-e^{-\lambda_i t})/\lambda_i \quad (\text{m}^2 \cdot \text{mrem}/\text{yr per } \mu\text{Ci}/\text{sec})$$

Where:

K' = a constant of unit conversion, 10^6 pCi/ μ Ci.

K'' = a constant of unit conversion, 8760 hr/year.

λ_i = the decay constant for the i th radionuclide, sec^{-1} .

t = the exposure period, 3.15×10^7 sec (1 year).

DFG_i = the ground plane dose conversion factor for the i th radionuclide (mrem/hr per pCi/ m^2).

The deposition rate onto the ground plane results in a ground plane concentration that is assumed to persist over a year with radiological decay the only operating removal mechanism for each radionuclide. The ground plane dose conversion factors for the i th radionuclide, DFG_i , are presented in Table E-6 of Regulatory Guide 1.109 (Ref. 6), in units of mrem/hr per pCi/ m^2 .

Resolution of the units yields:

$$P_i (\text{Ground}) = 8.76 \times 10^9 DFG_i (1-e^{-\lambda_i t})/\lambda_i.$$

5.2.1.3 Calculation of P_i (Food)

$$P_i = K'r \frac{Q_F(U_{ap})}{Y_p(\lambda_i + \lambda_w)} F_m DFL_i [e^{-\lambda_i t_f}] \quad (\text{m}^2 \cdot \text{mrem}/\text{yr per } \mu\text{Ci}/\text{sec})$$

where:

K' = a constant of unit conversion, 10^6 pCi/ μ Ci.

Q_F = the cow's consumption rate, in kg/day (wet weight).

U_{ap} = the infant's milk consumption rate, in liters/yr.

Y_p = the agricultural productivity by unit area, in kg/ m^2

F_m = the stable element transfer coefficients, in days/liter.

r = fraction of deposited activity retained on cow's feed grass.

DFL_i = the maximum organ ingestion dose factor for the i th radionuclide, in mrem/pCi.

λ_i = the decay constant for the i th radionuclide, in sec^{-1} .

λ_w = the decay constant for removal of activity on leaf and plant surfaces by weathering, 5.73×10^{-7} sec^{-1} (corresponding to a 14 day half-time).

t_f = the transport time from pasture to cow, to milk, to infant, in sec.

A fraction of the airborne deposition is captured by the ground plant vegetation cover. The captured material is removed from the vegetation (grass) by both radiological decay and weathering processes.

The values of Q_F , U_{ap} , and Y_p are provided in Regulatory Guide 1.109 (Ref. 6), Tables E-3, E-5, and E-15, as 50 kg/day, 330 liters/day and 0.7 kg/m², respectively. The value t_f is provided in Regulatory Guide 1.109 (Ref. 6), Table E-15, as 2 days (1.73x10⁵ seconds). The fraction, r , has a value of 1.0 for radioiodines and 0.2 for particulates, as presented in Regulatory Guide 1.109 (Ref. 6), Table E-15.

Table E-1 of Regulatory Guide 1.109 (Ref. 4) provides the stable element transfer coefficients, F_m , and Table E-14 provides the ingestion dose factors, DFL_i , for the infant's organs. The organ with the maximum value of DFL_i is to be used.

Resolution of the units yields:

$$P_i \text{ (food)} = 2.4 \times 10^{10} \frac{r F_m}{\lambda_i + \lambda_w} DFL_i [e^{-\lambda_i t_f}] \text{ (m}^2 \cdot \text{mrem/yr per } \mu\text{Ci/sec)}$$

for all radionuclides, except tritium.

The concentration of tritium in milk is based on its airborne concentration rather than the deposition rate.

$$P_i = K'' K''' F_m Q_F U_{ap} DFL_i [0.75(0.5/H)] \text{ (mrem/yr per } \mu\text{Ci/m}^3)$$

where:

K'' = a constant of unit conversion, 10³ gm/kg.

H = absolute humidity of the atmosphere, in gm/m³.

0.75 = the fraction of total feed that is water.

0.5 = the ratio of the specific activity of the feed grass water to the atmospheric water.

From Table E-1 and E-14 of Regulatory Guide 1.109 (Ref. 6), the values of F_m and DFL_i for tritium are 1.0x10⁻² day/liter and 3.08x10⁻⁷ mrem per pCi, respectively. Assuming an average absolute humidity of 8 grams/meter³, the resolution of units yields:

$$P_i \text{ (food)} = 2.4 \times 10^3 \text{ mrem/yr per } \mu\text{Ci/m}^3$$

for tritium, only.

5.3 DOSE LIMIT FOR IMPLEMENTING 10 CFR PART 50

In preparing proposed Technical Specifications, Figure 5.1-1* should consist of a map of the site area showing the unrestricted area boundary for gaseous effluents as defined in Section 5.2 of this manual. Guidelines for this figure are contained in Regulatory Guide 1.70, Section 2.1.1 (Ref. 21).

*See footnote on page 21.

5.3.1 REQUIREMENTS FOR IMPLEMENTING 10 CFR PART 50

The Standard Technical Specifications 3.11.2.2 and 3.11.2.3 implement 10 CFR Part 50, Appendix I, as follows:

"The air dose in unrestricted areas (see Figure 5.1-1)* due to noble gases released in gaseous effluents shall be limited to the following:

- a. During any calendar quarter, to ≤ 5 mrad for gamma radiation and ≤ 10 mrad for beta radiation;
- b. During any calendar year, to ≤ 10 mrad for gamma radiation and ≤ 20 mrad for beta radiation;

(The dose design objectives may be reduced based on expected public occupancy of areas, e.g., beaches and visitor centers within the unrestricted area boundary. (For PWRs only) the dose design objectives may be reduced based on predicted noble gas releases from the turbine building, if effluent sampling is not provided.)"

"The dose to an individual from radioiodines, radioactive materials in particulate form, and radionuclides other than noble gases with half-lives greater than 8 days in gaseous effluents released to unrestricted areas (see Figure 5.1-1)* shall be limited to the following:

- a. During any calendar quarter to ≤ 7.5 mrem; and
- b. During any calendar year to ≤ 15 mrem."

The ODCM should provide the mathematical relationships used to implement the Specifications. The relationships presented below are acceptable for inclusion in the ODCM. If other methods are selected to implement these Specifications, it is expected that the alternative method will include the same general features considered below.

The air dose in unrestricted area (see Figure 5.1-1)* due to noble gases released in gaseous effluents should be determined by the following expressions:

- a. During any calendar quarter, for gamma radiation:

$$3.17 \times 10^{-8} \sum_i \left[M_i \left[(\bar{x}/Q)_V \tilde{Q}_{iV} + (\bar{x}/q)_V \tilde{q}_{iV} \right] + [B_i \tilde{Q}_{iS} + b_i \tilde{q}_{iS}] \right] \leq 5 \text{ mrad, and}$$

During any calendar quarter, for beta radiation:

$$3.17 \times 10^{-8} \sum_i N_i \left[(\bar{x}/Q)_V \tilde{Q}_{iV} + (\bar{x}/q)_V \tilde{q}_{iV} + (\bar{x}/Q)_S \tilde{Q}_{iS} + (\bar{x}/q)_S \tilde{q}_{iS} \right] \leq 10 \text{ mrad, and}$$

- b. During any calendar year, for gamma radiation:

$$3.17 \times 10^{-8} \sum_i \left[M_i \left[(\bar{x}/Q)_V \tilde{Q}_{iV} + (\bar{x}/q)_V \tilde{q}_{iV} \right] + [B_i \tilde{Q}_{iS} + b_i \tilde{q}_{iS}] \right] \leq 10 \text{ mrad, and}$$

During any calendar year, for beta radiation:

$$3.17 \times 10^{-8} \sum_i N_i \left[(\bar{x}/Q)_V \tilde{Q}_{iV} + (\bar{x}/q)_V \tilde{q}_{iV} + (\bar{x}/Q)_S \tilde{Q}_{iS} + (\bar{x}/q)_S \tilde{q}_{iS} \right] \leq 20 \text{ mrad}$$

where:

M_i = The air dose factor due to gamma emissions for each identified noble gas radionuclide, in mrad/yr per $\mu\text{Ci}/\text{m}^3$.

*See footnote on page 21.

- N_i = The air dose factor due to beta emissions for each identified noble gas radionuclide, in mrad/yr per $\mu\text{Ci}/\text{m}^3$.
- $(\overline{x/Q})_v$ = $\frac{\text{sec}}{\text{m}^3}$. For vent releases. The highest calculated annual average relative concentration for area at or beyond the unrestricted area boundary for long term releases (greater than 500 hrs/year).
- $(\overline{x/q})_v$ = $\frac{\text{sec}}{\text{m}^3}$. For vent releases. The relative concentration for areas at or beyond the unrestricted area boundary for short term releases (equal to or less than 500 hrs/year).
- $(\overline{x/Q})_s$ = $\frac{\text{sec}}{\text{m}^3}$. For free-standing stack releases. The highest calculated annual average relative concentration for areas at or beyond the unrestricted area boundary for long term releases (greater than 500 hrs/year).
- $(\overline{x/q})_s$ = $\frac{\text{sec}}{\text{m}^3}$. For free-standing stack releases. The relative concentration for areas at or beyond the unrestricted area boundary for short term releases (equal to or less than 500 hrs/year).
- \tilde{q}_{is} = The average release of noble gas radionuclides in gaseous effluents, i, for short term releases (equal to or less than 500 hrs/year) from the free-standing stack, in μCi . Releases shall be cumulative over the calendar quarter or year as appropriate.
- \tilde{q}_{iv} = The average release of noble gas radionuclides in gaseous effluents, i, for short term releases (equal to or less than 500 hrs/year) from all vents, in μCi . Releases shall be cumulative over the calendar quarter or year as appropriate.
- \tilde{Q}_{is} = The average release of noble gas radionuclides in gaseous releases, i, for long term releases (greater than 500 hrs/year) from the free-standing stack, in μCi . Release shall be cumulative over the calendar quarter or year as appropriate.
- \tilde{Q}_{iv} = The average release of noble gas radionuclides in gaseous effluents, i, for long term releases (greater than 500 hrs/yr) from all vents, in μCi . Releases shall be cumulative over the calendar quarter or year as appropriate.
- B_i = The constant for long term releases (greater than 500 hrs/yr) for each identified noble gas radionuclide accounting for the gamma radiation from the elevated finite plume, derived in accordance with the dose methodology in Regulatory Guide 1.109, Appendix B, Section 1, in mrad/yr per $\mu\text{Ci}/\text{sec}$.
- b_i = The constant for short term releases (equal to or less than 500 hrs/yr) for each identified noble gas radionuclide accounting for the gamma radiation from the elevated finite plume, derived in accordance with the dose methodology in Regulatory Guide 1.109, Appendix B, Section 1, in mrad/yr per $\mu\text{Ci}/\text{sec}$.

3.17×10^{-8} = The inverse of the number of seconds in a year.

The dose to an individual from radioiodines, radioactive materials in particulate form, and radionuclides other than noble gases with half-lives greater than 8 days in gaseous effluents released to unrestricted areas (see Figure 5.1-1)* should be determined by the following expressions:

a. During any calendar quarter:

$$3.17 \times 10^{-8} \sum_i R_i [w_s \tilde{Q}_{is} + w_s \tilde{q}_{is} + w_v \tilde{Q}_{iv} + w_v \tilde{q}_{iv}] \leq 7.5 \text{ mrem, and}$$

*See footnote on page 21.

b. During any calendar year:

$$3.17 \times 10^{-8} \sum_i R_i [W_s \tilde{Q}_{is} + w_s \tilde{q}_{is} + W_v \tilde{Q}_{iv} + w_v \tilde{q}_{iv}] \leq 15 \text{ mrem}$$

where:

\tilde{Q}_i = The releases of radionuclides, radioactive materials in particulate form, and radionuclides other than noble gases in gaseous effluents, i, for long term releases greater than 500 hrs/yr, in μCi . Releases shall be cumulative over the calendar quarter or year as appropriate.

\tilde{q}_i = The releases of radionuclides, radioactive materials in particulate form and radionuclides other than noble gases in gaseous effluents, i, for short term releases equal to or less than 500 hrs/yr, in μCi . Releases shall be cumulative over the calendar quarter or year as appropriate.

W = The dispersion parameter for estimating the dose to an individual at the controlling location for long term releases (greater than 500 hrs/yr):

$$W = (\overline{\chi/Q}) \text{ for the inhalation pathway, in sec/m}^3.$$

$$W = (\overline{D/Q}) \text{ for the food and ground plane pathways in meters}^{-2}.$$

w = The dispersion parameter for estimating the dose to an individual at the controlling location for short term releases (equal to or less than 500 hrs/yr):

$$w = (\overline{\chi/q}) \text{ for the inhalation pathway in sec/m}^3.$$

$$w = (\overline{D/q}) \text{ for the food and ground plane pathway in meters}^{-2}.$$

3.17×10^{-8} = The inverse of the number of seconds in a year.

R_i = The dose factor for each identified radionuclide, i, in $\text{m}^2(\text{mrem/yr})$ per $\mu\text{Ci/sec}$ or mrem/yr per $\mu\text{Ci/m}^3$.

For the direction sectors with existing pathways within 5 miles from the unit, use the values of R_i for these pathways. If no real pathway exists within 5 miles from the center of the building complex, use the cow-milk R_i assuming that this pathway exists at the 4.5 to 5.0 mile distance in the worst sector. If the R_i for an existing pathway within 5 miles is less than a cow-milk R_i at 4.5 to 5.0 miles, then use the value of the cow-milk R_i at 4.5 to 5.0 miles. The pathway values used for calculating dose contributions shall be consistent with the results of the land use census performed pursuant to Specification 3.12.2. The controlling value of R_i for each radionuclide shall be determined and provided in tabular form in the ODCM. The parameters W and w shall correspond to the applicable pathway location.

SPECIAL NOTES: (1) If there is no free-standing stack, the factors denoted by the subscript, s, need not be considered. (2) In all cases, the tritium releases use the first W or w parameter, based on relative concentration (sec/m^3). (3) All radioiodines are assumed to be released in elemental form. If analysis includes the capability of determining the elemental and non-elemental forms in all releases, the food pathway parameters may be adjusted accordingly.

The following information is provided to further clarify the application of these Specifications and provide more information regarding the individual factors. The ODCM should include a detailed presentation of the calculational model and a complete tabulation of all values assigned to each parameter.

The noble gas Specification 3.11.2.2 is to be evaluated at the location in the unrestricted area where analyses of annual average air doses were found to be maximum. In the event that the analyses indicate different locations for the beta and gamma limitations, the location selected for consideration shall be that which minimizes the allowable release values due to gamma radiation.

The radioiodine and particulate Specification 3.11.2.3 is applicable to the location in the unrestricted area where the combination of existing pathways and receptor age groups indicates the maximum potential exposures. The inhalation and ground plane exposure pathways shall be considered to exist at all locations. The grass-cow-milk, grass-cow-meat, and vegetation pathways are considered based on their existence at the various locations. Of the various age groups, the infant or child receives the largest dose; thus, only these two age groups will be discussed. It is the intent, however, that a licensee undertake annual surveys of the age groups and the land use at the various locations about the site, reports these results according to Specification 3.12.2 and determines the applicable parameters of R_i for tabulation in the ODCM. The new parameters shall be submitted to the NRC for review and approval prior to implementation.

The M_i and N_i factors of the noble gas relationship relate the airborne concentration of the noble gas to the air dose rates assuming a semi-infinite cloud. These factors may be taken directly from Table B-1 of the Regulatory Guide 1.109 (Ref. 6), and the values therein have been multiplied by 10^6 to convert picocuries⁻¹ to microcuries⁻¹.

The factor, B_i , is defined in Section 5.2.1 of this manual. The corresponding short-term factor, b_i is computed following the same procedure replacing the meteorological variables, j , k , and l , for long-term releases with variables for short-term releases using the methodology provided in Regulatory Guide 1.111 (Ref. 8), NUREG-0324 (Ref. 13), and NUREG-75/087 (Ref. 23). Such information should be provided in tabular form in the ODCM.

In developing the R_i values, separate expressions are written for each of the potential pathways. These expressions are similar to those developed in Section 5.2.1 of this manual for P_i , and are denoted by $R_i^G[D/Q]$, $R_i^I[X/Q]$, $R_i^C[D/Q]$, $R_i^M[D/Q]$ and $R_i^V[D/Q]$, where the superscripts G, I, C, M, and V refer to ground plane, inhalation, cow's milk, meat and vegetation, respectively. The 'argument' notation, [], indicates the appropriate dispersion parameter, W , to be applied with the R_i factor. Note that the argument is not included in the following expressions. In the case of tritium, the dispersion parameter, W , is always taken as (\bar{x}/Q) . The R_i parameter is independent of long-term or short-term releases and should be provided in tabular form in the ODCM.

5.3.1.1 Inhalation Pathway Factor, $R_i^I[X/Q]$

$$R_i^I[X/Q] = K'(BR)_a (DFA_i)_a \text{ (mrem/yr per } \mu\text{Ci/m}^3\text{)}$$

where:

$$K' = \text{a constant of unit conversion, } 10^6 \text{ pCi}/\mu\text{Ci}.$$

$(BR)_a$ = the breathing rate of the receptor of age group (a), in m^3/yr .

$(DFA_i)_a$ = the maximum organ inhalation dose factor for the receptor of age group (a) for the i th radionuclide, in $mrem/pCi$. The total body is considered as an organ in the selection of $(DFA_i)_a$.

The breathing rates $(BR)_a$ for the various age groups are tabulated below, as given in Table E-5 of the Regulatory Guide 1.109 (Ref. 6).

<u>Age Group (a)</u>	<u>Breathing Rate (m^3/yr)</u>
Infant	1400
Child	3700
Teen	8000
Adult	8000

Inhalation dose factors $(DFA_i)_a$ for the various age groups are given in Tables E-7 through E-10 of Regulatory Guide 1.109 (Ref. 6).

5.3.1.2 Ground Plane Pathway Factor, R_i^G [D/Q]

$$R_i^G[D/Q] = K'K''(SF)DFG_i[(1-e^{-\lambda_i t})/\lambda_i] \text{ (m}^2 \cdot \text{mrem/yr per } \mu\text{Ci/sec)}$$

where:

K' = a constant of unit conversion, 10^6 $pCi/\mu Ci$.

K'' = a constant of unit conversion, 8760 $hr/year$.

λ_i = the decay constant for the i th radionuclide, sec^{-1} .

t = the exposure time, 4.73×10^8 sec (15 years).

DFG_i = the ground plane dose conversion factor for the i th radionuclide ($mrem/hr$ per pCi/m^2).

SF = the shielding factor (dimensionless).

A shielding factor of 0.7 is suggested in Table E-15 of Regulatory Guide 1.109 (Ref. 6). A tabulation of DFG_i values is presented in Table E-6 of Regulatory Guide 1.109 (Ref. 6).

5.3.1.3 Grass-Cow-Milk Pathway Factor, R_i^C [D/Q]

$$R_i^C[D/Q] = K' \frac{Q_F(U_{ap})}{\lambda_i + \lambda_w} F_m(r)(DFL_i)_a \left[\frac{f_p f_s}{Y_p} + \frac{(1-f_p f_s)e^{-\lambda_i t_h}}{Y_s} \right] e^{-\lambda_i t_f}$$

$(m^2 \cdot mrem/yr \text{ per } \mu\text{Ci/sec})$

where:

- K' = a constant of unit conversion, 10^6 pCi/ μ Ci.
 Q_F = the cow's consumption rate, in kg/day (wet weight).
 U_{ap} = the receptor's milk consumption rate for age (a), in liters/yr.
 Y_p = the agricultural productivity by unit area of pasture feed grass, in kg/m².
 Y_s = the agricultural productivity by unit area of stored feed, in kg/m².
 F_m = the stable element transfer coefficients, in days/liter.
 r = fraction of deposited activity retained on cow's feed grass.
 $(DFL_i)_a$ = the maximum organ ingestion dose factor for the *i*th radionuclide for the receptor in age group (a), in mrem/pCi.
 λ_i = the decay constant for the *i*th radionuclide, in sec⁻¹.
 λ_w = the decay constant for removal of activity on leaf and plant surfaces by weathering, 5.73×10^{-7} sec⁻¹ (corresponding to a 14 day half-life).
 t_f = the transport time from pasture to cow, to milk, to receptor, in sec.
 t_h = the transport time from pasture, to harvest, to cow, to milk, to receptor, in sec.
 f_p = fraction of the year that the cow is on pasture (dimensionless).
 f_s = fraction of the cow feed that is pasture grass while the cow is on pasture (dimensionless).

SPECIAL NOTE: The above equation is applicable in the case that the milk animal is a goat.

Milk cattle are considered to be fed from two potential sources, pasture grass and stored feeds. Following the development in Regulatory Guide 1.109 (Ref. 6), the values of f_p and f_s will be considered unity, in lieu of site specific information provided in the annual land census report by the licensee.

Tabulated below are the appropriate parameter values and their reference to Regulatory Guide 1.109 (Ref. 6). In the case that the milk animal is a goat, rather than a cow, refer to Regulatory Guide 1.109 for the appropriate parameter values.

<u>Parameter</u>	<u>Value</u>	<u>Table (Ref. 6)</u>
r (dimensionless)	1.0 for radiiodine	E-15
	0.2 for particulates	E-15
F_m (days/liter)	Each stable element	E-1
U_{ap} (liters/yr) - Infant	330	E-5
- Child	330	E-5
- Teen	400	E-5
- Adult	310	E-5
$(DFL_i)_a$ (mrem/pCi)	Each radionuclide	E-11 to E-14
Y_p (kg/m ²)	0.7	E-15
Y_s (kg/m ²)	2.0	E-15
t_f (seconds)	1.73×10^5 (2 days)	E-15
t_h (seconds)	7.78×10^6 (90 days)	E-15
Q_F (kg/day)	50	E-3

The concentration of tritium in milk is based on the airborne concentration rather than the deposition. Therefore, the R_i^C is based on $[X/Q]$:

$$R_i^C[X/Q] = K'K''F_m Q_{ap} U_{ap} (DFL_i)_a [0.75(0.5/H)] \text{ (mrem/yr per } \mu\text{Ci/m}^3\text{)}$$

where:

K'' = a constant of unit conversion, 10^3 gm/kg.

H = absolute humidity of the atmosphere, in gm/m^3 .

0.75 = the fraction of total feed that is water.

0.5 = the ratio of the specific activity of the feed grass water to the atmospheric water.

and other parameters and values are given above. The value of H may be considered as 8 grams/meter³, in lieu of site specific information (Ref. 6).

5.3.1.4 Grass-Cow-Meat Pathway Factor, $R_i^M[D/Q]$

The integrated concentration in meat follows in a similar manner to the development for the milk pathway, therefore:

$$R_i^M[D/Q] = K' \frac{Q_F(U_{ap})}{\lambda_i + \lambda_w} F_f(r)(DFL_i)_a \left[\frac{f_p f_s}{Y_p} + \frac{(1-f_p f_s)e^{-\lambda_i t_h}}{Y_s} \right] e^{-\lambda_i t_f}$$

($\text{m}^2 \cdot \text{mrem/yr per } \mu\text{Ci/sec}$)

where:

F_f = the stable element transfer coefficients, in days/kg.

U_{ap} = the receptor's meat consumption rate for age (a), in kg/yr.

t_f = the transport time from pasture to receptor, in sec.

t_h = the transport time from crop field to receptor, in sec.

Tabulated below are the appropriate parameter values and their reference to Regulatory Guide 1.109 (Ref. 6).

<u>Parameter</u>	<u>Value</u>	<u>Table (Ref. 6)</u>
r (dimensionless)	1.0 for radioiodine 0.2 for particulates	E-15 E-15
F_f (days/kg)	Each stable element	E-1
U_{ap} (kg/yr) - Infant	0	E-5
- Child	41	E-5
- Teen	65	E-5
- Adult	110	E-5
$(DFL_i)_a$ (mrem/pCi)	Each radionuclide	E-11 to E-14
Y_p (kg/m ²)	0.7	E-15

Parameter	Value	Table (Ref. 6)
γ_s (kg/m ²)	2.0	E-15
t_f (seconds)	1.73×10^6 (20 days)	E-15
t_h (seconds)	7.78×10^6 (90 days)	E-15
Q_F (kg/day)	50	E-3

The concentration of tritium in meat is based on its airborne concentration rather than the deposition. Therefore, the R_i^M is based on $[X/Q]$:

$$R_i^M[X/Q] = K' K'' F_f Q_F U_{ap} (DFL_i)_a [0.75(0.5/H)] \text{ (mrem/yr per } \mu\text{Ci/m}^3\text{)}$$

where all terms are defined above and Section 5.3.1.3 of this manual.

5.3.1.5 Vegetation Pathway Factor, $R_i^V[D/Q]$

The integrated concentration in vegetation consumed by man follows the expression developed in the derivation of the milk factor. Man is considered to consume two types of vegetation (fresh and stored) that differ only in the time period between harvest and consumption, therefore:

$$R_i^V[D/Q] = K' \left[\frac{(r)}{\gamma_v(\lambda_i + \lambda_w)} \right] (DFL_i)_a \left[U_a^L f_L e^{-\lambda_i t_L} + U_a^S f_g e^{-\lambda_i t_h} \right]$$

(m²·mrem/yr per $\mu\text{Ci/sec}$)

where:

K' = a constant of unit conversion, 10^6 pCi/ μCi .

U_a^L = the consumption rate of fresh leafy vegetation by the receptor in age group (a), in kg/yr.

U_a^S = the consumption rate of stored vegetation by the receptor in age group (a), in kg/yr.

f_L = the fraction of the annual intake of fresh leafy vegetation grown locally.

f_g = the fraction of the annual intake of stored vegetation grown locally.

t_L = the average time between harvest of leafy vegetation and its consumption, in seconds.

t_h = the average time between harvest of stored vegetation and its consumption, in seconds.

γ_v = the vegetation areal density, in kg/m².

and all other factors are defined in Section 5.3.1.3 of this manual.

Tabulated below are the appropriate parameter values and their reference to Regulatory Guide 1.109 (Ref. 6).

<u>Parameter</u>	<u>Value</u>	<u>Table (Ref. 6)</u>
r (dimensionless)	1.0 for radioiodines 0.2 for particulates	E-1 E-1
$(DFL_i)_a$ (mrem/pCi)	Each radionuclide	E-11 to E-14
U_a^L (kg/yr) - Infant	0	E-5
- Child	26	E-5
- Teen	42	E-5
- Adult	64	E-5
U_a^S (kg/yr) - Infant	0	E-5
- Child	520	E-5
- Teen	630	E-5
- Adult	520	E-5
f_L (dimensionless)	site specific (default = 1.0)	
f_g (dimensionless)	site specific (default = 0.76) (see Ref. 6, page 28)	
t_L (seconds)	8.6×10^4 (1 day)	E-15
t_h (seconds)	5.18×10^6 (60 days)	E-15
Y_v (kg/m ²)	2.0	E-15

The concentration of tritium in vegetation is based on the airborne concentration rather than the deposition. Therefore, the R_i^V is based on $[X/Q]$:

$$R_i^V[X/Q] = K'K'' U_a^L f_L + U_a^S f_g (DFL_i)_a [0.75(0.5/H)] \text{ (mrem/yr per } \mu\text{Ci/m}^3\text{)}.$$

where all terms have been defined above and in Section 5.3.1.3 of this manual.

The staff has developed a computer code PARTS for calculating the R_i parameters, which is described in Appendix D of this manual.

5.4 SPECIFICATION ON THE USE OF GASEOUS RADIOACTIVE WASTE MANAGEMENT SYSTEM

Standard Technical Specification 3.11.2.4 specifies that:

"The gaseous radwaste treatment system shall be OPERABLE. The appropriate subsystems shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected gaseous effluent releases from all release points to unrestricted areas (see Figure 5.1-1)* would result in a dose in any period of 31 days that exceeds 0.2 mrad for gamma radiation, 0.4 mrad for beta radiation, or 0.3 mrem to any organ for that same 31 day period."

The operability of the gaseous radioactive waste management system ensures that this system will be available for use whenever gaseous effluents require treatment prior to release to the environment. The term "gaseous radwaste treatment system" includes all of the installed and available gaseous radioactive waste management system equipment, as well as their controls, power, instrumentation and services that make the system functional. Equipment that is considered standby or redundant is also included, since the function is to assure operability. The action also permits alternate treatment paths using alternate subsystems and equipment to be used in the event that the normal treatment is inoperable.

*See footnote on page 21.

This Specification provides impetus to maintain and use the gaseous radioactive waste management system as required in 10 CFR 50.36a. Maintenance and use of the gaseous radioactive waste management system components are discussed in Section 2.2 of this manual. Since features; such as, free-standing stacks or elevated vents are not considered to be treatment systems, the NRC staff considers that BWR offgas systems must be used without bypassing normal treatment during reactor operation.

To determine if use of the installed equipment, other than in BWR offgas systems, is necessary, the licensee should calculate the expected dose to an individual in the unrestricted area by projecting the plant's cumulative gaseous effluent release over a 31-day period. These releases should include all potentially radioactive plant gaseous effluents from all gaseous radioactive waste management systems and ventilation exhaust treatment systems. Calculations should include a margin, based on operating data, for anticipated operational occurrences and should use the dose calculation models discussed in Section 5.3 of this manual. The dose impact from the projected 31-day release should be compared to 0.2 mrad for gamma radiation, 0.4 mrad for beta radiation, or 0.3 mrem to any organ. If the projection indicates these values will be exceeded, then installed radwaste treatment system components, which are capable of reducing the quantities or concentrations of radioactive materials in gaseous effluents and which are capable of reducing the projected impact to less than the values specified above, must be used. The values for the projected impact, given above, corresponds to approximately one forty-eighth of the annual design dose objective values of Appendix I, Section II.B and II.C of 10 CFR Part 50 in a month, and if continued for a year, these values would correspond to less than one-fourth the values limited by Specifications 3.11.2.2.b and 3.11.2.3.b. The calculation of projected cumulative dose impact that could result from the proposed operation may use the methodology provided in Section 5.3 of this manual.

5.5 SPECIFICATION ON EXPLOSIVE GAS MIXTURE LIMITATION

The Standard Technical Specifications for BWRs and PWRs contain Specification 3.11.2.6A and alternate Specification 3.11.2.6B for the limiting conditions for operation for systems designed to treat and store radioactive gases which also contain quantities of uncombined hydrogen and oxygen. Specification 3.11.2.6A applies to a system designed to withstand a hydrogen explosion. If all components of the system, from containment to the release point, are designed and tested to 20 times the normal operating pressure, the system is considered to be designed to withstand a hydrogen explosion. Alternate Specification 3.11.2.6B applies to a system not considered to be designed to withstand a hydrogen explosion.

The functional name, "waste gas holdup system," has been used in this manual to include the various system designs found in BWRs and PWRs which serve the same basic purpose, i.e., to remove radioactive waste gases from the reactor coolant, to treat and hold gases for radioactive decay, and to monitor and control the radioactive materials in the gaseous waste prior to final release.

The potentially explosive components of the waste gas holdup system may be effectively inerted by nitrogen or steam, treated and re-used in the plant or stored and released after delay. The treatment may involve hydrogen-oxygen recombiners, filters, holdup tanks, decay

pipes, charcoal adsorbers, and cryogenic stills. The Specification is provided to ensure that the concentrations of potentially explosive gases contained in the system are maintained outside the explosive envelope for hydrogen and oxygen (i.e., less than 4% H₂ by volume or less than 4% O₂ by volume). The alternate Specification 3.11.2.6B provides an additional setpoint limitation to ensure that the automatic dilution, inerting or recombiner control is functioning to maintain the relative concentration of components of potentially explosive gas mixtures outside one-half the above flammability limits (i.e., 2% H₂ and/or O₂). Based on the design, the licensee should specify the gas to be measured: hydrogen, oxygen or both hydrogen and oxygen.

5.6 SPECIFICATIONS UNIQUE TO LWR DESIGN FEATURES

The Standard Technical Specifications contain several Specifications unique to certain design features of PWRs and BWRs; in general, these Specifications contain limiting conditions for operation. The following Sections describe these limitations and the method for determining the limiting values.

5.6.1 PWR Gas Storage Tank Specification 3.11.2.7

Specification 3.11.2.7 requires that the quantity of radioactive gas in each gas storage tank at a PWR be limited to a predetermined curie content. It is not applicable to PWRs that use adsorption units for gas holdup, but is applicable for compressed gas storage and for cryogenic storage systems. The purpose of this Specification is to assure that, in the event of an uncontrolled release of the tank contents, the resulting total body exposure to an individual at the nearest exclusion area boundary will not exceed 0.5 rem.

Determination of the curie limit should consider the following expression;

$$\sum Q_{iT} \leq \frac{500 \text{ mrem} (3.15 \times 10^7 \text{ sec/year})}{10^6 \mu\text{Ci/Ci} \sum_i K_i (\bar{x}/Q)_{\text{DBA}} (\text{mrem}\cdot\text{sec}/\mu\text{Ci}\cdot\text{yr})}$$

where:

$\sum Q_{iT}$ = The sum quantity of all noble gas nuclides (i) in a gas storage tank based on a gas mixture resulting from gaseous wastes, in curies.

K_i = The total body dose factor due to gamma emissions for each identified noble gas radionuclide, in mrem/yr per $\mu\text{Ci}/\text{m}^3$. (See Section 5.2.1 of this manual.)

$(\bar{x}/Q)_{\text{DBA}}$ = The relative concentration at the exclusion area boundary used for evaluation of design basis accidents for ground release conditions, in sec/m^3 . Guidelines are provided in Standard Review Plan 2.3.4 (Ref. 23).

Normally the major radioactive nuclide constituent in PWR waste gas storage tanks is Xe-133. Radiation monitoring and sampling of these tanks should consider the Xe-133 (equivalent) concentration. Plant procedures shall not permit operation with communication between tanks.

Alternate Specification 3.11.2.7, "PWR Waste Gas Processing System," should be used for plants that use adsorption units for gas holdup prior to release. This Specification requires that the gross radioactivity in noble gases removed from the waste gas system by means of steam jet air ejectors (or other devices) and as measured prior to entering the adsorption systems at PWR plants shall be limited by a release rate alarm setpoint with indication in the main control room. The purpose of this pretreatment continuous radiation monitor setpoint is to provide reasonable assurance that the potential accident total body dose to an individual at the exclusion area boundary will not exceed a small fraction of the limits specified in 10 CFR Part 100 in the event this effluent is inadvertently discharged directly to the environment without treatment. Guidelines for determining the release rate limit are provided in Standard Review Plan 15.7.1 (Ref. 24), and guidance on the radiation monitoring instrumentation is given in Regulatory Guide 1.97 (Ref. 25).

5.6.2 BWR Main Condenser Evacuation System Specification 3.11.2.7

This Specification requires that the gross radioactivity in noble gases removed from the main condenser by means of steam jet air ejectors (or other devices) and as measured prior to entering the treatment, adsorption and delay systems at BWR plants shall be limited by a release rate alarm setpoint with indication in the main control room. The purpose of this pretreatment continuous radiation monitor setpoint is to provide reasonable assurance that the potential total body accident dose to an individual at the exclusion area boundary will not exceed a small fraction of the limits specified in 10 CFR Part 100 in the event this effluent including the radioactivity accumulated in the treatment system is inadvertently discharged directly to the environment without treatment.

The method for determining the specified rate limit, in $\mu\text{Ci}/\text{sec}$, should use the following assumptions:

1. The release of radioactive material from the fuel is postulated to have an isotopic composition of noble gases determined from noble gas source term distribution for a 3500 Mwt reactor. These values should be scaled linearly for reactors of higher and lower powers.
2. The assumptions related to the release of radioactive material from the system following the accident are:
 - a. For systems which are not fully detonation-resistant or for those systems which are equipped with rupture discs that have not been isolated or loop seals which do not vent back to the main condenser, release occurs from a break just downstream of the main condenser evacuation system. Release from the main condenser evacuation system is assumed to be at ground level and a delay of five minutes is assumed to account for radioactive decay during transit from the release point to the exclusion area boundary.

NOBLE GAS SOURCE TERM

Isotope	Approx. Half-Life	Source Term, $\mu\text{Ci}/\text{sec}$, at the main condenser evacuation system (SJAE) 0 Decay
Xe-140	13.7 s	1.1×10^6
Kr-90	33 s	9.8×10^5
Xe-139	41.0 s	9.8×10^5
Kr-89	3.2 m	4.6×10^5
Xe-137	3.8 m	5.3×10^5
Xe-138	14.0 m	3.1×10^5
Xe-135m	15.6 m	9.1×10^4
Kr-87	76 m	7.0×10^4
Kr-83m	1.86 hr	1.2×10^4
Kr-88	2.8 hr	7.0×10^4
Kr-85m	4.4 hr	1.1×10^4
Xe-135	9.2 hr	7.7×10^4
Xe-133m	2.3 d	1.0×10^3
Xe-133	5.27 d	2.9×10^4
Xe-131m	11.9 d	5.2×10^1
Kr-85	10.76 y	7.0×10^1
	Total	$\sim 4.7 \times 10^6$

- b. For systems which are detonation-resistant (i.e., rupture discs have been isolated and loop seals are vented back to the main condenser), release from the main condenser evacuation system exits via the normal release point.
- c. Activity release into the system continues for one hour following the accident at the Technical Specification limit unless positive means (such as automatic isolation) are provided to limit the releases from this source.
- d. Radioiodines and activation gases may be ignored.
- e. No deposition during downwind transport occurs.
- f. The total radioactive inventory (neglecting radioiodines and activation gases) in any delay lines and from the process equipment is released within a two-hour period with no decay.
- g. The total noble gas content of all charcoal delay beds is released over a period of two hours. A fractional release of the particulate inventory on the charcoal beds should be assumed. The rate of release is equal to the rate of absorption using the information contained in NUREG-0016 (Ref. 12).
- h. The main condenser air inleakage is 6 scfm for three shell main condensers.

3. The relative concentration, $(\bar{x}/Q)_{DBA}$, to be used is described in Section 5.6.1 of this manual.

Based on these assumptions, the applicant should backcalculate from a whole body dose of 2.5 rem to an individual at the exclusion area boundary to obtain the main condenser evacuation system rate limit, in $\mu\text{Ci}/\text{sec}$, having a noble gas isotopic distribution proportional to the noble gas source term, given above. For licensed BWR facilities that have a Technical Specification limit, in $\mu\text{Ci}/\text{sec}$, based on a 5 rem consequence criteria, a reevaluation of the specified limit is not required. Guidance on the radiation monitoring instrumentation is given in Regulatory Guide 1.97 (Ref. 25).

5.6.3 PWR Monitoring of Steam Generator Blowdown Flash Tank Vent

Standard Technical Specification 3.3.3.9, including Tables 3.3-12* and 4.3-12,* requires that PWRs continuously monitor the steam generator blowdown tank vent for gross noble gas radioactivity and continuously sample for radioactive iodines and particulates. Many PWRs with U-tube steam generators direct their blowdown to a blowdown treatment system without venting, so that item f in Tables 3.3-12 and 4.3-12 is not applicable. Others vent their flash tank to the main condenser, where the airborne radionuclides are either removed by condensing steam or drawn into the main condenser evacuation system, where they are then monitored prior to release to the atmosphere; therefore, this design provision makes item f not applicable. However, there are several operating PWR's that direct their blowdown to a flash tank which is vented directly to the atmosphere. Monitoring these releases presents serious difficulties, due to the presence of steam in the exhaust. In lieu of a flash tank vent radiation monitor, a determination of the release of radioiodine-131 via the flash tank vent can be made by calculating from a measured concentration in the secondary water by the following equation:

$$\dot{Q}_y = \bar{C}_y [R_{SGB}] f_{FT} (1 - SQ_{FTV})$$

where:

\dot{Q}_y = The release rate of radioiodine-131, y, from the steam generator flash tank vent, in $\mu\text{Ci}/\text{sec}$.

\bar{C}_y = The concentration of radioiodine-131, y, in the secondary coolant water averaged over not more than one week, in $\mu\text{Ci}/\text{ml}$.

R_{SGB} = The steam generator blowdown rate to the flash tank, in ml/sec .

f_{FT} = The fraction of blowdown flashed in the flash tank determined from a heat balance taken around the flash tank at the applicable reactor power level.

SQ_{FTV} = The measured steam quality in the flash tank vent; or an assumed value of 0.85, based on NUREG-0017 (Ref. 11).

If this option is chosen, the applicant shall perform this calculation every time measurements of secondary water radioiodine concentrations are required by Technical Specifications, and the calculated release shall be assumed at this calculated level until the next secondary water analysis is completed. These calculations shall be provided by the applicant in his semiannual effluent release report.

*See footnote on page 20.

REFERENCES

1. Title 10, "Energy," Chapter I, Code of Federal Regulations; Part 50, pages 250 to 327, U.S. Government Printing Office, Washington, D.C. 20402, January 1, 1977.
2. U.S. Nuclear Regulatory Commission, "Draft Radiological Effluent Technical Specifications for PWR's," USNRC NUREG-0472, Revision 1, Washington, D.C. 20555, October 1978.
3. U.S. Nuclear Regulatory Commission, "Draft Radiological Effluent Technical Specifications for PWR's," USNRC NUREG-0473, Revision 1, Washington, D.C. 20555, October 1978.
4. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications," Revision 4, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, August 1975.
5. Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations," Revision 3, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, May 1977.
6. Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," Revision 1, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, October 1977.
7. Regulatory Guide 1.110, "Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors," U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, March 1976.
8. Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors," Revision 1, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, July 1977.
9. Regulatory Guide 1.112, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Power Reactors," U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, April 1976.
10. Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I," Revision 1, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, April 1977.
11. U.S. Nuclear Regulatory Commission, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Pressurized Water Reactors (PWR-GALE Code)," USNRC Report NUREG-0017, Washington, D.C. 20555, April 1976.
12. U.S. Nuclear Regulatory Commission, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Boiling Water Reactors (BWR-GALE Code)," USNRC Report NUREG-0016, Washington, D.C. 20555, April 1976.
13. U.S. Nuclear Regulatory Commission, "XOQDOQ, Program for the Meteorological Evaluation of Routine Effluent Releases at Nuclear Power Stations," USNRC Report NUREG-0324, Washington, D.C. 20555, September 1977.
14. Title 10, "Energy," Chapter I, Code of Federal Regulations; Part 20, pages 144 to 172, U.S. Government Printing Office, Washington, D.C. 20402, January 1, 1977.
15. Title 10, "Energy," Chapter I, Code of Federal Regulations; Part 100, pages 409 to 421, U.S. Government Printing Office, Washington, D.C. 20402, January 1, 1977.
16. Title 40, "Protection of Environment," Chapter I, Code of Federal Regulations, Part 141, pages 169 to 182, U.S. Government Printing Office, Washington, D.C. 20402, January 1, 1977.
17. Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, June 1974.

18. Title 40, "Protection of Environment," Chapter I, Code of Federal Regulations, Part 190, Federal Register, Vol. 42, No. 9, pages 2858 to 2861, Washington, D.C. 20402, January 13, 1977.
19. "SKYSHINE, A Computer Program for Evaluating Effects of Structure Design on N-16 Gamma-Ray Dose Rates," Radiation Research Associates, Inc., Report RRA-T7209, Fort Worth, Texas, 76107, November 1972.
20. Regulatory Guide 4.15, "Quality Assurance for Radiological Monitoring Programs (Normal Operation) - Effluent Streams and the Environment," U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, December 1977.
21. Regulatory Guide 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants, LWR Edition," Revision 2, USNRC Report NUREG-75/094, Washington, D.C. 20555, September 1975.
22. International Commission on Radiological Protection, Report of ICRP Committee II on Permissible Dose for Internal Radiation, ICRP Publication 2, Pergamon Press, New York 10022, 1959.
23. U.S. Nuclear Regulatory Commission, "Short Term Diffusion Estimates," Section 2.3.4, Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants - LWR Edition, USNRC Report NUREG-75/087, Washington, D.C. 20555, November 1975.
24. U.S. Nuclear Regulatory Commission, "Waste Gas System Failure," Section 15.7.1 (Draft Revision) Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants - LWR Edition, USNRC Report NUREG-75/087, Washington, D.C. 20555, 1978.
25. Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," Revision 1, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, August 1977.

APPENDIX A

RATAFR CODE FOR PRESSURIZED WATER REACTORS Input Cards and Sample Calculation

1. Parameters Required for the PWR-RATAFR Code

Complete the cards from information given in the Applicant's Safety Analysis and Environmental Reports and in Section 2 of this Appendix.

a. CARD 1: Name of Reactor

Enter in spaces (33-60) the name of the reactor.

b. CARD 2: Thermal Power Level

Enter in spaces (73-80) the maximum thermal power level (MWT) evaluated for safety considerations in the Safety Analysis Report.

c. CARD 3: Mass of Coolant in Primary System

Enter in spaces (73-80) the mass of coolant (10^3 lbs) in the primary system at operating temperature and pressure.

d. CARD 4: Primary System Letdown Rate

Enter in spaces (73-80) the average letdown rate (gpm) from the primary system to the purification demineralizers.

e. CARD 5: Letdown Cation Demineralizer Flow Rate

Enter in spaces (73-80) the annual average flow rate (gpm) through the cation demineralizers for the control of cesium in the primary coolant. The average flow rate is determined by multiplying the average letdown rate (value entered on Card 4) by the fraction of time cation demineralizers are in service to obtain the average cation demineralizer flow rate.

f. CARD 6: Hydrological Travel Time

Enter in spaces (73-80) the travel time (days) it takes for the liquid waste of a failed tank to reach the nearest potable water supply or nearest surface water in an unrestricted area.

g. CARD 7: Hydrological Dilution Factor

Enter in spaces (73-80) the dimensionless value of:
The annual volume of water flowing past the potable water supply divided by the total volume of liquid waste in the failed tank (80% of design capacity).

h. CARDS 8-16: Liquid Tank Parameters

Tanks in two separate processing systems are considered in the PWR-RATAFR Code:

- (1) Shim Bleed Processing System, Cards 8-12;
- (2) Waste Drain Processing System, Cards 13-16.

In each of the processing systems considered, the Code can calculate the tank concentrations in either the collector tank or the evaporator bottoms tank. However, separate computer runs must be made for these two tank classifications. If it is desired, the Code can calculate concentrations in one tank of each of the processing systems in one computer run by making the appropriate entries in CARDS 8-16. If a tank in only one of the processing systems is to be considered, then the appropriate entries need only be made in CARDS 8-12 or 13-16, depending on which system contains the tank of interest.

Five input data cards define the major parameters for the failed tank in the shim bleed processing system. The first two cards (CARDS 8 and 9) describe the inputs to the tank. The shim bleed wastes are entered on CARD 8. For reactor designs which combine the shim bleed with other reactor grade wastes prior to processing, the other wastes are entered as equipment drain wastes on CARD 9.

Four input data cards define the major parameters for the failed tank in the waste drain processing system. The first card (CARD 13) describes the inputs to the tank. Essentially the same information is required on CARDS 8, 9 and 13, on CARDS 10 and 14, on CARDS 11 and 15, and on CARDS 12 and 16.

The following information explains the use of the parameters in this Appendix and information given in the SAR/ER to complete the input data cards.

For CARD 8, enter in spaces (42-49) the flow rate (gpd) of the inlet stream. The value of the shim bleed rate must always be entered, even if the tank being evaluated is not in this system, since it is used in determining the primary coolant concentrations. Do not enter inlet waste activity for the shim bleed since the activity for this waste stream is calculated by the Code.

The following information is required on CARDS 9 and 13 for both of the processing systems considered in the Code.

- (1) Enter in spaces (17-40) the name of the waste stream (e.g., clean wastes).
- (2) Enter in spaces (42-49) the flow rate (gpd) of the inlet stream.
- (3) Enter in spaces (57-61) the activity of the inlet stream expressed as a fraction of primary coolant activity (PCA).

For CARDS 10 and 14, the following information is required:

- (1) Enter in spaces (27-50) the name of the tank to be failed (e.g., shim bleed collector).
- (2) Enter in spaces (65-73) the volume (gallons) of the tank to be failed. If a tank in either of the two processing streams is not being considered, leave spaces (65-73) blank for that tank.

For CARDS 11 and 15, enter a 1 in space (80) if the tank to be failed is an evaporator bottoms tank. Otherwise leave space (80) blank.

CARDS 12 and 16 for both waste streams contain the overall system "tank" factors. The "tank" factors indicate the type of processing the waste has undergone prior to its entry into the tank. The tank factors should be entered as follows:

- (1) For a collector tank in either waste stream, without demineralizers upstream of the tank, values of TF=1.0 should be entered in the appropriate spaces.
- (2) For a collector tank in either waste stream with demineralizers upstream of the tank, or for an evaporator bottoms tank in either waste stream, TF's based on the description given in Section 2.a(3) should be entered in the appropriate spaces.

The appropriate spaces for the TFs are:

- (1) Enter in spaces (21-28) the TF for iodine.
- (2) Enter in spaces (34-41) the TF for cesium and rubidium.
- (3) Enter in spaces (47-54) the TF for other nuclides.

The following section explains the use of the parameters in this note and information given in the SAR/ER to complete data input CARDS 8-16.

2. Explanation of Parameters used in Filling out CARDS 8-16

a. Liquid Waste Flow Rates and Activities

(1) Shim Bleed Wastes and Equipment Drain Wastes

The flow rates of the shim bleed waste stream and equipment drain wastes processed with the shim bleed and the activity of the equipment drain wastes are based on information given in the SAR/ER. The activity of the shim bleed wastes is based on the primary coolant letdown system effluent activity and is calculated in the Code.

The activity of the combined inlet stream is calculated by the Code based on the weighted average of the composite stream entering the tanks.

(2) Waste Drain Tank Wastes

Flow rates and activities are calculated using the waste volumes and activities given in NUREG-0017, Table 1-2, "PWR Liquid Wastes" (Ref. 11).

These input flow rates are supplemented by the use of expected flows and activities more specific to the plant design as given in the SAR/ER. The individual streams are combined based on the radwaste treatment system described in the SAR/ER. Input activities are based on the weighted average activity of the composite stream entering the waste collection tanks. The input flow rates and activities are entered in units of gpd and fractions of PCA.

(3) Tank Factors

The tank factors indicate the type of processing the waste has undergone prior to its entry into the tank. The tank factors provide the capability to consider radionuclide removal by demineralizers or other treatment equipment prior to the tank. For evaporator bottoms tanks, the tank factors provide the capability to consider the effects of radionuclide concentration in the evaporator. Therefore, in determining the radionuclide concentration in the tank, the type of processing upstream of the tank must be considered and entered as tank factors on CARDS 12 and 16.

The following factors are considered in calculating overall tank factors for the systems.

(a) TFs are categorized by radionuclides.

Halogens

Cs, Rb

Other Nuclides

Note: TF of 1 is assumed for tritium.

(b) The system TF is the product of the individual equipment TF in each of the systems, e.g., the effect of the demineralizer removal, if any, is multiplied by the effect of the evaporator concentration, if any.

(c) Tank Factors for Demineralizers

The tank factors for demineralizers are entered in the same manner as decontamination factors (DFs) are entered in the PWR-GALE Code. Therefore, the values used for TFs for demineralizers are the same as those given in NUREG-0017, Table 1-3, "Decontamination Factors for PWR Liquid Waste Treatment Systems" (Ref. 11).

(d) Tank Failures for Evaporators

The tank factors for evaporators express the increase in concentration of radionuclides in the evaporator bottoms resulting from evaporator operation. The values entered on the CARDS are the ratio of the evaporator bottom stream flow to the evaporator inlet stream flow. Therefore, the TFs for evaporators are as follows:

<u>Evaporator</u>	<u>All Nuclides</u>
Waste Drain Stream	0.02
Shim Bleed Stream	0.02

1.	2.	3.	4.	5.	6.	7.	8.	9.	10.	11.	12.	13.	14.	15.	16.
CARD 1	CARD 2	CARD 3	CARD 4	CARD 5	CARD 6	CARD 7	CARD 8	CARD 9	CARD 10	CARD 11	CARD 12	CARD 13	CARD 14	CARD 15	CARD 16
NAME	PCWTH	PCVOL	LETDWN	CBFLR	HYTRTM	HYDF	SHIMBLEEC RATE	EQUIPMENT DRAINS INPUT	TANK NAME CONDENSATE HOLDUP	IS TANK IN SYSTEM A	DFI=	DIRTY WASTES INPUT	TANK NAME WASTE HOLDUP	IS TANK IN SYSTEM A	DFI=
NAME OF REACTOR	POWER LEVEL (MEGAWATTS)	MASS OF PRIMARY COOLANT (THOUSANDS LBS)	SYSTEM LETDOWN RATE (GPM)	LETDOWN CATION DEMINERALIZER FLOW (GPM)	HYDROLOGICAL TRAVEL TIME (DAYS)	HYDROLOGICAL DILUTION FACTOR	16200.	810.	TANK NAME CONDENSATE HOLDUP	IS TANK IN SYSTEM A	DFI=	1375.	TANK NAME WASTE HOLDUP	IS TANK IN SYSTEM A	DFI=
SAMPLE CASE PMR							GPD	GPD AT 1.0	TANK VOLUME	TANK 0 IF NO	1.0E05	GPD AT .075 PCA	TANK VOLUME	TANK 0 IF NO	1.0E04
4/78									250000.	1 IF YES			250000.	1 IF YES	
TYPE	PMR														
	3391.	561.	75.	7.5	1.	1000.									

SAMPLE CASE PWR 4/78

THERMAL POWER LEVEL (MEGAWATTS)	PWR
3391.0000	
PLANT CAPACITY FACTOR	
0.80	
MASS OF PRIMARY COOLANT (THOUSANDS LBS)	
561.0000	
PERCENT FUEL WITH CLADDING DEFECTS	
1.0000	
PRIMARY SYSTEM LETDOWN RATE (GPM)	
75.0000	
LETDOWN CATION DEMINERALIZER FLOW (GPH)	
7.5000	
HYDROLOGICAL TRAVEL TIME (DAYS)	
1.00	
HYDROLOGICAL DILUTION FACTOR	
1000.	
SHIMBLEED RATE	
1.62E+04	
EQUIPMENT DRAINS INPUT	
8.10E+02	
DIRTY WASTES INPUT	
1.38E+03	

FAILED TANK PARAMETERS

FAILED TANK	TANK FACTORS
CONDENSATE HOLDUP	I CS
WASTE HOLDUP	1.00E+03 1.00E+05
	250000. 1.00E+03 1.00E+04
	250000. 1.00E+03 1.00E+04
	OTHERS
	1.00E+05
	1.00E+04

SAMPLE CASE PWR 4/78 LIQUID TANK FAILURE
 NAME OF TANK FAILED: CONDENSATE HOLDUP
 VOLUME OF TANK FAILED: 20000. GAL. (80% OF TANK CAPACITY)
 HYDROLOGICAL TRAVEL TIME (DAYS): 1.00
 HYDROLOGICAL DILUTION FACTOR: 1000.

NUCLIDE	HALF-LIFE (DAYS)	PRIMARY COOLANT CONC. (UCI/ML)	10CFR20 LIMITS (UCI/ML)	FAILED TANK CONC. (UCI/ML)	CRITICAL RECEPTOR CONC. (UCI/ML)	FRACTION 10CFR20
CORROSION AND ACTIVATION PRODUCTS						
H 3	4.49E+03	1.00E+00	3.00E-03	1.00E+00	1.00E-03	0.3300
FISSION PRODUCTS						
I131	8.05E+00	2.18E+00	3.00E-07	2.00E-04	1.80E-07	0.6000
I133	8.75E-01	3.08E+00	1.00E-06	4.70E-05	2.10E-08	0.0210
ALL OTHERS		6.38E+00		2.40E-05	3.10E-09	0.0003
TOTAL EXCEPT TRITIUM		1.16E+01		2.70E-04	2.00E-07	0.6200

THE MAXIMUM QUANTITY OF TRITIUM IN THE TANK IS 2.3E+03 CURIES.

THE MAXIMUM QUANTITY OF CORROSION AND FISSION PRODUCTS (EXCLUDING TRITIUM) IN THE TANK IS 3.3E-01 CURIES.

SAMPLE CASE PHR 4/78 LIQUID TANK FAILURE
 NAME OF TANK FAILED: WASTE HOLDUP
 VOLUME OF TANK FAILED: 20000. GAL. (80% OF TANK CAPACITY)
 HYDROLOGICAL TRAVEL TIME (DAYS): 1.00
 HYDROLOGICAL DILUTION FACTOR: 1000.

NUCLIDE	HALF-LIFE (DAYS)	PRIMARY COOLANT CONC. (UCI/ML)	10CFR20 LIMITS (UCI/ML)	FAILED TANK CONC. (UCI/ML)	CRITICAL RECEPTOR CONC. (UCI/ML)	FRACTION 10CFR20
CORROSION AND ACTIVATION PRODUCTS						
H 3	4.49E+03	1.00E+00	3.00E-03	7.40E-02	7.40E-05	0.0250
FISSION PRODUCTS						
I131	8.05E+00	2.18E+00	3.00E-07	1.30E-05	1.20E-08	0.0400
ALL OTHERS		9.46E+00		6.70E-06	3.60E-09	0.0011
TOTAL EXCEPT TRITIUM		1.16E+01		2.00E-05	1.60E-08	0.0410

THE MAXIMUM QUANTITY OF TRITIUM IN THE TANK IS 2.3E+03 CURIES.

THE MAXIMUM QUANTITY OF CORROSION AND FISSION PRODUCTS (EXCLUDING TRITIUM) IN THE TANK IS 3.6E-01 CURIES.

APPENDIX B

RATAFR CODE FOR BOILING WATER REACTORS Input Cards and Sample Calculation

1. Parameters Required for the BWR-RATAFR Code

Complete the cards from information given in the Applicant's Safety Analysis and Environmental Reports and in Section 2 of this Appendix.

- a. CARD 1: Name or Reactor (SAR/ER)
Enter in spaces (33-60) the name of the reactor.
- b. CARD 2: Thermal Power Level (SAR/ER)
Enter in spaces (73-80) the maximum thermal power level (Mwt) evaluated for safety considerations in the Safety Analysis Report.
- c. CARD 3: Total Steam Flow Rate (SAR/ER)
Enter in spaces (73-80) the total steam flow rate from the reactor (10^6 lbs/hr).
- d. CARD 4: Mass of Coolant in Reactor Vessel (SAR/ER)
Enter in spaces (73-80) the mass of water in the reactor vessel (10^6 lbs).
- e. CARD 5: Cleanup Demineralizer Flow (SAR/ER)
Enter in spaces (73-80) the primary coolant flow rate (10^6 lbs/hr) through the reactor coolant cleanup system demineralizers.
- f. CARD 6: Condensate Demineralizer Regeneration Time
For deep bed condensate demineralizers, use 3.5 day regeneration frequency per demineralizer. If ultrasonic resin cleaning is used, assume 7-day regeneration frequency per demineralizer. Multiply the frequency by the number of demineralizers, and enter the calculated number of days in spaces (73-80). For filter/demineralizers (Powdex), enter zero in spaces (73-80).
- g. CARD 7: Fraction of Feed Water Through Condensate Demineralizer (SAR/ER)
Enter in spaces (73-80) the fraction of feedwater processed through the condensate demineralizers.
- h. CARD 8: Hydrological Travel Time
Enter in spaces (73-80) the travel time (days) it takes for the liquid waste of a failed tank to reach the nearest potable water supply or nearest surface water in an unrestricted area.
- i. CARD 9: Hydrological Dilution Factor
Enter in spaces (73-80) the dimensionless value of:
The annual volume of water flowing past the potable water supply divided by the total volume of liquid waste in the failed tank (80% of design capacity).

j. CARD 10-17: Liquid Tank Parameters

Tanks in two separate processing systems are considered in the BWR-RATAFR Code:

- (1) Waste Drain Processing System, CARDS 10-13.
- (2) Regenerant Solutions Processing System, CARDS 14-17.

In each of the processing systems considered, the Code can calculate the tank concentrations in either the collector tank or the evaporator bottoms tank. However, separate computer runs must be made for these two tank classifications. If it is desired, the Code can calculate concentrations in one tank of each of the processing systems in one computer run by making the appropriate entries in CARDS 10-17. If a tank in only one of the processing systems is to be considered, then the appropriate entries need only be made in CARDS 10-13 or 14-17 depending on which system the tank of interest is in.

Four input data cards are used to define the major parameters for the failed tank in each of the processing systems. Essentially, the same information is required on the four input data cards used for each of the processing systems. The instructions given in this section are applicable to both processing streams, with the following exception. The inlet waste activity is not entered on CARD 14 for the regenerant solutions wastes for systems using regenerable condensate demineralizers, since the activity is calculated by the Code.

The following information explains the use of the parameters in this Appendix and information given in the SAR/ER to complete the input data cards.

The following information is required on CARDS 10 and 14 for both of the streams considered in the Code.

- (1) Enter in spaces (18-40) the name of the waste stream (e.g., high-purity wastes).
- (2) Enter in spaces (42-49) the flow rate (gpd) of the inlet stream.
- (3) Enter in spaces (57-61) the activity of the inlet stream, expressed as a fraction of the primary coolant activity (PCA).

On CARD 14, do not enter the activity of the regenerant solutions waste inlet stream in spaces (57-61).

For CARDS 11 and 15, the following information is required:

- (1) Enter in spaces (27-50) the name of the tank to be failed (e.g., High Purity Collector).
- (2) Enter in spaces (65-73) the volume (gallons) of the tank to be failed. If a tank in either of the two processing streams is not being considered, leave spaces (65-73) blank for that tank.

For CARDS 12 and 16, enter a 1 in space (80) if the tank to be failed is an evaporator bottoms tank. Otherwise leave space (80) blank.

CARDS 13 and 17 for both waste streams contain the overall system "tank" factors. The "tank" factors indicate the type of processing the waste has undergone prior to its entry into the tank. The tank factors should be entered as follows:

- (1) For a collector tank in either waste stream, without demineralizers upstream of the tank, values of TF = 1.0 should be entered in the appropriate spaces.
- (2) For a collector tank in either waste stream with demineralizers upstream of the tank, or for an evaporator bottoms tank in either waste stream, TFs based on the description given in Section 2.b should be entered in the appropriate spaces.

The appropriate spaces for the TFs are:

- (1) Enter in spaces (21-28) the TF for iodine.
- (2) Enter in spaces (34-41) the TF for cesium and rubidium.
- (3) Enter in spaces (47-54) the TF for other nuclides.

The following section explains the use of the parameters in this note and information given in the SAR/ER to complete data input CARDS 10-17.

2. Explanation of Parameters used in Filling out Cards 10-17

a. Liquid Waste Flow Rates and Activities

Flow rates and activities are calculated using the waste volumes and activities given in NUREG-0016, Table 1-2, "BWR Liquid Wastes" (Ref. 12).

These input flows are supplemented by the use of expected flows and activities more specific to the plant design as given in the SAR/ER. The inlet streams are combined to form the principal waste streams (drain wastes and regenerant wastes) considered in this guide, based on the radwaste treatment system described in the SAR/ER.

Input activities are based on the weighted average activity of the composite stream entering the waste collection tanks.

b. Tank Factors

The tank factors indicate the type of processing the waste stream has undergone prior to its entry into the tank. The tank factors provide the capability to consider radionuclide removal by demineralizers prior to the waste stream input into the tank. For evaporator bottoms tanks, the tank factors provide the capability to consider the effects of radionuclide concentration in the evaporator. Therefore, in determining

the radionuclide concentration in a tank, the type of processing upstream of the tank must be considered and entered as tank factors on CARDS 13 and 17.

The following factors are considered in calculating overall tank factors for the systems:

- (1) TFs are categorized by radionuclides.

Halogens

Cs, Rb

Other Nuclides

Note: TF of 1 is assumed for tritium.

- (2) The system TF is the product of the individual equipment TF in each of the systems, e.g., the effect of the demineralizer removal, if any, is multiplied by the effect of the evaporator concentration, if any.

- (3) Tank Factors for Demineralizers

The tank factors for demineralizers are entered in the same manner as decontamination factors (DFs) are entered in the BWR-GALE Code. Therefore, the values used for TFs for demineralizers are the same as those given in NUREG-0016, Table 1-3, "Decontamination Factors for BWR Liquid Waste Treatment Systems" (Ref. 11).

- (4) Tank Factors for Evaporators

The tank factors for evaporators express the increase in concentration of radionuclides in the evaporator bottoms resulting from evaporator operation. The values entered on the cards are the ratio of the evaporator bottoms stream flow to the evaporator inlet stream flow. Therefore, the TFs for evaporators are entered as follows:

<u>Evaporator</u>	<u>All Nuclides</u>
Waste Drain Stream	0.01
Regenerant Waste Stream	0.05

1.	2.	3.	4.	5.	6.	7.	8.	9.	10.	11.	12.	13.	14.	15.	16.	17.
CARD 1	CARD 2	CARD 3	CARD 4	CARD 5	CARD 6	CARD 7	CARD 8	CARD 9	CARD 10	CARD 11	CARD 12	CARD 13	CARD 14	CARD 15	CARD 16	CARD 17
NAME	POWTH	GTO	WLIQ	GDE	REGEN	FFCDM	HYTRIM	HYDF	NAME OF REACTOR	SAMPLE CASE BMR	4/78	NAME OF REACTOR	SAMPLE CASE BMR	TYPE =	BMR	
THERMAL POWER LEVEL (MEGAWATTS)	TOTAL STEAM FLOW (MILLION LBS/HR)	MASS OF WATER IN REACTOR VESSEL (MILLION LBS)	CLFAN-UP DEMINERALIZER FLOW (MILLION LBS/HR)	CONDENSATE DEMINERALIZER REGENERATION TIME (DAYS)	FRACTION FEED WATER THROUGH CONDENSATE DEMIN	HYDROLOGICAL TRAVEL TIME (DAYS)	HYDROLOGICAL DILUTION FACTOR	HIGH PURITY WASTE INPUT	27800.	GPD AT .164 PCA		TANK NAME WASTE COLLECTION	TANK VOLUME 30000.	GAL	0	
IS TANK IN SYSTEM A BOTTOMS TANK 0 IF NO 1 IF YES	DFI = 1.0E-00DFCS = 1.0E-00DFD = 1.0E-00	REGENERATION SOLTNS INPUT 2550.	TANK NAME CONCENTRATOR WASTE	IS TANK IN SYSTEM A BOTTOMS TANK 0 IF NO 1 IF YES	DFI = 5.0E-02DFCS = 5.0E-02DFD = 5.0E-02							TANK VOLUME 5000.	GAL	1		

SAMPLE CASE BMR 4/78 LIQUID TANK FAILURE
 NAME OF TANK FAILED: WASTE COLLECTION
 VOLUME OF TANK FAILED: 24000. GAL. (80% OF TANK CAPACITY)
 HYDROLOGICAL TRAVEL TIME (DAYS): 2.00
 HYDROLOGICAL DILUTION FACTOR: 200000.

NUCLIDE	HALF-LIFE (DAYS)	PRIMARY COOLANT CONC. (UCI/ML)	10CFR20 LIMITS (UCI/ML)	FAILED TANK CONC. (UCI/ML)	CRITICAL RECEPTOR CONC. (UCI/ML)	FRACTION 10CFR20
CORROSION AND ACTIVATION PRODUCTS						
FISSION PRDUCTS						
TI31	8.09E+00	2.84E-02	3.00E-07	4.50E-03	1.90E-08	0.0630
TI33	8.75E-01	1.07E-01	1.00E-06	1.30E-02	1.30E-08	0.0130
ALL OTHERS		1.81E+00		4.90E-02	3.30E-08	0.0008
TOTAL EXCEPT TRITIUM		1.95E+00		6.60E-02	6.50E-08	0.0770

THE MAXIMUM QUANTITY OF TRITIUM IN THE TANK IS 5.0E+03 CURIES.

THE MAXIMUM QUANTITY OF CORROSION AND FISSION PRODUCTS (EXCLUDING TRITIUM) IN THE TANK IS 1.5E+01 CURIES.

SAMPLE CASE BWR 4/78 LIQUID TANK FAILURE
 NAME OF TANK FAILED: CONCENTRATOR WASTE
 VOLUME OF TANK FAILED: 4000. GAL. (80% OF TANK CAPACITY)
 HYDROLOGICAL TRAVEL TIME (DAYS): 2.00
 HYDROLOGICAL DILUTION FACTOR: 200000.

NUCLIDE	HALF-LIFE (DAYS)	PRIMARY COOLANT CONC. (UCI/ML)	10CFR20 LIMITS (UCI/ML)	FAILED TANK CONC. (UCI/ML)	CRITICAL RECEPTOR CONC. (UCI/ML)	FRACTION 10CFR20
CORROSION AND ACTIVATION PRODUCTS						
P 32	1.43E+01	1.96E-04	2.00E-05	2.00E-02	8.90E-08	0.0045
CR 51	2.78E+01	4.91E-03	2.00E-03	9.50E-01	4.50E-06	0.0023
MN 54	3.03E+02	5.90E-05	1.00E-04	2.30E+02	1.10E-07	0.0011
FE 55	9.50E+02	9.83E-04	8.00E-04	3.90E-01	2.00E-06	0.0025
CO 58	7.13E+01	1.97E-04	9.00E-05	6.00E-02	2.90E-07	0.0032
CO 60	1.92E+03	3.93E-04	3.00E-05	1.60E-01	8.00E-07	0.0270
ZN 65	2.45E+02	1.97E-04	1.00E-04	7.40E-02	3.70E-07	0.0037
FISSION PRODUCTS						
SR 89	5.20E+01	5.73E-04	3.00E-06	1.60E-01	7.60E-07	0.2500
SR 90	1.03E+04	3.44E-05	3.00E-07	1.40E-02	7.10E-08	0.2400
Y 90	2.67E+00	0.0	2.00E-05	1.40E-02	4.20E-08	0.0021
Y 91	5.88E+01	2.29E-04	3.00E-05	1.10E-01	5.20E-07	0.0170
MO 99	2.79E+00	1.13E-02	4.00E-05	1.80E-02	5.60E-08	0.0014
RU1C3	3.96E+01	1.15E-04	8.00E-05	2.70E-02	1.30E-07	0.0017
RU1C6	3.67E+02	1.72E-05	1.00E-05	6.70E-03	3.30E-08	0.0033
TE129M	3.40E+01	2.29E-04	2.00E-05	5.00E-02	2.40E-07	0.0120
I131	8.05E+00	2.84E-02	3.00E-07	2.20E+01	9.40E-05	310.0000
CS134	7.49E+02	1.72E-04	9.00E-06	3.80E-02	1.90E-07	0.0210
CS137	1.10E+04	4.02E-04	2.00E-05	9.10E-02	4.60E-07	0.0230
BA140	1.28E+01	2.29E-03	2.00E-05	2.00E-01	8.90E-07	0.0440
LA140	1.67E+00	0.0	2.00E-05	2.30E-01	5.00E-07	0.0250
CE141	3.24E+01	1.72E-04	9.00E-05	4.00E-02	1.90E-07	0.0021
PR143	1.37E+01	2.29E-04	5.00E-05	2.40E-02	1.10E-07	0.0021
CE144	2.84E+02	1.72E-05	1.00E-05	6.50E-03	3.30E-08	0.0033
ALL OTHERS		1.90E+00		2.50E-01	2.90E-07	0.0046
TOTAL						
EXCEPT TRITIUM		1.95E+00		2.50E+01	1.10E-04	310.0000

THE MAXIMUM QUANTITY OF TRITIUM IN THE TANK IS 5.0E+03 CURIES.

THE MAXIMUM QUANTITY OF CORROSION AND FISSION PRODUCTS (EXCLUDING TRITIUM) IN THE TANK IS 1.2E+00 CURIES.

APPENDIX C

RATAFR LISTING

This appendix contains the program listing in FORTRAN for the RATAFR Code, applicable to PWR's and BWR's using the input data cards from Appendix A or B of this manual. The nuclear data library and subroutines are available in card deck form from the Effluent Treatment Systems Branch, USNRC, (301)492-7775. The remainder of this appendix contains the RATAFR program listing.

```

1. C RATAF CODE FOR CALCULATING CONSEQUENCES OF RADIOACTIVE LIQUID
2. C TANK FAILURES. MODIFIED MARCH 1976 TO CALCULATE TANK ISOTOPIC
3. C CONCENTRATIONS TO PROVIDE AS INPUT FOR DETERMINING CONCENTRATIONS
4. C AT POTABLE WATER SUPPLIES. BASES FOR CODE IS THE LIQUID GALE
5. C CODE OF JULY 1975.
6. C
7. C INTEGER*2 NAME(3),ELE(99),STA(2),LOC,NONG,KD
8. C REAL*4 LETDMA,LETDWN,MCH3,MQH3,MCCFP,MQCFP
9. C COMMON/LABEL/ ELE,STA
10. C COMMON/MATRIX/AT(2500),LOC(2500),NONG(800),KD(800)
11. C COMMON/FLEX/FLUX(10),PMN,MOUT,INDEX,QXN,AXN,ERR,NOBLND,MZERO
12. C COMMON/PROCSS/ NPROS,PRATE(8),NDRPOS(8),NZPROS(8,20),PR(800)
13. C COMMON/EGXZFER(800),XZH(800),XTEMP(800),XNEW(10,800),
14. C B(800),D(800)
15. C COMMON/FLUXV/TI(20),POWER(10),TOCAP(800),FISS(100),DIS(800),ILITE,
16. C IACT,IFP,IFOT,NUN,INPT
17. C COMMON/OUT/NUCL(800),TITLE(20),Q(800),FG(800),CUTOFF(7),
18. C 1 PCW,BUR,IMP,FLUXJ,MSTAR,ALPHAN(100),SPUNF(100),ABUND(500),
19. C 2 BASIS(10),TCONST,TUNIT
20. C COMMON/MPC/MPC(800),AMP(800),WMPC(800)
21. C COMMON/CONP/PWC(800)
22. C COMMON/PCONC(800)
23. C DIMENSION PCONC(300),DMCONC(800),CWCONC(800),CMCONC(800),
24. C IRI(4(800))
25. C DIMENSION TKCONC(800),PER(800)
26. C DIMENSION PWCN(800),FRAC(800)
27. C DIMENSION SRTANK(6),DMTAK(6),CWTANK(6),RGTANK(6)
28. C DIMENSION WORD>6(14),WORD2(4(6)),REACTR(7)
29. C DATA PWR/,BWR/,BWR2/,BWR3/
30. C BCONC CONTAINS PRIMARY COULANT CONCENTRATIONS FOR BWR'S.
31. C PWCNC CONTAINS PRIMARY COULANT CONCENTRATIONS FOR PWR'S.
32. C
33. C READ NUCLEAR DATA AND CONSTRUCT TRANSITION MATRIX
34. C CALL MUDATA(NLIDE)
35. C
36. C
37. C
38. C
39. C
40. C
41. C
42. C
43. C
44. C
45. C
46. C
47. C
48. C
49. C
50. C
51. C
52. C
53. C
54. C
55. C
56. C
57. C
58. C
59. C
60. C
61. C
62. C
63. C
64. C

```

00000080

00000190
00000200
00000210
00000220
00000230
00000240
00000250
00000260
00000270
00000280
00000290

00000570
00000580

00000590
00000610
00000620
00000630
00000640

00000680
00000730
00003640
00000740

00000840

00000850
00000880
00000890
00000900

00000910
00000920
00000930
00000940
00000950
00000970
00000980
00000990
00001000
00001010
00001020
00001030
00001040
00001050
00001060

00001520
00001560

```
65. PCONC(J)=0.0
66. DWCONC(J)=0.0
67. RTNV(J)=0.0
68. CWCONC(J)=0.0
69. CMCONC(J)=0.0
70. TKCONC(J)=0.0
71. PWCON(J)=0.0
72. FRAC(J)=0.0
73. PER(J)=0.0
74. XZH(J)=0.0
75. 40 CONTINUE
76.
77. C
78. C
79. C
79. PRINT 9026
80. READ 9010,REACTR,TYPE
81. PRINT 9010,RFACTR,TYPE
82. READ 9011,WORD56,POM1
83. PRINT 9011,WORD56,POM1
84. PF=0.80
85. PRINT 9027
86. IF (TYPE.EQ.BWR) GO TO 50
87. READ DATA FOR PWR,NSS PARAMETERS AND TANK PARAMETERS
88. READ 9022,WORD56,PCVOL
89. PRINT 9022,WORD56,PCVOL
90. 700 FAILDB=1.0
91. FAILFA=0.12
92. FAILRA=FAILDR/FAILEA
93. PRINT 9028,FAILDB
94. READ 9022,WORD56,LETDWN
95. PRINT 9022,WORD56,LETDWN
96. READ 9022,WORD56,CBFLR
97. PRINT 9022,WORD56,CBFLR
98. READ 9085,WORD56,HYTRTM
99. PRINT 9085,WORD56,HYTRTM
100. READ 9090,WORD56,HYDF
101. PRINT 9090,WORD56,HYDF
102. READ 9013,WORD24,SBLDR
103. PRINT 9025,WORD24,SBLDR
104. CWA=1.0
105. READ 9013,WORD24,EDFLR,EDA
106. PRINT 9025,WORD24,EDFLR
107. READ 9023,SBTANK,SBTKSZ
108. READ 9086,KCV8TS
109. IF (KEVBTS.EQ.1) EVAPS=0.02
110. READ 9014,DFICM,DFCSCM,DFCW
111. READ 9013,WORD24,DWFLR,DWA
112. IF (DWFLR.EQ.0) GO TO 4446
113. PRINT 9025,WORD24,DWFLR
114. 4446 COVTINUE
115. READ 9023,DWTANK,DWTKSZ
116. READ 9036,KEVBTD
117. IF (KEVBTD.EQ.1)EVAPD=0.02
118. READ 9014,DFIDW,DFCSDW,DFDW
119. K=1
120. PRINT 9045
121. PRINT 9016
122. IF (SBTKSZ.EQ.0.0)GO TO 45
123. PRINT 9024,SBTANK,SBTKSZ,DFICM,DFCSCM,DFCW
124. GO TO 46
```



```

185. IF (RGTKSZ.EQ.0.0)GO TO 60
186. PRINT 9024, RGTANK, RGTKSZ, DFIRG, DFCSRG, DFRG
187. 60 CONTINUE
188. 77 DO 78 I=1, ITOT
189. B(I)=0.0
190. 78 CONTINUE
191.
192. C DFIDW=1.0
193. DFCSDW=1.0
194. DFUW=1.0
195. 240 CONTINUE
196. C
197. C CALCULATE PRIMARY COOLANT CONCENTRATIONS FOR PWRS
198. C
199. C IF ITYPE.EQ.BWR)GO TO 251
200. AFPTES=0.0
201. DO 242 I=1, ITOT
202. PCONC(I)=PWCONC(I)
203. PUMA=PWMI
204. PCVOA=PCVOL*IE3
205. LETDWA=LETDW*500.53
206. SBLDA=SBLDR*.3476
207. C8FLA=C8FLR*500.53
208. CHECK TO SEE IF PRIMARY PLANT PARAMETERS ARE WITHIN SPECIFIED
209. C RANGES
210. C IF (ABS(PCVOA-3400).GT.400.1)GO TO 243
211. C IF (ABS(PCVOA-5.5E5).GT.0.5001E5)GO TO 243
212. C IF (ABS(LETDWA-3.7E4).GT.0.5001E4)GO TO 243
213. C IF (ABS(SBLDA-625.).GT.375.1)GO TO 243
214. C IF (ABS(C8FLA-3750.).GT.3750.1)GO TO 243
215. GO TO 247
216. C CALCULATE PWR PRIMARY COOLANT ADJUSTMENT FACTORS
217. AFPTES=1.0
218. RHAL2=(LETDWA*0.9+0.1*SBLDA)/PCVOA
219. RCSRB2=(LETDWA*0.5+0.5*(SBLDA+C8FLA*0.9))/PCVOA
220. RCFP2=(LETDWA*0.9+C.1*(SBLDA+C8FLA*0.9))/PCVOA
221. RK2=161.76*PCVOA/PCVOA
222. DO 246 J=1, ITOT
223. IF (PCONC(J).EQ.0.0) GO TO 246
224. NZ=NUCL(J)/10000
225. DL=DIS(J)*3600.
226. IF (NZ.EQ.1) GO TO 246
227. IF (NZ.EQ.53.0R.NZ.EQ.35)GO TO 244
228. IF (NZ.EQ.37.0R.NZ.EQ.55)GO TO 245
229. PCONC(J)=PCONC(J)*RK2*(0.0612+DL)/(RCFP2+DL)
230. GO TO 246
231. 244 PCONC(J)=PCONC(J)*RK2*(0.0606+DL)/(RHAL2+DL)
232. GO TO 246
233. 245 PCONC(J)=PCONC(J)*RK2*(0.0371+DL)/(RCSRB2+DL)
234. 246 CONTINUE
235. 247 PCVOL=PCVOL*1000.*0.7/62.4
236. DO 279 I=1, ITOT
237. IF (PCONC(I).EQ.0.0)GO TO 279
238. FAIL=FAILRA
239. IF (I.LE.ILITE)FAIL=1.0
240. PCONC(I)=PCONC(I)/(DIS(I)*1.6283E13)*FAIL
241. 279 CONTINUE
242. C
243. C GO TO 400
244. C

```

00003290

00004850
00004860

00004880

00005480

```

245. C 251 CONTINUE
246. C CALCULATE BWR PRIMARY COOLANT CONCENTRATIONS
247. DO 2251 I=1,ITOT
248. 2251 PCONC(I)=BCONC(I)
249. POWA=POWI
250. PCVDA=PCVOL*IE6
251. LETDWA=LETDWN*IE6
252. STMFA=STMFR*IE6
253. FFCDM=FFCDM
254. CHECK TO SEE IF PLANT PARAMETERS ARE WITHIN SPECIFIED RANGES
255. IF(ABS(POWA-3400).GT.400.1)GO TO 252
256. IF(ABS(PCVDA-3.8E5).GT.0.4001E5)GO TO 252
257. IF(ABS(LETDWA-1.3E5).GT.0.2001E5)GO TO 252
258. IF(ABS(STMFA-1.5E7).GT.0.2001E7)GO TO 252
259. IF(ABS(FFCDA-0.9).GT.0.1001)GO TO 252
260. GO TO 256
261. C CALCULATE BWR ADJUSTMENT FACTORS
262. RHALZ=(LETDWA*0.9+FFCDA*STMFA*0.010)/PCVDA
263. RCSRB2=(LETDWA*0.5+FFCDA*STMFA*5E-4)/PCVDA
264. RCFP2=(LETDWA*0.9+FFCDA*STMFA*9E-4)/PCVDA
265. RK2=111.76*POWA/PCVDA
266. DO 255 J=1,ITOT
267. IF(PCONC(J).EQ.0.0) GO TO 255
268. NZ=NUCL(J)/1000
269. DL=DIS(J)*3600
270. IF (NZ.EQ.1) GO TO 255
271. IF (NZ.EQ.53.OR.NZ.EQ.35)GO TO 253
272. IF (NZ.EQ.37.OR.NZ.EQ.55)GO TO 254
273. PCONC(J)=PCONC(J)*RK2*(0.3434+DL)/(RCFP2+DL)
274. GO TO 255
275. 253 PCONC(J)=PCONC(J)*RK2*(0.1908+DL)/(RCSRB2+DL)
276. GO TO 255
277. 254 PCONC(J)=PCONC(J)*RK2*(0.1908+DL)/(RCSRB2+DL)
278. 255 CONTINUE
279. 256 PCVOL=PCVOL*100000./62.4
280. LETDWN=LETDWN*2000.
281. STMFR=STMFR*2851.
282. DO 2255 J=1,ITOT
283. OGRTE=OGRTE
284. IF (J.LE.ILITE)OGRTE=1.0
285. IF(PCONC(J).GT.0.0)PCONC(J)=PCONC(J)/(DIS(J)*1.6283E13)*OGRTE
286. 2255 CONTINUE
287. IF(IREGENT.GT.0.0) GO TO 257
288. GO TO 400
289.
290. C COMPUTE REMOVAL CONSTANT FOR CONDENSATE DEMINERALIZER IN BWR
291. CCDDM=0.9*STMFR*FPEF/(PCVOL*7.48*60.)*FFCDM
292. CSBDM=0.5*STMFR*FPEF/(PCVOL*7.48*60.)*FFCDM
293. DO 258 I=1,ITOT
294. NZ=NUCL(I)/1000
295. PR(I)=CCDDM
296. IF(NZ.EQ.53.OR.NZ.EQ.35)PR(I)=CCDDM*HEF/FPEF
297. IF(NZ.EQ.37.OR.NZ.EQ.55)PR(I)=CSBDM
298. XZHI=PCONC(I)*PR(I)*PCVOL*0.0283
299. B(I)=XZHI
300. XZHI(I)=XZHI*86400.
301. CONTINUE
302. XZERO(I)=0.
303. C CALCULATE INVENTORIES ON BWR CONDENSATE RESINS
304.

```

00003340
00003350

00005120

00005130

```

305. 290 T(1)=REGENT
306. CALL SOLVE
307. DO 295 I=1,ITOT
308. 295 RINV(I)=XTEMP(I)
309.
310. C THIS PORTION OF PROGRAM CALCULATES FAILED TANK CONCENTRATIONS
311. C
312. C 400 CONTINUE
313. RGMFRM=RGWFR*3785./1E6
314. DO 410 I=1,ITOT
315. 410 D(I)=-DIS(I)
316. 420 DO 430 J=1,ITOT
317. NZ=NUCL(J)/10000
318. IF (NZ.EQ.35.OR.NZ.EQ.54) GO TO 430
319. IF (TYPE.EQ.PWR)GO TO 425
320. CWCONC(J)=PCONC(J)*CWA
321. IF (NZ.EQ.1) CWCONC(J)=PCONC(J)
322. IF (RGMFR.EQ.0.0)GO TO 430
323. RINV(J)=RINV(J)/(REGENT*RGMFRM)
324. IF (NZ.EQ.1) RINV(J)=PCONC(J)
325. GO TO 430
326. 425 DFCVCS=10.
327. IF (NZ.EQ.1)DFCVCS=1.0
328. IF (NZ.EQ.37.OR.NZ.EQ.55) DFCVCS=2.
329. CWCONC(J)=PCONC(J)*(CWA*SBLDR/DFCVCS+ED*EDFLR)/(SBLDR+EDFLR)
330. DMCNOC(J)=PCONC(J)*DWA
331. 430 CONTINUE
332. C
333. C CALCULATE RADIOACTIVITY AFTER COLLECTION AT A CONSTANT RATE
334. IF (TYPE.EQ.BWR)GO TO 432
335. IF (SBLDR.GT.0.0)TC=SBTKSZ*0.8/((SBLDR+EDFLR)*EVAPS)
336. GO TO 434
337. 432 CONTINUE
338. IF (CWFLR.GT.0.0)TC=CWTKSZ*0.8/(CWFLR*EVAPC)
339. 434 CONTINUE
340. IF (DWFLR.GT.0.0)TD=DWTKSZ*0.8/(DWFLR*EVAPD)
341. IF (RGWFR.GT.0.0)TRG=(RGTKSZ*0.8-11900.*EVAPR)/(RGWFR*2.*EVAPR)
342. IF (TRG.LT.0.0) TRG=0.
343. C
344. CALL COLLECT(TC*86400.,CWCONC,ILITE,ITOT)
345. CALL COLLECT(TD*86400.,DMCONC,ILITE,ITOT)
346. 440 IF (TRG.LE.0.0)GO TO 450
347. CALL STORAG(TRG*86400.,RINV,ILITE,ITOT)
348. 450 DO 500 I=1,ITOT
349. NZ=NUCL(I)/10000
350. IF (NZ.EQ.1) GO TO 500
351. IF (NZ.EQ.35.OR.NZ.EQ.53)GO TO 460
352. IF (NZ.EQ.37.OR.NZ.EQ.55)GO TO 470
353. C
354. C CHEMICAL TREATMENT FOR OTHER CATIONS
355. C
356. CWCONC(I)=CWCONC(I)/DFCW
357. DMCONC(I)=DMCONC(I)/DFDM
358. RINV (I)=RINV (I)/DFRG
359. GO TO 500
360. C
361. C CHEMICAL TREATMENT FOR ANIONS
362. C
363. CWCONC(I)=CWCONC(I)/DFICW
364. DMCONC(I)=DMCONC(I)/DFIDW

```

00005420

00008320

00008340

00008360

00008470

00008480

00008490

00008500

00008520

00008580

00008650

00008660

00008670

00008680

00008700

00008790

00008800

00008810

00008840

```

365. RINV (I)=RINV (I)/DFIRG
366. GO TO 500
367.
368. C CHEMICAL TREATMENT FOR RB AND CS
369. C
370. C 470 CWCONC(I)=CWCONC(I)/DFCSCW
371. DWCONC(I)=DWCONC(I)/DFCSDW
372. RINV (I)=RINV (I)/DFCSR
373. C 500 CONTINUE
374. C
375. STS=S8TKSZ*0.8
376. DTS=DWTKSZ*0.8
377. CTS=CWTKSZ*0.8
378. RTS=RGTKSZ*0.8
379. C
380. IF (IREGENT.LT.0.001)GO TO 503
381. DO 502 I=1,ITOT
382. CWCUNC(I)=RINV(I)
383. C 502 CONTINUE
384. C 503 CONTINUE
385. IF (K.EQ.2) GO TO 508
386. CTKSZ=STS
387. IF (TYPE.EQ.8WR) CTKSZ=CTS
388. DO 507 I=1,ITOT
389. TKCONC(I)=CWCONC(I)
390. GO TO 512
391. C 508 DAM=1.6
392. IF (TYPE.EQ.8WR)GO TO 510
393. CTKSZ=DTS
394. DO 509 I=1,ITOT
395. TKCONC(I)=DWCONC(I)
396. GO TO 512
397. C 510 KATH=4.7
398. CTKSZ=RTS
399. DO 511 I=1,ITOT
400. TKCONC(I)=CMCONC(I)
401. C 512 CONTINUE
402. DO 540 I=1,ITOT
403. NZ=NUCL(I)/10000
404. IF (INZ.EQ.36.OR.NZ.EQ.54) GO TO 540
405. DISI=DIS(I)*1.6283E13
406. TKCONC(I)=TKCONC(I)*DISI
407. IF (TKCONC(I).GT.0.0) GO TO 535
408. PWCON(I)=0.0
409. GO TO 540
410. C 535 V=DIS(I)*HYTRTM*86400.
411. IF (V.GT.75.) V=75.
412. PWCON(I)=(TKCONC(I)/HYDF)*EXP(-V)
413. C 540 CONTINUE
414. DO 545 I=1,ITOT
415. C 545 FRAC(I)=PWCON(I)/MPC(I)
416. SAPIRIM=0.0
417. STANK=0.0
418. SCREC=0.0
419. SFRAC=0.0
420. PAPIRIM=0.0
421. PTANK=0.0
422. PCREC=0.0
423. PFRAC=0.0
424. MCH3=TKCONC(3)/FRAC(3)

```

00008910
00008920
00008930

00008960

00009410

00009600

00009620

00010010

```

425.  MQH3=MQH3*CTKSZ*3785./1.0E6
426.  IF (MQH3.GT.5000.) MQH3=5000.
427.  IF (TYPE.EQ.RWR) GO TO 553
428.  IF (K.EQ.1) PRINT 9076,REACTR,SBTANK,STS,HYTRM,HYDF
429.  IF (K.EQ.2) PRINT 9076,REACTR,DMTANK,DTS,HYTRM,HYDF
430.  GO TO 555
431.
432.
433.
434.
435.
436.
437.
438.
439.
440.
441.
442.
443.
444.
445.
446.
447.
448.
449.
450.
451.
452.
453.
454.
455.
456.
457.
458.
459.
460.
461.
462.
463.
464.
465.
466.
467.
468.
469.
470.
471.
472.
473.
474.
475.
476.
477.
478.
479.
480.
481.
482.
483.
484.

553 CONTINUE
    IF (K.EQ.1) PRINT 9076,REACTR,CWTANK,CTS,HYTRM,HYDF
    IF (K.EQ.2) PRINT 9076,REACTR,RGTANK,RTS,HYTRM,HYDF
555 CONTINUE
    PRINT 9077
    PRINT 9081
    KOUNTR=1
    I1=ILITE+IACT+1
    L=1
    DO 580 I=1,ITOT
      IF (I.EQ.I1)PRINT 9082
      NZ=NUCL(I)/10000
      IF (NZ.EQ.36.OR.NZ.EQ.54)GO TO 580
      DISI=DIS(I)*1.6283E+13
      APRIM=PCONC(I)*DISI
      WMPCL=WMPCL(I)
      TANK=TKCONC(I)
      CRECN=PWCON(I)
      FRACT=FRAC(I)
      NUCL1=NUCL(I)
      L=L+1
      IF(NZ.EQ.1) GO TO 560
      SAPRIM=SAPRIM+APRIM
      STANK=STANK+TANK
      SCRECN=SCRECN+CRECN
      SFRACI=SFRACI+FRACI
560 IF (FRACT.LT.0.001) GO TO 580
      IF (MOD(KOUNTR,50).NE.0) GO TO 570
      PRINT 9084, REACTR
      PRINT 9077
570 CALL NGAH(NUCL(I),NAME)
      THALF=8.0225E-6/DISI(I)
      ISUB=2
      IF (TANK.GT.1.) ISUB=1
      DIV=10.**((INT(ALOG10(TANK)))-ISUB)
      TANK1=AINT((TANK/DIV+0.5)*DIV)
      ISUB=2
      IF (CRECN.GT.1.0) ISUB=1
      DIV=10.**((INT(ALOG10(CRECN)))-ISUB)
      CRECN1=AINT((CRECN/DIV+0.5)*DIV)
      ISUB=2
      IF (FRACT.GT.1.0) ISUB=1
      DIV=10.**((INT(ALOG10(FRACT)))-ISUB)
      FRACT1=AINT((FRACT/DIV+0.5)*DIV)
      ISUBT=1
      PRINT 9078, NAME,THALF,APRIM,WMPCL,TANK1,CRECN1,FRACT1
      KOUNTR=KOUNTR+1
      IF(NZ.EQ.1) GO TO 580
      PAPRIM=PAPRIM+APRIM
      PTANK=PTANK+TANK
      PCRECN=PCRECN+CRECN
      PFRACI=PFRACI+FRACI
580 CONTINUE
      PAPRIM=SAPRIM-PAPRIM

```

00010170
00010180
00010190

00010220
00010250
00010260

00010350

00010470

00010650

00010700

00010720

00010830

```

485. PTANK=STANK-PTANK
486. PCREC=SCREC-PCREC
487. PFRAC=SFRAC-PFRAC
488. MOCFF=STANK/SFRAC
489. MOCFF=MOCFF*CTKSZ*3785./1.0E6
490. IF (MOCFF.GT.15.) MOCFF=15.
491. ISUBP=2
492. IF (PTANK.GT.1.0) ISUBP=1
493. DIV=10.**((INT(ALOG10(PTANK))-ISUBP)
494. PTANK=AINT(PTANK/DIV+0.5)*DIV
495. ISUBP=2
496. IF (PCREC.GT.1.0) ISUBP=1
497. DIV=10.**((INT(ALOG10(PCREC))-ISUBP)
498. PCREC=AINT(PCREC/DIV+0.5)*DIV
499. IF (PFRAC.EQ.0.0) GO TO 585
500. ISUBP=2
501. IF (PFRAC.GT.1.0) ISUBP=1
502. DIV=10.**((INT(ALOG10(PFRAC))-ISUBP)
503. PFRAC=AINT(PFRAC/DIV+0.5)*DIV
504.
505. 585 ISUBS=1
506. IF (STANK.LT.1.0) ISUBS=2
507. STANK=AINT(STANK/DIV+0.5)*DIV
508. ISUBS=1
509. IF (SCREC.LT.1.0) ISUBS=2
510. DIV=10.**((INT(ALOG10(SCREC))-ISUBS)
511. SCREC=AINT(SCREC/DIV+0.5)*DIV
512. ISUBS=1
513. IF (SFRAC.LT.1.0) ISUBS=2
514. DIV=10.**((INT(ALOG10(SFRAC))-ISUBS)
515. SFRAC=AINT(SFRAC/DIV+0.5)*DIV
516. PRINT 9079, PAPRIM,PTANK,PCREC,PFRAC
517. PRINT 9080, SAPRIM,STANK,SCREC,SFRAC
518. PRINT 9100, MQH3,MOCFF
519. IF (TYPE.EQ.8WR)GO TO 600
520. IF (K.EQ.2.OR.DWTKSZ.EQ.0.0)GO TO 30
521. GO TO 610
522. CONTINUE
523. IF (K.EQ.2.OR.RGTKSZ.EQ.0.0)GO TO 30
524. K=K+1
525. GO TO 503
526.
527. C
528. C
529. C
530. 9010 FORMAT(32X,7A4,16X,A4)
531. 9011 FORMAT(16X,13A4,A3,F9.4)
532. 9012 FORMAT(16X,14A4,F8.4)
533. 9013 FORMAT(16X,6A4,1X,F8.0,7X,F5.3)
534. 9014 FORMAT(20X,FR.0,2(5X,F8.0))
535. 9016 FORMAT('0',70X,'TANK FACTORS',/34X,'FAILED TANK',/13X,'VOLUME',/6X,
536. 'I',/9X,'CS',/7X,'OTHERS')
537. 9022 FORMAT(16X,14A4,F8.4)
538. 9023 FORMAT(26X,6A4,14X,F9.0)
539. 9024 FORMAT(30X,6A4,3X,F9.0,2X,3(1PE9.2,1X))
540. 9025 FORMAT(16X,6A4,' GPD',/28X,1PE8.2)
541. 9026 FORMAT(IH1)
542. 9027 FORMAT(16X,'PLANT CAPACITY FACTOR',/T75,'0.80')
543. 9028 FORMAT(16X,'PERCENT FUEL WITH CLADDING DEFECTS',/T75,F6.4)
544. 9030 FORMAT(16X,'FISSION PRODUCT CARRY-OVER FRACTION',/T75,F6.4/16X,
545. '1',/HALOGEN CARRY-OVER FRACTION',/T75,F6.4)
546.
547.
548.
549.
550.
551.
552.
553.
554.
555.
556.
557.
558.
559.
560.
561.
562.
563.
564.
565.
566.
567.
568.
569.
570.
571.
572.
573.
574.
575.
576.
577.
578.
579.
580.
581.
582.
583.
584.
585.
586.
587.
588.
589.
590.
591.
592.
593.
594.
595.
596.
597.
598.
599.
600.
601.
602.
603.
604.
605.
606.
607.
608.
609.
610.
611.
612.
613.
614.
615.
616.
617.
618.
619.
620.
621.
622.
623.
624.
625.
626.
627.
628.
629.
630.
631.
632.
633.
634.
635.
636.
637.
638.
639.
640.
641.
642.
643.
644.
645.
646.
647.
648.
649.
650.
651.
652.
653.
654.
655.
656.
657.
658.
659.
660.
661.
662.
663.
664.
665.
666.
667.
668.
669.
670.
671.
672.
673.
674.
675.
676.
677.
678.
679.
680.
681.
682.
683.
684.
685.
686.
687.
688.
689.
690.
691.
692.
693.
694.
695.
696.
697.
698.
699.
700.
701.
702.
703.
704.
705.
706.
707.
708.
709.
710.
711.
712.
713.
714.
715.
716.
717.
718.
719.
720.
721.
722.
723.
724.
725.
726.
727.
728.
729.
730.
731.
732.
733.
734.
735.
736.
737.
738.
739.
740.
741.
742.
743.
744.
745.
746.
747.
748.
749.
750.
751.
752.
753.
754.
755.
756.
757.
758.
759.
760.
761.
762.
763.
764.
765.
766.
767.
768.
769.
770.
771.
772.
773.
774.
775.
776.
777.
778.
779.
780.
781.
782.
783.
784.
785.
786.
787.
788.
789.
790.
791.
792.
793.
794.
795.
796.
797.
798.
799.
800.
801.
802.
803.
804.
805.
806.
807.
808.
809.
810.
811.
812.
813.
814.
815.
816.
817.
818.
819.
820.
821.
822.
823.
824.
825.
826.
827.
828.
829.
830.
831.
832.
833.
834.
835.
836.
837.
838.
839.
840.
841.
842.
843.
844.
845.
846.
847.
848.
849.
850.
851.
852.
853.
854.
855.
856.
857.
858.
859.
860.
861.
862.
863.
864.
865.
866.
867.
868.
869.
870.
871.
872.
873.
874.
875.
876.
877.
878.
879.
880.
881.
882.
883.
884.
885.
886.
887.
888.
889.
890.
891.
892.
893.
894.
895.
896.
897.
898.
899.
900.
901.
902.
903.
904.
905.
906.
907.
908.
909.
910.
911.
912.
913.
914.
915.
916.
917.
918.
919.
920.
921.
922.
923.
924.
925.
926.
927.
928.
929.
930.
931.
932.
933.
934.
935.
936.
937.
938.
939.
940.
941.
942.
943.
944.
945.
946.
947.
948.
949.
950.
951.
952.
953.
954.
955.
956.
957.
958.
959.
960.
961.
962.
963.
964.
965.
966.
967.
968.
969.
970.
971.
972.
973.
974.
975.
976.
977.
978.
979.
980.
981.
982.
983.
984.
985.
986.
987.
988.
989.
990.
991.
992.
993.
994.
995.
996.
997.
998.
999.
1000.

```



```

545. 9045 FORMAT(/,15X,'FAILED TANK PARAMETERS')
546. 9052 FORMAT(16X,'OFF-GAS RELEASE RATE(UC/SEC)',28X,F8.0)
547. 9076 FORMAT (1H1,20X,7A4,' LIQUID TANK FAILURE'/1H,21X,'NAME OF TANK F
548. LAILED: ',6A4/1H,21X,'VOLUME OF TANK FAILED: ',F9.0,' GAL. (80
549. 2% OF TANK CAPACITY)'/1H,21X,'HYDROLOGICAL TRAVEL TIME (DAYS):
550. 3',F8.2/1H,21X,'HYDROLOGICAL DILUTION FACTOR: ',F8.0)
551. 9077 FORMAT (1H0,28X,'PRIMARY',22X,'FAILED',7X,'CRITICAL'/29X,'COOLANT',
552. 1,7X,'ICFR20',9X,'TANK',8X,'RECEPTOR',6X,'FRACTION'/4X,'NUCLIDE',
553. 24X,'HALF-LIFE',5X,'CONC.',9X,'LIMITS',10X,'CONC.',7X,'CONC.',9X,
554. 3,'ICFR20'/15X,'(DAYS)',2X,4(6X,'(UCI/ML)'))
555. 9078 FORMAT (4X,A2,I3,A1,4X,1PE9.2,4(5X,1PE9.2),3X,0PF12.4)
556. 9079 FORMAT (4X,'ALL OTHERS',14X,1PE9.2,19X,1PE9.2,5X,1PE9.2,3X,0PF12.4
557. 1)
558. 9080 FORMAT (4X,'TOTAL'/4X,'EXCEPT TRITIUM',10X,1PE9.2,19X,1PE9.2,5X,1P
559. 1E9.2,3X,0PF12.4)
560. 9081 FORMAT (4X,'CORROSION AND ACTIVATION PRODUCTS')
561. 9084 FORMAT (1H0,4X,'FISSION PRODUCTS')
562. 9084 FORMAT (1H1,20X,7A4,' LIQUID TANK FAILURE (CONTINUED)')
563. 9085 FORMAT(16X,14A4,F8.2)
564. 9086 FORMAT (79X,I1)
565. 9090 FORMAT(16X,14A4,F8.0)
566. 9100 FORMAT (1H0,3X,'THE MAXIMUM QUANTITY OF TRITIUM IN THE TANK IS ',1
567. 1PE7.1,' CURIES.'/1H0,3X,'THE MAXIMUM QUANTITY OF CORROSION AND FIS
568. 2SION PRODUCTS (EXCLUDING TRITIUM) IN THE TANK IS ',1PE7.1,' CURIES
569. 3.')
```

00006230

```

570. END
571. BLOCK DATA
572. COMMON /CONB/BCONC(800)
573. COMMON/CONP/PWCONC(800)
574. DATA BCGNC/2*0.0,0.1,33*0,
575. 1 9E-3,13*0,2E-4,53*0,5E-3,4*0,6E-5,0.0,5E-2,3*0,
576. 11E-3,3*0,3E-5,0.0,2E-4,2*0,4E-4,7*0,1E-6,0.0,3E-4,2*0,3E-2,4*0,
577. 22E-4,3*0,2E-3,98*0,3E-4,64*0,7E-3,63*0,3E-3,4*0,5E-3,2*0,3E-3,
578. 319*0,5E-3,1E-4,3*0,6E-6,5*0,4E-3,0.0,4E-5,3*0,1E-2,6E-3,4*0,4E-3,
579. 411*0,7E-6,0.0,7E-6,6*0,5E-6,5*0,4E-3,2*0,2E-3,2E-2,8*0,9E-2,7*0,
580. 52E-5,3*0,8E-2,6*0,2E-3,4*0,3E-6,21*0,1E-6,104*0,4E-5,13*0,1E-4,
581. 60.0,5E-3,3*0,1E-5,3E-2,4*0,2E-2,5*0,7E-2,2*0,3E-5,2*0,2E-2,8*0,
582. 72E-5,3*0,7E-5,4*0,1E-2,4*0,1E-2,3*0,4E-4,4*0,1E-2,0.0,3E-5,3*0,
583. 86E-3,5E-3,7*0,3E-5,2*0,3E-6,10*0,3E-6,81*0/
584. DATA PWCONC/2*0.1,C,101*0,
585. 1 1.9F-3,4*0,3.1E-4,5*0,1.6E-3,3*0,1E-3,0.0,.016,
586. 12*0,2E-3,185*0,1.2E-3,63*0,4.8E-3,4*0,2.6E-3,2*0,3E-4,6*0,8.5E-5,
587. 28*0,2,4*0,3.5E-4,3*0,1E-5,0.0,1.2E-6,3*0,6.5E-4,3.6E-4,6.4E-5,9*000010680
588. 3,3,4E-5,11*0,6E-5,C,5E-5,15*0,0.084,0.048,16*0,4.5E-5,4.5E-5,14*0, 00010690
589. 41E-5,0.0,1E-5,103*0,2.9E-5,8*0,2.8E-4,8.5E-4,10*0,1.4E-3,1.6E-3,
590. 58*0,2.1E-3,3*0,2.5E-3,1E-3,1.27,5*0,0.027,1,4*0,38,5*0,0.047,2*0,
591. 6.025,2*0,19,8*0,0.13,3*0,0.18,0.16,12*0,2.2E-4,1.5E-4,5*0,7E-5,
592. 712*0,4E-5,5,2*0,3.3E-5,3*3E-5,91*0/
593. END
594. SUBROUTINE SOLVE
595. COMMON/EC/XZFR0(800), XZH(800),XTEMP(800),XNEM(10,800),
596. 8(800),D(800)
597. COMMON/FLEX/FLUX(10),MMN,MOUT,INDEX,QXN,AXN,ERR,NORLND,MZERO
598. COMMON/PROCESS/ MPROS,PRATE(8),NOPROS(8),NZPROS(8,20),PR(800)
599. COMMON/FLUXN/T(20),POWER(10),TOCAP(800),FISS(100),DIS(800),ILITE,
600. 1 IACT,1EPT,1TOT,NON,INPT
601. COMMON/OUT/NUCL(900),TITLE(20),Q(800),FG(800),CUTOFF(7),
602. 1 POW,BUR,UP,FLUXB,MSTAR,ALPHAN(100),SPONF(100),ABUND(500),
603. 2 BASIS(10),ICONST,TUNIT
604. DO 10 I=1,1TOT
00010990
00011000
00011010
00011020
00011030
00011040
00011050
00011060
00011070
00011080
00011090
```

```

605. D(I)=-DIS(I)
606. 10 XTEMP(I)=0.0
607. DELT=T(I)*TCONST
608. CALL DECAY(I,DELT,ITOT)
609. CALL TERM(DELT,I,ILITE,ITOT)
610. CALL EQUIL(I,ITOT)
611. DO 30 I=1,ITOT
612. XTEMP(I)=XNEW(I,I)
613. RETURN
614. END
615. SUBROUTINE TERM(T,M,ILITE,ITOT)
C
C TERM ADDS ONE TERM TO EACH ELEMENT OF THE SOLUTION VECTOR
C CSUM(J) IS THE CURRENT APPROXIMATION TO XNEW(M,J)
C CIMO(J) IS THE VECTOR CONTAINING THE LAST TERM ADDED TO EACH
C ELEMENT OF CSUM(J)
C CIMN(J) IS THE VECTOR CONTAINING 1/TON TIMES THE NEW TERM TO BE
C ADDED TO CSUM(J)
C CIMN(J) IS GENERATED FROM CIMO(I,J) BY A RECURSION RELATION:
C CIMN(J)=SUM OVER L OF (AP(J,L)*CIMO(L))
C AP(I,J) IS THE REDUCED TRANSITION MATRIX FOR THE LONG-LIVED
C NUCLIDES
C
C LOGICAL*1 LONG
C INTEGER*2 LOC,NONO,KD
C INTEGER*2 LOCP(2500)
C INTEGER*2 NOMP(800)
C INTEGER*2 NQ,NQU,NQUEUE
C REAL*8 RATE,RATM
C REAL*8 CIMN(800),CSUM(800),CIMNI
C DIMENSION AP(2500),CIB(800),CIMO(800)
C COMMON/SERIES/ XP(800),XPAR(800),LONG(800)
C COMMON/FLEX/FLUX(10),MMN,MOUT,INDEX,QXN,AXN,ERR,NQBLND,MZERO
C COMMON/EQ/XZERO(800),XZH(800),XTEMP(800),XNEW(10,800),
C 1 B(800),D(800)
C COMMON/MATRIX/A(2500),LOC(2500),NONO(800),KD(800)
C COMMON/DEBUG/AP
C COMMON/TERMD/DD(100),DXP(100),QUEUE(50),NQI(50),NQUEUE(50),NQ(800)
C NUL=0
C NN=0
C FIRST CONSTRUCT REDUCED TRANSITION MATRIX FOR LONG-LIVED ISOTOPES
C DO 220 L=1,ITOT
C IF(.NOT.LONG(L); GO TO 210
C NUM=NONO(L)
C IF(M.GT.MMN.OR.M.EQ.MZERO) NUM=KD(L)
C CIB(L)=B(L)
C IF(NUM.LE.NUL) GO TO 210
C NS=NN+1
C N=NUL
C NL=NUM-NUL
C DO 200 N1=1,NL
C N=N+1
C J=LOC(N)
C DJ=-D(J)
C THIS IS A TEST TO SEE IF ONE OF THE ASSYMPTOTIC SOLUTIONS APPLIES
C IF(.NOT.LONG(J)) GO TO 10
C NN=NN+1
664.

```



```

725. IF( ABS(PROD).GT.1.E-4) GO TO 60
726. USE THIS FORM FOR TWO NEARLY EQUAL HALF-LIVES
727. PROD=T*DK*XPJ*(1.0-0.5*(DL-DJ)*T)
728. GO TO 70
729. 60 PROD=(XPJ-XPL)/PROD
730. PROI=XPJ/DKR
731. 70 PI=1.0
732. S1=2./(DK*T)
733. DO 90 JK=1,II
734. IF(JK.EQ.KB) GO TO 90
735. S=1.0-DK/CD(JK)
736. IF( ABS(S).GT.1.E-4) GO TO 80
737. IF(ABS(DKR).GT.1.0E-4) PROD=PROI
738. S=S1
739. 80 PI=PI*S
740. IF(ABS(PI).GT.1.E25) GO TO 100
741. 90 CONTINUE
742. BATE=BATE+PROD/PI
743. 100 CONTINUE
744. C IF SUMMATION IS NEGATIVE, SET EQUAL TO ZERO AND PRINT MESSAGE
745. IF(BATE.LT.0.00) PRINT 9001, L,IM,BATE,BATM
746. 9001 FORMAT('BATE IS NEGATIVE IN TERM. THERE ARE MORE THAN TWO SHORT-LIVED NUCLEIDES IN A CHAIN WITH NEARLY EQUAL DIAGONAL ELEMENTS. /
747. 2, L,IM,BATE,BATM = ',2I5,1P2E12.5)
748. IF(BATE.LT.0.00) BATE=0.00
749. BATM=BATM+BATE
750. 110 CONTINUE
751. DRA=AKDJQ*DJ*(TRM-BATM)/TRM
752. GO TO 130
753. 120 DRA=AKDJQ*AMAX1(DRB,0.0)*DJ
754. 130 IF(NS.GT>NN) GO TO 150
755. DO 140 LJ=NS,NN
756. IF(LOCP(LJ).NE.J1) GO TO 140
757. AP(LJ)=AP(LJ)+DRA
758. GO TO 180
759. 140 CONTINUE
760. NN=NN+1
761. AP(NN)=DRA
762. LOCP(NN)=J1
763. GO TO 180
764. 160 IF(AKDJQ.LE.1.0E-06) GO TO 180
765. IF(NSAVE.GE.50) GO TO 180
766. 170 NSAVE=NSAVE+1
767. NQUEUE(NSAVE)=J1
768. QUEUE(NSAVE)=AKDJQ
769. NQU(NSAVE)=J
770. QUB(NSAVE)=DRB-1./(DJ*T)
771. CONTINUE
772. 180 IF(NSAVE.LE.0) GO TO 200
773. 190 J=NQUEUE(NSAVE)
774. QUE=QUEUE(NSAVE)
775. NQ(J)=NQ(NSAVE)
776. DRB=QUB(NSAVE)
777. CTRB(L)=CTRIB(L)+QUE*(J)*AMAX1(DRB,0.0)
778. NSAVE=NSAVE-1
779. GO TO 20
780. CONTINUE
781. 200 NUL=NULO(L)
782. 210 NQIP(L)=NN
783. 220 CONTINUE
784.

```

```

785. C FIND NORM OF MATRIX AND ESTIMATE ERROR AS DESCRIBED IN LAPIDUS 00012900
786. C AND LUNUS, OPTIMAL CONTROL OF ENGINEERING PROCESSES BLAISDELL 1967 00012910
787. C FIND THE MINIMUM OF THE MAXIMUM ROW SUM AND THE MAXIMUM COLUMN SUM 0000012920
788. ASUM = 0.0 00012930
789. ASUMJ = 0.0 00012940
790. NUL = 1 00012950
791. DO 250 I=1,ITOT 00012960
792. IF(.NOT.LONG(I)) GC TO 250 00012970
793. DI = -D(I)*T 00012980
794. AJ = DI 00012990
795. NUM = NCMPI(I) 00013000
796. IF(NUL.GT.NUM) GO TO 240 00013010
797. DO 230 N=NUL,NUM 00013020
798. AJ = AJ + AP(N) 00013030
799. AI = DI + DI 00013040
800. IF(AI.GT.ASUM) ASUM = AI 00013050
801. IF(AJ.GT.ASUMJ) ASUMJ = AJ 00013060
802. NUL = NCMPI(I) + 1 00013070
803. IF(ASUMJ.LT.ASUM) ASUM = ASUMJ 00013080
804. USE ASUM TO DECIDE HOW MANY TERMS ARE REQUIRED AND ESTIMATE ERROR 00013090
805. NLARGE = 3.5*ASUM + 5. 00013100
806. XLARGE = NLARGE 00013110
807. ERR1 = EXP(ASUM)*(ASUM*2.71828/XLARGE)**NLARGE/SORT(6.2832*XLARGE) 00013120
808. IF(ERR1.GT.1.E-3) PRINT 9002, ERR1, ASUM, NLARGE 00013130
809. 9002 FORMAT('OMAXIMUM ERROR GT 0.001, =F10.6,', TRACE = 'F10.4, 00013140
810. 1, NLARGE = 'I6) 00013150
811. C NEXT GENERATE MATRIX EXPONENTIAL SOLUTION 00013160
812. DO 260 I=1,ITOT 00013170
813. CSUM(I) = XTEMP(I) 00013180
814. CIMN(I) = XTEMP(I) 00013190
815. CONTINUE 00013200
816. ERR3 = 0.001*EPR 00013210
817. DO 310 NT=1,NLARGE 00013220
818. DO 270 I=1,ITOT 00013230
819. CIMO(I) = CIMN(I) 00013240
820. CONTINUE 00013250
821. TON = T/NT 00013260
822. NUL = 1 00013270
823. DO 300 I=1,ITOT 00013280
824. IF(.NOT.LONG(I)) GO TO 300 00013290
825. NUM = NUNPI(I) 00013300
826. CIMN1 = 0.0 00013310
827. IF(NT.EC.1) CIMN1 = CIM3(I) 00013320
828. IF(NUL.GT.NUM) GO TO 290 00013330
829. DO 280 N=NUL,NUM 00013340
830. J = LOCP(N) 00013350
831. CIMN1 = CIMN1 + AP(N)*CIMO(J) 00013360
832. CIMN1 = CIMN1 + D(I)*CIMO(I) 00013370
833. CIMN1 = TON*CIMN1 00013380
834. IF(DASS(CIMN1).LT.ERR3) CIMN1 = 0.00 00013390
835. CIMN(I) = CIMN1 00013400
836. CSUM(I) = CSUM(I) + CIMN1 00013410
837. NUL = NUNPI(I) + 1 00013420
838. CONTINUE 00013430
839. DO 320 I=1,ITOT 00013440
840. IF(CSUM(I).LT.ERR) CSUM(I) = 0.0 00013450
841. IF(LONG(I)) XNEW(M,I) = CSUM(I) 00013460
842. CONTINUE 00013470
843. RETURN 00013480
844. END 00013490

```

```

845. SUBROUTINE DECAY(M,T,ITOT)
846. DECADE TREATS SHORT-LIVED ISOTOPES AT BEGINNING OF CHAINS USING
847. BATEMAN EQUATIONS
848. LOGICAL*1 LONG
849. REAL*8 BATE
850. INTEGER*2 LOC,NONO,KD
851. INTEGER*2 NU,NQU,NQUEUE
852. COMMON/DEBUGG/AP(2500)
853. COMMON/SERIES/ XPI(800),XPAR(800),LONG(800)
854. COMMON/FLEX/FLUX(10),MMN,MOUT,INDEX,QXN,AXN,ERR,NDBLND,MZERO
855. COMMON/SEQ/XZFRU(800),XZH(800),XTEMP(800),XNEM(10,800),
      1 H(800),D(800)
856. COMMON/MATRIX/A(2500),LOC(2500),NONO(800),KD(800)
857. COMMON/TERM/D(LOC),DXP(100),QUEUE(50),NQU(50),NQUEUE(50),NQ(800)
858. DO 10 I=1,ITOT
859. XPAR(I)=C.0
860. LONG(I)=.FALSE.
861. XPI=0.0
862. DT=D(I)*T
863. IF(DT.LT.-50.) GO TO 10
864. IF(ABS(DT).LF.AXN) LONG(I)=.TRUE.
865. XPI=EXP(DT)
866. XP(I)=XPI
867. NUL=1
868. DO 160 L=1,ITOT
869. XTEM=0.0
870. DL=-D(L)
871. NUM=NONO(L)
872. IF(M.GT.MMN.OR.M.EQ.MZERO) NUM=KD(L)
873. IF(NUM.LT.NUL) GO TO 150
874. DO 140 N=NUL,NUM
875. J=LOC(N)
876. DJ=-D(J)
877. IF(LONG(J)) GO TO 140
878. USE THIS FORM FOR TWO NEARLY EQUAL HALF-LIVES
879. IF(ABS(DL/DJ-1.0).LE.1.0E-5) XTEM=XTEM+XTEMP(J)*A(N)*XP(J)*T
880. IF(ABS(DL/DJ-1.0).GT.1.0E-5)
      1 XTEM=XTEM+XTEMP(J)*A(N)*(XP(J)-XP(L))/(DL-DJ)
881.
882. QUE=A(N)/DJ
883. NQ(L)=0
884. NQ(J)=L
885. NSAVE=0
886. NUX=NONO(J)
887. IF(M.GT.MMN.OR.M.EQ.MZERO) NUX=KD(J)
888. NUF=1
889. IF(J.GT.1) NUF=NONO(J-1)+1
890. IF(NUF.GT.NUX) GO TO 130
891. DO 120 K=NUF,NUX
892. J1=LOC(K)
893. IF(LONG(J1)) GO TO 120
894. KP=J
895. IF(J1.EQ.NQ(KP)) GO TO 120
896. KP=NQ(KP)
897. IF(KP.NE.0) GO TO 30
898. DJ=-D(J1)
899. AKDJQ=A(K)/DJ*QUE
900. IF(AKDJQ.LE.1.0E-06) GO TO 120
901. NQ(J1)=J
902. I=1
903. KP=J1
904.

```

```

905. DD(I)=-D(KP)
906. DXP(I)=XP(KP)
907. KP=NQ(KP)
908. IF(KP.EQ.0) GO TO 50
909. I=I+1
910. IF(I.LE.100) GO TO 40
911. PRINT 9000, M,L,J1,J,AKDJQ
912. FORMAT('1',4I5,E12.5)
913. GO TO 130
914. BATE=0.00
915. I1=I-1
916. XPL=XP(L)
917. C D. R VONDY FORM OF BATEMAN EQUATIONS --- ORNL-TM-361
918. DO 100 KB=1,I1
919. XPJ=DXP(KB)
920. IF(XPL+XPJ.LT.ERR) GO TO 100
921. DK=DD(KR)
922. PROD=(DL/DK-1.0)
923. DKR=PRCD
924. IF(ABS(PROD).GT.1.E-4) GO TO 60
925. PRDD=T*DK*XPJ*(1.0-0.5*(DL-DJ)*T)
926. GO TO 70
927. PRDD=(XPJ-XPL)/PROD
928. PROL=XPJ/DKR
929. PI=1.0
930. S1=2./(DK*T)
931. DO 90 JK=1,I1
932. IF(JK.CQ.KB) GO TO 90
933. S=1.0-DK/DD(JK)
934. IF(ABS(S).GT.1.E-4) GO TO 80
935. USE THIS FORM FOR TWO NEARLY EQUAL HALF-LIVES
936. IF(ABS(DKR).GT.1.0E-4) PROD=PROL
937. S=S1
938. PI=PI*S
939. IF(ABS(PI).GT.1.E25) GO TO 100
940. CONTINUE
941. BATE=BATE+PROD/PI
942. CONTINUE
943. IF(BATE.LT.0.00) PRINT 9001, L,I,BATE,XTEM,XTEMP(J1),AKDJQ
944. FORMAT(' L,I,BATE,XTEM,XTEMP(J1),AKDJQ = ',2I5,1P4E12.5)
945. IF(BATE.LT.0.00) BATE=0.00
946. XTEM=XTEM+XTEMP(J1)*AKDJQ*BATE
947. IF(NSAVE-GE.50) GO TO 120
948. NSAVE=NSAVE+1
949. NQUEUE(NSAVE)=J1
950. QUEUE(NSAVE)=AKDJQ
951. NQU(NSAVE)=J
952. CONTINUE
953. IF(NSAVE.LE.0) GO TO 140
954. J=QUEUE(NSAVE)
955. QUE=QUEUE(NSAVE)
956. NQ(J)=NQU(NSAVE)
957. NSAVE=NSAVE-1
958. GO TO 20
959. CONTINUE
960. IF(LONG(L)) XPAR(L)=XTEM/XP(L)
961. NUL=NONO(L)+1
962. IF(.NOT.LONG(L)) XNEW(M,L)=XTEM+XTEMP(L)*XP(L)
963. CONTINUE
964. DO 170 I=1,ITOT
00014100
00014110
00014120
00014130
00014140
00014150
00014160
00014170
00014180
00014190
00014200
00014210
00014220
00014230
00014240
00014250
00014260
00014270
00014280
00014290
00014300
00014310
00014320
00014330
00014340
00014350
00014360
00014370
00014380
00014390
00014400
00014410
00014420
00014430
00014440
00014450
00014460
00014470
00014480
00014490
00014500
00014510
00014520
00014530
00014540
00014550
00014560
00014570
00014580
00014590
00014600
00014610
00014620
00014630
00014640
00014650
00014660
00014670
00014680
00014690

```

```

965. IF(LONG(I)) XTEMP(I)=XTEMP(I)+XPAR(I)
966. IF(.NOT.LONG(I)) XTEMP(I)=0.0
967. CONTINUE
968. RETURN
969. END
970. SUBROUTINE EQUIL(M,ITOT)
971. C
972. C EQUIL PUTS SHORT-LIVED DAUGHTERS IN EQUILIBRIUM WITH PARENTS
973. C EQUIL USES GAUSS-SEIDEL ITERATION TO GENERATE STEADY STATE
974. C CONCENTRATIONS
975. C
976. LOGICAL*1 LONG
977. INTEGER*2 LOC,NONO,KD
978. COMMON/EG/XZFR0(800),XZH(800),XTEMP(800),XNEW(10,800),
979. 1 B(800),D(800)
980. COMMON/MATRIX/A(2500),LOC(2500),NONO(800),KD(800)
981. COMMON/FLEX/FLUX(10),PMN,MOUT,INDEX,QXN,AXN,ERR,NOBLND,MZERO
982. COMMON/SERIES/XP(800),XPAR(800),LONG(800)
983. DO 10 I=1,ITOT
984. XPAR(I)=0.0
985. IF(.NOT.LONG(I)) GO TO 10
986. XTEMP(I)=XTEMP(I)*XP(I)
987. XPAR(I)=AMAX1(XNEW(M,I)-XTEMP(I),0.0)
988. CONTINUE
989. ITER=1
990. N=0
991. BIG=0.0
992. DO 60 I=1,ITOT
993. NUM=NONO(I)-N
994. DI=-D(I)
995. IF(LONG(I)) GO TO 50
996. XNW=B(I)
997. IF(M.GT.MMN.OR.M.EQ.MZERO) NUM=KD(I)-N
998. IF(NUM.EQ.0) GO TO 31
999. DO 30 K=1,NUM
1000. N=N+1
1001. J=LOC(N)
1002. DJ=-D(J)
1003. XJ=XPAR(J)
1004. IF(LONG(J)) XJ=XJ+XTEMP(J)/(1.0-DJ/DI)
1005. XNW=XNW+A(N)*XJ
1006. CONTINUE
1007. 31 XNW=XNW/DI
1008. IF(XNW.LT.1.0E-50) GO TO 40
1009. ARG=ABS((XNW-XPAR(I))/XNW)
1010. IF(ARG.GT.BIG) BIG=ARG
1011. XPAR(I)=XNW
1012. N=NONO(I)
1013. CONTINUE
1014. IF(BIG.LT.QXN ) GOTO 70
1015. ITER=ITER+1
1016. IF(ITER.LT.100) GO TO 20
1017. PRINT 9000
1018. STOP
1019. DO 80 I=1,ITOT
1020. IF(.NOT.LONG(I)) XNEW(M,I)=XNEW(M,I)+XPAR(I)
1021. 80 CONTINUE
1022. RETURN
1023. 9000 FORMAT(' GAUSS SEIDEL ITERATION DID NOT CONVERGE IN EQUIL')
1024. END

```



```

1145. C      IF(FPI .LT. ERR) GO TO 150
1146. M=M+1
1147. COEFF(M,I)=FPI*COEFF(M-1,I)
1148. NPROD(M,I)=NPROD(M-1,I)+1
1149. COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
1150. C
1151. C      TEST ISOMERIC TRANSITION
1152. C
1153. C      150 IF(FT .LT.ERR) GO TO 160
1154. M=M+1
1155. COEFF(M,I)=FT*A1
1156. NPROD(M,I)=NUCLI
1157. ABETA=ABETA-FT
1158. C
1159. C      TEST ALPHA EMISSION
1160. C
1161. C      160 IF(FA .LT. ERR) GO TO 170
1162. M=M+1
1163. COEFF(M,I)=FA*A1
1164. NPROD(M,I)=NUCLI-20040
1165. M=M+1
1166. COEFF(M,I)=COEFF(M-1,I)
1167. NPROD(M,I)=20040
1168. ABETA=ABETA-FA
1169. C
1170. C      TEST NEGATRON EMISSION
1171. C
1172. C      170 IF(ABETA.LT.1.E-4) GO TO 180
1173. M=M+1
1174. COEFF(M,I)=ABETA*A1
1175. NPROD(M,I)=NUCLI+10000
1176. C
1177. C      TEST NEGATRON EMISSION TO EXCITED STATE OF PRODUCT NUCLIDE
1178. C
1179. C      IF(FBI .LT. ERR)GO TO 180
1180. M=M+1
1181. COEFF(M,I)=FBI*COEFF(M-1,I)
1182. NPROD(M,I)=NPROD(M-1,I)+1
1183. COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
1184. C
1185. C      COMPUTE NEUTRON CAPTURE CROSS SECTIONS IN THREE REGIONS
1186. C
1187. C      180 KAP(I)=M
1188. DO 190 KI=1,6
1189. CAPT(KI) =0.0
1190. CAPT(1)=SIGNA
1191. CAPT(2)=SIGNP
1192. CAPT(4)=SIGNG*FNGL
1193. CAPT(3)=SIGNC-CAPT(4)
1194. CAPT(6)=SIGNP*FNZNI
1195. CAPT(5)=SIGNZN-CAPT(6)
1196. TOCAP(I)=0.0
1197. C      TOTAL NEUTRON CROSS SECTION FOR NUCLIDE(I)
1198. C      DO 220 K=1,6
1199. CAPKI=CAPT(K)
1200. IF(CAPKI.LT.ERR) GO TO 220
1201. M=M+1
1202. NPROD(M,I)=NUCLI+NUCAL(K)
1203. COEFF(M,I)=CAPKI
1204.

```

```

1205. TOCAP(I)=TOCAP(I)+CAPKI
1206. IF(K.NE.1) GO TO 210
1207. M=M+1
1208. COEFF(M,I)=COEFF(M-1,I)
1209. NPROD(M,I)=20040
1210. IF(K.NE.2) GO TO 220
210 M=M+1
1211. COEFF(M,I)=COEFF(M-1,I)
1212. NPROD(M,I)=10010
1213. CONTINUE
1214.
1215. 220 IF(MOD(NUCLI, 10).EQ.0) GO TO 250
1216. DO 240 K=1,M
1217. 240 NPROD(K,I)=NPROD(K,I)-1
1218. 250 PMAX(I)=M
1219. IF(M.GT.7) PRINT 9039, M
1220. DIS(I)=A1
1221. GO TO 40
1222. ILITE = I-1
1223. IACT=0
1224.
1225. C READ DATA ON ACTINIDES
1226. C
1227. C
1228. 270 READ(8,9034,END=450)NUCLI(I),DLAM,IU,FBI,FP,FPI,FT,FA,FSF,
1229. IQ(I),FG(I),DUMMY,WMPK(I),AMPK(I)
1230. DO 280 N=1,NLIBE
1231. 280 CONTINUE
1232. IF(NI.EQ.0) GO TO 300
1233. DO 290 N=1,NI
1234. 290 READ(8,9036) SKIP
1235. 300 IF(IT.EQ.0) GO TO 270
1236. M=0
1237. NUCLI=NUCLI(I)
1238. IF(NUCLI.EQ.0) GO TO 450
1239. DO 320 K=1,5
1240. IF(NUCLI.EQ.NSORS(K)) NSORS(K)=I
1241. CONTINUE
1242. CALL HALF(AI,IU)
1243. CALL NDAH(NUCLI,NAME)
1244. SIGNG=THEM*SIGF+RES*RIF
1245. SIGF=THEM*SIGF+RES*RIF+FAST*SIGFF
1246. SIGN2=N-SIGN2+FAST
1247. SIGN3=N-SIGN3+FAST
1248. IF(MOD(IACT,50).EQ.0) PRINT 9012, (TITLE(N),N=1,18)
1249. IF(MOD(IACT,50).EQ.0) PRINT 9024
1250. PRINT 9026, NAME, DLAM,FBI,FP,FPI,FT,FA,FSF,SIGNG,00017550
1251. 1 FNG1,SIGF,SIGN2N,SIGN3N,Q(I),FG(I)
1252. IACT=IACT+1
1253. C
1254. C TEST RADIOACTIVITY
1255. C
1256. C IF(AL.LT.ERR) GO TO 380
1257. ABETA=1.0
1258. TEST POSITRON EMISSION
1259. IF(FP.LT.ERR) GO TO 350
1260. ABETA=ABETA-FP
1261. M=M+1
1262. COEFF(M,I)=FP*A1
1263. NPROD(M,I)=NUCLI-10000
1264. C POSITRON EMISSION TO EXCITED STATE

```

```

1265. IF(FPI .LT. ERR)GO TO 350
1266. M=M+1
1267. COEFF(M,I)=FPI*COEFF(M-1,I)
1268. NPROD(M,I)=NPROD(M-1,I)+1
1269. COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
1270. ISOMERIC TRANSITION
1271. IF(FT .LT. ERR)GO TO 360
1272. M=M+1
1273. COEFF(M,I)=FT*A1
1274. NPROD(M,I)=NUCLI
1275. ABETA=ABETA-FT
1276. ALPHA EMISSION
1277. IF(FA .LT. ERR)GO TO 370
1278. M=M+1
1279. COEFF(M,I)=FA*A1
1280. NPROD(M,I)=NUCLI-20040
1281. M=M+1
1282. COEFF(M,I)=COEFF(M-1,I)
1283. NPROD(M,I)=20040
1284. ABETA=ABETA-FA
1285. BETA DECAY
1286. IF(ABETA.LT.1.E-4) GO TO 380
1287. M=M+1
1288. COEFF(M,I)=ABETA*A1
1289. NPROD(M,I)=NUCLI+10000
1290. IF(FBI .LT. FRR)GO TO 380
1291. M=M+1
1292. COEFF(M,I)=COEFF(M-1,I)*FBI
1293. COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
1294. NPROD(M,I)=NPROD(M-1,I)+1
1295.
1296.
1297.
1298.
1299.
1300.
1301.
1302.
1303.
1304.
1305.
1306.
1307.
1308.
1309.
1310.
1311.
1312.
1313.
1314.
1315.
1316.
1317.
1318.
1319.
1320.
1321.
1322.
1323.
1324.
C 350 IF(FPI .LT. ERR)GO TO 350
M=M+1
COEFF(M,I)=FPI*COEFF(M-1,I)
NPROD(M,I)=NPROD(M-1,I)+1
COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
ISOMERIC TRANSITION
IF(FT .LT. ERR)GO TO 360
M=M+1
COEFF(M,I)=FT*A1
NPROD(M,I)=NUCLI
ABETA=ABETA-FT
ALPHA EMISSION
IF(FA .LT. ERR)GO TO 370
M=M+1
COEFF(M,I)=FA*A1
NPROD(M,I)=NUCLI-20040
M=M+1
COEFF(M,I)=COEFF(M-1,I)
NPROD(M,I)=20040
ABETA=ABETA-FA
BETA DECAY
IF(ABETA.LT.1.E-4) GO TO 380
M=M+1
COEFF(M,I)=ABETA*A1
NPROD(M,I)=NUCLI+10000
IF(FBI .LT. FRR)GO TO 380
M=M+1
COEFF(M,I)=COEFF(M-1,I)*FBI
COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
NPROD(M,I)=NPROD(M-1,I)+1
C 360 IF(FA .LT. ERR)GO TO 370
M=M+1
COEFF(M,I)=FA*A1
NPROD(M,I)=NUCLI-20040
M=M+1
COEFF(M,I)=COEFF(M-1,I)
NPROD(M,I)=20040
ABETA=ABETA-FA
BETA DECAY
IF(ABETA.LT.1.E-4) GO TO 380
M=M+1
COEFF(M,I)=ABETA*A1
NPROD(M,I)=NUCLI+10000
IF(FBI .LT. FRR)GO TO 380
M=M+1
COEFF(M,I)=COEFF(M-1,I)*FBI
COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
NPROD(M,I)=NPROD(M-1,I)+1
C 370 IF(ABETA.LT.1.E-4) GO TO 380
M=M+1
COEFF(M,I)=ABETA*A1
NPROD(M,I)=NUCLI+10000
IF(FBI .LT. FRR)GO TO 380
M=M+1
COEFF(M,I)=COEFF(M-1,I)*FBI
COEFF(M-1,I)=COEFF(M-1,I)-COEFF(M,I)
NPROD(M,I)=NPROD(M-1,I)+1
C 380 KAP(I)=M
DO 39C K=1,6
CAPT(K )=0.0
CAPT(2)=SIGNG*FNG1
CAPT(1)=SIGNG-CAPT(2)
CAPT(4)=SIGN2N*FN2N1
CAPT(3)=SIGN2N-CAPT(4)
FISS(IACT)=SIGF
TOCAP(I)=0.0
DO 410 K=1,4
CAPKI=CAPT(K)
IF(CAPKI.LT.ERR) GO TO 410
M=M+1
TOCAP(I)=TOCAP(I)+CAPKI
COEFF(M,I)=CAPKI
NPROD(M,I)=NUCLI+NUCAL(K+2)
CONTINUE
TOCAP(I)=TOCAP(I)+FISS(IACT)
N-3N CROSS SECTION
A17=SIGV3N
IF(A17.LT.ERR) GO TO 420
M=M+1
COEFF(M ,I)= A17
NPROD(M ,I)= NUCLI-20
TOCAP(I)=TOCAP(I)+A17
IF(MOD(NUCLI,10).EQ.0) GO TO 440
DO 430 K=1,M

```

```

1325.
1326.
1327.
1328.
1329.
1330.
1331.
1332.
1333.
1334.
1335.
1336.
1337.
1338.
1339.
1340.
1341.
1342.
1343.
1344.
1345.
1346.
1347.
1348.
1349.
1350.
1351.
1352.
1353.
1354.
1355.
1356.
1357.
1358.
1359.
1360.
1361.
1362.
1363.
1364.
1365.
1366.
1367.
1368.
1369.
1370.
1371.
1372.
1373.
1374.
1375.
1376.
1377.
1378.
1379.
1380.
1381.
1382.
1383.
1384.

430 NPROD(K,I)=NPROD(K,I)-1
440 MMAX(I)=M
    IF(M.GT.7) PRINT 9039, M
    SPONF(I,ACTI)=FSF*A1*6.023E23
    ALPHAM(I,ACTI)=FA*A1*6.023E13*Q(I)**3.65
    DIS(I)=A1
    I=I+1
    GO TO 270
450 IL=0
    DO 460 K=1,5
460 TYLD(K)=0.0
C
C READ DATA FOR FISSION PRODUCTS
470 READ(8,9034,FND=690)NUCL(I),DLAM,IU,FBI,FP,FPI,FT,FA,FSF,
    IQ(I),FG(I),DUMMY,WPC(I),AMPC(I)
    DO 480 N=1,NLIBE
480 READ(8,9038) SIGNG,RING,FNGI,Y,IT
    IF(NL.EQ.0) GO TO 500
    DO 490 N=1,N1
490 READ(8,9036) SKIP
500 IF(IT.EQ.0) GO TO 470
510 M=0
    CALL HALF(A1,IU)
520 NUCL=NUCL(I)
    IF(NUCL.EQ.0) GO TO 690
    CALL NOAH(NUCL,NAME)
    IF(MOD(IL,50).EQ.0) PRINT 9012, (TITLE (N),N=1,18)
    SIGNG=THERM*SIGNG+RES*RING
530 IF(MOD( IL ,50).EQ.0) PRINT 9019
    PRINT 9021, NAME,
    DLAM,FBI,FP,FPI,FT,SIGNG,
    FNGI,Y,0(I),FG(I)
    GO TO 550
540 IF(MOD(IL,50).EQ.0) PRINT 9020
    PRINT 9022, NAME ,
    DLAM,FBI,FP,FPI,FT,SIGNG,FNGI,
    Y(2),Y(4),Y(5),Q(I),FG(I)
C
C TEST RADIOACTIVITY
550 IF(A1.LT.ERR) GO TO 600
    ABETA=1.0
    POSITRGN EMISSION
    A3=FP
    IF(A3.LT.ERR) GO TO 570
    ABETA=ABETA-A3
    API=A3*FPI
    AP=A3-API
    IF(AP.LT.ERR) GO TO 560
    M=M+1
    COEFF(M,I)=AP*A1
560 NPROD(M,I)=NUCL-10000
    IF(API.LT.ERR) GO TO 570
    M=M+1
    COEFF(M,I)=API*A1
    NPROD(M,I)=NUCL-9999
C ISOMERIC TRANSITION
570 IF(FT.LT.ERR) GO TO 580
    M=M+1
    COEFF(M,I)=FT*A1

```

```

1385. NPROD(M,I)=NUCLI
1386. ABETA=ABETA-FT
1387. NEGATRON EMISSION
1388. IF(ABETA.LT.1.0E-4) GO TO 600
1389. A2=FBI
1390. AB1=ABETA*A2
1391. AB=ABETA-AB1
1392. IF(AB.LT.1.E-4) GO TO 590
1393. M=M+1
1394. COEFF(M,I)=AR*A1
1395. NPROD(M,I)=NUCLI+1C000
1396. IF(AB1.LT.1.E-6) GO TO 600
1397. M=M+1
1398. COEFF(M,I)=AR1*A1
1399. NPROD(M,I)=NUCLI+1C001
1400.
1401.
1402.
1403.
1404.
1405.
1406.
1407.
1408.
1409.
1410.
1411.
1412.
1413.
1414.
1415.
1416.
1417.
1418.
1419.
1420.
1421.
1422.
1423.
1424.
1425.
1426.
1427.
1428.
1429.
1430.
1431.
1432.
1433.
1434.
1435.
1436.
1437.
1438.
1439.
1440.
1441.
1442.
1443.
1444.

C 580
IF(ABETA.LT.1.0E-4) GO TO 600
A2=FBI
AB1=ABETA*A2
AB=ABETA-AB1
IF(AB.LT.1.E-4) GO TO 590
M=M+1
COEFF(M,I)=AR*A1
NPROD(M,I)=NUCLI+1C000
IF(AB1.LT.1.E-6) GO TO 600
M=M+1
COEFF(M,I)=AR1*A1
NPROD(M,I)=NUCLI+1C001

C
C NEUTRON CAPTURE CRGSS SECTIONS FOR FISSION PRODUCTS USING THREE
C REGION APPROXIMATION
C
600 KAP(I)=M
DO 610 K=1,6
CAPT(K)=0.0
CAPT(2)=SIGNG*FNGL
CAPT(1)=SIGNG-CAPT(2)
TOCAP(I)=0.0
DO 620 K=1,2
CAPKI=CAPT(K)
IF(CAPKI.LT.ERR) GO TO 620
M=M+1
TOCAP(I)=TOCAP(I)+CAPKI
COEFF(M,I)=CAPKI
NPROD(M,I)=NUCLI+NUCAL(K+2)
620 CONTINUE
630 IF(MOD(NUCLI,10).EQ.0) GO TO 650
DO 640 K=1,M
NPROD(K,I)=NPROD(K,I)-1
IL=IL+1
DO 660 J=1,5
YJ=Y(J)*0.010
TYLD(J)=TYLD(J)+YJ
YIELD(J,IL)=YJ
IF(NLIBE.EQ.1.OR.NLIBE.EQ.4) GO TO 680
IF(NLIBE.EQ.3) YIELD(1,IL)=YJ
YIELD(3,IL)=YJ
680 MMAX(I)=M
IF(M.GT.7) PRINT 9039, M
DIS(I)=A1
I=I+1
GO TO 470
IFP=IL
690
C
C ALL DATA ON NUCLIDES HAS BEEN READ, BEGIN TO COMPUTE MATRIX COEFF
C
ITOT=I-1
C
C FIND PRODUCT NUCLIDES FOR REACTIONS OF LIGHT ELEMENTS
C
NON=0
DO 700 K=1,ITOT
NON(K)=0
700

```


1625. TEMPERATURE,,/,, N-2N - SIGN2N * FAST,,36X,,RES = RATIO 00021300
 1626. 80F RESONANCE FLUX PER LETHARGY UNIT TO THERMAL FLUX,,/,, 00021310
 9008 FORMAT,, N-ALPHA - SIGNA * THERM + RINA * RES + SIGNAF * FAST00021320
 1627. 1,,7X,,FAST = 1.45 * RATIO OF FAST (GT 1.0 MEV) TO THERMAL FLUX '00021330
 1628. 2/,, N-PROTON - SIGNP * THERM + RINP * RES + SIGNPF * FAST,,/,, 00021340
 1629. 9009 FORMAT,,59X,,REFERENCES,,/,, HALF LIVES, DECAY SCHEMES, AND 00021350
 1630. THERMAL POWERS,,/,, C M LEDERER, J M HOLLANDER, AND I PERLMAN,,TAB00021360
 1631. 2LE OF ISOTOPES - SIXTH EDITION,, JOHN WILEY AND SONS, INC (1967),,00021370
 1632. 3/,, B S DZHELEPOV AND L K PEKER,,DECAY SCHEMES OF RADIOACTIVE NUC00021380
 1633. 4LEI,, PERGAMON PRESS (1961),,/,, D T GOLDMAN AND JAMES R ROSSER '00021390
 1634. 5'CHART OF THE NUCLIDES,, NINTH EDITION GENERAL ELECTRIC CO (JULY 00021400
 1635. 61966),,/,, E D ARNOLD,,PROGRAM SPECTRA,, APPENDIX A OF ORNL-3576 00021410
 1636. 7(APRIL 1964),, 00021420
 9010 FORMAT,, CROSS SECTIONS AND FLUX SPECTRA,,/,, B E PRINCE,,NEUT00021430
 1638. IRON REACTION RATES IN THE MSRE SPECTRUM,, ORNL-4119, PP 79-83 (JUL00021440
 1639. 2Y 1967),,/,, B E PRINCE,,NEUTRON ENERGY SPECTRA IN MSRE AND MS8R,00021450
 1640. 3. ORNL-4191, PP 50-58 (DEC 1967),,/,, M D GOLDBERG ET AL,,NEUTRON00021460
 1641. 4 CROSS SECTIONS,, BNL-325, SECOND ED, SUPP NO 2 (MAY 1964 - AUG 1900021470
 1642. 566) ALSO EARLIER EDITIONS,,/,, H T KERR, UNPUBLISHED ERC COMPILATIO00021480
 1643. 6ON (FEB 1968),,/,, M K DRAKE,,A COMPILATION OF RESONANCE INTEGRAL00021490
 1644. 7S,, NUCLEONICS, VOL 24, NO 8, PP 108-111 (AUG 1966),,/,, BNWL STAF00021500
 1645. 8F,, INVESTIGATION OF N-2N CROSS SECTIONS,, BNC-98, PP 44-98 (JUNE00021510
 1646. 9 1965),, 00021520
 1647. 9011 FORMAT,,18A4,,13) 00021530
 1648. 9012 FORMAT,,1H1,,20X,,18A4) 00021540
 1649. 9013 FORMAT,, H ALTER AND C E WEBER,,PRODUCTION OF H AND HE IN METALS 00021550
 1650. 1DURING REACTOR IRRADIATION,, J NUCL MATLS, VOL 16, PP 68-73 (1965)00021560
 1651. 2,,/,, L L BENNETT,,RECOMMENDED FISSION PRODUCT CHAINS FOR USE IN 00021570
 1652. 3REACTOR EVALUATION STUDIES,, ORNL-TM-1658 (SEPT 1966),, 00021580
 9014 FORMAT,, FISSION PRODUCT YIELDS,,/,, M E MEER AND B F RIDER,, '00021590
 1653. 1SUMMARY OF FISSION PRODUCT YIELDS FOR U-235, U-238, PU-239, AND PU00021600
 1654. 2-241 AT THERMAL, FISSION SPECTRUM AND,,/,, 14 MEV NEUTRON ENERGI00021610
 1655. 3ES,, APED-5308-(REV.), (OCT. 1968),,/,, S KATCOFF,, FISSION PRODUCT00021620
 1656. 4YIELDS FROM NEUTRON INDUCED FISSION,, NUCLEONICS, VOL 18, NO 11, 00021630
 1657. 5NOV 1960),,/,, H O EUBEY,, REVIEW OF LOW-MASS ATOM PRODUCTION IN F00021640
 1658. 6AST REACTORS,, ANL-7434, (APRIL 1968) ' 00021650
 9015 FORMAT,,1H0,,20X,,LIGHT ELEMENTS, MATERIALS OF CONSTRUCTION, AND ACT00021660
 1660. 1IVATION PRODUCTS,,/,, '0 NUCL DLAM FBI FP FPI FT F00021670
 1661. 2A SIGH FNGI FNA FNP RITH FINA FINP SIGMEV FN2N100021680
 1662. 3 FFNA FFNP Q FG) 00021690
 9016 FORMAT,,1H0,,20X,,LIGHT ELEMENTS, MATERIALS OF CONSTRUCTION, AND ACT00021700
 1664. 1IVATION PRODUCTS,,/,, '0 NUCL DLAM FBI FP FPI FT F00021710
 1665. 2A SIGH FNGI FNGI SIGN2N FN2N1. SIGNA SIGNP Q FG ABU00021720
 1666. 3NDANCE') 00021730
 9017 FORMAT,,1H ,A2,,I3,,A1,,1PE9.2,0P5F6.3,1PE9.2,0P3F6.3,1PE9.2,0P2F6.3, 00021740
 1667. 1PE9.2,0P4F6.3,0PF5.2) 00021750
 9018 FORMAT,,1H0,,10X,,THERM= 'F10.5,5X,,RES= 'F10.5,5X,,FAST= 'F10.5, 00021760
 1668. 1//,,1X,,NEUTRON SOURCE= '5(110,5X),5X,,NLIBE= '13) 00021770
 9019 FORMAT,,1H0,,36X,,FISSION PRODUCTS,,/,, '0 NUCL DLAM FBI FP 00021780
 1669. 1 FPI FT SIGNG,,/,, FNGI Y23 Y25 Y02 Y00021790
 1670. 228 Y49 Q FG) 00021800
 9020 FORMAT,,1H0,,36X,,FISSION PRODUCTS,,/,, '0 NUCL DLAM FBI FP 00021810
 1671. 1 FPI FT SIGNG FNGI Y28 Y49 Q FG) 00021820
 9021 FORMAT,,1H ,A2,,I3,,A1,,1PE9.2,0P4F6.3, 1PE9.2,0PF6.3,1P5E9.2, 00021830
 1672. 10P2F6.3) 00021840
 9022 FORMAT,,1H ,A2,,I3,,A1,,1PE9.2,0P4F6.3,1PE9.2,0PF6.3,1P3E9.2,0P2F6.3) 00021850
 1673. 9023 FORMAT,,1H0,,32X,,ACTINIDES AND THEIR DAUGHTERS,,// 00021860
 1674. 1' NUCL DLAM FBI FP FPI FT FA FSF E+6 SIGNG R00021870
 1675. 2ING FNGI SIGF RIF SIGFF SIGN2N SIGN3N Q FG)00021880
 9024 FORMAT,,1H0,,32X,,ACTINIDES AND THEIR DAUGHTERS,,// 00021890

```

1685. 1, NUCL DLAM FBI FP FPI FT FA FSF E+6 SIGNG F00021900
1686. ZNG21 SIGF SIGM2N SIGN3N Q FG* 00021910
1687. 9025 FORMAT(1H, A2, I3, A1, 1PE9.2, 0P5F6.3, 6PF6.1, 1P2E9.2, 0PF6.3, 1P5E9.2, 00021920
1688. 0PF6.3, F5.2) 00021930
1689. 9026 FORMAT(1H, A2, I3, A1, 1PE9.2, 0P5F6.3, 6PF9.1, 1PE9.2, 0PF6.3, 1P3E9.2, 00021940
1690. 0PF7.3, F5.2) 00021950
1691. 9027 FORMAT('OSUM OF YIELDS OF ALL FISSION PRODUCTS =' , I5X, 1P3E9.2) 00021960
1692. 9028 FORMAT(15, 2X, 1PE10.3, 3X, E10.3, 5(2X, E10.3, 3X, 15)/(30X, 5(2X, E10.3, 00021970
1693. 3X, 15))) 00021980
1694. 9029 FORMAT('NON-ZERO MATRIX ELEMENTS AND THEIR LOCATIONS' / 00021990
1695. 1, I DIS(I) J CAP(I) A(I, J) J A(I, J) 00022000
1696. 2J A(I, J) J CAP(I) A(I, J) J A(I, J) J ' 00022010
1697. 9030 FORMAT(64HOSUM OF YIELDS OF ALL FISSION PRODUCTS 00022020
1698. 1, 1P5F9.2) 00022030
1699. 9031 FORMAT(5110) 00022040
1700. 9032 FORMAT(16, F5.3, I1, 5F3.3, E5.2, 3F3.3, E5.2, 2F3.3, E5.2, 3F3.3, F4.3, F3, 300022050
1701. 1, F6.4) 00022060
1702. 9033 FORMAT(1H, A2, I3, A1, 1PE9.2, 0P5F6.3, 1PE9.2, 0PF6.3, 1PE9.2, 0PF6.3, 00022070
1703. 1PE9.2, 0P2F6.3, F7.3) 00022080
1704. 9034 FORMAT(17, F9.3, I1, 5F5.3, 1PE9.2, 0P2F5.3, F7.3, 2E6.0) 00022090
1705. 9035 FORMAT(7X, F9.2, 3F5.3, F9.2, 2F5.3, F9.2, 3F5.3, 5X, I1) 00022100
1706. 9036 FORMAT(20A4) 00022110
1707. 9037 FORMAT(7X, 2F9.2, F5.3, 4F9.2, F4.3, F9.2, I1) 00022120
1708. 9038 FORMAT(7X, 2F9.2, F5.3, 5F9.2, 4X, I1) 00022130
1709. 9039 FORMAT('O WARNING, MOUT OF RANGE IN NUDATA, =' I5) 00022140
1710. 9040 FORMAT( 7X, F9.2, 3F8.6, F4.2, 2F3.1, F9.2, 3F5.3, 5X, I1) 00022150
1711. 9041 FORMAT('O NUN HAS EXCEEDED 2500, EQUAL TO '216) 00022160
1712. END 00022170
1713. SUBROUTINE COLLECT(TMB, CWASTE, ILITE, ITOT) 00022180
1714. COMMON/EQ/XZERO(800), XZH(800), XTEMP(800), XNEW(10, 800), 00022190
1715. 1 B(800), D(800) 00022200
1716. DIMENSION CWASTE(800) 00022210
1717. IF(TMB.LT.1) RETURN 00022220
1718. DO 10 I=1, ITOT 00022230
1719. B(I)=CWASTE(I) 00022240
1720. XTEMP(I)=0.0 00022250
1721. CALL DECAY(I, TMB, ITOT) 00022260
1722. CALL TERM(TMB, I, J, ITOT) 00022270
1723. CALL EQUIL(I, ITOT) 00022280
1724. DO 20 I=1, ITOT 00022290
1725. CWASTE(I)=XNEW(I, I)/TMB 00022300
1726. RETURN 00022310
1727. END 00022320
1728. SUBROUTINE STORAG(TMB, CWASTE, ILITE, ITOT) 00022330
1729. COMMON/EQ/XZFRO(900), XZH(800), XTEMP(800), XNEW(10, 800), 00022340
1730. 1 B(800), D(800) 00022350
1731. DIMENSION CWASTE(ITOT) 00022360
1732. IF(TMB.LT.1) RETURN 00022370
1733. DELT=TMB 00022380
1734. DO 10 I=1, ITOT 00022390
1735. B(I)=0.0 00022400
1736. XTEMP(I)=CWASTE(I) 00022410
1737. CALL DECAY(I, DELT, ITOT) 00022420
1738. CALL TERM(TMB, I, ILITE, ITOT) 00022430
1739. CALL EQUIL(I, ITOT) 00022440
1740. DO 20 I=1, ITOT 00022450
1741. CWASTE(I)=XNEW(I, I) 00022460
1742. RETURN 00022470
1743. END 00022480
1744. C PROGRAM BLOCK DATA 00022490

```

```

1745. BLOCK DATA
1746. INTEGER*2 ELE(99),STA(2)
1747. COMMON/LABEL/ ELE,STA
1748. DATA ELF/,H,HE,LI,Be,,B,,C,,N,,O,,F,,NE,,NA,,M00022530
1749. IG,AL,SI,P,S,CL,AR,K,CA,SC,TI,V,CR,MN00022540
1750. 2,FE,CO,NI,CU,ZN,GAG,GE,AS,SE,BR,KR,SR,00022550
1751. 3,Y,ZR,NB,KC,TC,RU,RH,PD,AG,CD,IN,SN,SB,00022560
1752. 4,TE,I,XE,CS,BA,LA,CE,PR,ND,PM,SM,EU,GD,00022570
1753. 5,TB,DY,HG,ER,TM,YB,LU,HF,TA,W,RE,OS,IR,P00022580
1754. 6,T,AU,HG,TL,Pb,BI,PO,AT,ARN,FR,RA,AC,TH,PA00022590
1755. 7,U,NP,PU,AM,CM,BK,CF,ES//
1756. DATA STA/,,M //
1757. END
1758. SUBROUTINE HALF(A,I)
1759. SUBROUTINE HALF CONVERTS HALF-LIFE TO DECAY CONSTANT (1/SEC)
1760. DIMENSION C(9)
1761. DATA C/6.9315E-01,1.1552E-02,1.9254E-04,8.0226E-08,0.0,00022660
1762. 1 2.1965E-11,2.1965E-14,2.1965E-17/
1763. IF(A.GT.0.0) GO TO 10
1764. IF(1.EQ.6) GO TO 20
1765. A=9.99
1766. RETURN
1767. A=C(I)/A
1768. RETURN
1769. A=0.0
1770. RETURN
1771. END
1772. SUBROUTINE NOAH(NUCL,NAME)
1773. SUBROUTINE NOAH CONVERTS SIX DIGIT IDENTIFIER TO ALPHAMERIC SYMBOL 00022780
1774. INTEGER*2 NAME(3)
1775. INTEGER*2 ELE(99),STA(2)
1776. COMMON/LABEL/ ELE,STA
1777. IS=MOD(NUCL,10)+1
1778. NZ =NUCL/1000
1779. MW=NUCL/10-NZ *1000
1780. NAME(1)=ELE(NZ)
1781. NAME(2)=MW
1782. NAME(3)=STA(IS)
1783. RETURN
1784. END
00022500
00022510
00022520
00022530
00022540
00022550
00022560
00022570
00022580
00022590
00022600
00022610
00022620
00022630
00022640
00022650
00022660
00022670
00022680
00022690
00022700
00022710
00022720
00022730
00022740
00022750
00022760
00022770
00022780
00022790
00022800
00022810
00022820
00022830
00022840
00022850
00022860
00022870
00022880
00022890

```


APPENDIX D

COMPUTER PROGRAMS FOR DOSE PARAMETERS

The following computer programs provide the NRC staff method for calculating various parameters used in the Technical Specifications.

PARTS, a computer program to calculate technical specification dose parameters for the iodine and particulate portions of gaseous effluents; available from the Radiological Assessment Branch of the Nuclear Regulatory Commission, Washington, D.C. 20555.

RABFIN, a computer program to calculate technical specification dose parameters for the noble gas portion of gaseous effluents; available from the Radiological Assessment Branch of the Nuclear Regulatory Commission, Washington, D.C. 20555.

LADTAP, a computer program to calculate the doses from radioactive effluents released to the hydrosphere; the program and a modification to calculate the technical specification dose parameters for radionuclides in liquid effluents is available from the Radiological Assessment Branch of the Nuclear Regulatory Commission, Washington, D.C. 20555.

The remainder of this Appendix contains the PARTS program listing, and modifying the routines to the LADTAP Code.

LADTAP MODIFICATIONS

An option has been added to the staff's code LADTAP to tabulate the A_{i_T} factors of Section 4.3.1 of this manual. Listed below are the routines which have been modified, the changes are indicated by the 'CHANGE 1' notation in columns 73-80.

To execute this option the standard LADTAP input is prepared with the following departures:

1. The 50 mile population (card 3 of the LADTAP input deck) should be set negative.
2. The release of all nuclides should be set to one Ci/yr.

Only the input data defining the ALARA determination of LADTAP is necessary. Following the input data deck structure of Enclosure 1 to the LADTAP program, data beyond card number 7 need not be prepared for the option.

PARTS INPUT DECK TABLE

Card No.	Format	Variable	Columns	Description
1	20 A4	Name	1-80	Plant Title Card, Name, Docket No. and Plant Type
2	F 5.2	H	1-5	Humidity absolute (default value = 8.0 gr/m ³)
	F 5.2	YL	6-10	Yield of leafy vegetables for human consumption (default value = 2.0 Kg/m ²)
	F 5.2	YV	11-15	Yield of produce other than leafy vegetables (default value = 2.0 Kg/m ²)
	F 5.2	YP	16-20	Agricultural productivity of animal pasture feed (default value = 0.70 Kg/m ² wet weight)
	F 5.2	YC	21-25	Agricultural crop productivity of animal feed other than pasture grass (default value = 2.0 Kg/m ² wet weight)
	F 5.2	QC	26-30	Milk cow and beef cattle consumption rate for feed or forage (default value = 50 Kg/day wet weight)
	F 5.2	QG	31-35	Goat consumption rate for feed or forage (default value = 6 Kg/day wet weight)
	E 8.2	DOQ	41-48	Annual average relative deposition rate (D/Q), determined for a specific plant airborne release and site location (meters ⁻²) (Optional Use For Reference & Information Only)
3	I1	IAGE	1	Identification of Controlling Age Group (1 is Infant, 2 is Child, 3 is Teen and 4 is Adult)
	I1	IORG	2	Identification of Controlling Organ (1 is Thyroid, 2 is the critical organ)
	6A4	ZLOC	3-26	Receptor Location identification, Name, Compass Sector and Distance
	F 5.0	GF	27-31	Fraction of yr. humans are exposed to ground surface radiation (default value = 1.0)
	F 5.0	ZIN	32-36	Annual occupancy factor for the inhalation pathway (default value = 1.0)
	F 5.0	FV	37-41	Fraction of yr. leafy vegetables are grown (default value = 1.0)
	F 5.0	FP	42-46	Fraction of yr. cows are on pasture (default value = 1.0)
	F 5.0	FG	47-51	Fraction of produce from local garden (default value = 0.76)
	F 5.0	FPF	52-56	Fraction of daily intake of cows derived from pasture while on pasture (default value = 1.0)
	F 5.0	FGT	57-61	Fraction of yr. goats are on pasture (default value = 1.0)
	F 5.0	FPG	62-66	Fraction of daily intake of goat from pasture while on pasture (default value = 1.0)
	F 5.0	FB	67-71	Fraction of yr. beef cattle are on pasture (default value 1.0)
	F 5.0	FBF	72-76	Fraction of daily intake of beef cattle derived from pasture while on pasture (default value = 1.0)


```

1  HL'CK DATA BLKDAT
   COMMUN/ELEMEN/IELEM(100)
   INTEGER IELEM
   COMMUN/POPUL/PERA,PERT,PEKC,US
   DATA IELEM/
5  1' H , ' P H E , ' L I , ' R E , ' B , ' I C , ' I N , ' I O , ' F , ' I N E , ' I N A , ' M G , ' A L , '
   2' S I , ' P , ' S , ' C L , ' A H , ' K , ' C A , ' S C , ' T I , ' V , ' C R I , ' M N , ' F E , '
   3' C H , ' M I , ' C U , ' Z N , ' G A , ' G E , ' P A S , ' S E , ' B R , ' K R I , ' R R I , ' S H , ' Y , '
4' Z R , ' H R , ' M O , ' T C , ' R U , ' R H , ' P D , ' A G , ' C D , ' I N , ' S N , ' S B , ' T E , '
10  S I , ' X E , ' C S , ' B A , ' L A , ' C E , ' P R , ' I N D I , ' P H , ' S H , ' E U , ' G D , ' T B , '
   6' D Y , ' H U , ' E R , ' T M , ' Y B , ' L U , ' H F , ' T A , ' M , ' R E , ' O S , ' I R , ' P T , '
   7' A U , ' M G , ' I L , ' P B , ' B I , ' P U , ' A T , ' R N , ' F R , ' R A , ' A C , ' T H , ' P A , '
   8' U , ' N P , ' I P U , ' A M , ' C M , ' B K , ' C F , ' E S , ' F M , '
   DATA PERA/0.66/
   DATA PERT/0.14/
   DATA PERC/0.20/
   DATA US/2.6E+08/
   END
15

```

```

BLKDAT 2
BLKDAT 3
BLKDAT 4
BLKDAT 5
BLKDAT 6
BLKDAT 7
BLKDAT 8
BLKDAT 9
BLKDAT 10
BLKDAT 11
BLKDAT 12
BLKDAT 13
BLKDAT 14
BLKDAT 15
BLKDAT 16
BLKDAT 17
BLKDAT 18
BLKDAT 19

```

```

1 1 PROGRAM LADTAP(INPUT,OUTPUT,TAPE5=INPUT,TAPE6=OUTPUT,TAPE10)
COMMON Q(200),PL,CFS,NSUR,LT,RECU(200),LIST(200,4),LCT,LZ ,CON,KIT
+ ,PIP
5 COMMON/PUPUL/PERA,PERT,PERC,US
DIMENSION FACCF(100),FACCI(100),FACCA(100),SACCF(100),SACCI(100),
+ SACCA(100)
DIMENSION IITITLE(20)
DATA FACCF/
9.0E-01,1.0E+00,5.0E-01,2.0E+00,2.2E-01,4.6E+03,1.5E+05,9.2E-01,
+ 1.0E+01,1.0E+00,1.0E+02,5.0E+01,1.0E+01,2.5E+00,1.0E+05,7.5E+02,
+ 5.0E+01,1.0E+00,1.0E+03,4.0E+01,2.0E+00,1.0E+03,1.0E+01,2.0E+02,
+ 4.0E+02,1.0E+02,5.0E+01,1.0E+02,5.0E+03,3.3E+02,3.3E+03,
+ 1.0E+02,1.7E+02,4.2E+02,1.0E+00,2.0E+03,3.0E+01,2.5E+01,3.3E+00,
+ 3.0E+04,1.0E+01,1.5E+01,1.0E+01,1.0E+01,1.0E+01,2.3E+00,2.0E+02,
+ 1.0E+05,3.0E+03,1.0E+00,4.0E+02,1.5E+01,1.0E+00,2.0E+03,4.0E+00,
+ 2.5E+01,1.0E+00,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,
+ 2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,3.5E+00,
+ 3.0E+04,1.2E+03,1.2E+02,1.0E+01,1.0E+01,1.0E+02,3.3E+01,1.0E+03,
+ 1.0E+04,1.0E+02,1.5E+01,5.0E+02,1.5E+01,1.0E+00,4.0E+02,5.0E+01,
+ 2.5E+01,3.0E+01,1.5E+01,2.0E+00,1.0E+01,3.5E+00,2.5E+01,2.5E+01,
+ 2.5E+01,2.5E+01,1.0E+01,1.0E+01/
DATA FACCI/
9.0E-01,1.0E+00,4.0E+01,1.0E+01,5.0E+01,9.1E+03,1.5E+05,9.2E-01,
+ 1.0E+02,1.0E+00,2.0E+02,1.0E+02,6.3E+01,2.5E+01,2.0E+04,1.0E+02,
+ 1.0E+02,1.0E+00,8.3E+02,3.3E+02,1.0E+03,3.0E+03,3.0E+03,2.0E+03,
+ 9.0E+04,3.0E+03,2.0E+02,1.0E+02,4.0E+02,1.0E+04,6.7E+02,3.3E+01,
+ 4.0E+01,1.7E+02,3.3E+02,1.0E+00,1.0E+03,1.0E+02,1.0E+03,6.7E+00,
+ 1.0E+02,1.0E+01,5.0E+00,3.0E+02,3.0E+02,3.0E+02,7.7E+02,2.0E+03,
+ 1.0E+05,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,
+ 1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,6.7E+00,
+ 6.7E+02,1.0E+01,6.0E+01,3.0E+02,3.0E+02,3.0E+02,5.0E+01,1.0E+05,
+ 1.5E+04,1.0E+02,2.4E+01,2.0E+04,5.0E+00,1.0E+00,1.0E+02,2.5E+02,
+ 1.0E+03,5.0E+02,1.1E+02,6.0E+01,4.0E+02,1.0E+02,1.0E+03,1.0E+03,
+ 1.0E+03,1.0E+03,1.0E+02,1.0E+02/
DATA FACCA/
9.0E-01,1.0E+00,3.0E+00,2.0E+01,2.2E+00,4.6E+03,1.3E+04,9.2E-01,
+ 2.0E+00,1.0E+00,5.0E+02,1.0E+02,4.2E+02,1.3E+02,5.0E+05,1.0E+02,
+ 5.0E+01,1.0E+00,6.7E+02,1.3E+02,1.0E+04,3.0E+02,1.0E+02,4.0E+03,
+ 1.0E+04,1.0E+03,2.0E+02,5.0E+01,2.0E+03,2.0E+04,1.7E+03,3.3E+01,
+ 3.0E+03,1.0E+03,5.0E+01,1.0E+00,1.0E+05,5.0E+02,5.0E+03,1.0E+03,
+ 8.0E+02,1.0E+03,4.0E+01,2.0E+03,2.0E+02,2.0E+02,2.0E+02,1.0E+03,
+ 1.0E+05,1.0E+02,1.5E+03,1.0E+02,4.0E+01,1.0E+00,5.0E+02,5.0E+02,
+ 5.0E+03,4.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,
+ 8.0E+02,1.2E+03,2.4E+02,2.0E+02,2.0E+02,2.0E+02,3.3E+01,1.0E+03,
+ 1.0E+05,2.0E+02,2.4E+01,2.0E+03,4.0E+01,1.0E+00,6.0E+01,2.5E+03,
+ 5.0E+03,1.5E+03,1.1E+03,5.0E+01,3.0E+02,3.5E+02,5.0E+03,5.0E+03,
+ 5.0E+03,5.0E+03,1.0E+03,1.0E+03/
DATA SACCF/
9.0E-01,1.0E+00,5.0E-01,2.0E+02,7.7E-01,1.0E+01,1.0E+03,6.0E+04,9.6E-01,
+ 3.0E+00,1.0E+00,6.7E-02,7.7E-01,1.0E+01,1.0E+03,2.0E+04,1.7E+00,
+ 1.3E-02,1.0E+00,1.1E+01,5.0E+01,2.0E+00,1.0E+03,1.0E+01,4.0E+02,
+ 5.5E+02,3.0E+03,1.0E+02,1.0E+02,6.7E+02,2.0E+03,3.3E+02,3.3E+03,
+ 3.3E+02,4.0E+03,1.5E+02,1.0E+00,8.3E+00,2.0E+00,2.5E+01,2.0E+02,
+ 3.0E+04,1.0E+01,1.0E+01,3.0E+00,1.0E+01,1.0E+01,3.3E+03,3.0E+03,
+ 1.0E+05,3.0E+03,4.0E+03,1.0E+01,1.0E+01,1.0E+00,4.0E+01,1.0E+01,

```

```

+2.5E+01,1.0E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,
+2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,2.5E+01,
+3.0E+04,3.0E+01,4.0E+00,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,
+1.0E+04,3.0E+02,1.5E+01,3.0E+02,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,
+2.5E+01,1.0E+04,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,
+2.5E+01,2.5E+01,1.0E+01,1.0E+01,
DATA SACCI/
+9.3E+01,1.0E+00,5.0E-01,2.0E+02,4.4E+01,1.4E+03,1.7E+04,9.6E+01,
+3.6E+00,1.0E+00,1.9E+00,1.7E-01,6.0E+01,3.3E+01,3.0E+04,4.4E+01,
+1.9E+02,1.0E+00,6.6E+00,1.3E+01,1.0E+04,1.0E+03,5.0E+03,2.0E+03,
+4.0E+02,2.0E+04,1.0E+03,2.5E+02,1.7E+03,5.0E+04,6.7E+02,1.7E+04,
+3.3E+02,1.0E+03,3.1E+00,1.0E+00,1.7E+01,2.0E+01,1.0E+03,4.0E+01,
+1.0E+02,1.0E+01,5.0E+01,1.0E+03,2.0E+03,2.0E+03,3.3E+03,2.5E+05,
+1.0E+05,1.0E+03,5.0E+00,1.0E+05,5.0E+01,1.0E+00,2.5E+01,1.0E+02,
+1.0E+03,6.0E+02,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,
+1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,2.0E+01,
+1.7E+04,3.0E+01,6.0E+01,2.0E+03,2.0E+03,2.0E+03,3.3E+03,3.3E+04,
+1.5E+04,1.0E+03,2.4E+01,5.0E+03,5.0E+01,1.0E+00,2.0E+01,1.0E+02,
+1.0E+03,2.0E+03,1.0E+01,1.0E+01,1.0E+01,1.0E+01,1.0E+01,2.0E+02,1.0E+03,1.0E+03,
+1.0E+03,1.0E+03,1.0E+01,1.0E+01,
DATA SACCA/
+9.3E+01,1.0E+00,3.0E+00,1.0E+03,2.2E+00,1.4E+03,1.0E+03,1.0E+04,9.6E+01,
+1.4E+00,1.0E+00,9.5E+01,7.7E-01,6.0E+02,6.7E+01,3.0E+03,4.4E+01,
+7.6E+02,1.0E+00,2.6E+01,5.0E+00,1.0E+05,2.0E+03,1.0E+02,2.0E+03,
+5.5E+03,7.3E+02,1.0E+03,2.5E+02,1.0E+03,1.0E+03,1.7E+03,3.3E+02,
+1.7E+03,1.0E+03,1.5E+00,1.0E+00,1.7E+01,1.0E+01,1.0E+01,5.0E+03,1.0E+03,
+5.0E+02,1.0E+01,4.0E+03,2.0E+03,2.0E+03,2.0E+03,2.0E+03,2.0E+02,1.0E+03,
+1.0E+05,1.0E+02,1.5E+03,1.0E+03,1.0E+03,1.0E+03,1.0E+03,5.0E+01,5.0E+02,
+5.0E+03,6.0E+02,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,
+5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,5.0E+03,2.0E+03,
+1.0E+03,3.0E+01,2.4E+02,2.0E+03,2.0E+03,2.0E+03,3.3E+03,3.3E+01,1.0E+03,
+1.0E+05,5.0E+03,2.4E+01,2.0E+03,4.0E+03,1.0E+00,2.0E+01,1.0E+02,
+5.0E+03,3.0E+03,6.0E+00,6.7E+01,6.0E+00,1.0E+03,5.0E+03,5.0E+03,
+5.0E+03,5.0E+03,6.0E+01,6.0E+01/
10 FORMAT(2X,19A4,A2)
20 FORMAT(4F10.0)
24 FORMAT(11U,2E10,2,I10)
25 FORMAT(1H0,2X,'DISCHARGER',1PE8.2,' CFS',10X,'SOURCE TERM MULTIPLI
+ERE',4X,2)
26 FORMAT(1H0,2X,'FRESHWATER SITE')
27 FORMAT(1H0,2X,'SALTWATER SITE')
30 FORMAT(1H0,' 50-MILE POPULATION',1PE8.2,5X,'FRACTION --- ADULT=
+',0PF4.2//,49X,'TEENAGERS',F4.2//,49X,'CHILDEN',F4.2)
28 FORMAT(1H1)
PL=15.0
IPRNT=1
JSH=1
22 HEAD(5,10)TITLE
IF(EUF(5).NE.0)GOTO500
PRINT 28
READ 24,L1,CFS,UML,LCT
IF(UML.EQ.0.) UML=1.
READ 20,POP,TR
IF(TR.GT.0.) READ 20,PERA,PERT,PERC
PRINT 10,TITLE
PRINT 25,CFS,UML
IF(POP.GT.0) PRINT 30,POP,PERA,PERT,PERC

```

```

115 IF(LT.EQ.0) PRINT 26
    IF(LT.GT.0) PRINT 27
    IF(JSH.EQ.1) CALL REDDF(IPRINT)
    JSRUSH+2
    CALL PLOP(MUSE,4)
120 CALL SOURCE(UML)
    IF(LT.EQ.0) CALL ALARA(FACCF,FACCI,FACCA)
    IF(LT.GT.0) CALL ALARA(SACCF,SACCI,SACCA)
    IF(LT.EQ.0) CALL WHY(FACCF,1,1)
    IF(LT.GT.0) CALL WHY(FACCF,1,2)
125 IF(LT.EQ.0) CALL WHY(FACCI,2,1)
    IF(LT.GT.0) CALL WHY(FACCI,2,2)
    IF(LT.EQ.0) CALL WHY(SACCF,1,1)
    IF(LT.GT.0) CALL WHY(SACCF,1,2)
130 IF(LT.GT.0) CALL WHY(SACCI,2,1)
    IF(LT.GT.0) CALL WHY(SACCI,2,2)
    CALL WATER
    CALL ACTIVE
    CALL FLOWD
    IF(LT.EQ.0) CALL WHO(FACCF,FACCI,FACCA)
    IF(LT.GT.0) CALL WHO(SACCF,SACCI,SACCA)
135 CALL PLOP(DOSE,3)
    GO TO 22
    500 STOP
    END
LADTAP 117
LADTAP 118
LADTAP 119
LADTAP 120
LADTAP 121
LADTAP 122
LADTAP 123
LADTAP 124
LADTAP 125
LADTAP 126
LADTAP 127
LADTAP 128
LADTAP 129
LADTAP 130
LADTAP 131
LADTAP 132
LADTAP 133
LADTAP 134
LADTAP 135
LADTAP 136
LADTAP 137
LADTAP 138
LADTAP 139
LADTAP 140
LADTAP 141

```

```

1  SUBROUTINE HEDDF(IPRNT)
   COMMON/SURCE/IZ(300),IMASS(300),META(300),MLIB4,MLIBT,MLIBG,MLIBI
   COMMON/ELEMFN/ELEM(100)
   COMMON G(200),PL,CFS,NSUR,LT,RECD(200),LIST(200,4),LCT,LZ ,CON,KIT
   + ,PUJ
   COMMON/DFLIB/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   +EFF(300,8)
   DIMENSION LS(20)
10  FORMAT(IH1)
11  FORMAT(/,/)
12  FORMAT(2X,19A4,A2)
13  FORMAT(' CHILD DOSE FACTORS')
14  FORMAT(' ADULT DOSE FACTORS')
15  FORMAT(' TEENAGER DOSE FACTORS')
16  FORMAT(1X,213,A1,5E8.0)
17  FORMAT(10E8.0)
18  FORMAT(' ADULT HEADING GUES HERE')
19  FORMAT(14,A2,14,A1,1P12E9.2,/,56X,1P7E9.2,/,10X,1P8E10.2)
20  FORMAT(' TEENAGER HEADING GUES HERE')
21  FORMAT(14,A2,14,A1,1P7E9.2,/,11X,1P7E9.2)
22  FORMAT(' INFANT DOSE FACTORS')
23  FORMAT(' DOSE FACTOR LIBRARY CONTAINS',I4,' ENTRIES')
24  FORMAT(1X,9E8.2)
25  C** READ ADULT DOSE FACTOR LIBRARY
   READ (10,12)LS
   K=0
39  K=K+1
   READ(10,10)IZ(K),IMASS(K),META(K),TAU(K),EXG(K,2),EXS(K,2),EXG(K,1
   +),EXS(K,1)
   TAU(K)=TAU(K)*3600.
   IF(IZ(K)745,45,40
40  READ(10,17)(DFL(K,J),J=1,7)
   READ(10,17)(DFA(K,J),J=1,7)
   READ(10,24)(EFF(K,J),J=1,8)
   GOTO 39
35  NLIBA=K-1
   K=K-1
40  C** READ TEENAGER DOSEFACTOR LIBRARY
   READ(10,12)LS
49  K=K+1
   READ(10,16)IZ(K),IMASS(K),META(K)
   IF(IZ(K)55,55,50
50  READ(10,17)(DFL(K,J),J=1,7)
   READ(10,17)(DFA(K,J),J=1,7)
   GOTO 49
45  NLIBT=K-1
   K=K-1
50  C** READ CHILD DOSE FACTOR LIBRARY
   READ(10,12)LS
59  K=K+1
   READ(10,16)IZ(K),IMASS(K),META(K)
   IF(IZ(K)65,65,60
60  READ(10,17)(DFL(K,J),J=1,7)
   READ(10,17)(DFA(K,J),J=1,7)
   GOTO 59
55  NLIBC=K-1
   K=K-1

```

```

60      HEAD(10,16)IZ(K),IMASS(K),META(K)
        IF(IZ(K)68,68,67)
67      HEAD(10,17)(DFL(K,J),J=1,7)
        HEAD(10,17)(DFA(K,J),J=1,7)
        GO TO 66
65      NLIBI=K-1
        IF(IPRNT.GT.0)GOTO 1000
        C** PRINT OUT ADULT DOSE FACTORS
        PRINT 10
        PRINT 11
        PRINT 14
        PRINT 18
        DU 70 K=1,NLIBI
        KK=IZ(K)
70      PRINT 19, IZ(K),IELEM(KK),IMASS(K),META(K),TAUC(K),(EXG(K,J),J=1,2)
        1,(EXS(K,J),J=1,2),(DFL(K,J),J=1,7),(DFA(K,J),J=1,7),(EFF(K,J),J=1,
        28)
        KI=NLIBI+1
        C** PRINT OUT TEENAGER DOSE FACTORS
        PRINT 10
        PRINT 11
        PRINT 15
        PRINT 20
        DU 80 K=1,NLIBI
        KK=IZ(K)
80      PRINT 21,IZ(K),IELEM(KK),IMASS(K),META(K),(DFL(K,J),J=1,7),(DFA(K,
        J),J=1,7)
        KI=NLIBI+1
        C** PRINT OUT CHILD DOSE FACTORS
        PRINT 10
        PRINT 11
        PRINT 13
        PRINT 20
        DU 90 K=1,NLIBI
        KK=IZ(K)
90      PRINT 21,IZ(K),IELEM(KK),IMASS(K),META(K),(DFL(K,J),J=1,7),(DFA(K,
        J),J=1,7)
        KI=NLIBI+1
        C** PRINT OUT INFANT DOSE FACTORS
        PRINT 10
        PRINT 11
        PRINT 22
        PRINT 20
        DU 100 K=1,NLIBI
        KK=IZ(K)
100     PRINT 21,IZ(K),IELEM(KK),IMASS(K),META(K),(DFL(K,J),J=1,7),(DFA(K,
        CJ),J=1,7)
        PRINT 23,NLIBI
        1000 RETURN
        END

```

```

1 SURRUUTINE SOURCE(UML) SOURCE 4
COMMON Q(200),PL,CFS,NSUR,LT,RECO(200),LIST(20,4),LCT,LZ ,CON,KIT SOURCE 5
+ ,PUP SOURCE 6
+EFF(300,A) SOURCE 7
COMMON/DFL16/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EX8(300,2), SOURCE 8
COMMON/SOURCE/IZ(300),IMASS(300),META(300),NLIBA,NLIBT,NLIBC,NLIBI SOURCE 9
COMMON/ELEM/TELEM(100) SOURCE 10
DIMENSION IM(5),NUM(12),ISDR(20) BPL01 7
DATA NUM/1,10,11,12,13,14,15,16,17,18,19,1M1/ SOURCE 14
11 FORMAT(REIU,0) SOURCE 15
12 FORMAT(2X,1944,A2) SOURCE 16
13 FORMAT(1 GRIEF ,A2,5A1,1PE10.2) SOURCE 17
21 FORMAT(2X,A2,5A1,1X,E10.0) SOURCE 18
22 FORMAT(1M ,14,A2,14,A1,1X,1P12F9.2) SOURCE 19
23 FORMAT(IH0,1TOTAL NUMBER IN SOURCE TERM IS ,14,5X,1TOTAL RELEASE SOURCE 20
+IS ,1PE10.4) SOURCE 21
30 FORMAT(1M ,1NUCLIDE CURIE/YEAR NONE LIVER TOTAL BODY THYR SOURCE 22
+IID KIDNEY LUNG GI-LLI SKIN TOTAL BODY RECON') SOURCE 23
31 FORMAT(IH0,21X,1 SHORELINE SOURCE 24
+ SOURCE 25
35 FORMAT(1M ,45X,1(MREM/PCI INTAKE),22X,1(MREM/HR)/(PCI/M**2),1) SOURCE 26
32 FORMAT(IH1,30X,1 * * * TEENAGER DOSE FACTORS * * * *) SOURCE 27
33 FORMAT(IH0,30X,1 * * * CHILD DOSE FACTORS * * * *) SOURCE 28
34 FORMAT(IH0,30X,1 * * * INFANT DOSE FACTORS * * * *) SOURCE 29
36 FORMAT(IH0,30X,1 * * * ADULT DOSE FACTORS * * * *) SOURCE 30
READ 12,ISUR SOURCE 31
PRINT 12,ISUR SOURCE 32
120 SOURCE 33
75 I=1+1. SOURCE 34
READ 21,1A,1M,00 SOURCE 35
IF(00)101,101,76 SOURCE 36
76 K=1 SOURCE 37
MASSE0 SOURCE 38
MET=NUM(1) SOURCE 39
DU 90 JJ=1.5 SOURCE 40
J=JJ SOURCE 41
IF(IM(J).EQ.NUM(1))GOTO 90 SOURCE 42
IF(IM(J).NE.NUM(12))GOTO 78 SOURCE 43
GOTO 90 SOURCE 44
78 K=K+1 SOURCE 45
DU 80 LE2,11 SOURCE 46
80 IF(NUM(1).EQ.IM(J))GOTO 85 SOURCE 47
PRINT 13,1A,1M,00 SOURCE 48
I=I+1 SOURCE 49
GOTO 75 SOURCE 50
85 MASSE=MASS*(L=2)*10.**K SOURCE 51
90 CURTINE SOURCE 52
C** FIND Z OF NUCLIDE SOURCE 53
DU 91 IK=1,100 SOURCE 54
91 IF(ELEM(IK).EQ,IA)GOTO 92 SOURCE 55
PRINT 13,1A,1M,00 SOURCE 56
I=I+1 SOURCE 57
GOTO 75 SOURCE 58
C** FINE NUCLIDE IN ADULT, 4IMHARY SOURCE 59
92 DU, 95, LL=1,NLIBA SOURCE 60

```

```

60 IF (IZ(LL).NE.IK)GOTO 95
   IF (IMASS(LL).NE.MASS)GOTO 95
   IF (META(LL).NE.MET)GOTO 95
   GOTO 96
95 CONTINUE
   I=I+1
   PRINT 13,IA,IM,00
   GOTO 75
65 96 LIST(I,1)=LL
   C** FIND NUCLIDE IN TEENAGER LIBRARY
   KI=NLIHA+1
   DO 97 LL=KI,NL18T
   IF (IZ(LL).NE.IK)GOTO 97
   IF (IMASS(LL).NE.MASS)GOTO 97
   IF (META(LL).NE.MET)GOTO 97
   GOTO 98
97 CONTINUE
   LIST(I,2)=LIST(I,1)
   GOTO 99
75 98 LIST(I,2)=LL
   DFL(LL,5)=DFL(LIST(I,1),5)
   DFA(LL,5)=DFA(LIST(I,1),5)
   C** FIND NUCLIDE IN CHILD DOSE FACTOR LIBRARY
   KI=NLIHT+1
   DO 106 LL=KI,NLIHC
   IF (IZ(LL).NE.IK)GOTO 106
   IF (IMASS(LL).NE.MASS)GOTO 106
   IF (META(LL).NE.MET)GOTO 106
   GOTO 107
106 CONTINUE
   LIST(I,3)=LIST(I,2)
   GOTO 108
90 107 LIST(I,3)=LL
   DFL(LL,5)=DFL(LIST(I,1),5)
   DFA(LL,5)=DFA(LIST(I,1),5)
108 CONTINUE
   C** FIND NUCLIDE IN INFANT DOSE FACTOR LIBRARY
   KI=NLIHC+1
   DO 110 LL=KI,NLIHI
   IF (IZ(LL).NE.IK)GOTO 110
   IF (IMASS(LL).NE.MASS)GOTO 110
   IF (META(LL).NE.MET)GOTO 110
   GOTO 115
110 CONTINUE
   LIST(I,4)=LIST(I,3)
   GOTO 120
105 115 LIST(I,4)=LL
   DFL(LL,5)=DFL(LIST(I,1),5)
   DFA(LL,5)=DFA(LIST(I,1),5)
120 CONTINUE
   RI=RT+00
   Q(I)=QR*UML
   GOTO 75
110 101 NSOH=I-1
   C** PRINT OUT SOURCE TERM
   C**CALCULATES ANY RECONCENTRATION OF RADIONUCLIDES
   CALL RECON

```



```

115      PRINT 36
        PRINT 31
        PRINT 35
        PRINT 30
        DO 105 I=1,NSUR
          LL=LIST(I,1)
          IK=IZ(LL)
105      PRINT 22,IK,IELEM(IK),IMASS(LL),META(LL),0(I),(DFL(LL,J),J=1,7),
          +FXG(LL,1),FXG(LL,2),RECU(I)
          DO 210 JT=2,4
            IF(JT.EQ.4) PRINT 34
            IF(JT.EQ.3) PRINT 33
            IF(JT.EQ.2) PRINT 32
            PRINT 31
            PRINT 35
            PRINT 30
            DO 200 I=1,NSUR
              LL=LIST(I,1)
              IK=IZ(LL)
              LA=LIST(I,JT)
              IF(LA.EQ.LL) GO TO 200
              PRINT 22,IK,IELEM(IK),IMASS(LL),META(LL),0(I),(DFL(LA,J),J=1,7)
200      CONTINUE
210      CONTINUE
          OT=OT+UHL
        PRINT 23,NSOR,OT
        RETURN
        END
SOURCE 116
SOURCE 119
SOURCE 120
SOURCE 121
SOURCE 122
SOURCE 123
SOURCE 124
SOURCE 125
SOURCE 126
SOURCE 127
SOURCE 128
SOURCE 129
SOURCE 130
SOURCE 131
SOURCE 132
SOURCE 133
SOURCE 134
SOURCE 135
SOURCE 136
SOURCE 137
SOURCE 138
SOURCE 139
SOURCE 140
SOURCE 141
SOURCE 142
SOURCE 143
SOURCE 144
SOURCE 145

```



```

60      HUILEDILU
      IF(KK.GT.1)DWD=DILU
      IF(KK.GT.1) PRINT 160,LUCA
      IF(KK.GT.1) TD=1
      C
      PRINT 60
      PRINT 50
      PRINT 40
      CALL UUT(1,ACCF,ACCI,ACCA,T,FIUS,CRUS,ALUS,MUSE,SMU,SMU,RUSE,DILU,
      +DWD,SHD,SWD,BDIL,SWF,KK,N,TD)
      IF(KK.GT.0.AND.LCT.GT.0)PRINT 130
      IF(KK.GT.1) PRINT 160,LUCA
      IF(KK.EQ.0.AND.LCT.GT.0)PRINT 10
      PRINT 80
      PRINT 50
      PRINT 40
      CALL UUT(2,ACCF,ACCI,ACCA,T,TAF,TAC,TAA,TAM,TAS,TASM,TAB,DILU,DWD,
      +SHD,SWD,HDIL,SWF,KK,N,TD)
      IF(KK.GT.0.AND.LCT.GT.0)PRINT 130
      IF(KK.GT.1) PRINT 160,LUCA
      IF(KK.EQ.0.AND.LCT.GT.0)PRINT 10
      PRINT 70
      PRINT 50
      PRINT 40
      CALL UUT(3,ACCF,ACCI,ACCA,T,CHF,CHC,CHA,CHW,CHS,CHSW,CHB,DILU,DWD,
      +SHD,SWD,BDIL,SWF,KK,N,TD)
      IF(TDW.EQ.0.UR.LT.GT.0) GO TO 145
      IF(KK.GT.0.AND.LCT.GT.0)PRINT 130
      IF(KK.GT.1) PRINT 160,LUCA
      IF(KK.EQ.0.AND.LCT.GT.0)PRINT 10
      PRINT 90
      PRINT 50
      PRINT 40
      CALL UUT(4,ACCF,ACCI,ACCA,T,TDF,TDC,TDA,TDW,TDS,TDSM,TDH,DILU,DWD,
      +SHD,SWD,BDIL,SWF,KK,N,TD)
      145 KK=KK+2
      GO TO 150
      200 CONTINUE
      RETURN
      END
54      ALARA
55      ALARA
56      ALARA
57      ALARA
58      ALARA
59      ALARA
60      ALARA
61      ALARA
62      ALARA
63      ALARA
64      ALARA
65      ALARA
66      ALARA
67      ALARA
68      ALARA
69      ALARA
70      ALARA
71      ALARA
72      ALARA
73      ALARA
74      ALARA
75      ALARA
76      ALARA
77      ALARA
78      ALARA
79      ALARA
80      ALARA
81      ALARA
82      ALARA
83      ALARA
84      ALARA
85      ALARA
86      ALARA
87      ALARA
88      ALARA
89      ALARA
90      ALARA
91      ALARA
92      ALARA
93      ALARA
94      ALARA
95      ALARA
96      ALARA
97      ALARA
98      ALARA

```

```

1  SUBROUTINE OUT(KUP,ACCF,ACCI,ALUS,CRUS,ALUS,MUSE,SHU,SMU,RU OUT
+SE,DILU,DMD,SHD,SMD,RDIL,SWF,KN,N,10) OUT
COMMON D(200),PL,CFS,NSUP,LT,HECO(200),LIST(200,4),LCT,LZ,CON,KIT OUT
+ ,POP OUT
5  DIMENSION W(3),X(3),Y(3),Z(3),H(3),FDOSE(8),COURSE(R),ADUSE(8),SDOS OUT
IE(8),SDDO(8),BDUSE(8),TDOSE(R),A(3),H(3),C(3),MDOSE(R) OUT
DIMENSION ACCF(100),ACCI(100),ACCA(100),DOSE(200,R) OUT
C
C
10  CONTROLS THE CALLING OF THE CALCULATIONAL SUBROUTINES WITH THE
C APPROPRIATE USAGE PARAMETERS FOR THE PARTICULAR AGE GROUP
C
DATA Y/'TOTA','L',' ',' ',' '
DATA X/'INVE','RTEH','RATE',' '
DATA Z/'ALGA','E',' ',' '
DATA C/'HOAT','ING',' ',' '
DATA H/'DRIN','KING',' ',' '
DATA W/'FISH',' ',' '
DATA A/'SHOK','ELIN','E',' '
DATA B/'SWIM','MING',' '
10  FORMAT(IH,3A4,15X,1P7E15.2)
20  FURNAT(IH,3A4,15X,1P7E15.2)
T2=1+24.
T3=10+12.
LZ=0
KIT=0
CALL AQUA(W,DILU,FIUS,T2,FDOSE,KOP,ACCF)
CALL AQUA(X,DILU,CRUS,T2,CDOSE,KOP,ACCI)
IF(CRUS.GT.0.0) PRINT 10,X,(CDOSE(J),J=2,8)
CALL AQUA(Z,DILU,ALUS,T2,ADUSE,KOP,ACCA)
IF(ALUS.GT.0.0) PRINT 10,Z,(ADUSE(J),J=2,8)
CALL OPINK(DMD,T3,MUSE,MDOSE,KOP)
IF(LT.EQ.0) PRINT 10,H,(MDOSE(J),J=2,8)
CALL SHOPE(A,SWF,SHD,T,SHU,SDOUSE)
PRINT 20,A,SDOUSE
GEOM=1.
CALL SWIM(H,SMD,T,SMU,GEOM,SMDN)
IF(SMU.GT.0.0) PRINT 20,B,SMDN
GEOM=2.
CALL SWIM(C,RDIL,T,BUSE,GEOM,BDOSE)
IF(BUSE.GT.0.0) PRINT 20,C,BDOSE
DO 40 J=1,H
40  TDOSE(J)=FDOSE(J)+CDOSE(J)+ADUSE(J)+SDOUSE(J)+SDDO(J)+HDOS
1E(J)
PRINT 20,Y,TDOSE
60  FORMAT(1H0,12X,'USAGE (KG/YR,HR/YR)
+',10X,'SHOREWIDTH FACTOR',F3.1)
70  FORMAT(1H,3A4,7X,F8.1,10X,F10.1,3X,F10.2)
PRINT 60,SWF
PRINT 70,W,FIUS, DILU,T2
IF(CRUS.GT.0.0) PRINT 70,X,CRUS,DILU,T2
IF(ALUS.GT.0.0) PRINT 70,Z,ALUS,DILU,T2
IF(MUSE.GT.0.0) AND(LT.EQ.0) PRINT 70,H,MUSE,DMD,T3
IF(SHU.GT.0.0) PRINT 70,A,SHU,SHD,T
IF(SMU.GT.0.0) PRINT 70,B,SMU,SMD,T

```

SUBROUTINE OUT

76/76 OPT=1

FTN 4.5+414

10/14/78 15.10.083

PAGE

2

```
IF(BUSE.GT.0.0) PRINT 70,C,BUSE,BDIL,T  
KIT#10  
80 CONTINUE  
IF(LCT.GT.0)CALL PERDOS(W,TD0SE,DD0SE)  
RETURN  
END
```

```
54 UUT  
59 UUT  
60 UUT  
61 UUT  
62 UUT  
63 UUT  
64 UUT
```

```

1 SURROUTINE WHU(ACCF,ACCI,ACCA)
  COMMON /G(200),PL,CFS,NSUR,LT,REC(200),LIST(200,4),LCT,LZ,CON,KIT
  +,POP
5 DIMENSION EXT(8),EXI(8),TDUSE(8),TYPE(3),A(3),B(3),C(3),D(3),W(3),
  CX(3),Z(3),ACCF(100),ACCI(100),ACCA(100)
  C
  C
  C CONTROLS THE CALLING OF THE CALCULATIONAL SUBROUTINES FOR THE
  C PRIMARY AND SECONDARY BIUTA DOSES
  C
15 DATA CSWF/2.0/
  DATA RAT,RAC,HERON,DUCK/6.,14.,11.,5./
  DATA RATMAS,RACMAS,HERMAS,DUCKMAS/1000.,12000.,4600.,1000./
  DATA RATUSE,RACUSE,HERUSE,DUCKUSE/100.,200.,600.,100./
  C BIUTA TYPES
  DATA A/'MUSK','RAF','/'
  DATA B/'RACC','UON','/'
  DATA C/'HERU','N','/'
  DATA D/'DUCK','I','/'
  DATA W/'FISH','I','/'
  DATA X/'INVE','RTEB','RATE','/'
  DATA Z/'ALGA','E','/'
  C
25 70 FORMAT(1H0,' ',1)
  20 FORMAT(1H,3A,1P3E15.2)
  60 FORMAT('DILUTION=',1PE10.2,10X,'TRANSIT TIMES',E10.2,' HRT')
  10 FURMAT(1H,'+', '* DOSE TO BIUTA * *')
  30 FURMAT(1H0,20X,'MRADS PER YEAR')
  40 FURMAT(1H0,19X,'INTERNAL EXTERNAL TOTAL')
  50 FURMAT(1E10.0)
  CUN:1.
  PRINT 10
  PRINT 30
35 READ 50,DILU,T
  IF(DILU.EQ.0.)GO TO 100
  KIT=0
  LZ=0
  PRINT 70
  PRINT 60,DILU,T
  PRINT 40
  CALL CRITTR(W,DILU,T,TDUSE,ACCF)
  CALL SHPRE(TYPE,CSWF,DILU,T,4380.,EXT)
  CALL SWIM(TYPE,DILU,T,8760.,1.,EXI)
  TEXT=EXT(2)+EXI(2)
  TUT=TEXT+TDUSE(1)
  PRINT 20,W,TDUSE(1),TEXT,TOT
  CALL CRITTR(X,DILU,T,TDUSE,ACCI)
  CALL SHPRE(TYPE,CSWF,DILU,T,8760.,EXT)
  CALL SWIM(TYPE,DILU,T,8760.,1.,EXI)
  TEXT=EXT(2)+EXI(2)
  TUT=TEXT+TDUSE(1)
  PRINT 20,X,TDUSE(1),TEXT,TOT
  CALL CRITTR(Z,DILU,T,TDUSE,ACCA)
  CALL SWIM(TYPE,DILU,T,8760.,1.,EXI)
  TOT=EXI(2)+TDUSE(1)
  PRINT 20,Z,TDUSE(1),EXI(2),TOT
  CALL EAT(A,RAT,RATMAS,RATUSE,DILU,T,TDUSE,ACCA)

```

```

60 CALL SHORE(TYPE,CSMF,DILU,T,2922,,EXT)
   CALL SWIM(TYPE,DILU,T,2922,,1,,EXI)
   TEXT=EXT(2)+EXI(2)
   TOT=TEXT+TDUSE(1)
   PRINT 20,A,TDUSE(1),TEXT,TOT
65 CALL EAT(H,HAC,RACMAS,RACUSE,DILU,T,TDUSE,ACCI)
   CALL SHORE(TYPE,CSMF,DILU,T,2191,,EXT)
   TOT=EXT(2)+TDUSE(1)
   PRINT 20,B,TDUSE(1),EXT(2),TOT
   CALL EAT(C,HERMID,HERMAS,HERUSE,DILU,T,TDUSE,ACCF)
   CALL SHORE(TYPE,CSMF,DILU,T,2922,,EXT)
   CALL SWIM(TYPE,DILU,T,2920,,2,,EXI)
   TEXT=EXT(2)+EXI(2)
   TOT=TEXT+TDUSE(1)
   PRINT 20,C,TDUSE(1),TEXT,TOT
70 CALL EAT(D,DUCK,DUCHAS,DUCCUSE,DILU,T,TDUSE,ACCA)
   CALL SHORE(TYPE,CSMF,DILU,T,4383,,EXT)
   TEXT=EXT(2)+EXI(2)*3./2.
   TOT=TEXT+TDUSE(1)
   PRINT 20,D,TDUSE(1),TEXT,TOT
   KITE=70
   IF(LCT.GT.0) CALL PERDUS(A,TDUSE,DOUSE)
   GO TO 80
80 100 CONTINUE
   RETURN
   END

```



```

1 SUBROUTINE WHY(ACC,IAN)
COMMON/PUPUL/PERA,PRT,PERC,IUS
COMMON Q(200),PL,CFS,NSUR,LT,PECO(200),LIST(200,4),LCT,LZ ,CON,MIT
+ ,PUP
5 DIMENSION ACC(100),X(3),W(3),CATH(20),DILU(20),T(20),TDOSE(6),DOSE
+(200,8),A(3),H(3),C(3),TYPE(3),CONC(200)
DIMENSION D(3),PD(7)
C
C CONTRIBS THE CALCULATION OF THE SPORT
C AND COMMERCIAL FISH AND INVERTEBRATE
C POPULATION DOSES=ACTUAL CALCULATIONS
C DONE BY SUBROUTINES PAFD AND CENT
15 DATA X/INVE1,IR ,',',
DATA W/FISH',',
DATA A/ADUL',',
DATA H/TEEN',',AGEH',',
DATA C/CHILD',',D ,',',
DATA D/TOTAL',',L ,',',
20 FORMAT(1H ,64,1PBE10.2)
20 FORMAT(1H0,'PATHWAY AGE GROUP USAGE RONE LIVER
+ TOTAL BODY THYROID KIDNEY LUNG GI-LLI')
25 FORMAT(1H0,34X,'-----DOSE (MAN-REM)-----
+-----')
30 FORMAT(1H0,'DILUTION CATCH TIME(HR)=INCLUDES FOOD PROCESSING
+ TIME OF 1,1PER,2,1 HR,5X,'POPULATION',E8.2)
30 FORMAT(1H ,53X,'MAN-REM')
30 FORMAT(1H1,55X,'* * * INVERTEBRATE CONSUMPTION POPULATION DOSES
+ * * *')
30 FORMAT(1H1,35X,'* * * FISH CONSUMPTION POPULATION DOSES * *
+*')
35 FORMAT(1H0,'-----SPORTFISH HARVEST-----')
35 FORMAT(1H0,'-----COMMERCIAL HARVEST-----')
35 FORMAT(1H0,'-----NEPA DOSES-----')
35 FORMAT(1H0,'NOTE==TUATL NEPA DOSE MUST INCLUDE SPORT CATCH, DOSES
+HELLO ARE FOR COMMERCIAL CATCH ONLY')
40 FORMAT(1H ,1PBE10.2)
40 FORMAT(1H0,'AVERAGE INDIVIDUAL CONSUMPTION (KG/YR) ADULT',
+1PE,2,5X,'TEEN',E8.2,5X,'CHILD',E8.2)
45 P(AMT,AU,IU,CU)BAMT/(AU*PERA+TURPERT+CURPERC)
AUSE(PER,AU)BPECU*PERA*AU
TUSE(PER,IU)BPEU*PERT*IU
CUSE(PER,CU)BPECU*PERC*CU
CUM=1000.
LH=0
IF(N,EQ,1) NN=1
IF(N,EQ,2) NN=2
IF(I,EQ,1,AND,LT,EG,0) HARV=4.0E+06
IF(I,EQ,2,AND,LT,EG,0) HARV=2.30E+06
IF(I,EQ,1,AND,LT,GT,0) HARV=6.58E+08
IF(I,EQ,2,AND,LT,GT,0) HARV=4.10E+08
IF(N,EQ,1) FPT=68.
IF(N,EQ,2) FRT=240.
JM=1

```

```

60      80 HEAD 70,CATH(J),DILU(J),I(J)
        T(J)=T(J)+FPT
        J=J+1
        M=J-1
        IF(DJLU(M).EQ.0.) GO TO 85
        GO TO 80
85      M=M-1
        IF(M.EQ.0) GO TO 100
        IF(I.EQ.1)PRINT 65
        IF(I.EQ.2)PRINT 64
        PRINT 61
        IF(N.EQ.1)PRINT 66
        IF(N.EQ.2)PRINT 67
        PRINT 30
        PRINT 20
        GO TO (21,22),I
75      21 AU=6.9
        TU=5.2
        DO 17 J=1,3
17      TYPE(J)=M(J)
        CU=2.2
        GO TO 86
90      22 AU=1.
        TU=0.75
        CU=0.33
        DO 16 J=1,3
16      TYPE(J)=X(J)
        GO TO 86
86      AMT=0.
        IF(N.EQ.1) NL=1
        IF(N.EQ.2) NL=2
        DO 87 I=1,M
87      AMT=AMT+CATH(I)
        IF(N.EQ.1)PEU=PEU+AMT*AU,TU*CU
        IF(N.EQ.2)PEU=PEU*PUP
        LZ=0
        KIT=0
        USE=AU*PEU*AU
        CALL CENT(T,CATH,DILU,M,CUNC,AMT,HARV,NL)
        CALL PAFD(TYPE,ACC,CUNC,I,USE,TDOSE,NN,LM)
        PRINT 15,TYPE,A,USE,(TDOSE(JK),JK=1,7)
        DO 89 J=1,7
100     89 PD(J)=TDOSE(J)
        SUM=USE
        USE=TUSE*(PEU,TU)
        CALL PAFD(TYPE,ACC,CUNC,2,USE,TDOSE,NN,LM)
        PRINT 15,TYPE,B,USE,(TDOSE(JK),JK=1,7)
        DO 91 J=1,7
105     91 PD(J)=PB(J)+TDOSE(J)
        SUM=SUM+USE
        USE=CUSE*(PEU,CU)
        CALL PAFD(TYPE,ACC,CUNC,3,USE,TDOSE,NN,LM)
        PRINT 15,TYPE,C,USE,(TDOSE(JK),JK=1,7)
        DO 92 J=1,7
110     92 PD(J)=PC(J)+TDOSE(J)
        SUM=SUM+USE
    
```

```

115 PRINT 15,TYPE,0,SUM,(PD(J),J=1,7)
    IF(LM,GT,0)GO TO 100
    PRINT 60,FPT,PEO
    DO 90 J=1,M
120 PRINT 71,UILU(J),CATH(J),T(J)
    PRINT 75,AU,TU,CU
    KITS=30
    IF(LCT,GT,0)CALL PERDOS(M,TDOSE,DOSE)
    IF(N.EQ,1)GO TO 100
    LM=10
    NL=1
125 PRINT 68
    PRINT 69
    PRINT 30
    PRINT 20
    PEO=P(AHT,AU,TU,CU)
    GO TO 88
130 CONTINUE
    RETURN
    END

```

CARD NR. SEVERITY DETAILS DIAGNOSIS OF PROBLEM

```

73 I AN IF STATEMENT MAY BE MORE EFFICIENT THAN A 2 OR 3 BRANCH COMPUTED GO TO STATEMENT.
74 I THERE IS NO PATH TO THIS STATEMENT.

```



```

1 SUBROUTINE ACTIVE
  CUMM(10,200),PL,CFS,NSUR,LT,RECU(200),LIST(200,4),LCT,LZ ,CON,MIT
  + ,POP
5 DIMENSION S(3),SW(3),D(3),A(3),B(3),C(3),TDOUSE(8)
  DATA S/'SHOR','ELIN','E ' /
  DATA SW/'SWIM','MING',' ' /
  DATA A/'HAI','ING',' ' /
  DATA D/'TUT','L PO','PUL ' /
10 FORMAT(1H0,'LUCATION= ',3A4)
  15 FORMAT(1H0,'DILUTIONS=',E8.2,10X,'TRANSIT TIME=',E8.2,' HR',10X,'S#
  +E',F3.1)
  16 FORMAT(1H1,35X,'* * * RECREATION POPULATION DOSES * * *
  +*1)
  20 FORMAT(1H0,'PATHWAY AGE GROUP USAGE SKIN
  + TOTAL BODY THYROID')
  30 FORMAT(1H1,'* * * POPULATION DOSES * * *1)
  40 FORMAT(1H ,48X,' _____DOSE(MAN-REM)_____')
  50 FORMAT(1H0,' _____')
  +
  60 FORMAT(1H0,'DILUTIONS=',E8.2,10X,'TRANSIT TIME=',E8.2,' HR')
  70 FORMAT(4E10.0,3A4)
  75 FORMAT(3E10.0,3A4)
  PRINT 14
  CUM=1000.
  JL=10
100 READ 70,SHU,DILU,T,SWF,(LUCA(J),J=1,3)
  PRINT 50
  IF(DILU.EQ.0.)GO TO 110
  LZ=0
  PRINT 40
  KIT=0.
  PRINT 20
  CALL SHURE(S,SWF,DILU,T,SHU,TDOUSE)
  PRINT 10,S,A,SHU,(TDOSE(J),J=1,3)
  PRINT 40,LUCA
  PRINT 15,DILU,T,SWF
  KIT=40
  IF(JL.LT.0)GO TO 100
  IF(LCT.GT.0)CALL PERDOS(M,TDOUSE,DOUSE)
  JL=-10
  GO TO 100
110 READ 75,SHU,DILU,T,(LUCA(J),J=1,3)
  PRINT 50
  IF(DILU.EQ.0.)GO TO 120
  LZ=0
  KIT=0.
  PRINT 40
  PRINT 20
  GENM=1.
  CALL SWIM(SW,DILU,T,SHU,GEOM,TDOUSE)
  PRINT 10,SW,A,SHU,(TDOSE(J),J=1,3)
  PRINT 40,LUCA
  PRINT 60,DILU,T
  KIT=40
  IF(LCT.GT.0)GO TO 110

```

```

60      42 PD(JM)=PD(JM)+TDOSE(JM)
        DU 57 JM=2,8
        57 CUM(JM)=CUM(JM)+TDOSE(JM)
           TUS=USE
           USE=TUSE(P,TU)
           CALL DRINK(DILU,T,USE,TDOSE,2)
           PRINT 10,TYPE,P,USE,(TDOSE(JK),JK=2,8)
65      DU 43 JM=2,8
        43 PD(JM)=PD(JM)+TDOSE(JM)
           DU 58 JM=2,8
        58 CUM(JM)=CUM(JM)+TDOSE(JM)
           TUS=TUS+USE
           USE=CUSE(P,CU)
           CALL OPTNK(DILU,T,USE,TDOSE,3)
           PRINT 10,TYPE,C,USE,(TDOSE(JK),JK=2,8)
70      DU 44 JM=2,8
        44 PD(JM)=PD(JM)+TDOSE(JM)
           DU 59 JM=2,8
        59 CUM(JM)=CUM(JM)+TDOSE(JM)
           TUS=TUS+USE
           EUS=FUS+TUS
           PRINT 10,TYPE,D,TUS,(PD(JM),JM=2,8)
           KIT=20
           PRINT 55,P,DILU,T
           PRINT 60,AU,TU,CU
           IF(LCT.GT.0) CALL PERDUS(TYPE,TDOSE,DUSE)
80      GU TO 40
65      100 CONTINUE
           IF(PD(2).GT.0.0) PRINT 66
           IF(PD(2).GT.0.0) PRINT 20
           IF(PD(2).GT.0.0) PRINT 10,TYPE,E,EUS,(CUM(JM),JM=2,8)
           PRINT 65
           PRINT 20
           USE=2.2
           DU 70 I=1,NSOR
           M=LIST(I,1)
           IF(M.EQ.1) CALL TRTIUM(Q(M),POP,H3B,H3T)
           IF(M.EQ.1) PRINT 10,TRI,D,USE,H3B,H3T,H3B,H3B,H3B,H3B,H3B
70      CONTINUE
           RETURN
           END
95      WATER 54
           WATER 60
           WATER 61
           WATER 62
           WATER 63
           WATER 64
           WATER 65
           WATER 66
           WATER 67
           WATER 68
           WATER 69
           WATER 70
           WATER 71
           WATER 72
           WATER 73
           WATER 74
           WATER 75
           WATER 76
           WATER 77
           WATER 78
           WATER 79
           WATER 80
           WATER 81
           WATER 82
           WATER 83
           WATER 84
           WATER 85
           WATER 86
           WATER 87
           WATER 88
           WATER 89
           WATER 90
           WATER 91
           WATER 92
           WATER 93
           WATER 94
           WATER 95
           WATER 96
           WATER 97
           WATER 98
           WATER 99

```

```

60      IF (LCT.RI.0)CALL PERDUS(M,TDUSE,DUSE)
        JL=10
        GO TO 110
        120 READ 75,HUSE,DILU,T,(LOCA(J),J=1,3)
        PRINT 50
        IF (DILU.EQ.0.0)GO TO 130
        LZ=0
        KIY=0
        GEOME2.0
        PRINT 40
        PRINT 20
        CALL SWIMCO,DILU,T,BUSE,GEOM,TDUSE)
        PRINT 10,D,A,HUSE,(TDUSE(J),J=1,3)
        PRINT 80,LOCA
        PRINT 60,DILU,T
        C
        C
75      GO TO 120
        130 CONTINUE
        RETURN
        END

```

```

ACTIVE 54
ACTIVE 60
ACTIVE 61
ACTIVE 62
ACTIVE 63
ACTIVE 64
ACTIVE 65
ACTIVE 66
ACTIVE 67
ACTIVE 68
ACTIVE 69
ACTIVE 70
ACTIVE 71
ACTIVE 72
ACTIVE 73
ACTIVE 74
ACTIVE 75
ACTIVE 76
ACTIVE 77
ACTIVE 78
ACTIVE 79

```

```

1  SUBROUTINE AQUA (CRITH,DILU,USF,T,TDUSE,JJ,ACC)
   COMMON/DFLIB/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   +EFF(300,4)
5  COMMON R(200),PL,CFS,NSUP,LT,REC0(200),LIST(200,4),LCT,LZ ,CON,KIT
   + ,PUP
   COMMON/SURCE/IZ(300),IMASS(300),META(300),NLIBA,NLIBA,NLIBT,NLIBC,NLIBI
   DIMENSION ACC(100),TDUSE(8),DOUSE(200,8),CRITR(3)
   DO 8 J=1,8
8  TDUSE(J)=0.0
   IF(USE.EQ.0.)GO TO 50
   DO 10 I=1,NSUR
10  DOSE(I,1)=0.
   L=LIST(I,JJ)
   LM=LIST(I,1)
   MU=IZ(LM)
   ARGU=TAU(LM)*T
   IF(ARGU.GT.20.)GO TO 20
   FACT=1119.*0(I)*REC(I)/CFS/DILU*EXP(-ARGU)*USE*ACC(MD)/CON
   GO TO 30
20  FACT=0.0
   DO 40 J=2,8
30  DO 40 J=2,8
   L=J-1
   DOSE(I,J)=DFL(LL,L)*FACT
40  TDUSE(J)=TDUSE(J)+DOSE(I,J)
25  10 CONTINUE
   IF(LCT.GT.0) CALL PERD09(CRITR,TDUSE,DOUSE)
50  CONTINUE
   RETURN
   END

```

AQUA 2
 AQUA 3
 AQUA 4
 AQUA 5
 AQUA 6
 AQUA 7
 AQUA 8
 AQUA 9
 AQUA 10
 AQUA 11
 AQUA 12
 CHANGE1 1
 AQUA 13
 AQUA 14
 AQUA 15
 AQUA 16
 AQUA 17
 AQUA 18
 AQUA 19
 AQUA 20
 AQUA 21
 AQUA 22
 AQUA 23
 AQUA 24
 AQUA 25
 AQUA 26
 AQUA 27
 AQUA 28
 AQUA 29


```

1  SUBROUTINE DRINK(DMD,T,USE,TDUSE,JJ)
   CUPMIN(200),PL,CFS,NSUR,LT,RECU(200),LIST(200,4),LCT,LZ ,CON,MIT
   + ,POP
5  COMMON/SOICE/IZ(300),IMASS(300),META(300),NLIBA,NLIBT,NLIBC,NLIBI
   COMMON/DFLTH/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   +EFF(300,6)
   COMMON/TRANS/DILM
   DIMENSION TDUSE(N),DOSE(200,4),TYPE(3)
10  DATA TYPE/'DRIN','KING',',',,
   DILM = DMD
   DO 8 J=1,6
8  TDUSE(J)=0.0
   IF(USE.EQ.0.)GO TO 50
   IF(LT.GT.0) GO TO 50
   DO 10 I=1,NSUR
15  DOSE(I,1)=0.
   LL=LIST(I,JJ)
   LM=LIST(I,1)
   NU=IZ(LM)
   ARGU=TAU(LM)*T
20  IF(ARGU.GT.20.) GO TO 20
   FACT=1119.*U(1)*RECU(I)/CFS/DMD*EXP(=ARGU)*USE/CON
   GO TO 30
25  FACT=0.0
   DO 40 J=2,6
   L=J-1
   DOSE(I,J)=DFL(LL,L)*FACT
40  TDUSE(J)=TDUSE(J)+DOSE(I,J)
   DO CONTINUE
30  IF (LCT.GT.0) CALL PERDUS(TYPE,TDUSE,DOSE)
   IF(CUN.GT.10.)CALL PLOP(DOSE,7)
50  CONTINUE
   RETURN
   END
DRINK 2
DRINK 3
DRINK 4
DRINK 5
DRINK 6
DRINK 7
CHANGE1 2
DRINK 8
DRINK 9
CHANGE1 3
DRINK 10
DRINK 11
DRINK 12
DRINK 13
DRINK 14
CHANGE1 4
DRINK 15
DRINK 16
DRINK 17
DRINK 18
DRINK 19
DRINK 20
DRINK 21
DRINK 22
DRINK 23
DRINK 24
DRINK 25
DRINK 26
DRINK 27
DRINK 28
DRINK 29
DRINK 30
DRINK 31
DRINK 32

```

```

1  SURROUTINE SHURE(TYPE,SWF,DILU,T,USE,TDUSE) SHURE 2
   CUMMIN W(200),PL,CFS,NSUR,LT,RECU(200),LIST(200,4),LCT,LZ ,CUM,KIT SHURE 3
   +,PUP SHURE 4
5  CUMMIN/DFLIB/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2), SHURE 5
   +EFF(300,K) SHURE 6
   CUMMIN/SURCE/IZ(300),IMASS(300),META(300),NLIBA,NLIBT,NLIBC,NLIBI SHURE 7
   DIMENSION TYPE(3),TDUSE(8),DOSE(200,8) SHURE 8
   DO 8 J=1,8 SHURE 9
8  TDUSE(J)=0.0 SHURE 10
   IF(USE.EP.0.)GO TO 50 SHURE 11
   DO 10 I=1,NSUR SHURE 12
   LM=LIST(I,1) SHURE 13
   M=IZ(LM) SHURE 14
   ARGUMENT(LM)*PL*8760. SHURE 15
   IF(ARGU.GT.100.)JANGU=100 SHURE 16
   TP=TAU(LM)*T SHURE 17
   IF(TP.GT.100)TP=100 SHURE 18
   FACT=1/0000.*(1.093/TAU(LM)/24.)*Q(I)*RECU(I)/CFS/DILU*EXP(-TP)*SWF SHURE 19
   C*(1.-EXP(-ANGU))*USE/CUM SHURE 20
   DO 20 J=1,8 CHANGE1 5
   IF(J.GT.2)GUTU 15 CHANGE1 6
   DOSE(I,J)=FACT*EXG(LM,J) SHURE 22
   TDUSE(J)=TDUSE(J)+DOSE(I,J) CHANGE1 7
   GUTU 20 CHANGE1 8
25  DOSE(I,J)=DOSE(I,2) CHANGE1 9
   20 CONTINUE CHANGE1 10
   10 CONTINUE SHURE 24
   IF(USE.GT.1000. .AND. CON.LT.10.) GO TO 50 SHURE 25
   IF(LCT.GT.0)CALL PERDUS(TYPE,TDUSE,DOSE) SHURE 26
30  50 CONTINUE SHURE 27
   DO 40 J=3,8,1 SHURE 28
   40 TDUSE(J)=TDUSE(2) SHURE 29
   IF(CON.GT.10. .AND. USE.GT.0.)CALL PLOP(DOSE,5) SHURE 30
   RETURN SHURE 31
   END SHURE 32

```

```

1  SUBROUTINE SWIM(TYPE,DILU,T,USE,GEOM,TDUSE)
   COMMON (PNO),PL,CFS,NSUR,LT,RECN(200),LIST(200,4),LCT,LZ ,CON,KIT
   + ,POP
5  COMMON/DFLH/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   +EFF(300,4)
   COMMON/SURCE/IZ(300),IMASS(300),META(300),NLIBA,NLITB,NLITC,NLITD
   DIMENSION TDUSE(8),DUSE(200,8)
   DO 8 J=1,4
8  TDUSE(J)=0.0
   IF(USE.EQ.0.)GO TO 50
   IF(DILU.EQ.0.)GO TO 50
   DO 10 I=1,NSUR
   LM=LIST(I,1)
   MD=IZ(LM)
   ARGU=TAU(LM)*T
   IF(ARGU.GT.20.)GO TO 20
   FACT=1119.*W(1)*RECN(I)/CFS/DILU*EXP(-ARGU)/GEOM*USE/CON
   GO TO 30
20  FACT=0.0
30  DUSE(I,1)=EXS(LM,1)*FACT
   DUSE(I,2)=EXS(LM,2)*FACT
   DO 15 J=3,4
15  DUSE(I,J)=DUSE(I,2)
   TDUSE(1)=TDUSE(1)+DUSE(I,1)
10  TDUSE(2)=TDUSE(2)+DUSE(I,2)
   IF(INT(GEOM).EQ.2) GO TO 50
   IF(USE.GT.1000.)AND(CON.LT.10.) GO TO 50
   IF(LCT.GT.0) CALL PERDUS(TYPE,TDUSE,DUSE)
   IF(CON.GT.10.)CALL PLUP(DUSE,5)
30  CONTINUE
   DO 40 J=3,8,1
40  TDUSE(J)=TDUSE(2)
   RETURN
   END

```

```

1  SUBROUTINE CRITR(CRITR,DILU,T,TDUSE,ACC)
    COMMON G(200),PL,CFS,NSUR,LT,RECO(200),LIST(200,4),LCT,LZ ,LUN,KIT
    + ,PUP
5  COMMON/SOURCE/IZ(300),IMASS(300),META(300),MLIBA,MLIBT,MLIBC,MLIHI
    COMMON/DFLIR/DFL(300,7),DFA(300,7),EXG(300,2),TAUC(300),EXS(300,2),
    +EFF(300,8)
    DIMENSION TDUSE(8),ACC(100),CRITR(3),DOUSE(200,8)
    DO 8 J=1,8
6  TDUSE(J)=0.0
    DO 10 I=1,NSUR
7  LM=LIST(I,1)
    MU=IZ(LM)
    ARGU=TAU(LM)*T
    IF (ARGU.GT.40.)GO TO 9
    FACT=21.*G(I)*RECO(I)/CFS/DILU*EXP(-ARGU)*EFF(LM,2)
    GO TO 11
9  FACT=0.0
    TDUSE(I,1)=FACT*ACC(MU)
10 TDUSE(I,1)=TDUSE(I,1)+DOUSE(I,1)
    IF (LCT.GT.0) CALL PERDOS(CRITR,TDUSE,DOUSE)
    RETURN
    END
    CRITR 2
    CRITR 3
    CRITR 4
    CRITR 5
    CRITR 6
    CRITR 7
    CRITR 8
    CRITR 9
    CRITR 10
    CRITR 11
    CRITR 12
    CRITR 13
    CRITR 14
    CRITR 15
    CRITR 16
    CRITR 17
    CRITR 18
    CRITR 19
    CRITR 20
    CRITR 21
    CRITR 22
    CRITR 23
    
```

```

1  SUBROUTINE EAT(BIOT,RAD,MASS,CONS,DILU,T,TDUSE,ACC)
   COMMON D(200),PL,CFS,NSUR,LT,RECN(200),LIST(200,4),LCT,LZ,CIN,KIT
   +,PIP
5  COMMON/NLIB/DPL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   +EFF(300,8)
   COMMON/SINICE/IZ(300),IMASS(300),META(300),NLIBA,NLIBT,NLIBC,NLIBI
   REAL MASS
   DIMENSION BIOT(4),STAN(9),ACC(100),TDUSE(8),DUSE(200,8)
   DATA STAN/0.,1.,4.,2.,3.,5.,7.,10.,20.,30./
10  TDUSE(1)=0.0
   TDUSE(2)=0.0
   DO 10 J=2,8
   IF(RAD.LE.1.4)GO TO 50
   J1=J-1
   J2=(STAN(J1)+STAN(J))/2.
   J3=J+1
   J4=(STAN(J)+STAN(J3))/2.
   IF(RAD.GE.PT.AND.RAD.LT.TP)GO TO 50
10  CONTINUE
50  LEJ=1
   DO 20 I=1,NSUR
   MD=LIST(I,1)
   MT=IZ(MD)
   ARGU=TAU(MD)*T
25  IF(ARGU.GT.40.)GO TO 9
   IF(EFF(MD,A).EQ.0.)GO TO 9
   FACT=2.44E+07*(I)*RECN(I)/CFS/DILU*EXP(-ARGU)*CONS/MASS/EFF(MD,8)
   +DPL(MD,3)*EFF(MD,L)
   GO TO 8
9  FACT=0.0
8  CONTINUE
   DOSE(I,1)=FACT*ACC(MT)
20  TDUSE(1)=TDUSE(1)+DUSE(I,1)
   IF(LCT.GT.0)CALL PERDUS(BIOT,TDUSE,DOSE)
35  100 CONTINUE
   RETURN
   END

```

```

1  SUBROUTINE PAFD(TYPE,ACC,CUNC,JJ,USE,TDUSE,NN,LM) PAFD 2
   CUMMIN/SOURCE/IZ(300),IMASS(300),META(300),NL10A,NL10T,NL10C,NL10I PAFD 3
   COMMON G(200),PL,CFS,NSUR,LT,RECU(200),LIST(200,4),LCT,LZ,CON,KIT PAFD 4
   +,POP PAFD 5
5  CUMMIN/DEL1H/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2), PAFD 6
   +EFF(300,4) PAFD 7
   DIMENSION TYPE(3),ACC(100),CUNC(200) PAFD 8
   DIMENSION DUSE(200,6),TDUSE(4) PAFD 9
   DO 50 KJ=1,NSUR PAFD 10
   ML=LIST(KJ,1) PAFD 11
   LL=LIST(KJ,JJ) PAFD 12
   MU=IZ(ML) PAFD 13
   FACT=CUNC(KJ)*ACC(MU)/CUH*1100. PAFD 14
   DO 60 J=1,7 PAFD 15
   IF(KJ.EQ.1)TDUSE(J)=0. PAFD 16
   DUSE(KJ,J)=FACT*DFL(LL,J)*USE PAFD 17
   TDUSE(J)=TDUSE(J)+DUSE(KJ,J) PAFD 18
60 CONTINUE PAFD 19
50 CONTINUE PAFD 20
   IF(LM.EQ.0.AND.NN.EQ.1)CALL PLOP(DUSE,6) PAFD 21
   IF(LM.EQ.0.AND.NN.EQ.2)CALL PLOP(DUSE,6) PAFD 22
   IF(LCT.GT.0)CALL PERDUS(TYPE,TDUSE,DUSE) PAFD 23
   RETURN PAFD 24
   END

```

```

1  SUBROUTINE LENT(T,CATH,DILU,J,CONC,AMT,HARV,N)
    COMMON Q(200),PL,CFS,NSUR,LT,REC(200),LIST(200,4),LCT,LZ ,CON,KJT
    + ,PIP
    COMMON/SURCE/IZ(300),IMASS(300),META(300),NLIHA,NLIHT,NLIBC,NLIHI
5  COMMON/DFLIR/DFL(300,7),DFA(300,7),EXG(300,2),TAH(300),EXS(300,2),
    +EFF(300,8)
    DIPENSTON  CATH(20),DILU(20),T(20),CONC(200)
    DJ 20 K=1,J
    DO 40 KK=1,NSUR
10  IF(K.EQ.1)CONC(KK)=0.0
    L=LIST(KK,1)
    MU=IZ(LM)
    ARGUSTAU(LM)*T(K)
15  IF(ARGU.GT.20.)GO TO 40
    IF(N.EQ.2) CONC(KK)=CONC(KK)+CATH(K)*U(KK)*REC(KK)/CFS/DILU(K)*
    +EXP(-ARGU)/HARV
    IF(N.EQ.1)CONC(KK)=CONC(KK)+CATH(K)*Q(KK)*REC(KK)/CFS/DILU(K)*EXP
    +(-ARGU)/AMT
    40 CONTINUE
    20 CONTINUE
    RETURN
    END
    CENT 2
    CENT 3
    CENT 4
    CENT 5
    CENT 6
    CENT 7
    CENT 8
    CENT 9
    CENT 10
    CENT 11
    CENT 12
    CENT 13
    CENT 14
    CENT 15
    CENT 16
    CENT 17
    CENT 18
    CENT 19
    CENT 20
    CENT 21
    CENT 22
    CENT 23

```

```

1  SUBROUTINE TRTIUM(CIYR,P,H3R,H3T)
   COMMON N(200),PL,CFS,NSUP,LT,RECN(200),LIST(200,4),LCT,LZ ,CON,KIT
   + ,PIP
5  COMMON/PDRUL/PERA,PERT,PERC,US
   CUMMUN/DFLID/DFL(300,7),DFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   +FF(300,8)
   DATA HYDRU/2.7E+19/
   DATA CONSUM/800./
   ARGU=TAU(1)*PL*#760.
10  IF(ARGU.GT.40.)ARGU=40.
   H3CON=CIYH*1.0E+12*(1.-EXP(-ARGU))/(TAU(1)*#760.*HYDRU)
   H3EH3CON=CONSUM*DFL(1,3)/CON*US
   H3T=H3CON*CONSUM*DFL(1,4)/CON*US
   RETURN
   END
15

```



```

60      DL=DILU(1)
      DO 107 J=1,M
      IF (DILU(J).LT.DL) DL=DILU(J)
107    IF (DILU(J).LT.DL) TMST(J)
      PRINT 10
      GO 106 J=1,M
106    TTIG=TTIG+PRND(J)
      IF (K7.GT.0)GO TU 160
      GO TU (120,130,140,150),N
      PRINT 85
      RETURN
70      AC=190.
      TC=240.
      CC=200.
      ACIN=520.
      TCIN=630.
      CCIN=520.
      HLD1=140.
      HLD=340.
      GO TU 160
130     AC=30.
      TC=20.
      CC=10.
      ACIN=64.
      TCIN=42.
      CCIN=26.
      HLD=50.
      HLD1=24.
      GO TU 160
140     AC=110.
      TC=200.
      CC=170.
      ACIN=310.
      TCIN=400.
      CCIN=330.
      HLD=96.
      HLD1=48.
      GO TU 160
150     AC=95.
      TC=59.
      CC=37.
      ACIN=110.
      TCIN=65.
      CCIN=41.
      HLD=480.
      HLD1=480.
160     CONTINUE
105     IF (N.EQ.1) PRINT 60,VEG
      IF (N.EQ.2) PRINT 60,LY
      IF (N.EQ.3) PRINT 60,MLK
      IF (N.EQ.4) PRINT 60,MET
      IF (N.GT.4) PRINT 85
      P=TTIG/(AC*PERA+TC*PERT+CC*PERC)
110     IP=TTIG/(AC*PERA+TC*PERT+CC*PERC)
      IF (P.GT.PUP)P=PUP
      KIT=0
      LZ=0
54     FLUDD
60     FLUDD
61     FLUDD
62     FLUDD
63     FLUDD
64     FLUDD
65     FLUDD
66     FLUDD
67     FLUDD
68     FLUDD
69     FLUDD
70     FLUDD
71     FLUDD
72     FLUDD
73     FLUDD
74     FLUDD
75     FLUDD
76     FLUDD
77     FLUDD
78     FLUDD
79     FLUDD
80     FLUDD
81     FLUDD
82     FLUDD
83     FLUDD
84     FLUDD
85     FLUDD
86     FLUDD
87     FLUDD
88     FLUDD
89     FLUDD
90     FLUDD
91     FLUDD
92     FLUDD
93     FLUDD
94     FLUDD
95     FLUDD
96     FLUDD
97     FLUDD
98     FLUDD
99     FLUDD
100    FLUDD
101    FLUDD
102    FLUDD
103    FLUDD
104    FLUDD
105    FLUDD
106    FLUDD
107    FLUDD
108    FLUDD
109    FLUDD
110    FLUDD
111    FLUDD
112    FLUDD
113    FLUDD
114    FLUDD
115    FLUDD

```

```

115 CALL CENT(T,PHOD,DILU,M,CUNC,AMT,TFMG,2)
    CALL FPRD(TYPE,CINC,HOLD,HLD1,ACUN,IRRIG,YIELD,GRUM,TFMG,TTIG,TP
    +,AC,TDUSE,AALD,AAND,N,1,P,TM,DL)
    PRINT 41
    PRINT 40
120 CALL FPRD(TYPE,CUNC,HOLD,HLD1,TCUN,IRRIG,YIELD,GRUM,TFMG,TTIG,TP
    +,TC,TDUSE,TALD,TAND,N,2,P,TH,DL)
    PRINT 60,H,(TDUSE(J),J=1,7)
125 CALL FPRD(TYPE,CUNC,HOLD,HLD1,CCON,IRRIG,YIELD,GRUM,TFMG,TTIG,TP
    +,CC,TDUSE,CALD,CAND,N,3,P,TM,DL)
    PRINT 60,C,(TDUSE(J),J=1,7)
    PRINT 43,DL,TM
    PRINT 42
    PRINT 91
    PRINT 40
130 PRINT 60,A,(AAND(JM),JM=1,7)
    PRINT 60,B,(TAND(JM),JM=1,7)
    PRINT 60,C,(CAND(JM),JM=1,7)
    DU 165 JM=1,7
135 GOTO(JM)=AAND(JM)+TAND(JM)+CAND(JM)
    PRINT 60,D,(GOTO(JM),JM=1,7)
    PRINT 90
    PRINT 40
    PRINT 60,A,(AALD(JM),JM=1,7)
    PRINT 60,B,(TALD(JM),JM=1,7)
    PRINT 60,C,(CALD(JM),JM=1,7)
    DU 166 JM=1,7
166 RAD(JM)=AALD(JM)+TALD(JM)+CALD(JM)
    PRINT 60,U,(RAD(JM),JM=1,7)
    PRINT 96,TYPE,IRRIG,YIELD,GRUM,TFMG,TTIG
    DU 170 J=1,M
170 PRINT 84,DILU(J),PRHD(J),TI(J)
    PRINT 97,ACUN,TCUN,CCUN,HLD1
    PRINT 98,AC,TC,CC,HOLD
    KITS30
    IF(LCT.GT.0) CALL PERDUS(TYPE,TDUSE,DUSE)
    GO TO 100
200 CONTINUE
    RETURN
    END
155

```

FLOOD 116
 FLOOD 117
 FLOOD 118
 FLOOD 119
 FLOOD 120
 FLOOD 121
 FLOOD 122
 FLOOD 123
 FLOOD 124
 FLOOD 125
 FLOOD 126
 FLOOD 127
 FLOOD 128
 FLOOD 129
 FLOOD 130
 FLOOD 131
 FLOOD 132
 FLOOD 133
 FLOOD 134
 FLOOD 135
 FLOOD 136
 FLOOD 137
 FLOOD 138
 FLOOD 139
 FLOOD 140
 FLOOD 141
 FLOOD 142
 FLOOD 143
 FLOOD 144
 FLOOD 145
 FLOOD 146
 FLOOD 147
 FLOOD 148
 FLOOD 149
 FLOOD 150
 FLOOD 151
 FLOOD 152
 FLOOD 153
 FLOOD 154
 FLOOD 155
 FLOOD 156

CARD NR. SEVERITY DETAILS DIAGNOSIS OF PROBLEM
 67 I THERE IS NO PATH TO THIS STATEMENT.

```

1  SUBROUTINE F00D(TYPE,CUNC,HOLD,MLDI,CUNSUM,IRRIG,YIELD,GROW,
   *TMC,TTC,TP,C,TDUSE,ALD,AND,N,JJ,P,TM,DL)
   *CUMMIN(4(200)),PL,CFS,NSUR,LT,RFCU(200),ALIST(200,4),LCT,LZ ,CUM,KIT
   * ,POP
5  CUMMIN/PHIPUL/PEMA,PENT,PEMG,US
   CUMMIN/DFLIH/DEL(300,7),DEFA(300,7),EXG(300,2),TAU(300),EXS(300,2),
   *EFF(300,8)
   CUMMIN/SOURCE/IZ(300),IMASS(300),META(300),NLIBA,NLIBT,NLIBC,NLIBI
10  REAL IPRIG
   DIMENSION ALD(7),AND(7),TYPE(3),TDUSE(6),DUSE(200,6),POOL(200,8)
   DIMENSION CUNC(200)
   DIMENSION SUTL(100),ZMET(100),ZMLK(100)
   DATA Q1,Q2,Q3,Q4/50,60,70,80,90,100,110,120,130,140,150,160,170,180,190,200,210,220,230,240,250,260,270,280,290,300,310,320,330,340,350,360,370,380,390,400,410,420,430,440,450,460,470,480,490,500,510,520,530,540,550,560,570,580,590,600,610,620,630,640,650,660,670,680,690,700,710,720,730,740,750,760,770,780,790,800,810,820,830,840,850,860,870,880,890,900,910,920,930,940,950,960,970,980,990,1000,1010,1020,1030,1040,1050,1060,1070,1080,1090,1100,1110,1120,1130,1140,1150,1160,1170,1180,1190,1200,1210,1220,1230,1240,1250,1260,1270,1280,1290,1300,1310,1320,1330,1340,1350,1360,1370,1380,1390,1400,1410,1420,1430,1440,1450,1460,1470,1480,1490,1500,1510,1520,1530,1540,1550,1560,1570,1580,1590,1600,1610,1620,1630,1640,1650,1660,1670,1680,1690,1700,1710,1720,1730,1740,1750,1760,1770,1780,1790,1800,1810,1820,1830,1840,1850,1860,1870,1880,1890,1900,1910,1920,1930,1940,1950,1960,1970,1980,1990,2000,2010,2020,2030,2040,2050,2060,2070,2080,2090,2100,2110,2120,2130,2140,2150,2160,2170,2180,2190,2200,2210,2220,2230,2240,2250,2260,2270,2280,2290,2300,2310,2320,2330,2340,2350,2360,2370,2380,2390,2400,2410,2420,2430,2440,2450,2460,2470,2480,2490,2500,2510,2520,2530,2540,2550,2560,2570,2580,2590,2600,2610,2620,2630,2640,2650,2660,2670,2680,2690,2700,2710,2720,2730,2740,2750,2760,2770,2780,2790,2800,2810,2820,2830,2840,2850,2860,2870,2880,2890,2900,2910,2920,2930,2940,2950,2960,2970,2980,2990,3000,3010,3020,3030,3040,3050,3060,3070,3080,3090,3100,3110,3120,3130,3140,3150,3160,3170,3180,3190,3200,3210,3220,3230,3240,3250,3260,3270,3280,3290,3300,3310,3320,3330,3340,3350,3360,3370,3380,3390,3400,3410,3420,3430,3440,3450,3460,3470,3480,3490,3500,3510,3520,3530,3540,3550,3560,3570,3580,3590,3600,3610,3620,3630,3640,3650,3660,3670,3680,3690,3700,3710,3720,3730,3740,3750,3760,3770,3780,3790,3800,3810,3820,3830,3840,3850,3860,3870,3880,3890,3900,3910,3920,3930,3940,3950,3960,3970,3980,3990,4000,4010,4020,4030,4040,4050,4060,4070,4080,4090,4100,4110,4120,4130,4140,4150,4160,4170,4180,4190,4200,4210,4220,4230,4240,4250,4260,4270,4280,4290,4300,4310,4320,4330,4340,4350,4360,4370,4380,4390,4400,4410,4420,4430,4440,4450,4460,4470,4480,4490,4500,4510,4520,4530,4540,4550,4560,4570,4580,4590,4600,4610,4620,4630,4640,4650,4660,4670,4680,4690,4700,4710,4720,4730,4740,4750,4760,4770,4780,4790,4800,4810,4820,4830,4840,4850,4860,4870,4880,4890,4900,4910,4920,4930,4940,4950,4960,4970,4980,4990,5000,5010,5020,5030,5040,5050,5060,5070,5080,5090,5100,5110,5120,5130,5140,5150,5160,5170,5180,5190,5200,5210,5220,5230,5240,5250,5260,5270,5280,5290,5300,5310,5320,5330,5340,5350,5360,5370,5380,5390,5400,5410,5420,5430,5440,5450,5460,5470,5480,5490,5500,5510,5520,5530,5540,5550,5560,5570,5580,5590,5600,5610,5620,5630,5640,5650,5660,5670,5680,5690,5700,5710,5720,5730,5740,5750,5760,5770,5780,5790,5800,5810,5820,5830,5840,5850,5860,5870,5880,5890,5900,5910,5920,5930,5940,5950,5960,5970,5980,5990,6000,6010,6020,6030,6040,6050,6060,6070,6080,6090,6100,6110,6120,6130,6140,6150,6160,6170,6180,6190,6200,6210,6220,6230,6240,6250,6260,6270,6280,6290,6300,6310,6320,6330,6340,6350,6360,6370,6380,6390,6400,6410,6420,6430,6440,6450,6460,6470,6480,6490,6500,6510,6520,6530,6540,6550,6560,6570,6580,6590,6600,6610,6620,6630,6640,6650,6660,6670,6680,6690,6700,6710,6720,6730,6740,6750,6760,6770,6780,6790,6800,6810,6820,6830,6840,6850,6860,6870,6880,6890,6900,6910,6920,6930,6940,6950,6960,6970,6980,6990,7000,7010,7020,7030,7040,7050,7060,7070,7080,7090,7100,7110,7120,7130,7140,7150,7160,7170,7180,7190,7200,7210,7220,7230,7240,7250,7260,7270,7280,7290,7300,7310,7320,7330,7340,7350,7360,7370,7380,7390,7400,7410,7420,7430,7440,7450,7460,7470,7480,7490,7500,7510,7520,7530,7540,7550,7560,7570,7580,7590,7600,7610,7620,7630,7640,7650,7660,7670,7680,7690,7700,7710,7720,7730,7740,7750,7760,7770,7780,7790,7800,7810,7820,7830,7840,7850,7860,7870,7880,7890,7900,7910,7920,7930,7940,7950,7960,7970,7980,7990,8000,8010,8020,8030,8040,8050,8060,8070,8080,8090,8100,8110,8120,8130,8140,8150,8160,8170,8180,8190,8200,8210,8220,8230,8240,8250,8260,8270,8280,8290,8300,8310,8320,8330,8340,8350,8360,8370,8380,8390,8400,8410,8420,8430,8440,8450,8460,8470,8480,8490,8500,8510,8520,8530,8540,8550,8560,8570,8580,8590,8600,8610,8620,8630,8640,8650,8660,8670,8680,8690,8700,8710,8720,8730,8740,8750,8760,8770,8780,8790,8800,8810,8820,8830,8840,8850,8860,8870,8880,8890,8900,8910,8920,8930,8940,8950,8960,8970,8980,8990,9000,9010,9020,9030,9040,9050,9060,9070,9080,9090,9100,9110,9120,9130,9140,9150,9160,9170,9180,9190,9200,9210,9220,9230,9240,9250,9260,9270,9280,9290,9300,9310,9320,9330,9340,9350,9360,9370,9380,9390,9400,9410,9420,9430,9440,9450,9460,9470,9480,9490,9500,9510,9520,9530,9540,9550,9560,9570,9580,9590,9600,9610,9620,9630,9640,9650,9660,9670,9680,9690,9700,9710,9720,9730,9740,9750,9760,9770,9780,9790,9800,9810,9820,9830,9840,9850,9860,9870,9880,9890,9900,9910,9920,9930,9940,9950,9960,9970,9980,9990,10000,10010,10020,10030,10040,10050,10060,10070,10080,10090,10100,10110,10120,10130,10140,10150,10160,10170,10180,10190,10200,10210,10220,10230,10240,10250,10260,10270,10280,10290,10300,10310,10320,10330,10340,10350,10360,10370,10380,10390,10400,10410,10420,10430,10440,10450,10460,10470,10480,10490,10500,10510,10520,10530,10540,10550,10560,10570,10580,10590,10600,10610,10620,10630,10640,10650,10660,10670,10680,10690,10700,10710,10720,10730,10740,10750,10760,10770,10780,10790,10800,10810,10820,10830,10840,10850,10860,10870,10880,10890,10900,10910,10920,10930,10940,10950,10960,10970,10980,10990,11000,11010,11020,11030,11040,11050,11060,11070,11080,11090,11100,11110,11120,11130,11140,11150,11160,11170,11180,11190,11200,11210,11220,11230,11240,11250,11260,11270,11280,11290,11300,11310,11320,11330,11340,11350,11360,11370,11380,11390,11400,11410,11420,11430,11440,11450,11460,11470,11480,11490,11500,11510,11520,11530,11540,11550,11560,11570,11580,11590,11600,11610,11620,11630,11640,11650,11660,11670,11680,11690,11700,11710,11720,11730,11740,11750,11760,11770,11780,11790,11800,11810,11820,11830,11840,11850,11860,11870,11880,11890,11900,11910,11920,11930,11940,11950,11960,11970,11980,11990,12000,12010,12020,12030,12040,12050,12060,12070,12080,12090,12100,12110,12120,12130,12140,12150,12160,12170,12180,12190,12200,12210,12220,12230,12240,12250,12260,12270,12280,12290,12300,12310,12320,12330,12340,12350,12360,12370,12380,12390,12400,12410,12420,12430,12440,12450,12460,12470,12480,12490,12500,12510,12520,12530,12540,12550,12560,12570,12580,12590,12600,12610,12620,12630,12640,12650,12660,12670,12680,12690,12700,12710,12720,12730,12740,12750,12760,12770,12780,12790,12800,12810,12820,12830,12840,12850,12860,12870,12880,12890,12900,12910,12920,12930,12940,12950,12960,12970,12980,12990,13000,13010,13020,13030,13040,13050,13060,13070,13080,13090,13100,13110,13120,13130,13140,13150,13160,13170,13180,13190,13200,13210,13220,13230,13240,13250,13260,13270,13280,13290,13300,13310,13320,13330,13340,13350,13360,13370,13380,13390,13400,13410,13420,13430,13440,13450,13460,13470,13480,13490,13500,13510,13520,13530,13540,13550,13560,13570,13580,13590,13600,13610,13620,13630,13640,13650,13660,13670,13680,13690,13700,13710,13720,13730,13740,13750,13760,13770,13780,13790,13800,13810,13820,13830,13840,13850,13860,13870,13880,13890,13900,13910,13920,13930,13940,13950,13960,13970,13980,13990,14000,14010,14020,14030,14040,14050,14060,14070,14080,14090,14100,14110,14120,14130,14140,14150,14160,14170,14180,14190,14200,14210,14220,14230,14240,14250,14260,14270,14280,14290,14300,14310,14320,14330,14340,14350,14360,14370,14380,14390,14400,14410,14420,14430,14440,14450,14460,14470,14480,14490,14500,14510,14520,14530,14540,14550,14560,14570,14580,14590,14600,14610,14620,14630,14640,14650,14660,14670,14680,14690,14700,14710,14720,14730,14740,14750,14760,14770,14780,14790,14800,14810,14820,14830,14840,14850,14860,14870,14880,14890,14900,14910,14920,14930,14940,14950,14960,14970,14980,14990,15000,15010,15020,15030,15040,15050,15060,15070,15080,15090,15100,15110,15120,15130,15140,15150,15160,15170,15180,15190,15200,15210,15220,15230,15240,15250,15260,15270,15280,15290,15300,15310,15320,15330,15340,15350,15360,15370,15380,15390,15400,15410,15420,15430,15440,15450,15460,15470,15480,15490,15500,15510,15520,15530,15540,15550,15560,15570,15580,15590,15600,15610,15620,15630,15640,15650,15660,15670,15680,15690,15700,15710,15720,15730,15740,15750,15760,15770,15780,15790,15800,15810,15820,15830,15840,15850,15860,15870,15880,15890,15900,15910,15920,15930,15940,15950,15960,15970,15980,15990,16000,16010,16020,16030,16040,16050,16060,16070,16080,16090,16100,16110,16120,16130,16140,16150,16160,16170,16180,16190,16200,16210,16220,16230,16240,16250,16260,16270,16280,16290,16300,16310,16320,16330,16340,16350,16360,16370,16380,16390,16400,16410,16420,16430,16440,16450,16460,16470,16480,16490,16500,16510,16520,16530,16540,16550,16560,16570,16580,16590,16600,16610,16620,16630,16640,16650,16660,16670,16680,16690,16700,16710,16720,16730,16740,16750,16760,16770,16780,16790,16800,16810,16820,16830,16840,16850,16860,16870,16880,16890,16900,16910,16920,16930,16940,16950,16960,16970,16980,16990,17000,17010,17020,17030,17040,17050,17060,17070,17080,17090,17100,17110,17120,17130,17140,17150,17160,17170,17180,17190,17200,17210,17220,17230,17240,17250,17260,17270,17280,17290,17300,17310,17320,17330,17340,17350,17360,17370,17380,17390,17400,17410,17420,17430,17440,17450,17460,17470,17480,17490,17500,17510,17520,17530,17540,17550,17560,17570,17580,17590,17600,17610,17620,17630,17640,17650,17660,17670,17680,17690,17700,17710,17720,17730,17740,17750,17760,17770,17780,17790,17800,17810,17820,17830,17840,17850,17860,17870,17880,17890,17900,17910,17920,17930,17940,17950,17960,17970,17980,17990,18000,18010,18020,18030,18040,18050,18060,18070,18080,18090,18100,18110,18120,18130,18140,18150,18160,18170,18180,18190,18200,18210,18220,18230,18240,18250,18260,18270,18280,18290,18300,18310,18320,18330,18340,18350,18360,18370,18380,18390,18400,18410,18420,18430,18440,18450,18460,18470,18480,18490,18500,18510,18520,18530,18540,18550,18560,18570,18580,18590,18600,18610,18620,18630,18640,18650,18660,18670,18680,18690,18700,18710,18720,18730,18740,18750,18760,18770,18780,18790,18800,18810,18820,18830,18840,18850,18860,18870,18880,18890,18900,18910,18920,18930,18940,18950,18960,18970,18980,18990,19000,19010,19020,19030,19040,19050,19060,19070,19080,19090,19100,19110,19120,19130,19140,19150,19160,19170,19180,19190,19200,19210,19220,19230,19240,19250,19260,19270,19280,19290,19300,19310,19320,19330,19340,19350,19360,19370,19380,19390,19400,19410,19420,19430,19440,19450,19460,19470,19480,19490,19500,19510,19520,19530,19540,19550,19560,19570,19580,19590,19600,19610,19620,19630,19640,19650,19660,19670,19680,19690,19700,19710,19720,19730,19740,19750,19760,19770,19780,19790,19800,19810,19820,19830,19840,19850,19860,19870,19880,19890,19900,19910,19920,19930,19940,19950,19960,19970,19980,19990,20000,20010,20020,20030,20040,20050,20060,20070,20080,20090,20100,20110,20120,20130,20140,20150,20160,20170,20180,20190,20200,20210,20220,20230,20240,20250,20260,20270,20280,20290,20300,20310,20320,20330,20340,20350,20360,20370,20380,20390,20400,20410,20420,20430,20440,20450,20460,20470,20480,20490,20500,20510,20520,20530,20540,20550,20560,20570,20580,20590,20600,20610,20620,20630,20640,20650,20660,20670,20680,20690,20700,20710,20720,20730,20740,20750,20760,20770,20780,20790,20800,20810,20820,20830,20840,20850,20860,20870,20880,20890,20900,20910,20920,20930,20940,20950,20960,20970,20980,20990,21000,21010,21020,21030,21040,21050,21060,21070,21080,21090,21100,21110,21120,21130,21140,21150,21160,21170,21180,21190,21200,21210,21220,21230,21240,21250,21260,21270,21280,21290,21300,21310,21320,21330,21340,21350,21360,21370,21380,21390,21400,21410,21420,21430,21440,21450,21460,21470,21480,21490,21500,21510,21520,21530,21540,21550,21560,21570,21580,21590,21600,21610,21620,21630,21640,21650,21660,21670,21680,21690,21700,21710,21720,21730,21740,21750,21760,
```

```

60 IF (J, F0, 3) TERM=PERC
   IF (J, F0, 2) TERM=PERT
   DO 8 J=1,8
     * TDUSE(J)=0.0
     IF (N, EQ, 0) GO TO 50
     TRANS=1.
     DO 10 I=1, NSNR
       M=LIST(I, I)
       MT=I/(MO)
       LL=LIST(I, JJ)
       CNC1=0.0
       ARGU=TAU(MO)*TM
       IF (ARGU, GT, 20.) GO TO 25
       CNC1=0(J)*REC(I)/CFS/UL*EXP(-ARGU)*1100.
25 CONTINUE
       IF (J, EQ, 1) CNC1(1)=CUNC(1)*1100.
       DECAY=0.693/14.*24.*TAU(MO)
       ARGU=DECAY*GRUW
       IF (ARGU, GT, 20.) ARGU=20.
       LEAF=FRAC*TRANS/YIELD*(1.-EXP(-ARGU))/(DECAY*30.)
       ARG1=TAU(MO)*PL*8766.
       IF (ARG1, GT, 20.) ARG1=20.
       ROOT=SUJL(MT)*(1.-EXP(-ARG1))/(TAU(MO)*240.*730.)
       PCUN=CUNC(1)*IRRI*(LEAF*ROOT)
       PCN1=CNC1+IRRI*(LEAF*ROOT)
       IF (MO, EQ, 1) PCN1=CNC1
       IF (MO, EQ, 1) PCN1=CNC1
       ARGU=TAU(MO)*HOLD
       ARG1=TAU(MO)*HOLD1
       IF (ARGU, GT, 60.) ARGU=60.
       IF (ARG1, GT, 60.) ARG1=60.
       IF (N, GT, 2) GO TO 30
       FCN1=PCN1*EXP(-ARG1)
       FCUN=PCN1*EXP(-ARGU)
       GO TO 40
30 IF (N, EQ, 3) FCUN=(01*PCUN+02*CUNC(1))*ZMLK(MT)+EXP(-ARG1)
   IF (N, EQ, 3) FCN1=(01*PCN1+02*CNC1)*ZMLK(MT)+EXP(-ARG1)
   IF (N, EQ, 4) FCUN=(03*PCUN+04*CUNC(1))*ZMET(MT)+EXP(-ARGU)
   IF (N, EQ, 4) FCN1=(03*PCN1+04*CNC1)*ZMET(MT)+EXP(-ARG1)
   IF (N, EQ, 3, AND, MO, EQ, 1) FCUN=(0.0028*CUNC(1)/0.28)*(38.+60.)
   IF (N, EQ, 3, AND, MO, EQ, 1) FCN1=(0.0028*CNC1/0.28)*(38.+60.)
   IF (N, EQ, 4, AND, MO, EQ, 1) FCUN=(0.0041*CUNC(1)/0.32)*(28.+50.)
   IF (N, EQ, 4, AND, MO, EQ, 1) FCN1=(0.0041*CNC1/0.32)*(28.+50.)
   GO TO 40
C
40 DO 60 J=1,7
   IF (I, EQ, 1) ALD(J)=0.
   IF (I, EQ, 1) AND(J)=0.
   DOSE(I, J)=DFL(LL, J)*FCN1*CONSUM
   FACI=TERM*PAC
   FCTI=TERM*TPAC
   POOL(I, J)=DFL(LL, J)*FCUN*FACT/1000.
   POLI=DFL(LL, J)*PCUN*FCTI*TFMG/TTIG/1000.
   ALD(J)=ALD(J)+POOL(I, J)
   AND(J)=AND(J)+POLI
60 TDUSE(J)=TDUSE(J)+DOSE(I, J)
10 CONTINUE

```

```
115      CALL PL0P(P00L,6)
      IF(LCT.GT.0)CALL PERDUS(TYPE,TDUSE,DOSE)
50      CONTINUE
      RETURN
      END
```

```
FOOD 116
FOOD 117
FOOD 118
FOOD 119
FOOD 120
```

```

1 SURROUTINE PERDUS(SPECIE, TIME, DUSE)
COMMON D(200), PL, CFS, NSUK, LT, RECU(200), LIST(200,4), LCT, LZ, CON, MIT
+ , PUP
COMMON/ELEMN/IELEM(100)
COMMON/SORCE/IZ(300), IMASS(300), META(300), PLIHA, NLIHT, NLIHC, NLIHI
DIMENSION SPECIE(3), TDUSE(8), DUSE(200,8), PATH(8,3), SF(12), A1(7,8),
+20, A2(7,8,20), A3(7,8,20), ISUT(7,8,20), IMET(7,8,20), NADS(7,8)
DIMENSION H1(7,8,20), H2(7,8,20), H3(7,8,20)
COMMON/TRANS/DIEM
DIMENSION DUS(8,100,8)
C
DATA SET/MI,IM2,IM3,IM4,IMS,IM6,IM7,IM8,IM9,IM10,IM,IMX/
10 FURMAT(IH0,'PATHWAY THYROID SKIN KIDNEY BONE LUNG LI
+VER TOTAL HUCY THYROID BODY')
15 FURMAT(IH0,'PATHWAY GI-LLI')
DATA BLAK,' /'
110 FURMAT(IH0,'AGE GROUP TOTAL BODY THYROID')
115 FURMAT(IH0,'AGE GROUP SKIN TOTAL BODY')
120 FURMAT(IH0,'* ISOTOPE CONTRIBUTION * LIVER *')
130 FURMAT(IH0,'AGE GROUP HUNE TOTAL
+BODY THYROID KIDNEY LUNG GI-LLI')
IF(LZ.GT.0) GO TO 50
DU 65 J81,7
DU 66 K81,8
DU 80 M81,8
DU 90 I81,20
A1(J,K,I)=BLAK
A2(J,K,I)=BLAK
A3(J,K,I)=BLAK
ISUT(J,K,I)=BLAK
H1(J,K,I)=BLAK
H2(J,K,I)=BLAK
H3(J,K,I)=BLAK
IMET(J,K,I)=BLAK
90 CONTINUE
80 CONTINUE
70 CONTINUE
50 IF(KIT.EQ.40) GO TO 300
IF(KIT.EQ.30) GO TO 300
IF(KIT.EQ.10) GO TO 200
IF(KIT.EQ.50) GO TO 300
IF(KIT.EQ.20) GO TO 300
IF(KIT.EQ.70) GO TO 200
LZ=LZ+1
PATH(LZ,1)=SPECIE(1)
PATH(LZ,2)=SPECIE(2)
PATH(LZ,3)=SPECIE(3)
DU 20 J81,NSUK
DU=LIST(J,I)
NIZ(MI)
INDR = IMASS(MI)/100
IFN = (IMASS(MI)-INDR*100)/10
LUPIT = IMASS(MI)-INDR*100-IFN*10

```

```

        IF (IHUN,GT,0,AND,IITEM,LU,0) ITEM = 10
        IF (IHUN,EG,0,AND,IITEM,EU,0) ITPN = 11
        IF (IHUN,EG,0) IHUN = 11
        IF (IUNIT,EG,0) IUNIT = 10
        C1=SET(IHUN)
        C2=SET(IITEM)
        C3=SET(IUNIT)
        DO 30 JJ=1,6
            LUMADS(LZ,JJ)=101.94*CF9*DIUSE(J,JJ)
        IF (L,GT,20)GOTO 30
        IF (TIMEF(JJ),LT,1,F=10)GOTO 30
        PERDIUSEF(J,JJ)/TIMEF(JJ)*100.
        IF (PEK,LT,1.)GOTO 30
        ITPN=INT(PEK/10.)
        IUNIT=INT(PEK)-ITEM*10
        IF (IITEM,EG,0) ITPN=11
        IF (IUNIT,EG,0) IUNIT=10
        A1(LZ,JJ,L)=SET(IITEM)
        A2(LZ,JJ,L)=SET(IUNIT)
        A3(LZ,JJ,L)=SET(I12)
        ISUT(LZ,JJ,L)=IELEM(MT)
        H1(LZ,JJ,L)=EC1
        H2(LZ,JJ,L)=C2
        H3(LZ,JJ,L)=C3
        IMET(LZ,JJ,L)=META(MI)
        NADS(LZ,JJ)=NADS(LZ,JJ)+1
    30 CONTINUE
    20 CONTINUE
    RETURN
    C * * INDIVIDUAL PERCENTAGE
    200 CONTINUE
    210 FORMAT(1H0,3A4,8(4X,A2,1X,4A1,1X,3A1))
    220 FORMAT(1H ,12X,8(4X,A2,1X,4A1,1X,3A1))
    PRINT 120
    IF (KIT,EG,10) PRINT 10
    IF (KIT,EG,70) PRINT 15
    DO 240 K=1,LZ
        LUMENADS(K,1)
        DO 250 JS=1,M
            IF (NADS(K,JS),GT,LUM) LUM=NADS(K,JS)
        250 CONTINUE
        IF (LUM,GT,20) LUM=20
        PRINT 210,(PATH(K,KL),KL=1,3),(ISUT(K,J,1),B1(K,J,1),B2(K,J,1),B3(
        +K,J,1),IMET(K,J,1),A1(K,J,1),A2(K,J,1),A3(K,J,1),J=1,6)
        DO 260 K=2,LUM
            PRINT 220,(ISUT(K,J,KJ),H1(K,J,KJ),B2(K,J,KJ),IMET(K,J,
        +KJ),A1(K,J,KJ),A2(K,J,KJ),A3(K,J,KJ),J=1,6)
        260 CONTINUE
    240 CONTINUE
        IF (PHI,AGE,0.) IHE TUNN
        PRINT 599
    599 FORMAT(1H1,32X,1'ABLE 4.11-2'/24X,1'LIMUID EFFLUENT DOSE PARAMETERS
        +1/37X,1'A(1)PHRE/HR PER UCI/ML'/16X,1'HADIUNUCLIDE',6X,1'TOTAL BUDY
        +1,5X,1'CALITICAL INGAN'/)
        DO 503 I=1,NSIM
    
```

18 CHANGE1
 19 CHANGE1
 20 CHANGE1
 21 CHANGE1
 22 CHANGE1
 23 CHANGE1
 24 CHANGE1
 25 PERDUS
 26 CHANGE1
 27 CHANGE1
 28 CHANGE1
 29 PERDUS
 30 CHANGE1
 31 PERDUS
 32 CHANGE1
 33 CHANGE1
 34 PERDUS
 35 CHANGE1
 36 PERDUS
 37 PERDUS
 38 PERDUS
 39 PERDUS
 40 PERDUS
 41 PERDUS
 42 PERDUS
 43 PERDUS
 44 PERDUS
 45 PERDUS
 46 PERDUS
 47 PERDUS
 48 PERDUS
 49 PERDUS
 50 PERDUS
 51 PERDUS
 52 PERDUS
 53 PERDUS
 54 PERDUS
 55 PERDUS
 56 PERDUS
 57 PERDUS
 58 PERDUS
 59 PERDUS
 60 PERDUS
 61 PERDUS
 62 PERDUS
 63 PERDUS
 64 PERDUS
 65 PERDUS
 66 PERDUS
 67 PERDUS
 68 PERDUS
 69 PERDUS
 70 CHANGE1
 71 CHANGE1
 72 CHANGE1
 73 CHANGE1
 74 PERDUS
 75 PERDUS
 76 PERDUS
 77 PERDUS
 78 PERDUS
 79 PERDUS
 80 PERDUS
 81 PERDUS
 82 PERDUS
 83 PERDUS
 84 PERDUS
 85 PERDUS
 86 PERDUS
 87 PERDUS
 88 PERDUS
 89 PERDUS
 90 PERDUS
 91 PERDUS
 92 PERDUS
 93 PERDUS
 94 PERDUS
 95 PERDUS
 96 PERDUS
 97 PERDUS
 98 CHANGE1
 99 PERDUS
 100 PERDUS
 101 PERDUS
 102 PERDUS
 103 PERDUS
 104 PERDUS
 105 CHANGE1
 106 CHANGE1
 107 CHANGE1
 108 CHANGE1
 109 CHANGE1
 110 CHANGE1


```

115      MUBLIST(J,1)
      MZIZ(MH)
      NUS1 = DUS(1,J,4) + DUS(2,J,4)
      CDUS = 0
      DO 510 J=2,4
      DUS2 = DUS(1,J,JJ) + DUS(2,J,JJ)
      IF(DUS2.GT.CDUS) CDUS = DUS2
510 CONTINUE
505 PRINT 505,FELEM(MT),IMASS(MH),META(MH),DUS1,CDUS
505 FORMAT(1H,10X,A2,14,A1,9X,1PE4,2,9X,F8.2)
503 CONTINUE
506 PRINT 506
506 FORMAT(//////)
507 PRINT 507,(PATH(2,KL), KL*1,3),DILW
507 FORMAT(2X,3A4,1 DILUTION IN ADDITION TO THAT FOR FISH#,F5.1)
130      C * IRRIGATED FLOODS AND PUD.
      320 FORMAT(1H,12X,2(4X,A2,1X,4A1,1X,3A1))
300 CONTINUE
510 FORMAT(1H,12X,7(4X,A2,1X,4A1,1X,3A1))
380 FORMAT(1H0,'ADULT')
390 FORMAT(1H0,'TEENAGER')
595 FORMAT(1H0,'CHILD')
      PRINT 120
      IF(KIT.EQ.20) PRINT 130
      IF(KIT.EQ.50) PRINT 110
      IF(KIT.EQ.30) PRINT 130
      IF(KIT.EQ.40) PRINT 115
      DO 340 K=1,LZ
      IF(K.EQ.1)PRINT 360
      IF(K.EQ.2)PRINT 390
      IF(K.EQ.3) PRINT 395
      LUM=NADS(K,1)
      DO 350 JS=1,7
      IF(NADS(K,JS).GT.LOW)LUM=NADS(K,JS)
350 CONTINUE
      IF(LUM.GT.20)LUM=20
      DO 360 K=1,L0W
      IF(KIT.EQ.30) PRINT 310,(ISOT(K,J,KJ),B1(K,J,KJ),B2(K,J,KJ),B3(K,J
      +KJ),IMET(K,J,KJ),A1(K,J,KJ),A2(K,J,KJ),A3(K,J,KJ),J=1,7)
      IF(KIT.EQ.20) PRINT 310,(ISOT(K,J,KJ),B1(K,J,KJ),B2(K,J,KJ),B3(K,J
      +KJ),IMET(K,J,KJ),A1(K,J,KJ),A2(K,J,KJ),A3(K,J,KJ),J=2,8)
      IF(KIT.EQ.50)PRINT 320,(ISOT(K,J,KJ),B1(K,J,KJ),B2(K,J,KJ),B3(K,J
      +KJ),IMET(K,J,KJ),A1(K,J,KJ),A2(K,J,KJ),A3(K,J,KJ),J=3,4)
      IF(KIT.EQ.40) PRINT 320,(ISOT(K,J,KJ),B1(K,J,KJ),B2(K,J,KJ),B3(K,J
      +KJ),IMET(K,J,KJ),A1(K,J,KJ),A2(K,J,KJ),A3(K,J,KJ),J=1,2)
360 CONTINUE
340 CONTINUE
      RETURN
      ENDO

```

```

1 SURROUTINE PLUP(DUSE,N)
  INTERMTELYM
  LOGICAL META
  COMMON/SOURCE/IZ(300),IMASS(300),META(300),NL,IHA,NLIHT,NLIHC,NLIHI
  COMMON/ELEMEN/ELEM(100)
  COMMON Q(200),PL,CFS,NSUR,LT,RECU(200),LIST(200),LCT,LZ ,CGR,KIT
  + ,POP
  DIMENS(IN DUSE(200,4),CUREAD(200,4))
10 FORMAT(1H1,20X,'* * * COST-BENEFIT ANALYSIS * * *')
11 FORMAT(1H0,' NUCLEIDE          RELEASE          MAN-REM DOSE-----')
  +-----MAN-REM PER CURIE-----')
12 FORMAT(1H ,13X,'1'  CI/YR      I TOTAL BODY I  THYROID I TOTAL BOD
  +Y I THYROID I')
13 FORMAT(1H ,14,A2,14,A1,' I ',E8.2,' I ',E8.2,' I ',E8.2,
  +I ',E8.2,' I ',E8.2,' I')
14 FORMAT(1H0,' TOTAL',20X,1PER.2,5X,E8.2)
  IF(N.EQ.6,OR,N.EQ.7)GO TO 15
  IF(N.EQ.5)GO TO 20
  IF(N.EQ.3) GO TO 100
  IF(N.EQ.4) GO TO 50
15 DO 30 J=1,NSUR
  CUREAD(J,1)=CUREAD(J,1)+DUSE(J,N=3)
  CUREAD(J,2)=CUREAD(J,2)+DUSE(J,N=2)
30 CONTINUE
  RETURN
20 DO 40 J=1,NSUR
  CUREAD(J,1)=CUREAD(J,1)+DUSE(J,2)
  CUREAD(J,2)=CUREAD(J,2)+DUSE(J,2)
40 CONTINUE
  RETURN
50 DO 60 J=1,200
  CUREAD(J,1)=0.
  CUREAD(J,2)=0.
60 CONTINUE
  TOT=0.0
  TOT=0.0
  RETURN
100 PRINT 10
  PRINT 11
  PRINT 12
  DO 110 J=1,NSUR
  LL=LIST(J,1)
  IK=IZ(LL)
  CI=CUREAD(J,1)/Q(J)
  CIT=CUREAD(J,2)/Q(J)
  PRINT 13,IK,ELEM(IK),IMASS(LL),META(LL),0(J),CUREAD(J,1),CUREAD(J
  +2),CI,CIT
  TOT=TOT+CUREAD(J,1)
  TOT=TOT+CUREAD(J,2)
110 CONTINUE
  PRINT 14,TOT,TOT
  RETURN
  END

```

BLKCK	ADDRESS	LENGTH
/ELEMEN/	100	104
/POPUL/	244	4
HLKDAT	250	0
LADTAP	250	1623
/SOURCE/	2073	1610
/HPLTR/	3703	17644
REDDF	23547	1306
SOURCE	25055	724
RECIN	26001	244
ALARA	26245	1000
PUT	27245	3714
MMO	33161	626
4MY	34007	4516
WATER	40525	637
ACTIVE	41364	513
AQUA	42077	3242
/TRANS/	45341	1
DRINK	45342	3252
SMORE	50614	3311
SWIM	54125	3266
CRITTR	57413	3213
EAT	62626	3264
PAFU	66112	3240
CENT	71352	121
TRTIUM	71473	40
FLUID	71533	1743
FLUID	73476	7410
PEROUS	103106	40467
PLIP	143575	3533
/FCL.C./	147130	23
/J8.10./	147153	134
COUNTRY#	147307	1
COMINE	147310	44
FIF	147354	20
FECMRK#	147374	41
FLTIN#	147435	154
FLTOUT#	147611	314
FHTAP#	150125	372
FORSYS#	150517	556
FURUTL#	151275	16
GETFIT#	151313	43
INCIMP#	151356	257
INPC#	151635	173
KIDERE	152030	467
KWAKEN#	152517	454
OUTCE	153173	171
OUTCOM#	153364	203
GUTDER#	153567	14
ALNG	153603	77
EXP	153702	100
SYSAID#	154002	1
SYS#19T	154003	62

S C U P E 2 L O A D M A P
XTUI# 154065 10
// 154075 2271

LADTAP TEST BECK FUM LIQUID EFFLUENT 1/S IN FRESH WATER

DISCHARGE=2.00E+01 CFS SOURCE TERM MULTIPLIER=1.00E+00

FRESHWATER SITE
TECH SPEC NUCLEIDES

NU RECONCENTRATION OF NUCLEIDES

* * * ADULT DOSE FACTORS * * *

NUCLIDE	CURIE/YEAR	INGESTION DOSE FACTORS (MHREM/PIC INTAKE)										SHORELINE (MHREM/HR)/(PCI/MH*2)		
		BONE	LIVER	TOTAL BODY	HYDROD	KIDNEY	LUNG	GI-LLI	SKIN	TOTAL BODY	MFCIN			
3H	3	1.00E+00	0.	1.34E-07	1.34E-07	1.34E-07	1.34E-07	1.34E-07	1.34E-07	1.34E-07	1.34E-07	0.	0.	1.00E+00
15P	32	1.00E+00	1.93E-04	1.21E-05	7.47E-06	0.	0.	0.	0.	0.	2.17E-05	0.	0.	1.00E+00
24CR	51	1.00E+00	0.	0.	2.66E-09	1.59E-09	5.87E-10	3.53E-09	6.69E-07	2.60E-10	2.20E-10	2.20E-10	2.20E-10	1.00E+00
25FN	54	1.00E+00	0.	4.57E-06	4.73E-07	0.	1.36E-06	0.	1.40E-05	6.40E-09	5.40E-09	1.00E+00	1.00E+00	1.00E+00
26PF	55	1.00E+00	6.20E-06	2.79E-05	7.33E-06	0.	0.	3.23E-05	1.07E-05	0.	0.	1.00E+00	1.00E+00	1.00E+00
26FE	59	1.00E+00	4.34E-06	1.03E-05	3.92E-06	0.	0.	2.86E-06	3.40E-05	9.40E-09	8.00E-09	1.00E+00	1.00E+00	1.00E+00
27CO	58	1.00E+00	0.	7.46E-07	1.67E-06	0.	0.	0.	1.51E-05	8.20E-09	7.00E-09	1.00E+00	1.00E+00	1.00E+00
27CN	60	1.00E+00	0.	2.15E-06	4.72E-06	0.	0.	0.	4.02E-05	2.00E-08	1.70E-08	1.00E+00	1.00E+00	1.00E+00
30ZN	65	1.00E+00	4.65E-06	1.54E-05	6.97E-06	0.	1.03E-05	0.	9.70E-06	4.60E-09	4.00E-09	1.00E+00	1.00E+00	1.00E+00
30ZN	69M	1.00E+00	1.70E-07	4.09E-07	3.73E-08	0.	2.48E-07	0.	2.49E-05	3.40E-09	2.90E-09	1.00E+00	1.00E+00	1.00E+00
37KH	86	1.00E+00	0.	2.11E-05	9.64E-06	0.	0.	0.	4.04E-06	7.20E-10	6.30E-10	1.00E+00	1.00E+00	1.00E+00
38SR	89	1.00E+00	3.09E-04	0.	8.65E-06	0.	0.	0.	1.02E-04	6.50E-13	5.60E-13	1.00E+00	1.00E+00	1.00E+00
38SR	90	1.00E+00	7.61E-03	0.	1.66E-03	0.	0.	0.	1.02E-04	2.60E-12	2.20E-12	1.00E+00	1.00E+00	1.00E+00
39Y	90	1.00E+00	9.63E-04	0.	2.58E-10	0.	0.	0.	7.76E-05	2.70E-11	2.40E-11	1.00E+00	1.00E+00	1.00E+00
39Y	91	1.00E+00	1.41E-07	0.	3.74E-09	0.	0.	0.	3.03E-05	5.40E-09	5.00E-09	1.00E+00	1.00E+00	1.00E+00
40ZR	95	1.00E+00	3.41E-04	2.76E-09	6.61E-09	0.	1.54E-08	0.	1.05E-04	6.40E-09	5.50E-09	1.00E+00	1.00E+00	1.00E+00
40ZR	97	1.00E+00	1.64E-09	3.39E-10	1.56E-10	0.	5.12E-10	0.	2.10E-05	6.00E-09	5.10E-09	1.00E+00	1.00E+00	1.00E+00
41PH	95	1.00E+00	6.23E-09	3.46E-09	1.56E-09	0.	3.43E-09	0.	9.99E-05	2.20E-09	1.90E-09	1.00E+00	1.00E+00	1.00E+00
42MN	90	1.00E+00	0.	4.31E-06	6.20E-07	0.	9.77E-06	0.	2.16E-05	4.20E-09	3.60E-09	1.00E+00	1.00E+00	1.00E+00
44KH	103	1.00E+00	1.65E-07	0.	7.98E-09	0.	7.07E-07	0.	1.78E-04	1.80E-09	1.50E-09	1.00E+00	1.00E+00	1.00E+00
44KH	106	1.00E+00	2.75E-06	0.	3.48E-07	0.	5.32E-06	0.	6.04E-05	2.10E-08	1.40E-08	1.00E+00	1.00E+00	1.00E+00
47AG	110M	1.00E+00	1.60E-07	1.48E-07	6.60E-08	0.	2.91E-07	0.	2.56E-05	2.60E-12	2.30E-12	1.00E+00	1.00E+00	1.00E+00
48CD	113M	1.00E+00	0.	3.19E-06	1.02E-07	0.	3.50E-06	0.	6.33E-05	6.46E-06	0.	1.00E+00	1.00E+00	1.00E+00
50SN	123	1.00E+00	3.11E-05	5.10E-07	7.60E-07	4.36E-07	0.	0.	2.43E-05	1.00E-08	9.00E-09	1.00E+00	1.00E+00	1.00E+00
50SN	126	1.00E+00	8.46E-05	1.64E-06	2.41E-06	4.92E-07	0.	0.	7.95E-05	1.50E-08	1.30E-08	1.00E+00	1.00E+00	1.00E+00
51SR	124	1.00E+00	2.41E-06	5.30E-06	1.11E-06	6.79E-09	0.	0.	1.97E-05	3.50E-09	3.10E-09	1.00E+00	1.00E+00	1.00E+00
51SR	125	1.00E+00	2.23E-06	2.40E-06	4.48E-07	1.98E-09	0.	0.	1.07E-05	4.80E-11	3.50E-11	1.00E+00	1.00E+00	1.00E+00
52TE	125M	1.00E+00	2.64E-06	9.73E-07	3.59E-07	8.07E-07	1.09E-05	0.	2.27E-05	1.30E-12	1.10E-12	1.00E+00	1.00E+00	1.00E+00
52TE	127M	1.00E+00	6.74E-06	2.37E-06	8.26E-07	1.73E-06	2.75E-05	0.	5.79E-05	9.00E-10	7.70E-10	1.00E+00	1.00E+00	1.00E+00
52TE	129M	1.00E+00	1.15E-05	4.30E-06	1.62E-06	3.95E-06	4.80E-05	0.	8.40E-05	9.90E-09	8.40E-09	1.00E+00	1.00E+00	1.00E+00
52TE	131M	1.00E+00	1.74E-06	4.47E-07	7.06E-07	1.34E-06	4.58E-06	0.	7.71E-05	2.00E-09	1.70E-09	1.00E+00	1.00E+00	1.00E+00
52TF	132	1.00E+00	2.53E-05	1.64E-06	1.53E-06	1.80E-06	1.58E-05	0.	1.57E-06	3.40E-09	2.80E-09	1.00E+00	1.00E+00	1.00E+00
53I	131	1.00E+00	4.14E-04	5.96E-06	3.41E-06	1.95E-03	1.02E-05	0.	2.14E-06	4.50E-09	3.70E-09	1.00E+00	1.00E+00	1.00E+00
53I	133	1.00E+00	1.43E-06	2.48E-06	7.57E-07	4.77E-04	4.60E-05	1.59E-05	2.59E-06	1.40E-08	1.20E-08	1.00E+00	1.00E+00	1.00E+00
55CS	134	1.00E+00	6.22E-05	1.48E-04	1.21E-04	0.	0.	0.	2.92E-06	1.70E-08	1.50E-08	1.00E+00	1.00E+00	1.00E+00
55CS	136	1.00E+00	6.51E-06	2.57E-06	1.85E-05	0.	3.71E-05	1.23E-05	4.18E-06	4.90E-09	4.20E-09	1.00E+00	1.00E+00	1.00E+00
55CS	137	1.00E+00	7.48E-05	1.09E-04	7.15E-05	0.	6.68E-09	1.46E-08	9.25E-05	1.70E-08	1.50E-08	1.00E+00	1.00E+00	1.00E+00
56HA	140	1.00E+00	2.03E-05	2.55E-04	1.34E-06	0.	0.	0.	2.42E-05	2.40E-09	2.10E-09	1.00E+00	1.00E+00	1.00E+00
57LA	140	1.00E+00	2.50E-09	1.26E-09	3.34E-10	0.	0.	0.	9.25E-05	1.70E-08	1.50E-08	1.00E+00	1.00E+00	1.00E+00
58CF	141	1.00E+00	9.37E-09	6.34E-09	7.18E-10	0.	2.94E-04	0.	2.42E-05	6.20E-10	5.50E-10	1.00E+00	1.00E+00	1.00E+00
58CF	143	1.00E+00	1.65E-09	1.22E-09	1.35E-10	0.	5.34E-10	0.	4.50E-05	2.50E-09	2.20E-09	1.00E+00	1.00E+00	1.00E+00
58CF	144	1.00E+00	4.89E-07	2.04E-07	2.62E-08	0.	1.21E-07	0.	1.65E-04	3.70E-10	3.20E-10	1.00E+00	1.00E+00	1.00E+00
58PR	143	1.00E+00	9.21E-09	3.70E-09	4.57E-10	0.	2.13E-09	0.	4.03E-05	0.	0.	1.00E+00	1.00E+00	1.00E+00
93NP	239	1.00E+00	1.20E-09	1.18E-10	6.46E-11	0.	3.65E-10	0.	2.40E-05	1.10E-09	9.50E-10	1.00E+00	1.00E+00	1.00E+00

* * * TEENAGER DOSE FACTORS * * *

NUCLIDE	CURIE/YEAR	MUNE	INGESTION DOSE FACTORS (MREM/PCI INTAKE)					SHURELINE (MREM/HR)/(PCI/M**2)		
			LIVER	TOTAL BODY	THYROID	KIDNEY	LUNG	GT-LLI	SKIN	TOTAL BODY
3H	1.00E+00	0.	1.06E-07	1.06E-07	1.06E-07	1.34E-07	1.06E-07	1.06E-07	1.06E-07	
27CO	1.00E+00	0.	9.92E-07	2.20E-06	0.	0.	0.	0.	1.30E-05	
27CO	1.00E+00	0.	2.76E-06	6.30E-06	0.	0.	0.	0.	3.31E-05	
38SR	1.00E+00	4.60E-04	0.	1.32E-05	0.	0.	0.	0.	4.99E-05	
38SR	1.00E+00	1.04E-02	0.	2.57E-05	0.	0.	0.	0.	2.20E-04	
39Y	1.00E+00	3.30E-04	0.	8.87E-10	0.	0.	0.	0.	1.09E-04	
39Y	1.00E+00	1.96E-07	0.	5.23E-09	0.	0.	0.	0.	3.75E-05	
40ZR	1.00E+00	3.72E-08	1.24E-08	8.66E-09	0.	1.54E-08	0.	0.	2.68E-05	
41KA	1.00E+00	7.24E-09	4.36E-09	2.46E-09	0.	3.43E-09	0.	0.	1.78E-05	
44RU	1.00E+00	2.37E-07	0.	1.06E-07	0.	7.07E-07	0.	0.	1.85E-05	
44RU	1.00E+00	4.00E-06	0.	5.03E-07	0.	5.32E-06	0.	0.	1.81E-04	
50SN	1.00E+00	4.38E-05	7.22E-07	1.08E-06	5.78E-07	0.	0.	0.	6.31E-05	
52TE	1.00E+00	3.83E-06	1.37E-06	5.08E-07	1.04E-06	1.09E-05	0.	0.	1.07E-05	
52TE	1.00E+00	1.66E-05	6.15E-06	2.01E-06	5.30E-06	4.80E-05	0.	0.	5.80E-05	
52TE	1.00E+00	3.55E-06	2.22E-06	2.10E-06	2.36E-06	1.58E-05	0.	0.	8.00E-05	
53I	1.00E+00	5.57E-06	7.87E-06	4.69E-06	2.27E-03	1.02E-05	0.	0.	1.49E-06	
53I	1.00E+00	2.03E-06	3.44E-06	1.06E-06	6.25E-04	4.33E-06	0.	0.	2.50E-06	
55CS	1.00E+00	8.05E-05	1.94E-04	9.06E-05	0.	4.80E-05	2.35E-05	2.24E-06	2.24E-06	
55CS	1.00E+00	1.07E-04	1.44E-04	5.05E-05	0.	3.71E-05	1.91E-05	1.92E-06	1.92E-06	
56RA	1.00E+00	2.83E-05	3.48E-08	1.82E-06	0.	8.68E-09	2.33E-08	4.14E-06	4.14E-06	
57LA	1.00E+00	3.48E-09	1.72E-09	4.55E-10	0.	-0.	0.	0.	9.48E-05	
58CF	1.00E+00	1.26E-08	8.46E-09	9.70E-10	0.	2.94E-09	0.	0.	2.29E-05	
58CE	1.00E+00	7.22E-07	2.96E-07	3.83E-08	0.	1.21E-07	0.	0.	1.70E-04	

* * * CHILD DOSE FACTORS * * *

NUCLIDE	CURIE/YEAR	MUNE	INGESTION DOSE FACTORS (MREM/PCI INTAKE)					SHURELINE (MREM/HR)/(PCI/M**2)		
			LIVER	TOTAL BODY	THYROID	KIDNEY	LUNG	GT-LLI	SKIN	TOTAL BODY
3H	1.00E+00	0.	2.03E-07	2.03E-07	2.03E-07	1.34E-07	2.03E-07	2.03E-07	2.03E-07	
27CO	1.00E+00	0.	1.85E-06	5.58E-06	0.	0.	0.	0.	1.10E-05	
27CO	1.00E+00	0.	5.17E-06	1.54E-05	0.	0.	0.	0.	2.86E-05	
38SR	1.00E+00	1.38E-03	0.	3.95E-05	0.	0.	0.	0.	5.15E-05	
38SR	1.00E+00	1.72E-02	0.	4.30E-03	0.	0.	0.	0.	2.29E-04	
39Y	1.00E+00	4.21E-08	0.	1.13E-09	0.	0.	0.	0.	1.20E-04	
39Y	1.00E+00	5.65E-07	0.	1.56E-08	0.	0.	0.	0.	7.77E-05	
40ZR	1.00E+00	1.04E-07	2.42E-08	2.20E-08	0.	1.54E-08	0.	0.	2.50E-05	
41KA	1.00E+00	1.93E-08	8.32E-09	6.11E-09	0.	3.43E-09	0.	0.	1.44E-05	
44RU	1.00E+00	6.78E-07	0.	2.74E-07	0.	7.07E-07	0.	0.	1.78E-05	
44RU	1.00E+00	1.19E-05	0.	1.44E-06	0.	5.52E-06	0.	0.	1.85E-04	
50SN	1.00E+00	1.31E-04	1.64E-06	3.22E-06	1.73E-06	0.	0.	0.	6.50E-05	
52TE	1.00E+00	1.10E-05	3.04E-06	1.52E-06	3.20E-06	1.09E-05	0.	0.	1.10E-05	
52TE	1.00E+00	4.95E-05	1.38E-05	7.65E-06	1.58E-05	4.80E-05	0.	0.	5.96E-05	
52TE	1.00E+00	1.02E-05	4.50E-06	5.42E-06	6.62E-06	1.58E-05	0.	0.	7.89E-05	
53I	1.00E+00	1.63E-05	1.67E-05	1.26E-05	5.43E-03	1.02E-05	0.	0.	1.43E-06	
53I	1.00E+00	5.98E-06	7.38E-06	2.90E-06	1.78E-03	4.33E-06	0.	0.	2.99E-06	
55CS	1.00E+00	2.24E-04	3.77E-04	6.02E-05	0.	4.80E-05	4.19E-05	2.04E-06	2.04E-06	
55CS	1.00E+00	3.12E-04	3.02E-04	4.50E-05	0.	3.71E-05	3.54E-05	1.84E-06	1.84E-06	
56RA	1.00E+00	6.26E-05	7.25E-08	4.85E-06	0.	8.68E-09	4.32E-08	4.21E-06	4.21E-06	
57LA	1.00E+00	1.01E-08	3.52E-09	1.19E-09	0.	-0.	0.	0.	1.00E-04	
58CE	1.00E+00	3.76E-08	1.88E-08	2.80E-09	0.	2.94E-09	0.	0.	2.36E-05	
58CE	1.00E+00	2.14E-06	6.70E-07	1.14E-07	0.	1.21E-07	0.	0.	1.74E-04	

INGESTION DOSE FACTORS SHUMLINK
 (MREM/PCI INTAKE) (MREM/HR)/(PCI/M**2)
 RECON

NUCLIDE	CURIE/YEAR	BONE	LIVER	INTAL HUDY	THYROID	KIDNEY	LUNG	GI-LLI	SKIN	TOTAL HUDY	RECON
3	1.00E+00	0.	3.07E-07	3.07E-07	3.07E-07	1.34E-07	3.07E-07	3.07E-07	0.	0.	0.
27CU	1.00E+00	0.	3.78E-06	4.26E-06	0.	0.	0.	9.79E-06	0.	0.	0.
27CO	1.00E+00	0.	1.07E-05	2.56E-05	0.	0.	0.	2.64E-05	0.	0.	0.
34SP	1.00E+00	2.93E-03	0.	6.42E-05	0.	0.	0.	5.40E-05	0.	0.	0.
34SR	1.00E+00	2.51E-02	0.	6.40E-04	0.	0.	0.	2.43E-04	0.	0.	0.
39Y	1.00E+00	8.97E-08	0.	2.41E-04	0.	0.	0.	1.29E-04	0.	0.	0.
39Y	1.00E+00	1.25E-06	0.	3.33E-08	0.	0.	0.	8.27E-05	0.	0.	0.
40ZR	1.00E+00	2.11E-07	5.32E-08	3.78E-08	0.	1.54E-08	0.	2.38E-05	0.	0.	0.
41NR	1.00E+00	3.49E-08	1.75E-08	1.03E-08	0.	3.43E-09	0.	1.40E-05	0.	0.	0.
44KH	1.00E+00	1.41E-06	0.	4.95E-07	0.	7.07E-07	0.	1.74E-05	0.	0.	0.
44KH	1.00E+00	2.54E-05	0.	3.12E-06	0.	5.32E-06	0.	1.97E-04	0.	0.	0.
50SN	1.00E+00	2.79E-04	4.33E-06	6.88E-06	4.33E-06	0.	0.	6.91E-05	0.	0.	0.
52TF	1.00E+00	2.43E-05	4.19E-06	3.24E-06	8.00E-06	1.09E-05	0.	1.17E-05	0.	0.	0.
52TF	1.00E+00	1.05E-04	3.61E-05	1.60E-05	3.95E-05	4.40E-05	0.	6.33E-05	0.	0.	0.
52TF	1.00E+00	2.13E-05	1.05E-05	9.76E-06	1.55E-05	1.58E-05	0.	8.08E-05	0.	0.	0.
53I	1.00E+00	3.42E-05	4.07E-05	2.34E-05	1.31E-02	1.02E-05	0.	1.53E-06	0.	0.	0.
53I	1.00E+00	1.26E-05	1.44E-05	5.54E-06	4.35E-03	4.33E-06	0.	3.27E-06	0.	0.	0.
55CS	1.00E+00	4.58E-04	8.24E-04	6.97E-05	0.	4.80E-05	9.42E-05	1.96E-06	0.	0.	0.
55CS	1.00E+00	6.53E-04	7.31E-04	4.20E-05	0.	3.71E-05	8.61E-05	1.89E-06	0.	0.	0.
56HA	1.00E+00	1.74E-04	1.75E-07	8.99E-06	0.	8.68E-09	1.07E-07	4.43E-06	0.	0.	0.
57LA	1.00E+00	2.12E-08	4.37E-09	2.16E-09	0.	0.	0.	1.04E-04	0.	0.	0.
58CE	1.00E+00	8.00E-08	4.91E-08	5.75E-09	0.	2.94E-09	0.	2.38E-05	0.	0.	0.
58CE	1.00E+00	4.49E-06	1.77E-06	2.42E-07	0.	1.21E-07	0.	1.85E-04	0.	0.	0.

TOTAL NUMBER IN SOURCE TERM IS 44 TOTAL RELEASE IS 4.4000E+01

* * * AS LOW AS REASONABLY ACHIEVABLE * * *

A D U L T D I S E S

DUSE (MHREM PER YEAR INTAKE)

PATHWAY	SKIN	H/INE	TOTAL BODY	THYROID	KIDNEY	LUNG	GI-LLI
FISH	2.27E+04	2.12E+03	1.45E+03	4.26E+01	3.06E+02	7.50E+01	3.66E+03
DRINKING	3.46E+02	1.65E+01	8.73E+01	8.98E+01	1.02E+01	1.23E+01	6.99E+01
SHORELINE	6.49E+00	6.49E+00	6.49E+00	6.49E+00	6.49E+00	6.49E+00	6.49E+00
TOTAL	2.30E+04	2.14E+03	1.54E+03	1.39E+02	3.23E+02	9.38E+01	3.74E+03

USAGE (KG/YR,HR/YR)	DILUTION	TIME(HR)	SHOREWIDTH FACTOR
FISH	1.0	24.00	1.0
DRINKING	1.0	12.00	1.0
SHORELINE	1.0	-0.00	1.0

* * * ISOTOPE CONTRIBUTION * * *

PATHWAY	SKIN	BONE	LIVER	TOTAL BODY	THYROID	KIDNEY	LUNG	GI-LLI
FISH	P 32 90 126	P 32 95% Zn 65 1% Rb 86 2% Cs 134 16% Cs 136 2% Cs 137 12%	P 32 63% Zn 65 1% Rb 86 2% SH 90 4% Cs 134 19% Cs 136 2% Cs 137 11%	P 32 57% Zn 65 1% Rb 86 1% SH 90 4% Cs 134 19% Cs 136 2% Cs 137 11%	SN 123 3% SN 126 4% TE 127M 1% TE 129M 4% TE 132 1% I 131 73% I 133 .8%	Zn 65 7% TE 125M 1% TE 127M 4% TE 129M 7% TE 132 1% Cs 134 36% Cs 136 10% Cs 137 28%	FE 55 5% CS 134 49% CS 136 5% CS 137 3%	P 32 66% NB 95 19% SN 123 6% SN 126 2%

DRINKING

P 32 2%	SH 90 87%	I 131 84%	Zn 65 4%	FE 55 10%	P 32 1%
SR 89 3%	MN 54 1%	I 133 14%	MU 99 3%	SH 125 77%	FE 59 1%
SR 90 89%	FE 55 6%	CS 134 5%	RU 106 2%	CS 134 5%	CU 60 2%
	FE 59 2%	CS 137 3%	CU 113M 1%	CS 137 4%	SK 89 2%
	Zn 65 3%		TE 125M 4%		SH 90 5%
	Rb 86 5%		TE 127M 11%		Y 90 5%
	TE 129M 1%		TE 129M 19%		Y 91 4%
	I 131 1%		TE 131M 2%		Zn 65 1%
	Cs 134 36%		TE 132 5%		ZR 97 3%
	Cs 136 6%		I 131 3%		NB 95 1%
	Cs 137 27%		I 133 1%		RU 103 1%
			CS 134 19%		RU 106 10%
			CS 136 5%		AB 110M 3%
			CS 137 14%		CD 113M 1%

SHORELINE

MN 54 1%	SN 123 6%	SN 126 34%	SN 125 3%	SN 126 1%	SB 124 4%
CU 60 27%	AG 110M 4%	SH 125 9%	CS 134 8%	SB 125 1%	TE 127M 1%
CU 60 2%	SN 126 31%	CS 134 8%	CS 137 13%	TE 129M 3%	TE 129M 3%
AG 110M 4%	SN 125 3%	CS 134 8%	CS 137 13%	TE 127M 1%	TE 129M 3%
SN 123 6%	SH 125 3%	CS 134 8%	CS 137 13%	TE 127M 1%	TE 129M 3%
SN 126 31%	CS 134 8%	CS 134 8%	CS 137 13%	TE 127M 1%	TE 129M 3%
SH 125 9%	CS 134 8%	CS 134 8%	CS 137 13%	TE 127M 1%	TE 129M 3%
CS 134 8%	CS 137 13%	CS 134 8%	CS 137 13%	TE 127M 1%	TE 129M 3%
CS 137 13%		CS 134 8%	CS 137 13%	TE 127M 1%	TE 129M 3%

TABLE 4.11-2
LIQUID EFFLUENT DOSE PARAMETERS

RADIOISOTOPE	TOTAL BODY A(I), REM/HR PER UCI/ML	CRITICAL ORGAN
H 3	1.14E+01	1.14E+01
P 32	1.71E+06	4.41E+07
CR 51	1.46E+00	3.64E+02
MN 54	9.07E+02	1.45E+04
FE 55	2.36E+03	1.04E+04
FE 59	1.25E+03	1.08E+04
CU 58	3.36E+02	3.04E+03
CU 60	9.58E+02	8.16E+03
ZN 65	3.39E+04	7.49E+04
RN 69M	5.55E+01	3.70E+04
RH 86	4.62E+04	9.91E+04
SR 89	1.36E+03	4.75E+04
SR 90	2.89E+05	1.18E+06
Y 90	3.04E+02	1.22E+04
Y 91	5.37E+01	1.10E+04
ZR 95	5.99E+01	2.75E+03
ZR 97	8.40E+03	5.65E+03
NA 95	9.59E+01	1.48E+06
MO 99	7.55E+01	9.19E+02
KU 103	8.47E+00	2.29E+03
RU 106	3.73E+01	1.91E+04
AG 110M	7.80E+00	5.35E+03
CD 113M	5.74E+01	1.44E+04
SN 123	5.50E+03	4.58E+05
SN 124	1.75E+04	6.15E+05
SH 124	9.45E+01	6.77E+03
SH 125	3.84E+01	2.00E+04
TE 125M	3.70E+02	1.12E+04
TE 127M	8.55E+02	2.85E+04
TE 129M	1.86E+03	5.91E+04
TE 131M	4.33E+02	5.15E+04
TE 132	1.31E+03	6.55E+04
I 131	3.84E+02	2.20E+05
I 133	5.45E+01	3.43E+04
CS 134	5.89E+05	7.21E+05
CS 136	8.55E+04	1.19E+05
CS 137	3.48E+05	5.31E+05
BA 140	1.21E+02	3.77E+03
LA 140	3.59E+02	9.93E+03
CE 141	6.08E+02	2.05E+03
CE 143	8.93E+03	3.02E+03
CE 144	2.24E+00	1.41E+04
PR 143	6.31E+02	5.56E+03
NP 239	5.79E+03	2.15E+03

DRINKING DILUTION IN ADDITION TO THAT FOR FISH 1.0

ADDENDUM

Setpoint Calculations

The radiological effluent Technical Specifications require alarm/trip setpoints for radiation monitors and flow measurement devices for each effluent line. Setpoint values are to be calculated to assure that alarm and trip actions occur prior to exceeding the limits of 10 CFR 20 at the release point to the unrestricted area. The calculated alarm and trip action setpoints to be specified in the ODCM for each radioactive liquid effluent line monitor and flow measurement device must satisfy the following equation:

$$\frac{cf}{F+f} \leq C$$

where:

C = the effluent concentration limit (Specification 3.11.1) implementing 10 CFR 20 for the site, in $\mu\text{Ci/ml}$

c = the setpoint, in $\mu\text{Ci/ml}$, of the radioactivity monitor measuring the radioactivity concentration in the effluent line prior to dilution and subsequent release; the setpoint, which is proportional to the volumetric flow of the effluent line and inversely proportional to the volumetric flow of the dilution stream plus the effluent stream, represents a value which, if exceeded, would result in concentrations exceeding the limits of 10 CFR 20 in the unrestricted area

f = the flow setpoint as measured at the radiation monitor location, in volume per unit time, but in the same units as F, below

F = the dilution water flow setpoint as measured prior to the release point, in volume per unit time.

[Note that if no dilution is provided, $c \leq C$. Also, note that when (F) is large compared to (f), then $F+f \approx F$.]

The equation is satisfied when the following alarm/trip setpoints are provided for each effluent line in the ODCM:

$$f \leq \frac{CF}{c} \quad (\text{in ml/sec; for example}).$$

$$F \leq \frac{cf}{C} \quad (\text{in ml/sec; for example}).$$

$$c \leq \frac{CF}{f} \quad (\text{in } \mu\text{Ci/ml; for example}).$$

Some plants may be operated using a fixed value for one or more of these three variables, c, f or F.

Example 1

By using a constant capacity radwaste system discharge pump (on the undiluted stream) the value of (f) is fixed; therefore, the setpoints to be given in the ODCM are:

$$f = \text{_____ ml/sec (fixed)}$$

$$F > \text{_____ ml/sec} = CF/C$$

$$c = \text{_____} \times F \text{ } \mu\text{Ci/ml} \leq CF/f$$

If $C = 3 \times 10^{-8} \text{ } \mu\text{Ci/ml}$, $f = 4000 \text{ ml/sec}$ and $F > 4 \times 10^6 \text{ ml/sec}$, the radiation monitor setpoint is calculated as follows:

$$c \leq CF/f$$

$$= \frac{(3 \times 10^{-8})F}{4000} = 7.5 \times 10^{-12} F \text{ } \mu\text{Ci/ml}.$$

If F is measured at some value in excess of the limiting value (the limiting value is $4 \times 10^6 \text{ ml/sec}$ in this example), then c may be established proportionately. If $F = 8 \times 10^6 \text{ ml/sec}$, the alarm setpoint is:

$$c = 7.5 \times 10^{-12} F (\mu\text{Ci/ml per ml/sec})(\text{ml/sec})$$

$$= 7.5 \times 10^{-12} (8 \times 10^6) = 6 \times 10^{-5} \text{ } \mu\text{Ci/ml}.$$

In this case, the alarm setpoint for the radioactive liquid effluent line monitor can be established at $6 \times 10^{-5} \text{ } \mu\text{Ci/ml}$, provided that an automatic isolation/control trip action occurs to satisfy the condition:

$$c/F < 7.5 \times 10^{-12} \text{ } \mu\text{Ci/ml per ml/sec}.$$

Example 2

By using a constant capacity dilution pump (on the dilution stream prior to a mixing box), the value of (F) is fixed; therefore, the setpoints to be given in the ODCM are:

$$f < \text{_____ ml/sec} = CF/c$$

$$F = \text{_____ ml/sec (fixed)}$$

$$c = \text{_____} \times (1/f) \mu\text{Ci/ml} \leq CF/f$$

If $C = 3 \times 10^{-8} \text{ } \mu\text{Ci/ml}$, $F = 4 \times 10^6 \text{ ml/sec}$ and $f < 4000 \text{ ml/sec}$, the radiation monitor setpoint is calculated as follows:

$$c \leq CF/f$$

$$= \frac{(3 \times 10^{-8} \times 4 \times 10^6)}{f} = 0.12(1/f) \text{ } \mu\text{Ci/ml}.$$

If f is measured at some value less than the limiting value (the limiting value is 4000 ml/sec in this example), then c may be established proportionately. If $f = 1000 \text{ ml/sec}$, the alarm setpoint is:

$$c = 0.12(1/f)(\mu\text{Ci/sec})(\text{sec/ml})$$

$$= \frac{0.12}{1000} = 1.2 \times 10^{-4} \text{ } \mu\text{Ci/ml}.$$

In this case, the alarm setpoint for the radioactive liquid effluent line monitor can be established at 1.2×10^{-4} $\mu\text{Ci/ml}$, provided that an automatic isolation/control trip action occurs to satisfy the condition:

$$cf > 0.12 \mu\text{Ci/sec.}$$

Value of c

A detailed description of the method to be used to obtain the value of (c) should be provided in the ODCM. Since (c) is dependent on the radionuclide distribution, yields, calibration and the monitor's parameters, each of these variables should be considered and the fixed or adjustable setpoint method of determination described in the ODCM for each effluent monitor. This may be accomplished by tabulation. Changes to the ODCM shall be provided in the SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT.

NRC FORM 335 (7-77)		U.S. NUCLEAR REGULATORY COMMISSION BIBLIOGRAPHIC DATA SHEET		1. REPORT NUMBER (Assigned by DDC) NUREG-0133	
4 TITLE AND SUBTITLE (Add Volume No., if appropriate) "Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants" A Guidance Manual for Users of Standard Technical Specifications				2. (Leave blank)	
7. AUTHOR(S) J. S. Boegli, R. R. Bellamy, W. L. Britz, R. L. Waterfield				3 RECIPIENT'S ACCESSION NO N/A	
9 PERFORMING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code) Division of Site Safety and Environmental Analysis U.S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Washington, D.C. 20555				5. DATE REPORT COMPLETED MONTH YEAR October 1978	
12. SPONSORING ORGANIZATION NAME AND MAILING ADDRESS (Include Zip Code) N/A				DATE REPORT ISSUED MONTH YEAR October 1978	
				6 (Leave blank)	
				8 (Leave blank)	
				10 PROJECT/TASK/WORK UNIT NO N/A	
				11. CONTRACT NO N/A	
13 TYPE OF REPORT N/A		PERIOD COVERED (Inclusive dates) N/A			
15 SUPPLEMENTARY NOTES				14. (Leave blank)	
16. ABSTRACT (200 words or less) This guidance manual provides the NRC staff methodology for calculating parameters for limiting conditions of operation required in the radiological effluent Technical Specifications for light-water-cooled nuclear power plants. It provides guidance in using the model specifications reported in NUREG-0472 (Rev. 1)* and NUREG-0473 (Rev. 1),* applicable to operating PWR and BWR licensees, and users of the Standard Technical Specifications packages available for various vendor designs. The manual addresses the implementation of the Regulations and current NRC staff positions as related to the radioactive waste management systems, effluent control and radiological monitoring programs and provides equations, references, computer codes, and guidelines pertinent to these limiting conditions for operation. *October 1978					
17. KEY WORDS AND DOCUMENT ANALYSIS N/A			17a DESCRIPTORS		
17b. IDENTIFIERS/OPEN-ENDED TERMS					
18 AVAILABILITY STATEMENT Unlimited		19. SECURITY CLASS (This report) 20 SECURITY CLASS (This page)		21 NO. OF PAGES 22 PRICE \$	

