Mr. William R. Campbell, Jr.
Chief Nuclear Officer and Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: MID-CYCLE PERFORMANCE REVIEW AND INSPECTION PLAN – BROWNS FERRY NUCLEAR STATION UNITS 1, 2 AND 3

Dear Mr. Campbell:

On August 14, 2008, the NRC staff completed its performance review of the Browns Ferry Nuclear Station Units 1, 2 and 3 for the first half of the calendar year 2008 assessment cycle. Our technical staff reviewed performance indicators (PIs) for the most recent quarter and inspection results over the previous twelve months. The purpose of this letter is to inform you of our assessment of your safety performance during this period and our plans for future inspections at your facility.

This performance review and the enclosed inspection plan do not include security information. A separate letter designated and marked as “Official Use Only – Security Related Information” will include the Physical Protection/Security cornerstone performance review and resultant inspection plan.

Plant performance for the most recent quarter, at Browns Ferry Units 1, 2 and 3, was within the Licensee Response Column of the NRC’s Action Matrix, based on all inspection findings being classified as having very low safety significance (Green) and all PIs indicating performance at a level requiring no additional NRC oversight (Green). However for the first quarter of the 2008 year assessment cycle, the Unplanned Scrams per 7000 Critical Hours PI for Unit 1 indicated performance characterized as being of very low safety significance (White) in the Initiating Events Cornerstone. This is an artifact of this Unit 1 PI being of substantial safety significance (Yellow) and Unit 1 being in the Degraded Cornerstone Column of the NRC’s Action Matrix for the fourth quarter of the 2007 year assessment cycle. Your staff has informed us of your readiness after September 8, 2008, for the NRC to conduct a supplemental inspection to review the actions taken to address the Unit 1 PI being of substantial safety significance (Yellow). Therefore, in addition to the normal reactor oversight process (ROP) baseline inspections at your facility, we plan to conduct a Supplemental Inspection in accordance with Inspection Procedure 95002 in December 2008 for the previous Yellow PI. Furthermore, we also plan on conducting several non-ROP inspections, which involve: emergency response organization drill/exercise performance indicator program review (Temporary Instruction (TI) 2515/175); emergency diesel generator technical specification requirements (TI 2515/176); operator licensing examinations; and the Independent Spent Fuel Storage Installation. In addition,
TI 2515/173, Review of the Implementation of the Industry Ground Water Protection Voluntary Initiative, will be performed at your facility prior to August 30, 2010. In the near term, we will contact you to assess your readiness for this inspection and then finalize our schedule accordingly.

In our 2007 Annual Assessment Letter, dated March 3, 2008, a substantive cross-cutting issue was identified in the area of Problem Identification and Resolution (PI&R) with a cross-cutting theme in the aspect of appropriate and timely corrective action (P.1(d)). Criteria established for clearing the substantive cross-cutting issue were: during 2008, no greater-than-green findings or a small number of findings associated with (P.1(d)), and evidence of improvements as described in the licensee’s Turn Around Plan and Problem Evaluation Report (PER)136489. In the first two quarters of the current assessment cycle, no findings were identified with an aspect of P.1(d). Your staff has informed us of your readiness for our evaluation of your improvements described in your Turn Around Plan and PER 136489 after September 8, 2008. We plan to inspect these improvements during the biennial P&IR inspection scheduled for September and October 2008. Until this inspection is satisfactorily completed and the other criteria met, this substantive cross-cutting issue remains open.

The enclosed inspection plan details the inspections, less those related to physical protection, scheduled through December 31, 2009. The inspection plan is provided to minimize the resource impact on your staff and to allow for the resolution of any scheduling conflicts and personnel availability issues well in advance of inspector arrival onsite. Routine resident inspections are not listed due to their ongoing and continuous nature. The inspections in the last nine months of the inspection plan are tentative and may be revised at the end-of-cycle review meeting. In accordance with 10 CFR 2.390 of the NRC's “Rules of Practice,” a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

If circumstances arise which cause us to change the inspection plan, we will contact you to discuss the change as soon as possible. Please contact Mr. Eugene Guthrie at (404) 562-4662 with any questions you may have regarding this letter or the inspection plan.

Sincerely,

/RA/
Leonard D. Wert, Director
Division of Reactor Projects

Docket Nos.: 50-259, 50-260, 50-296
License Nos.: DPR-33, DPR-52, DPR-68

Enclosure: Browns Ferry Inspection/Activity Plan

cc w/encl: (See page 3)
cc w/encl:
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Kirksey E. Whatley  
Director  
Office of Radiation Control  
Alabama Dept. of Public Health  
Electronic Mail Distribution

(cc w/encl cont’d – see page 4)
cc w/encl cont'd:
Steven M. Douglas
General Manager
Browns Ferry Site Operations
Electronic Mail Distribution
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