UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931

December 13, 2000

South Carolina Electric & Gas Company
ATTN: Mr. Stephen A. Byrne
Vice President, Nuclear Operations
Virgil C. Summer Nuclear Station
P. O. Box 88
Jenkinsville, SC 29065

SUBJECT: MEETING ANNOUNCEMENT - VIRGIL C. SUMMER NUCLEAR STATION REACTOR COOLANT SYSTEM (RCS) HOT LEG WELD CRACK

Dear Mr. Byrne:

This letter confirms communication between Mr. Melvin Browne of your staff and myself, concerning a meeting which has been scheduled for December 20, 2000, at 1:00 p.m. The purpose of this meeting is to discuss the laboratory analysis results for the flawed weld and any extent of condition. Enclosed is a list of talking points to facilitate this discussion. This list was provided to you by the resident inspector on December 12, 2000. The location of the meeting will be the Sam Nunn Atlanta Federal Center, 61 Forsyth Street, SW, Atlanta, GA 30303, room 24T20 on the 24th floor. This meeting is an open meeting as per "Staff Meeting Open to the Public; Final Policy Statement" (September 20, 1994; 59 FR 48340).

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available **electronically** for public inspection in the NRC Public Document Room **or** from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

Should you have any questions concerning this meeting, please contact me at (404) 562-4605.

Sincerely,

/RA/

Kerry D. Landis, Chief
Reactor Projects Branch 5
Division of Reactor Projects

Docket No. 50-395
License No. NPF-12

Enclosure: Talking Points

cc w/enclosure: See page 2

cc w/enclosure: Continued
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TALKING POINTS

Discuss the root cause of the cracking found in the "A" hot leg nozzle weld.

Discuss the significance of the multiple eddy current (EC) indications found in this weld. This should also include a discussion as to why such cracks would not propagate into the nozzle and/or piping, and the significance of the observed circumferential cracking.

Discuss the significance of the EC indications found in the other 5 nozzle welds, and why it would be acceptable to operate for another cycle with these indications. This should include a discussion of the expected residual stress fields and other relevant parameters that could affect the potential for axial and/or circumferential crack growth.

Discuss the acceptability of the V. C. Summer inservice inspection (ISI) program in light of the apparent inability to detect by ultrasonic (UT) examination the EC indications in the nozzle welds, especially as it applies to other dissimilar weld sites in the reactor coolant system (RCS). This should include your plans for follow-up inspections and/or other activities during future availabilities.

Discuss the adequacy of performing a pre-service UT examination of the replaced weld from the outside diameter (OD) to establish a baseline with the intent to perform subsequent ISI examinations from the inside diameter (ID).

Discuss the adequacy of your leakage detection system, especially as to its ability to detect future similar cracks. If the present system is inadequate, discuss your plans, if any, for enhanced leakage detection procedures or systems for the next cycle of operation.

Enclosure